

Program EVALPLOT
(Version 2018-1)

by

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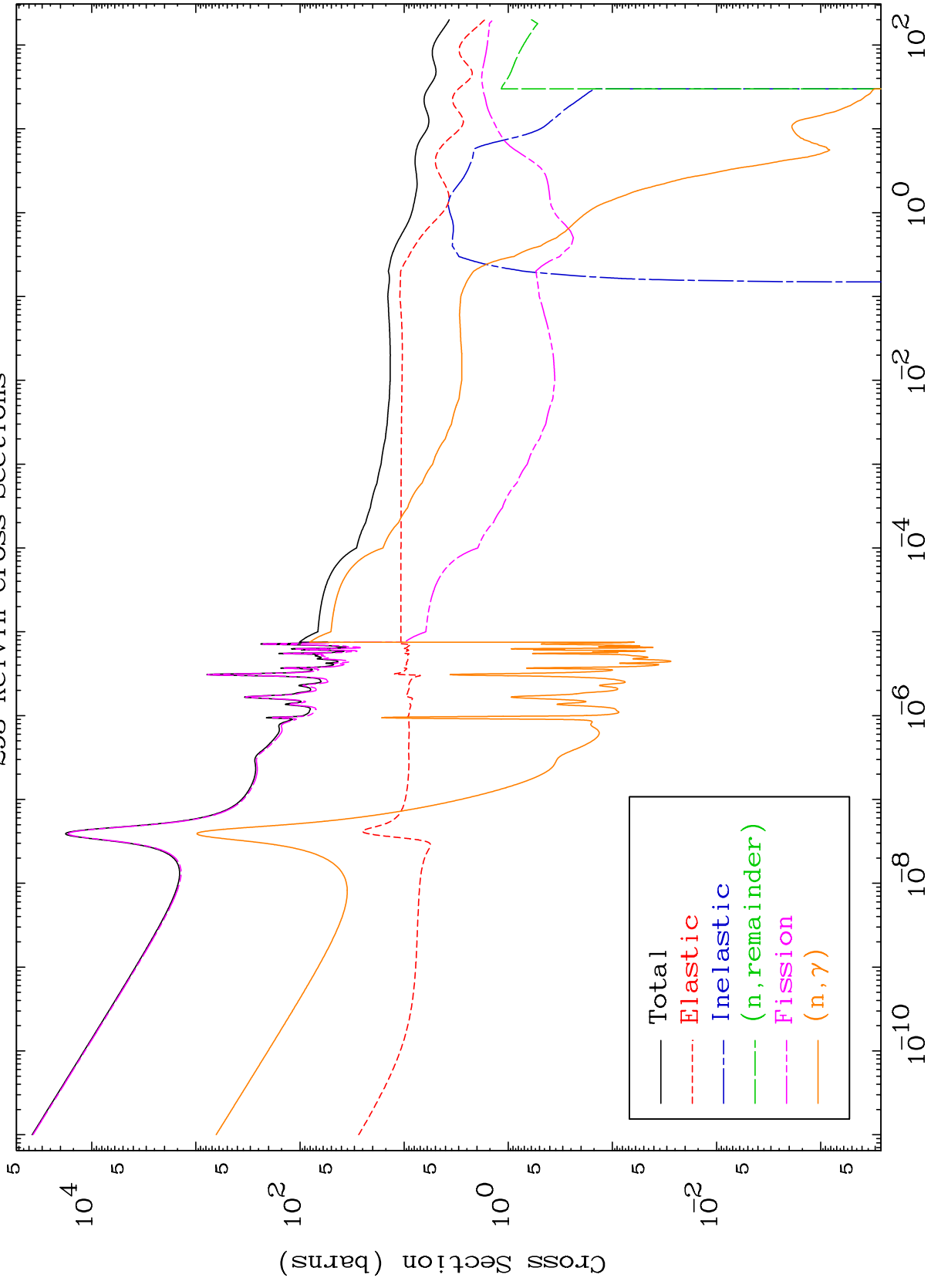
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9957

Major
293 Kelvin Cross Sections

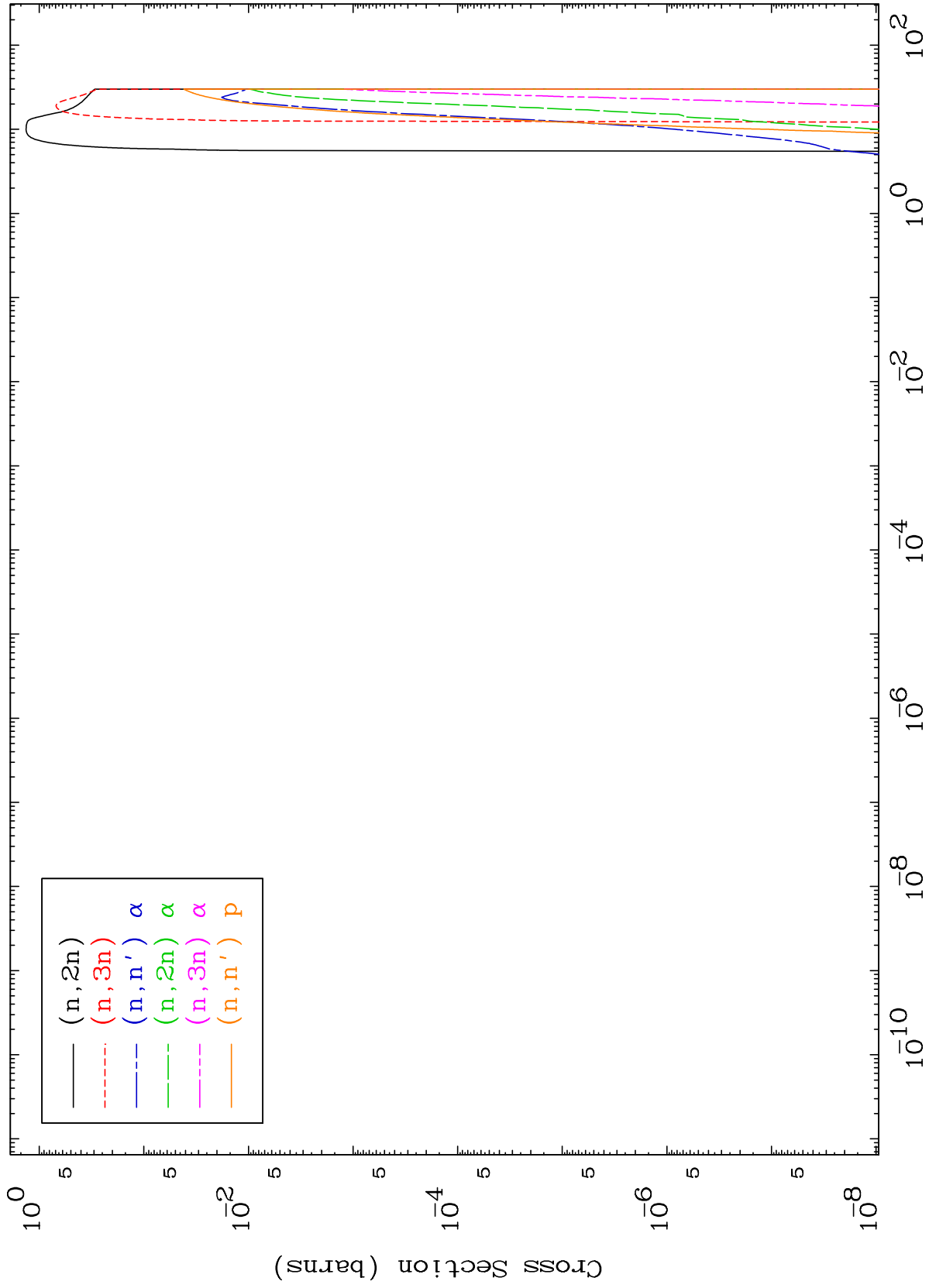
101-Md-256



MAT 9957

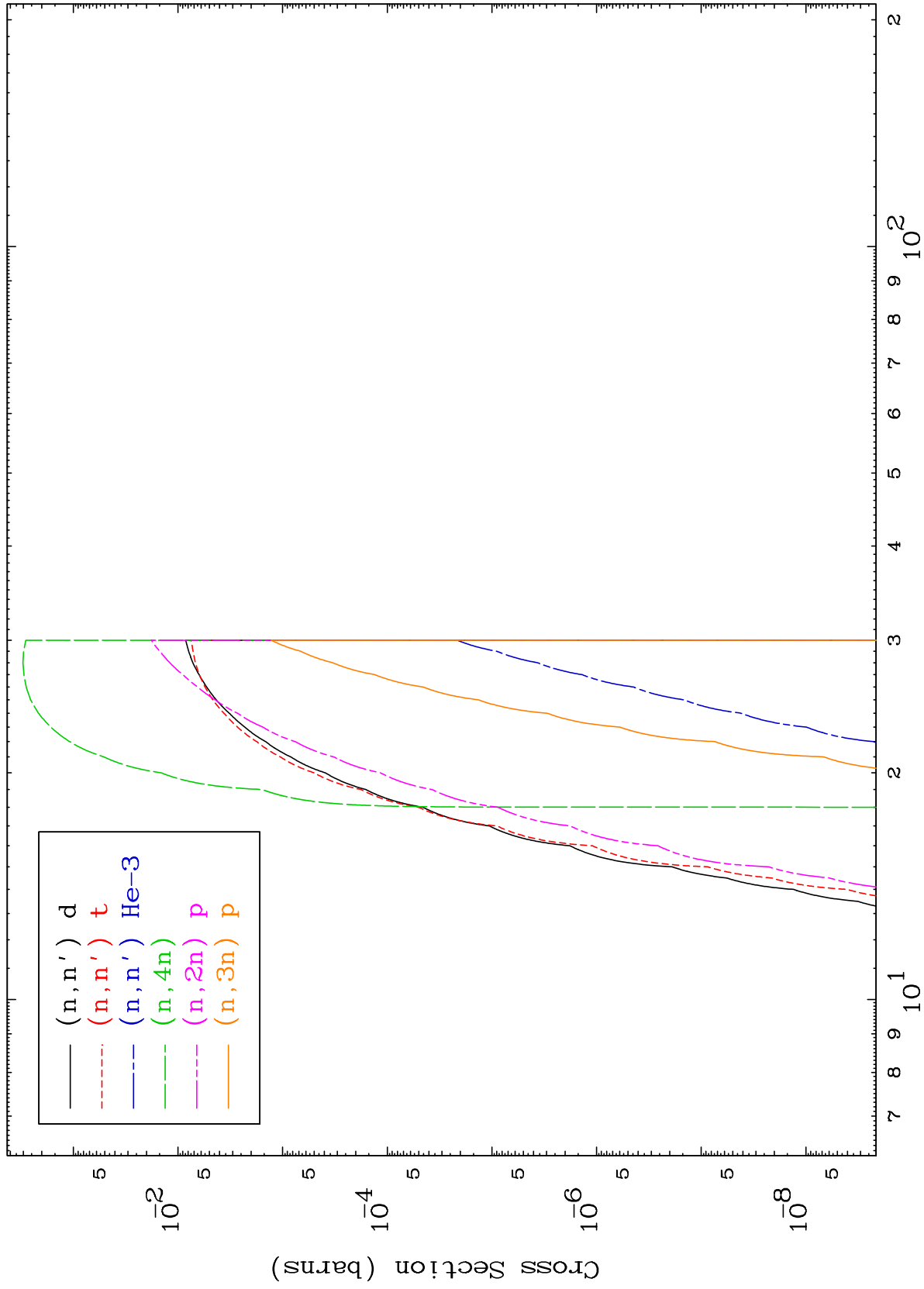
Neutron Production
293 Kelvin Cross Sections

101-Md-256



2

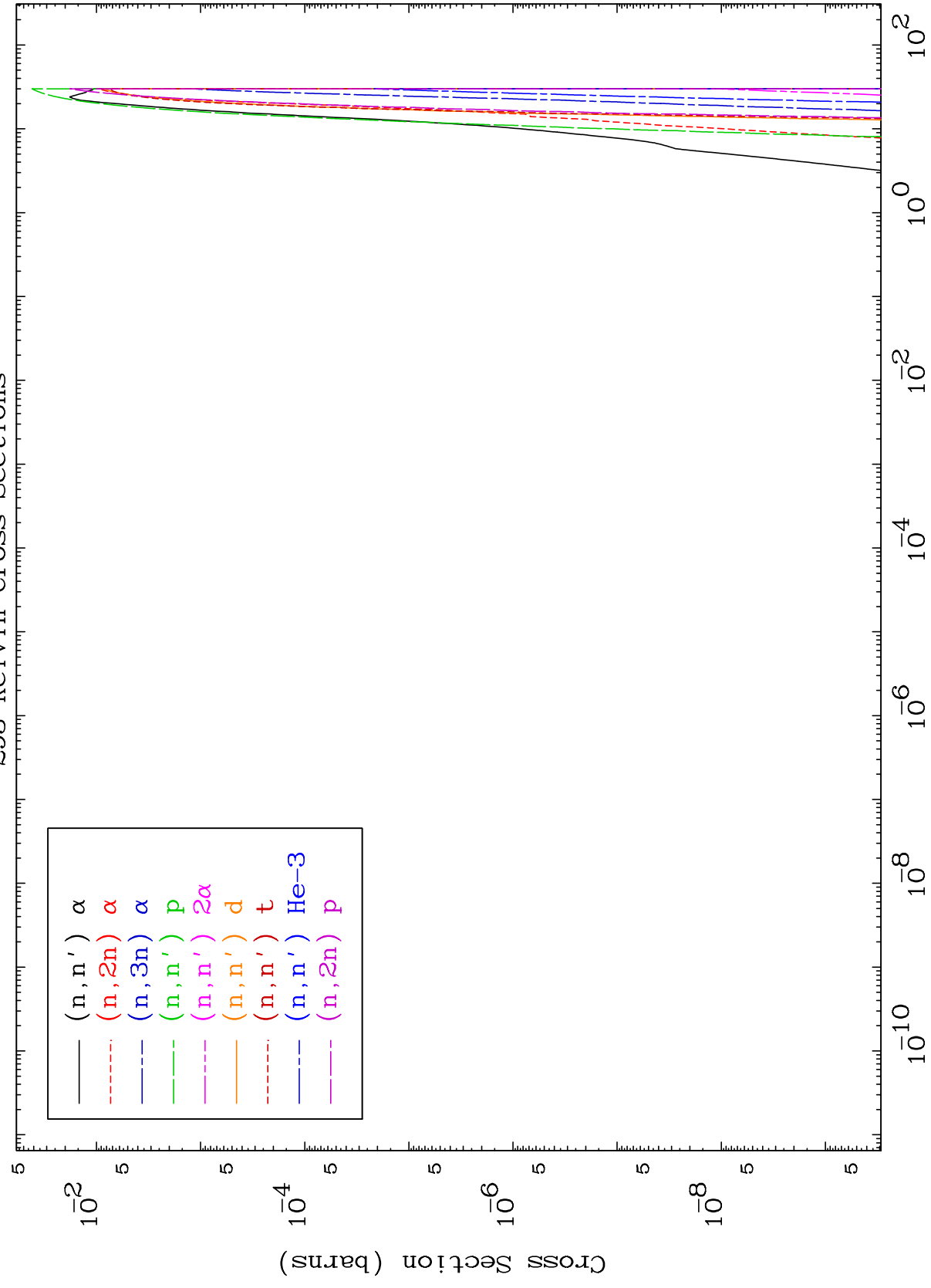
101-Md-256



MAT 9957

293 Kelvin Cross Sections
Charged Particle

101-Md-256



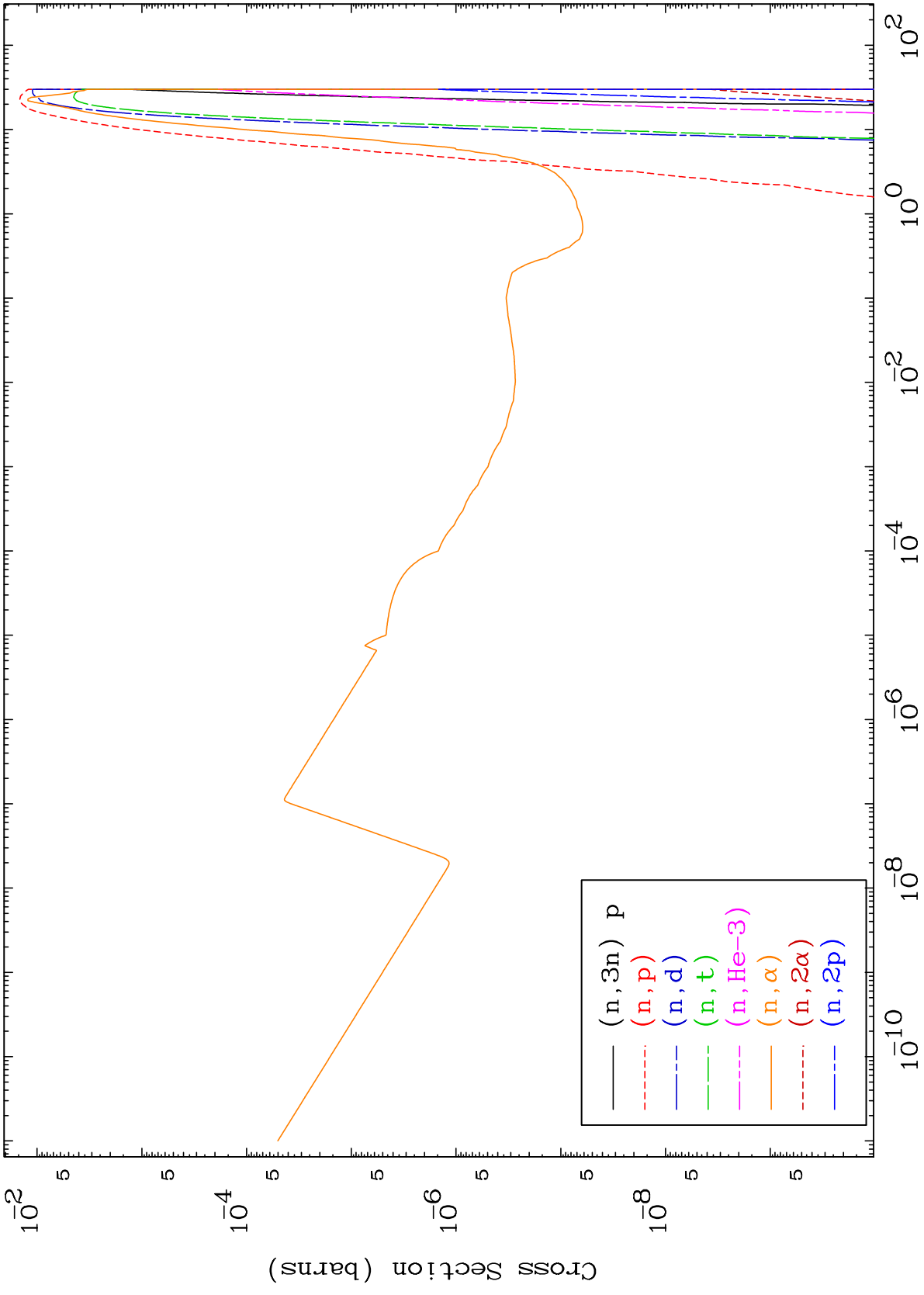
101-Md-256

Incident Energy (MeV)

MAT 9957

Charged Particle
293 Kelvin Cross Sections

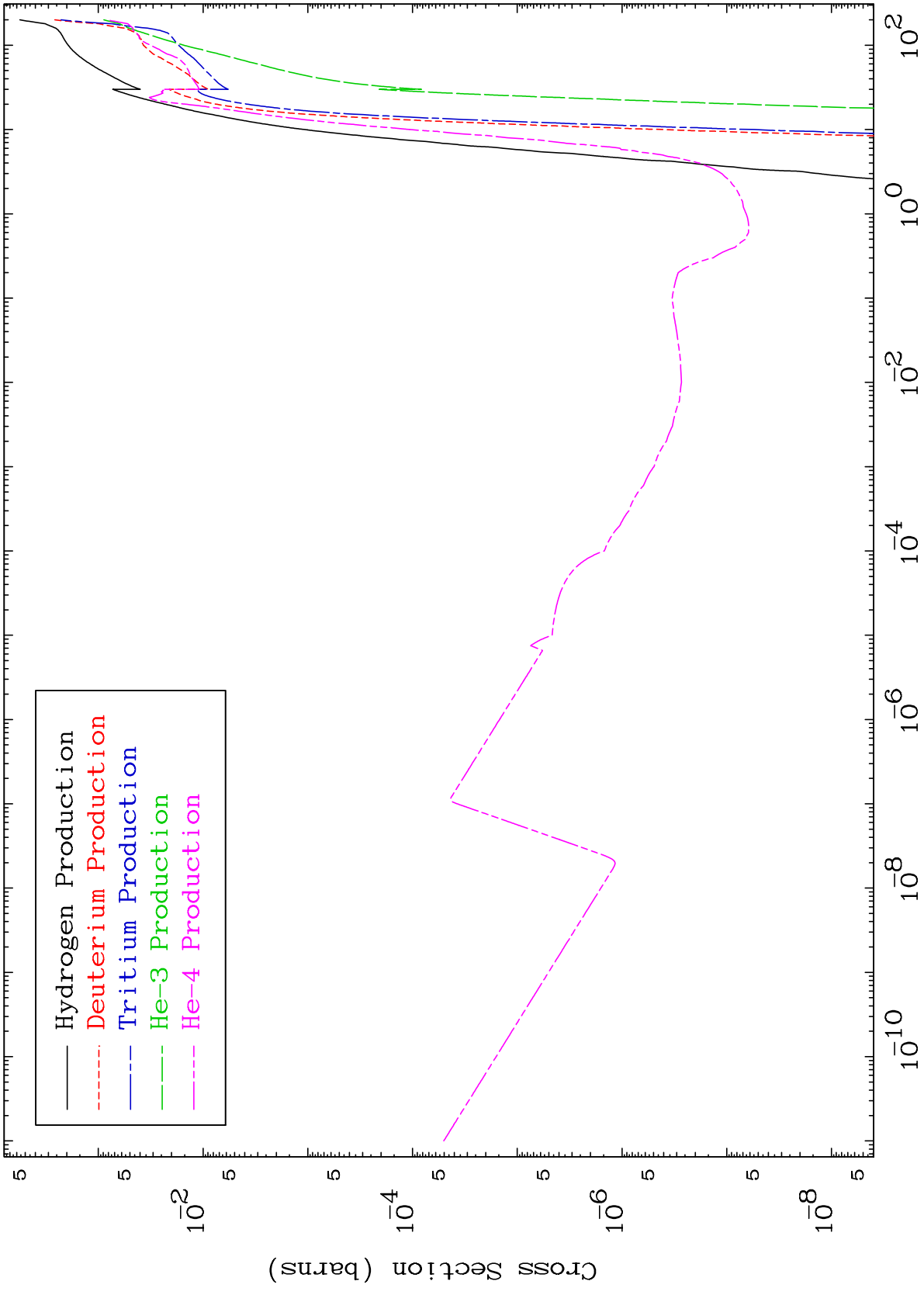
101-Md-256



MAT 9957

Particle Production
293 Kelvin Cross Sections

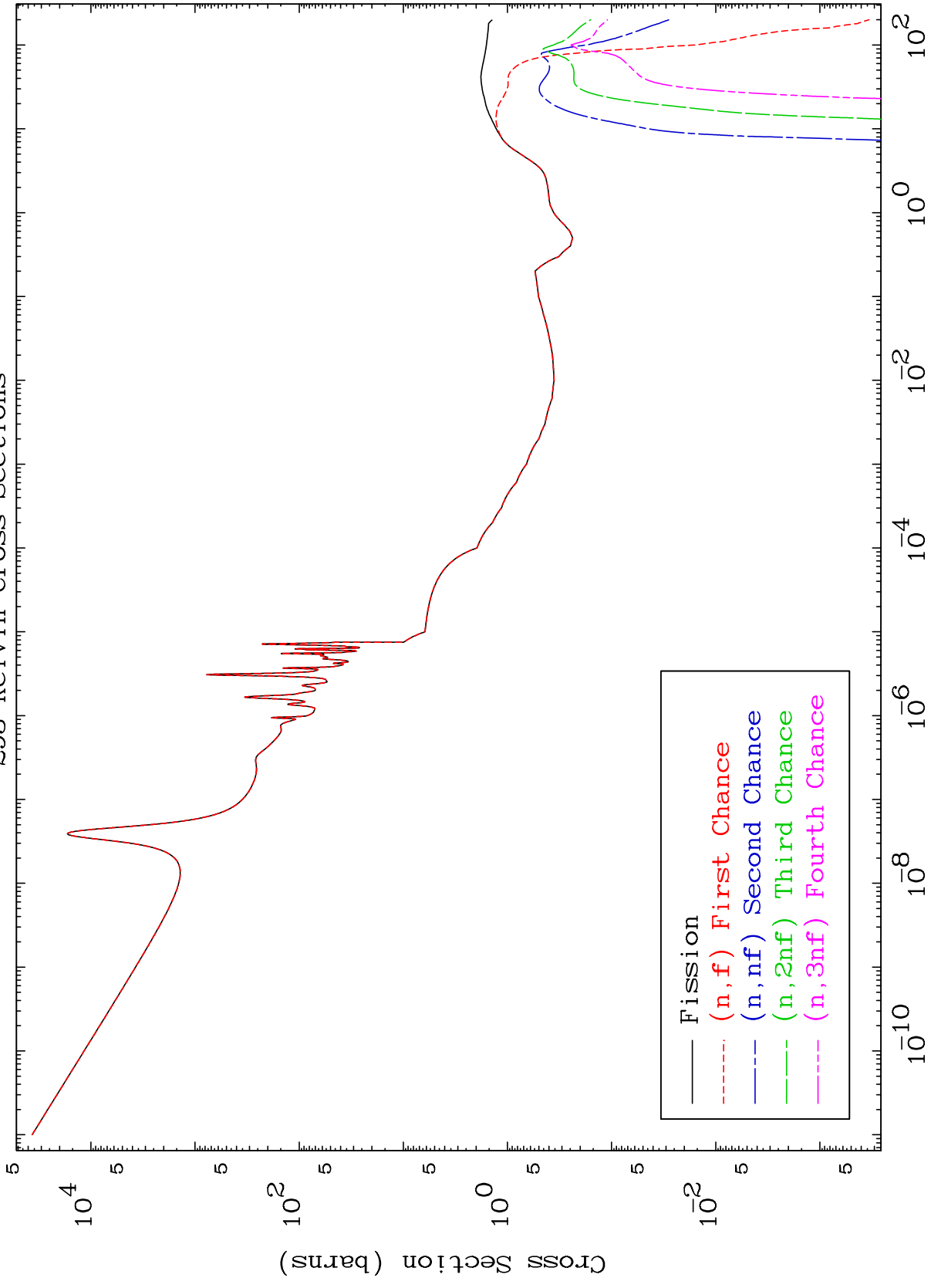
101-Md-256



MAT 9957

Fission
293 Kelvin Cross Sections

101-Md-256



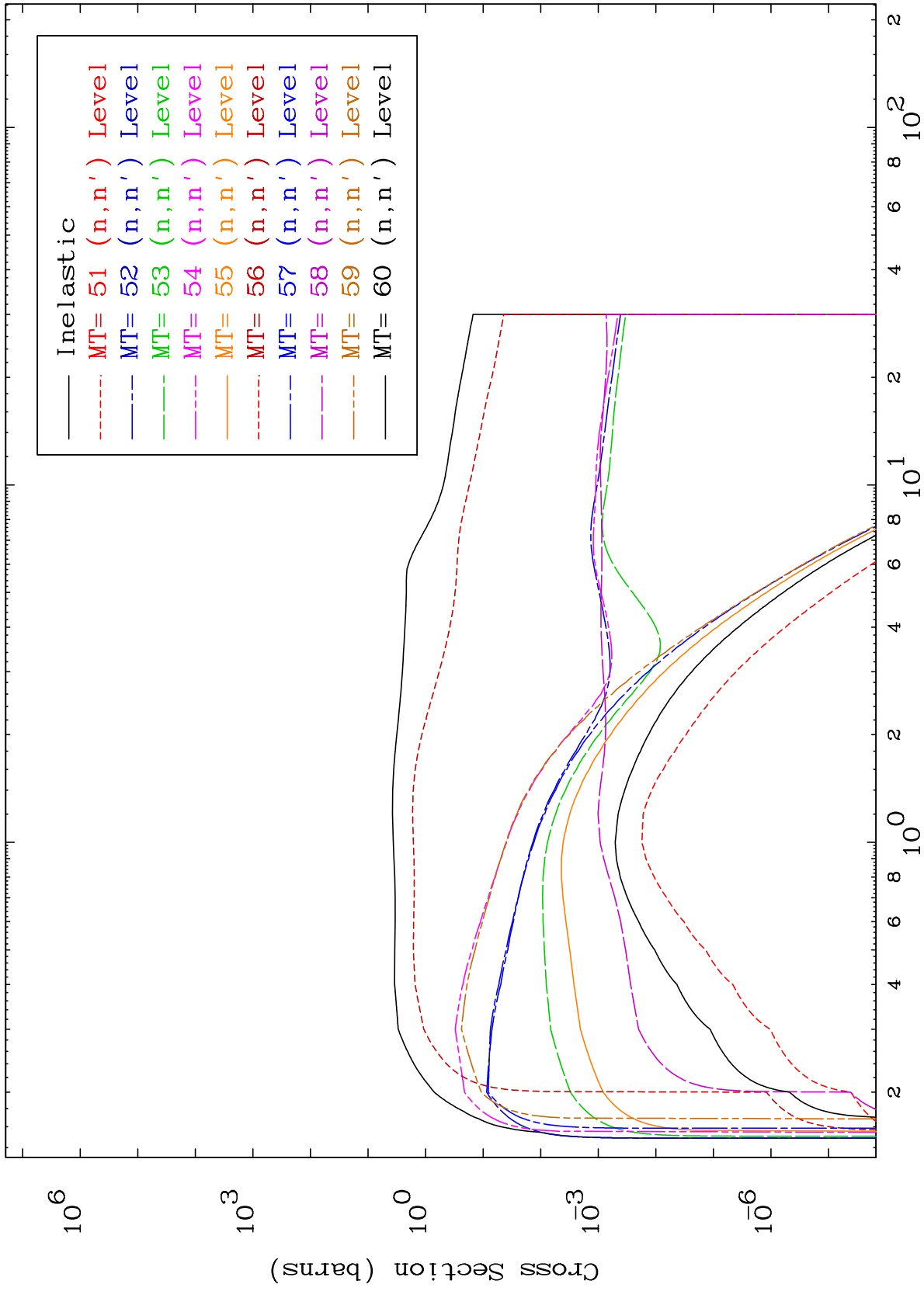
101-Md-256

MAT 9957

(n,n') Level

101-Md-256

293 Kelvin Cross Sections



8

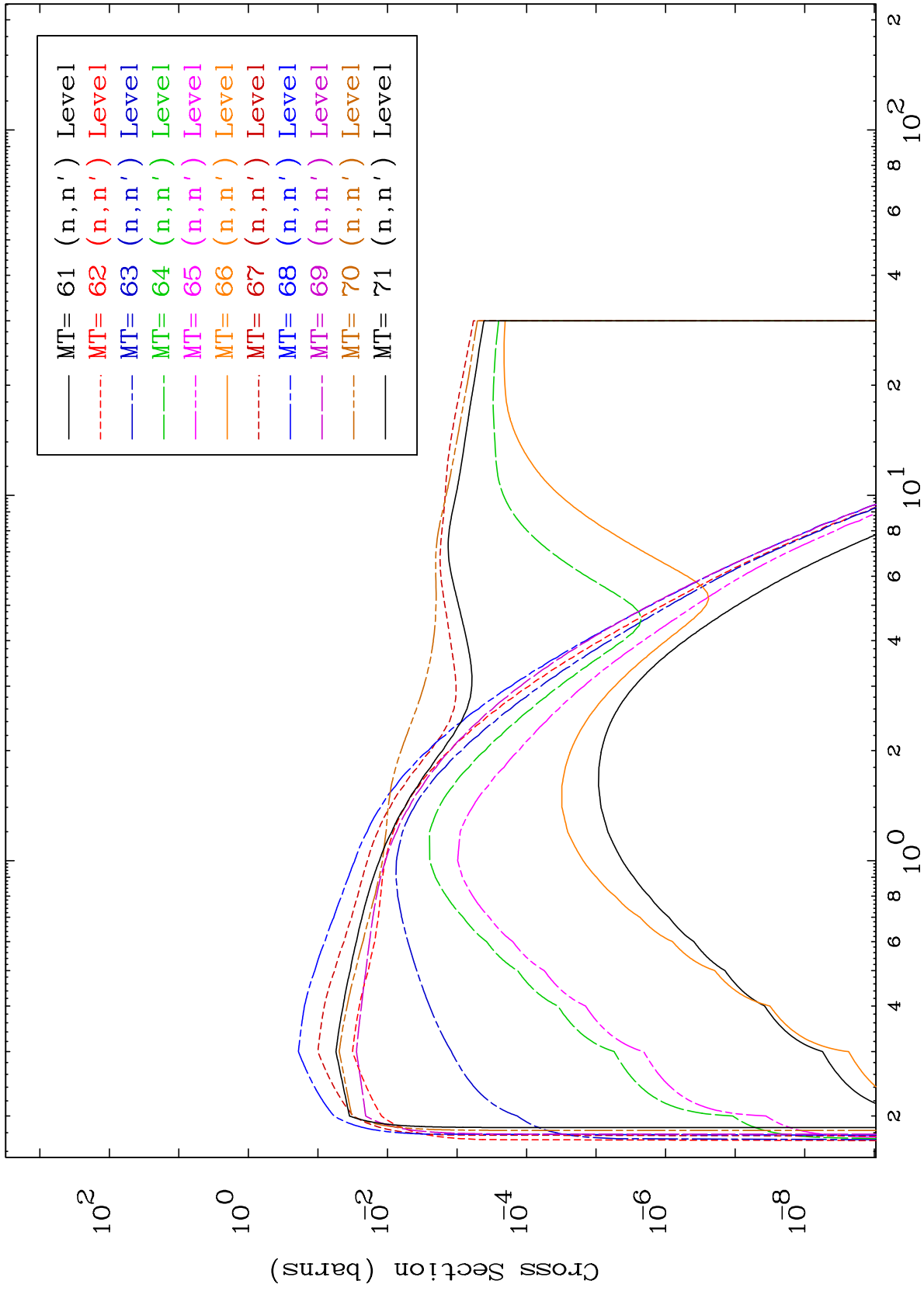
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,n') Level
293 Kelvin Cross Sections

101-Md-256



9

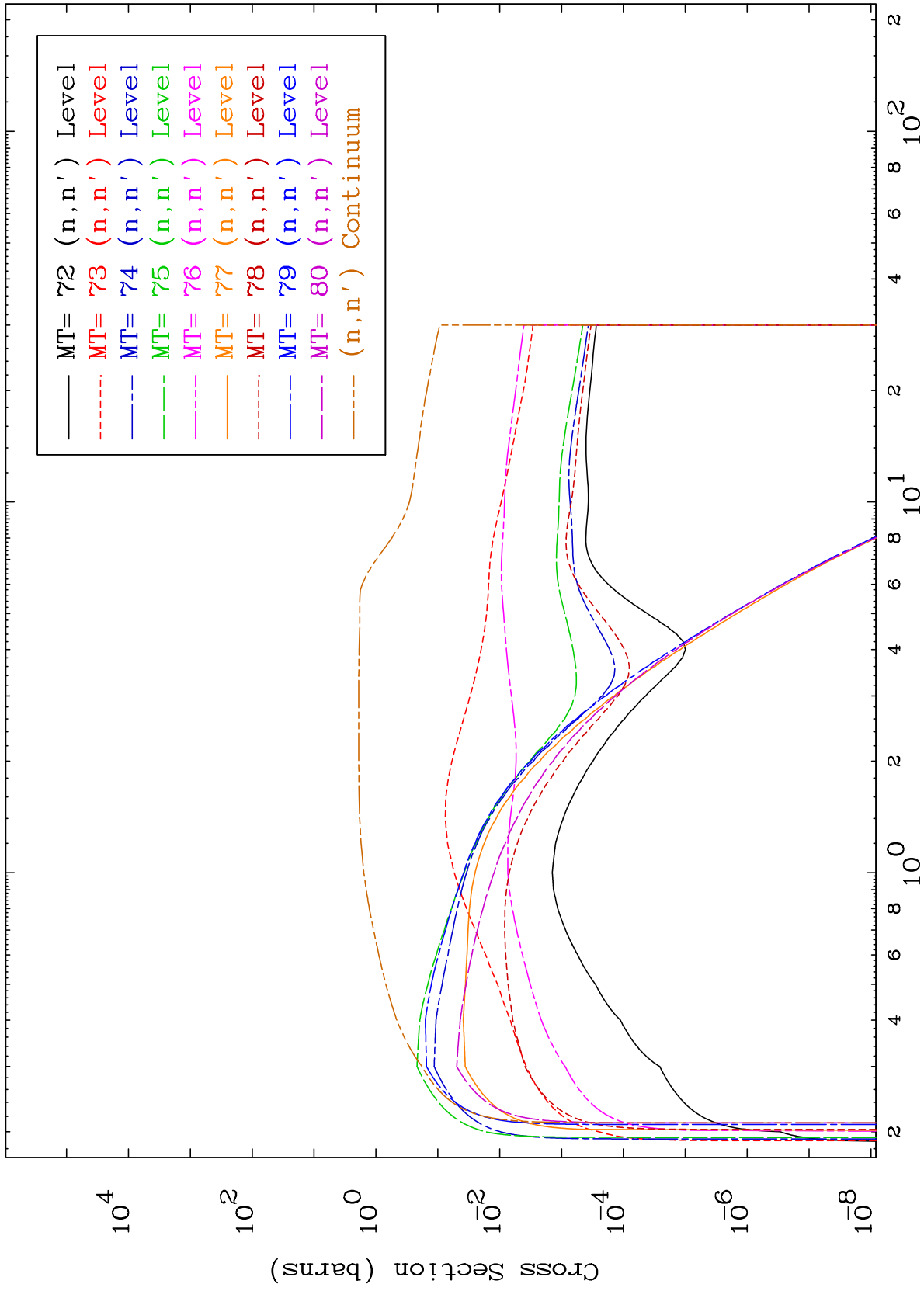
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,n') Level
293 Kelvin Cross Sections

101-Md-256



10

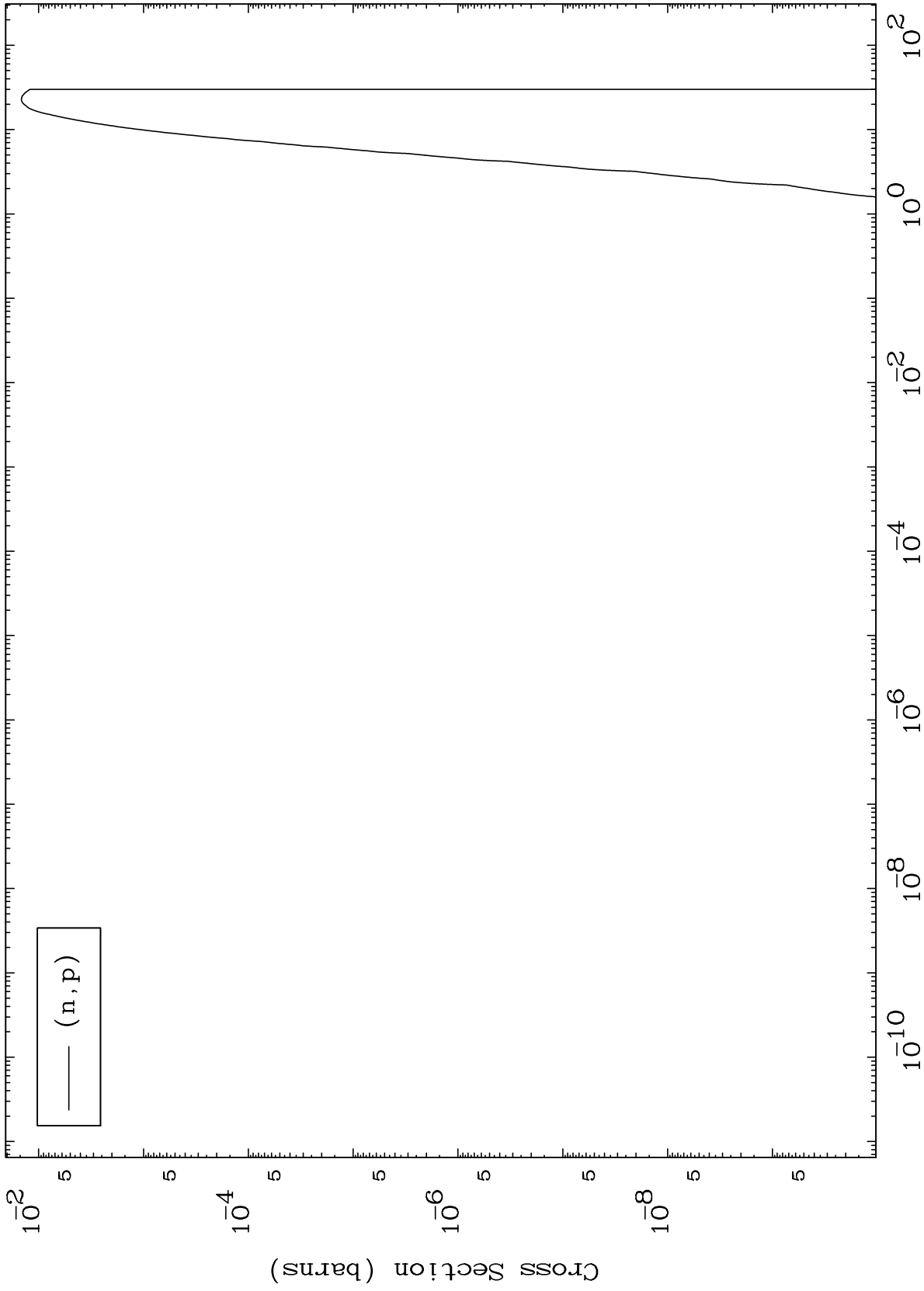
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,p) Levels
293 Kelvin Cross Sections

101-Md-256

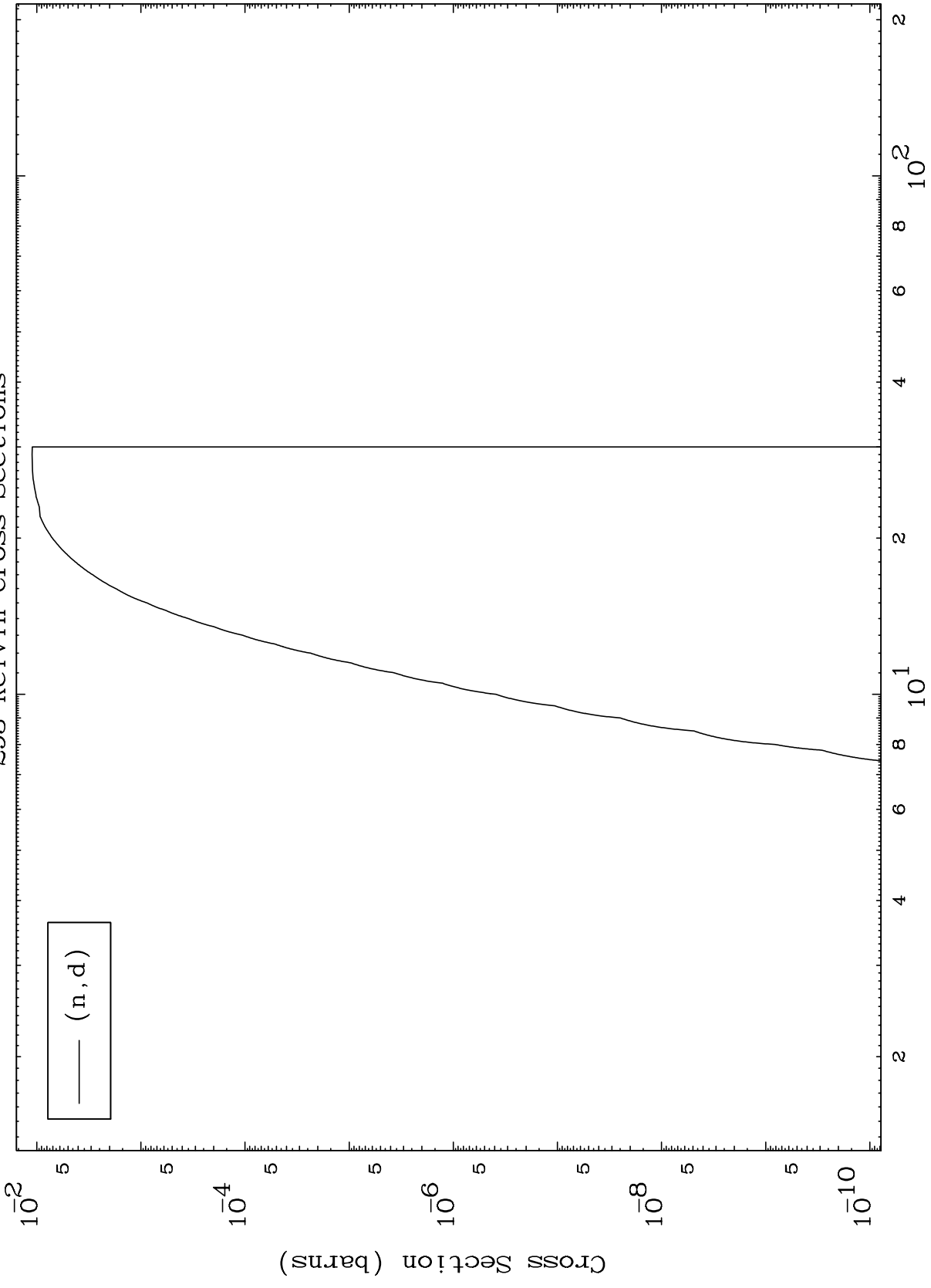


101-Md-256

MAT 9957

(n,d) Levels
293 Kelvin Cross Sections

101-Md-256



12

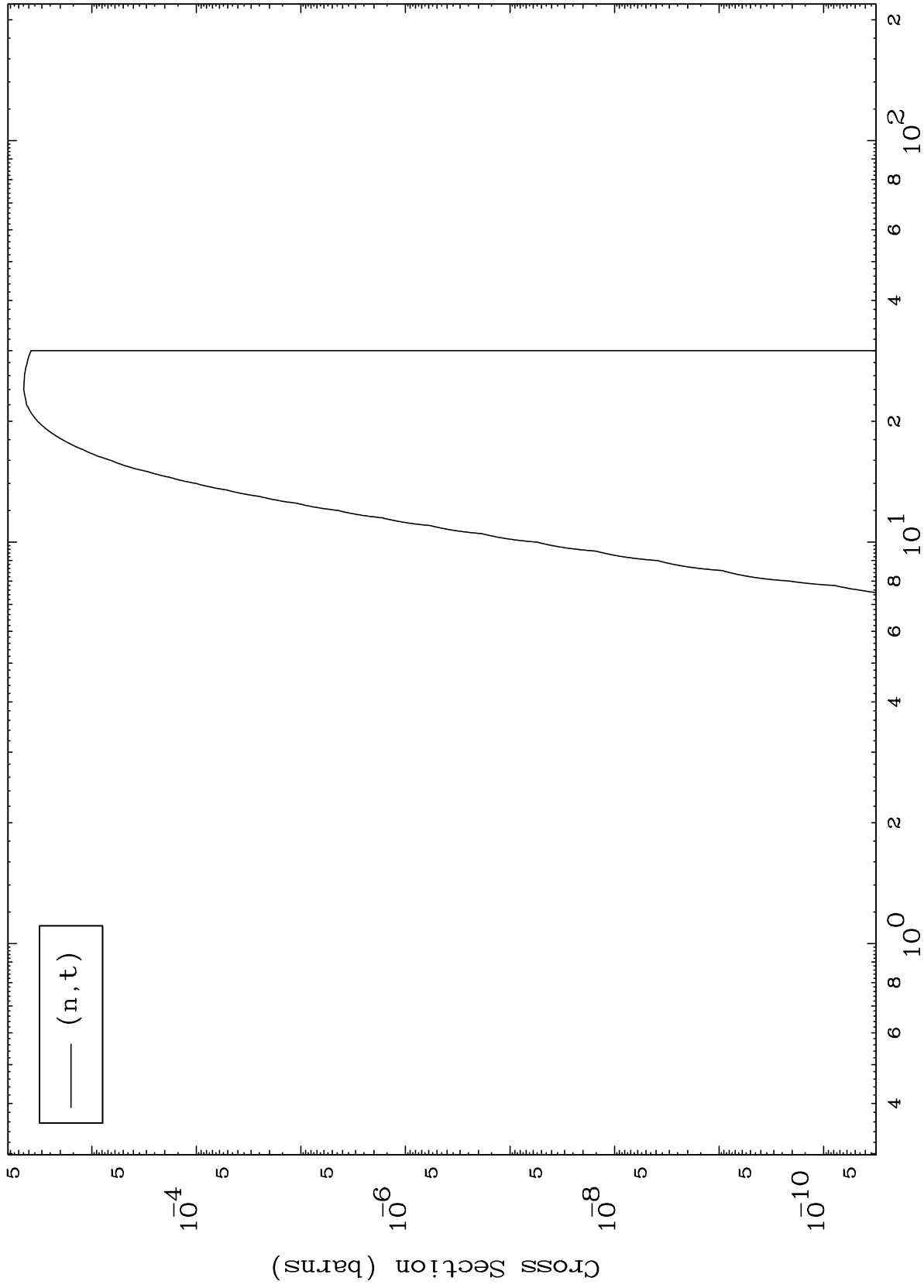
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,t) Levels
293 Kelvin Cross Sections

101-Md-256



13

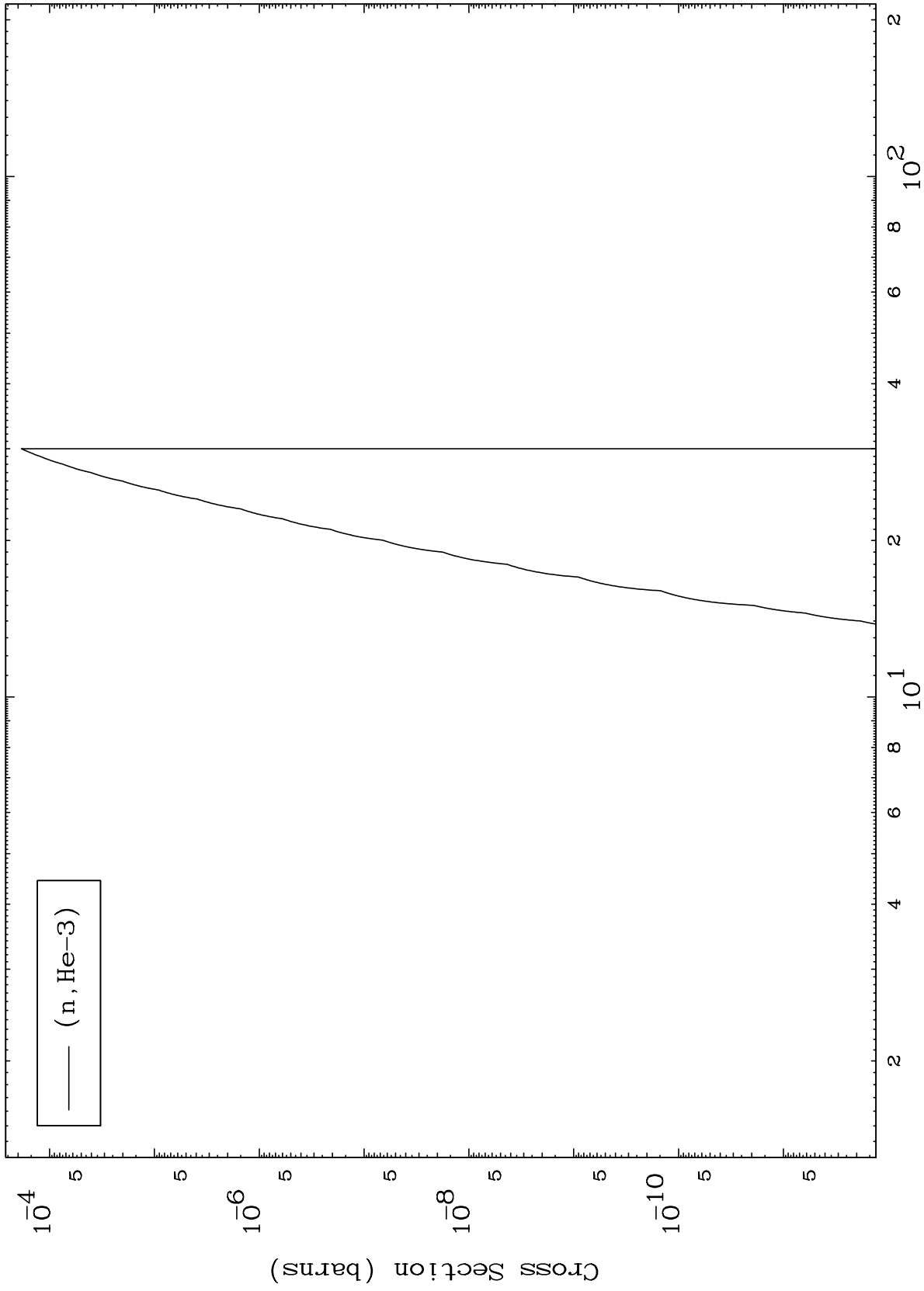
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-256



14

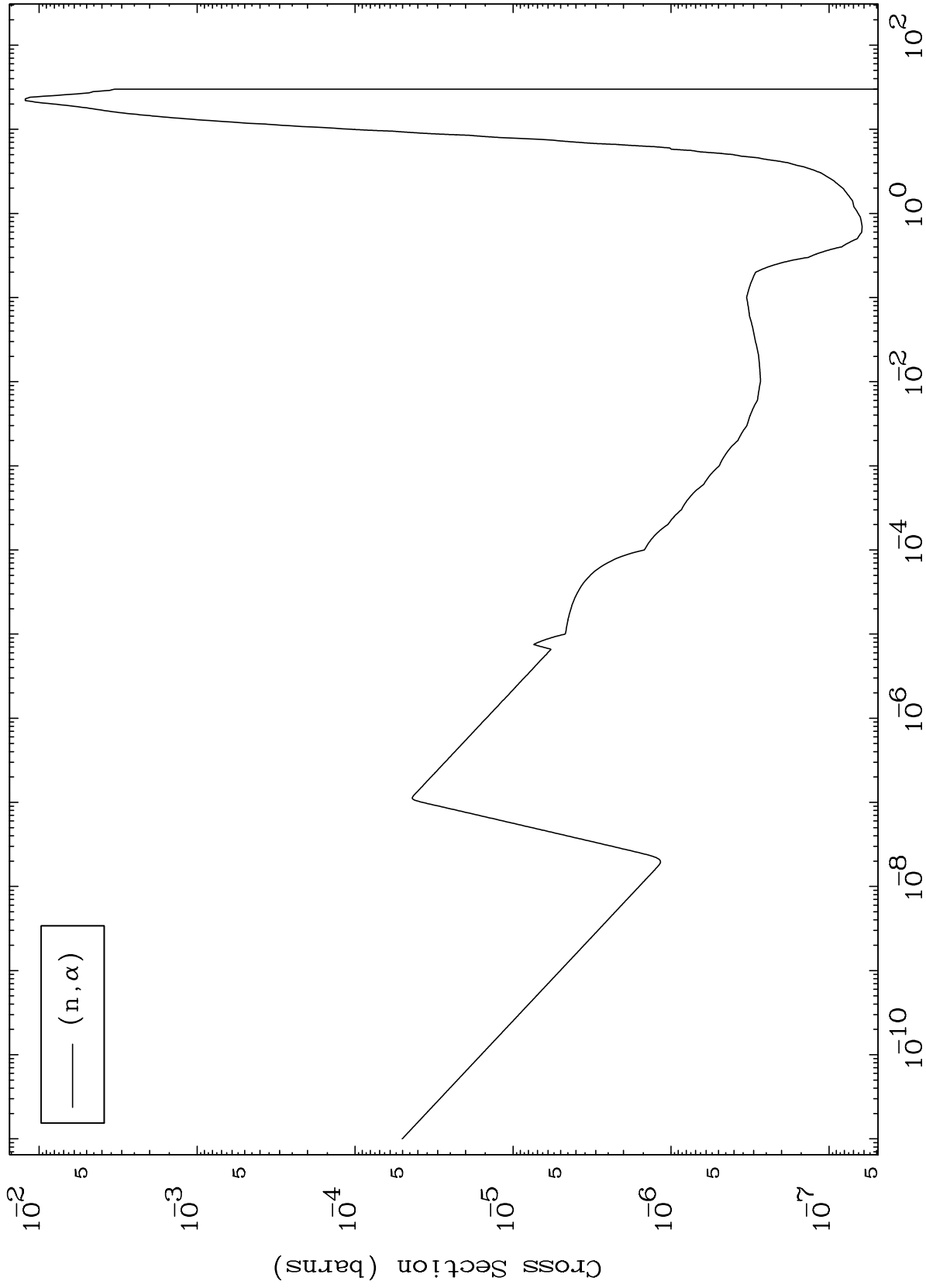
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,α) Levels
293 Kelvin Cross Sections

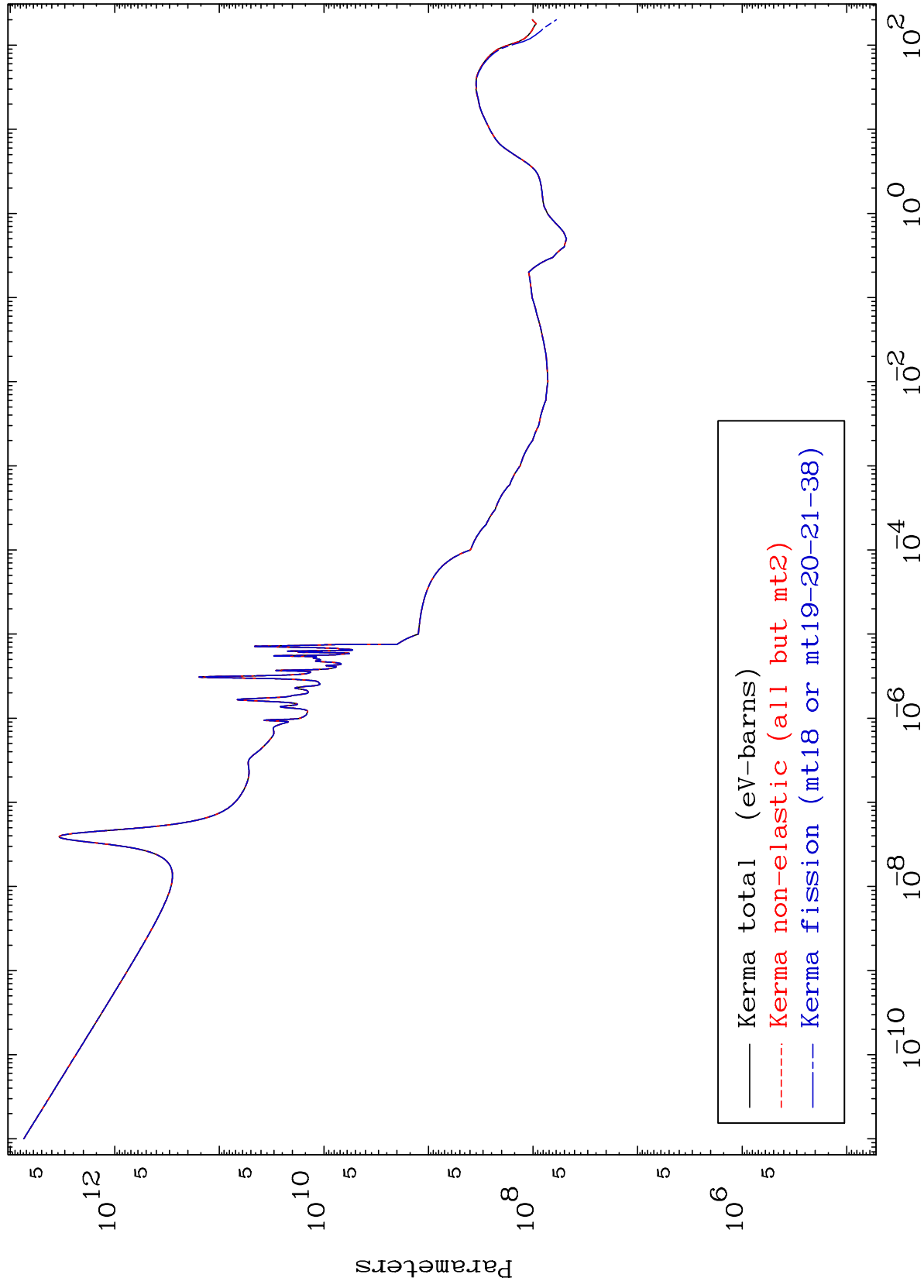
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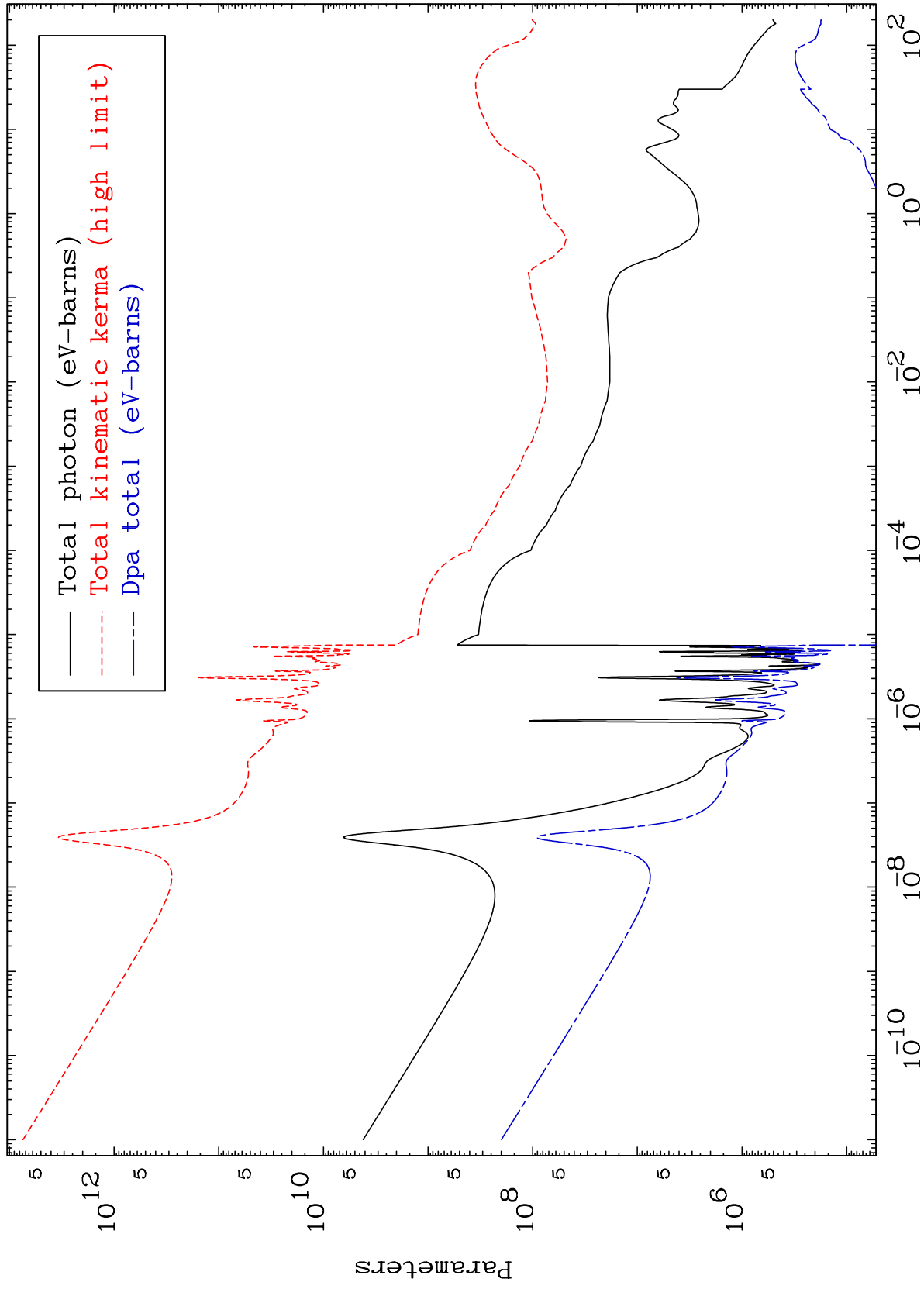


15

Incident Energy (MeV)

101-Md-256

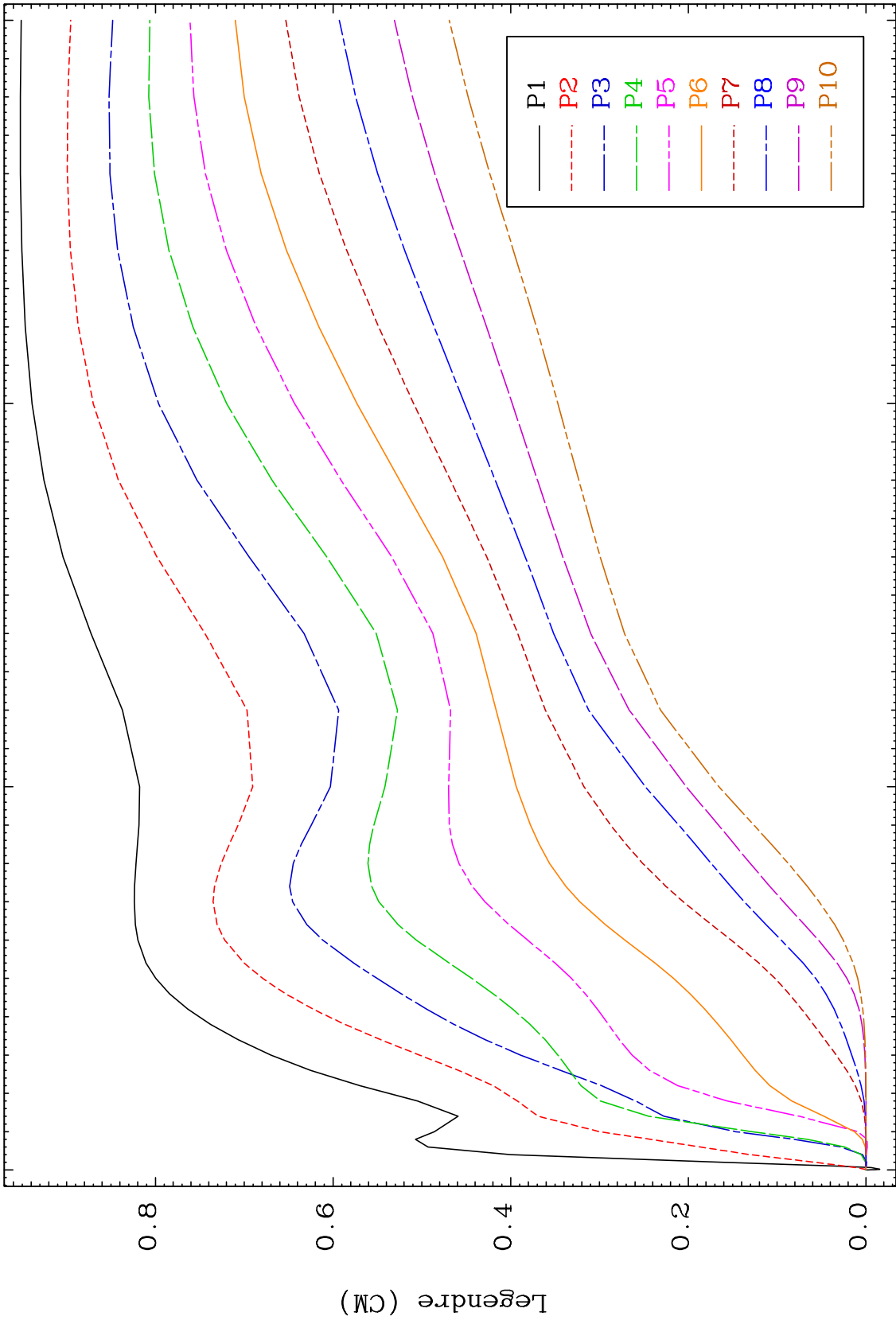




MAT 9957

Elastic Legendre Coefficients

101-Md-256



18

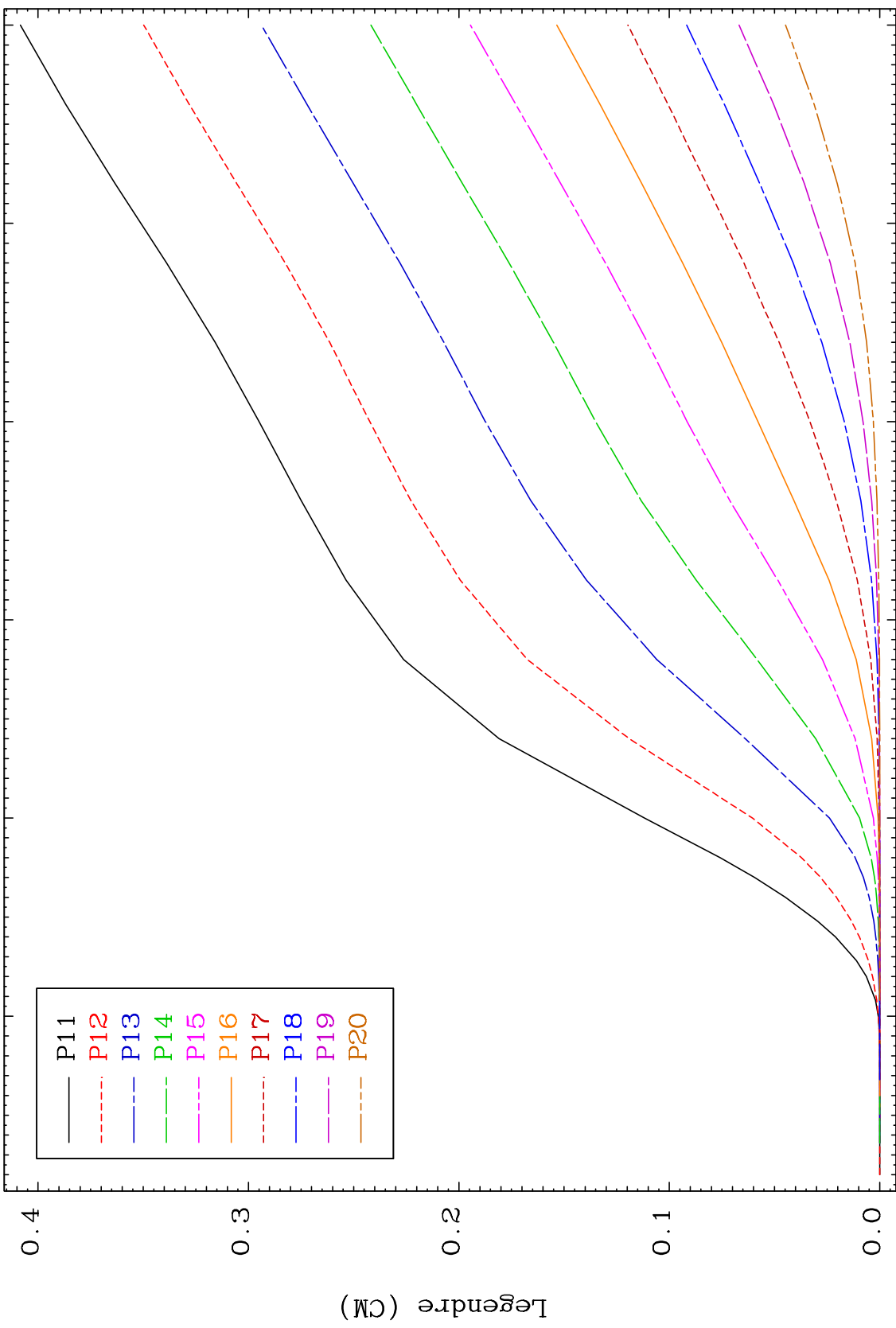
Incident Energy (MeV)

101-Md-256

MAT 9957

Elastic Legendre Coefficients

101-Md-256



19

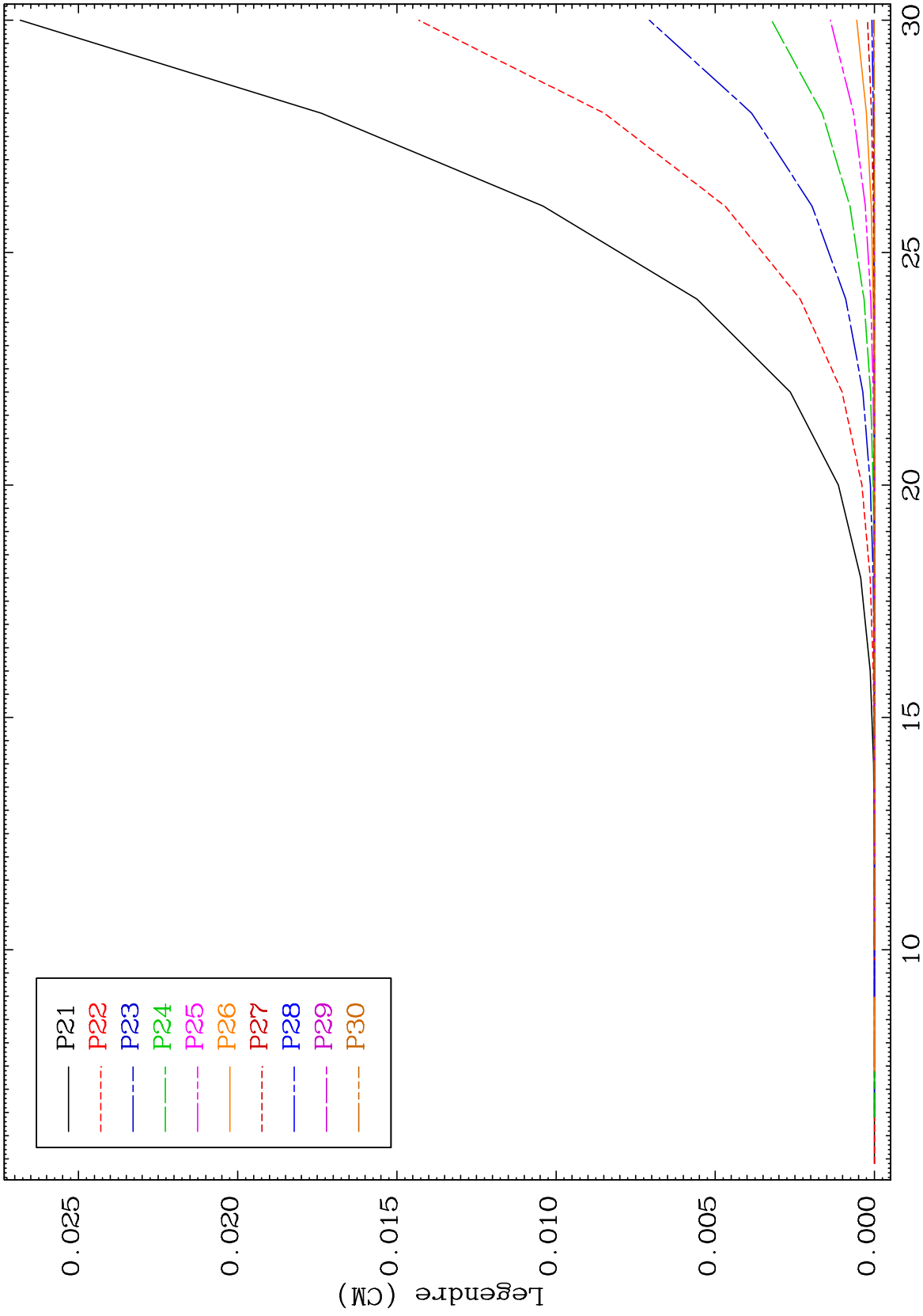
Incident Energy (MeV)

101-Md-256

MAT 9957

Elastic Legendre Coefficients

101-Md-256



20

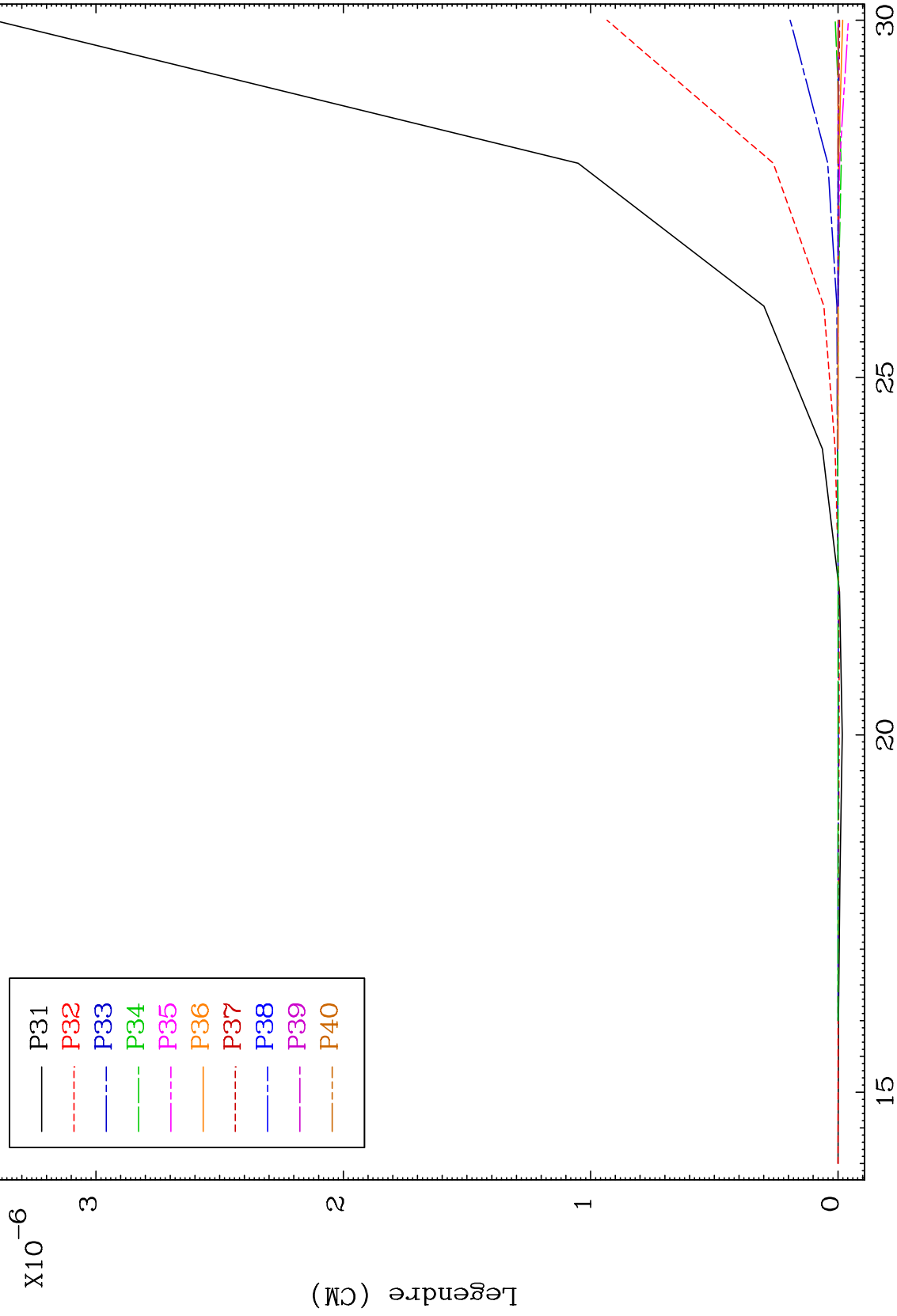
Incident Energy (MeV)

101-Md-256

MAT 9957

Elastic Legendre Coefficients

101-Md-256



21

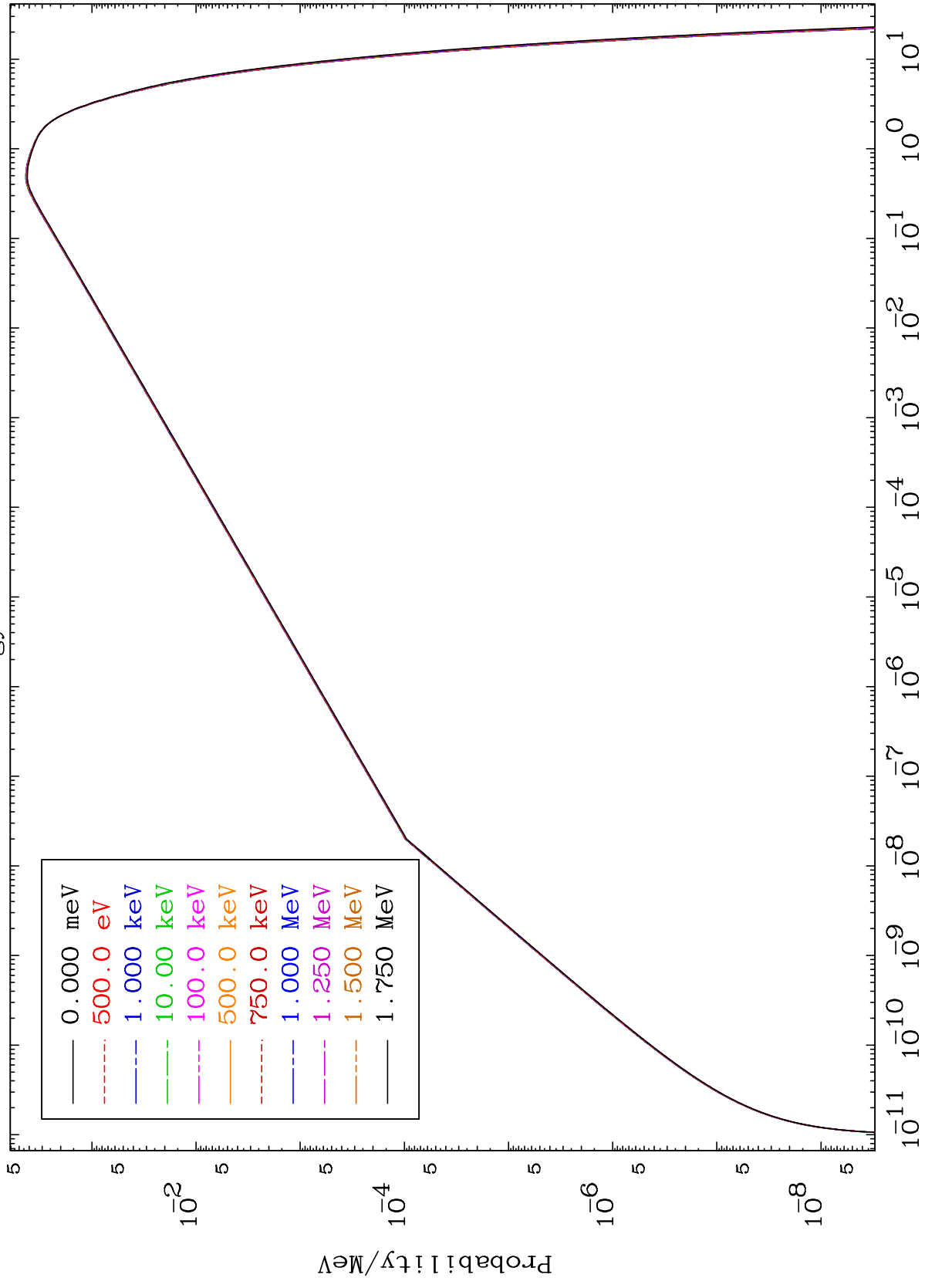
Incident Energy (MeV)

101-Md-256

MAT 9957

Fission Energy Distribution

101-Md-256

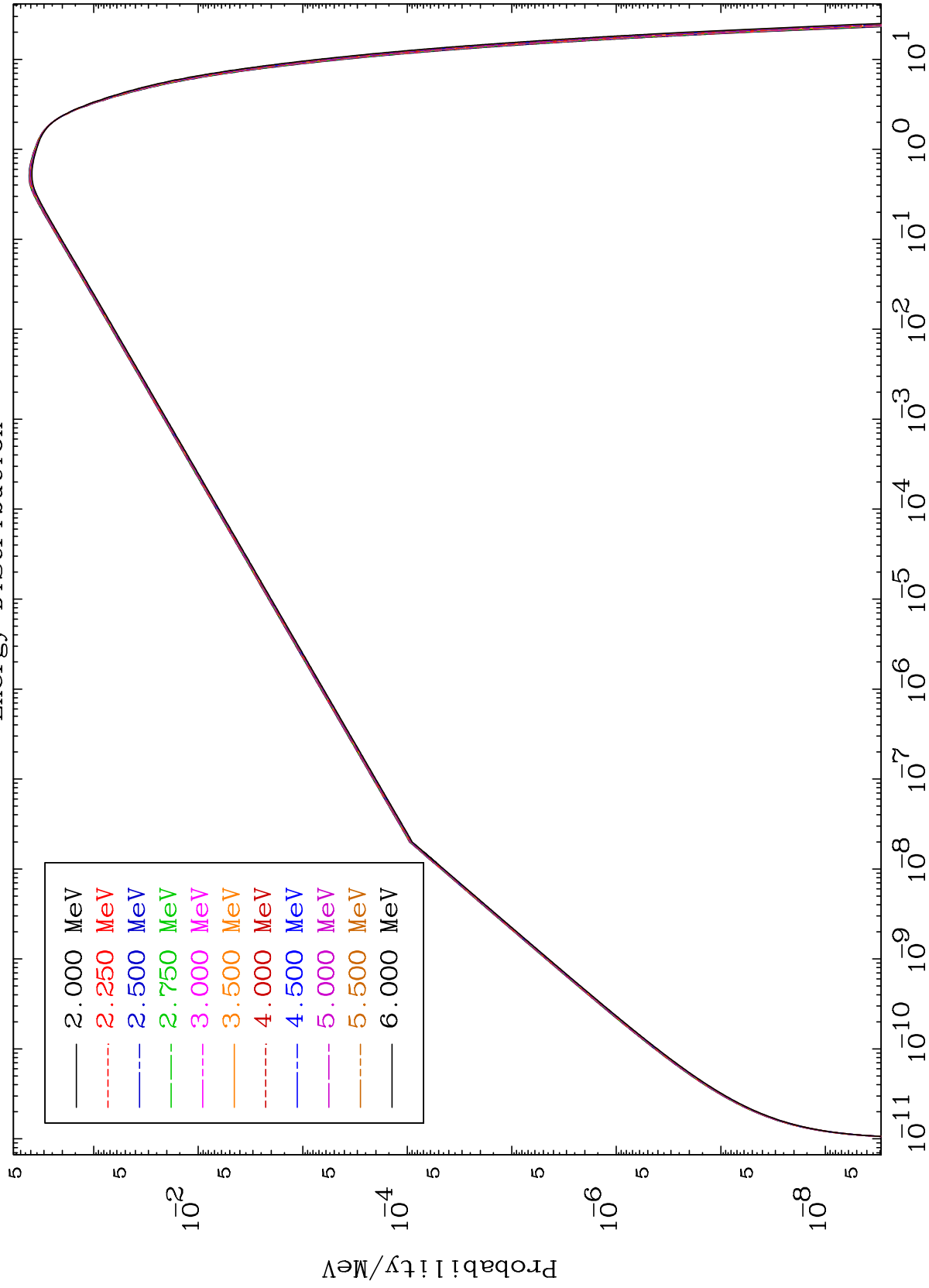


101-Md-256

MAT 9957

Fission Energy Distribution

101-Md-256

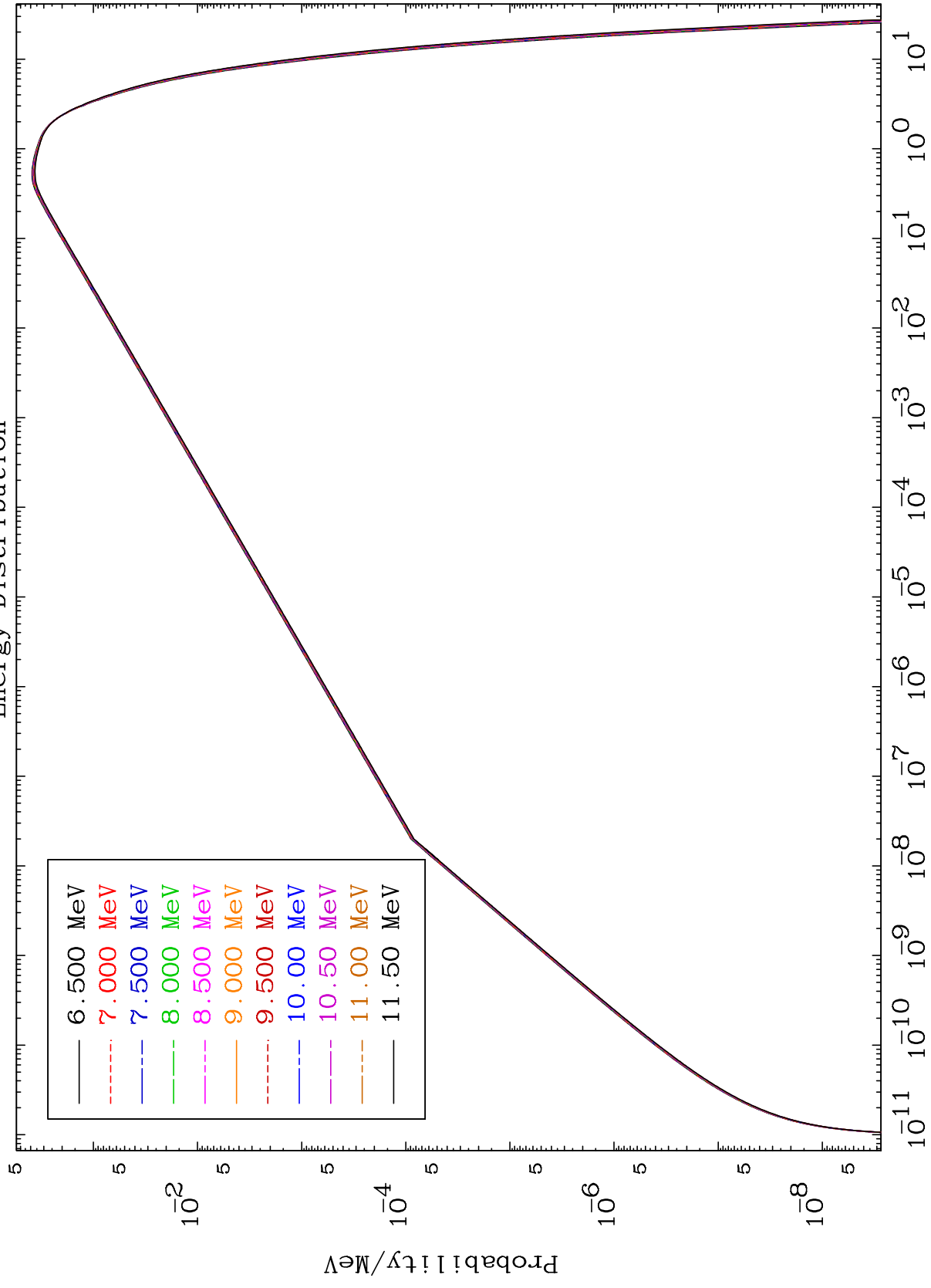


101-Md-256

MAT 9957

Fission Energy Distribution

101-Md-256

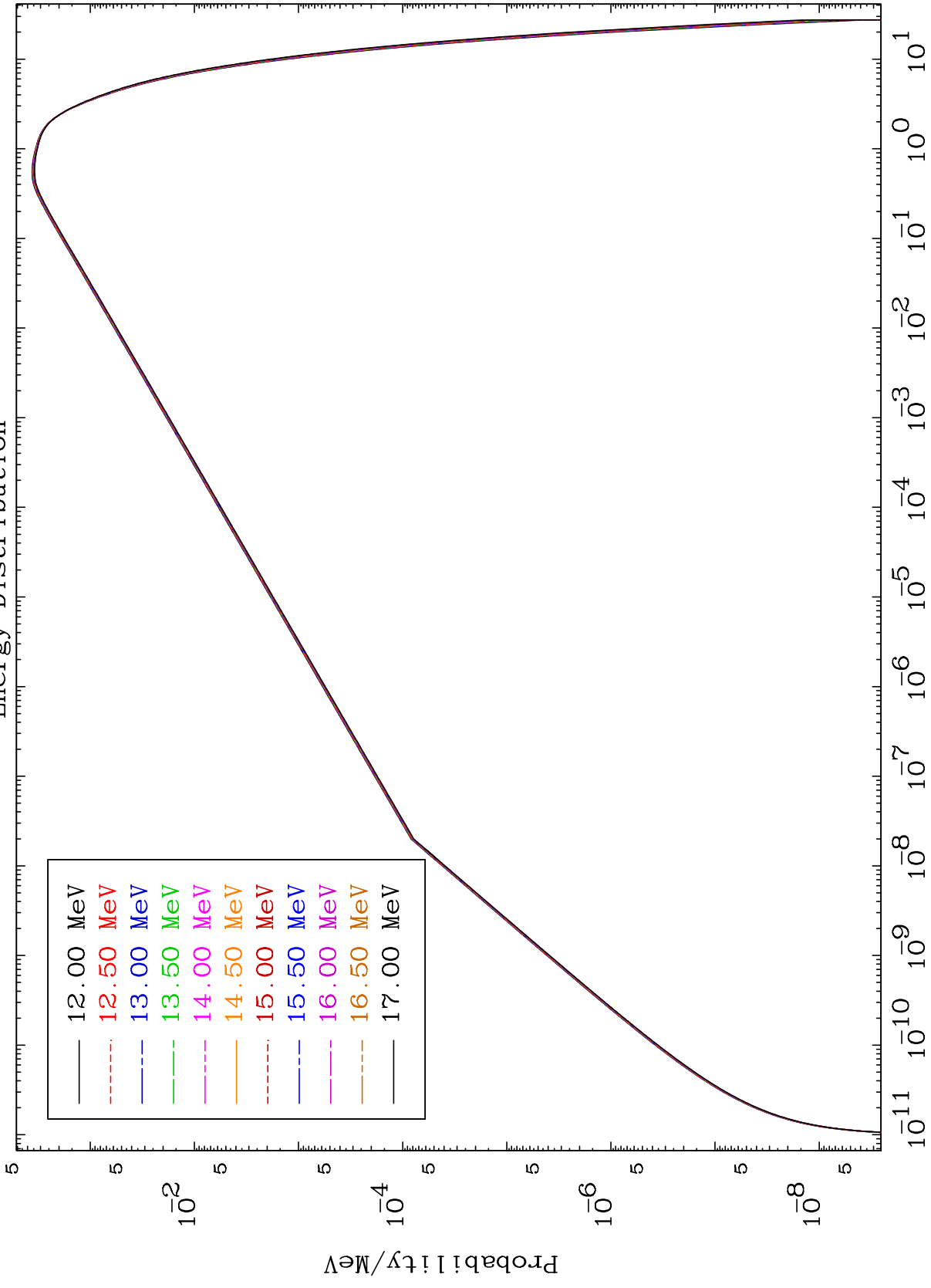


101-Md-256

MAT 9957

Fission Energy Distribution

101-Md-256

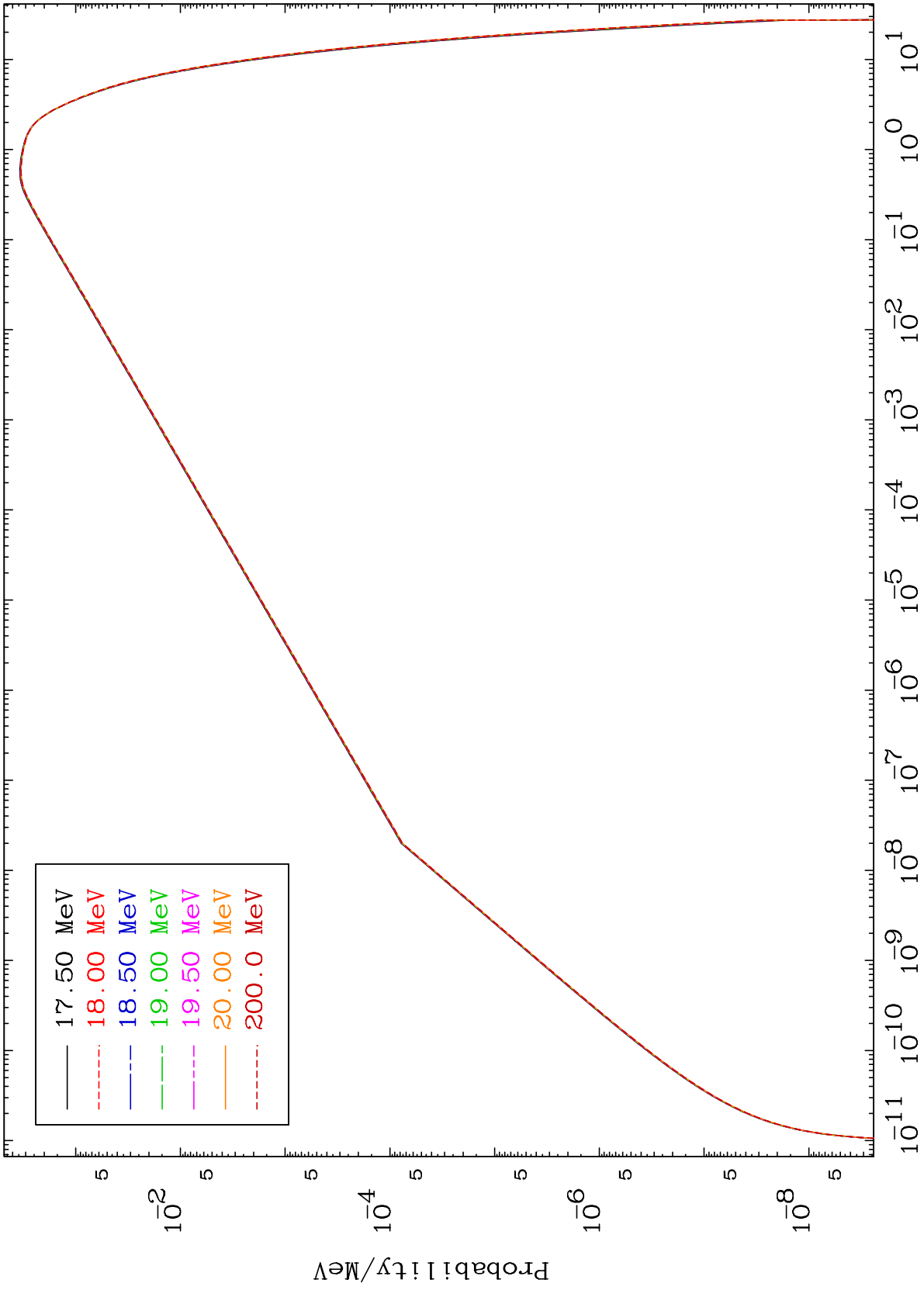


101-Md-256

MAT 9957

Fission Energy Distribution

101-Md-256



26

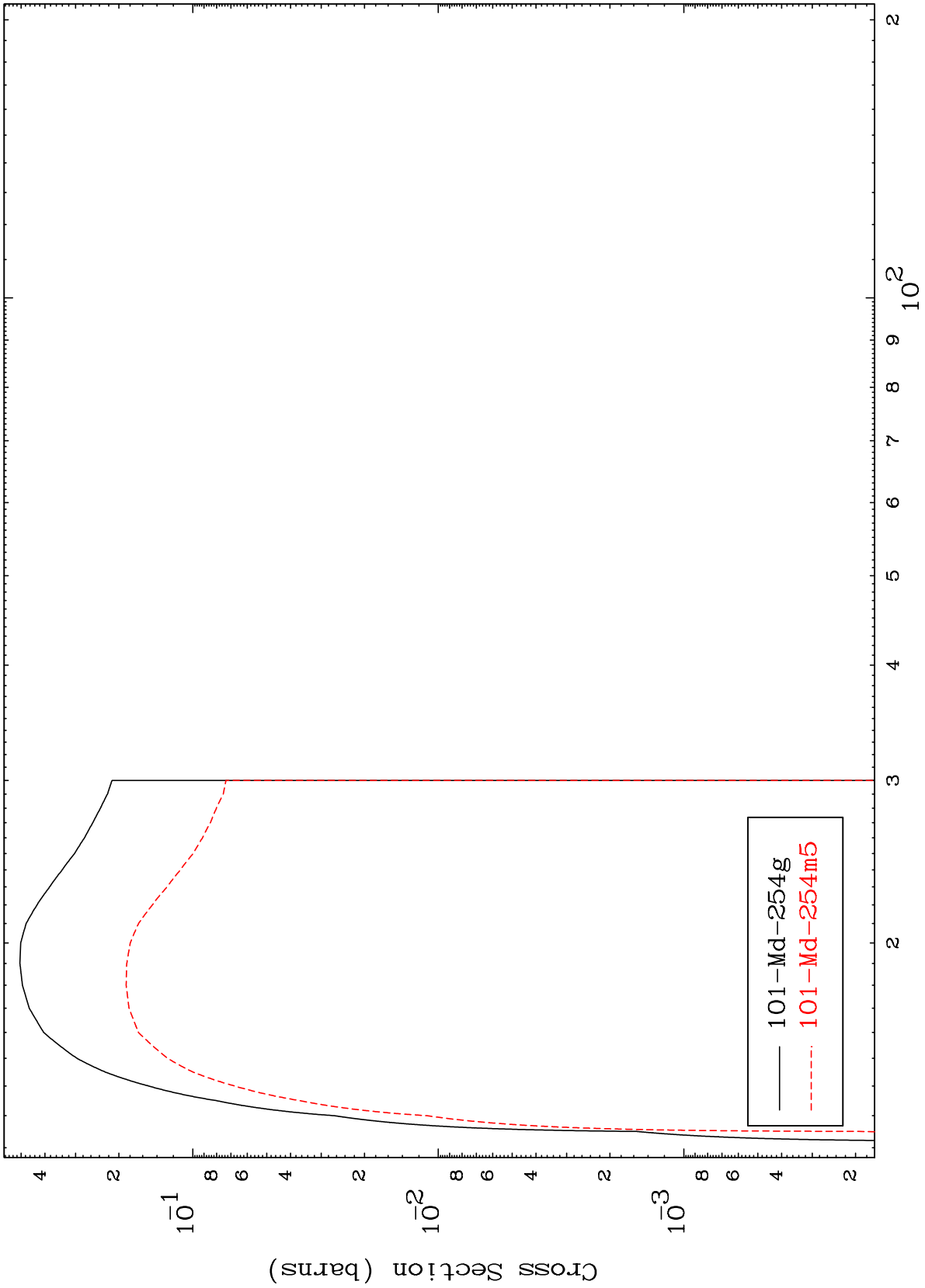
101-Md-256

MAT 9957

(n,3n)

101-Md-256

Radionuclide Production Cross Section



27

Incident Energy (MeV)

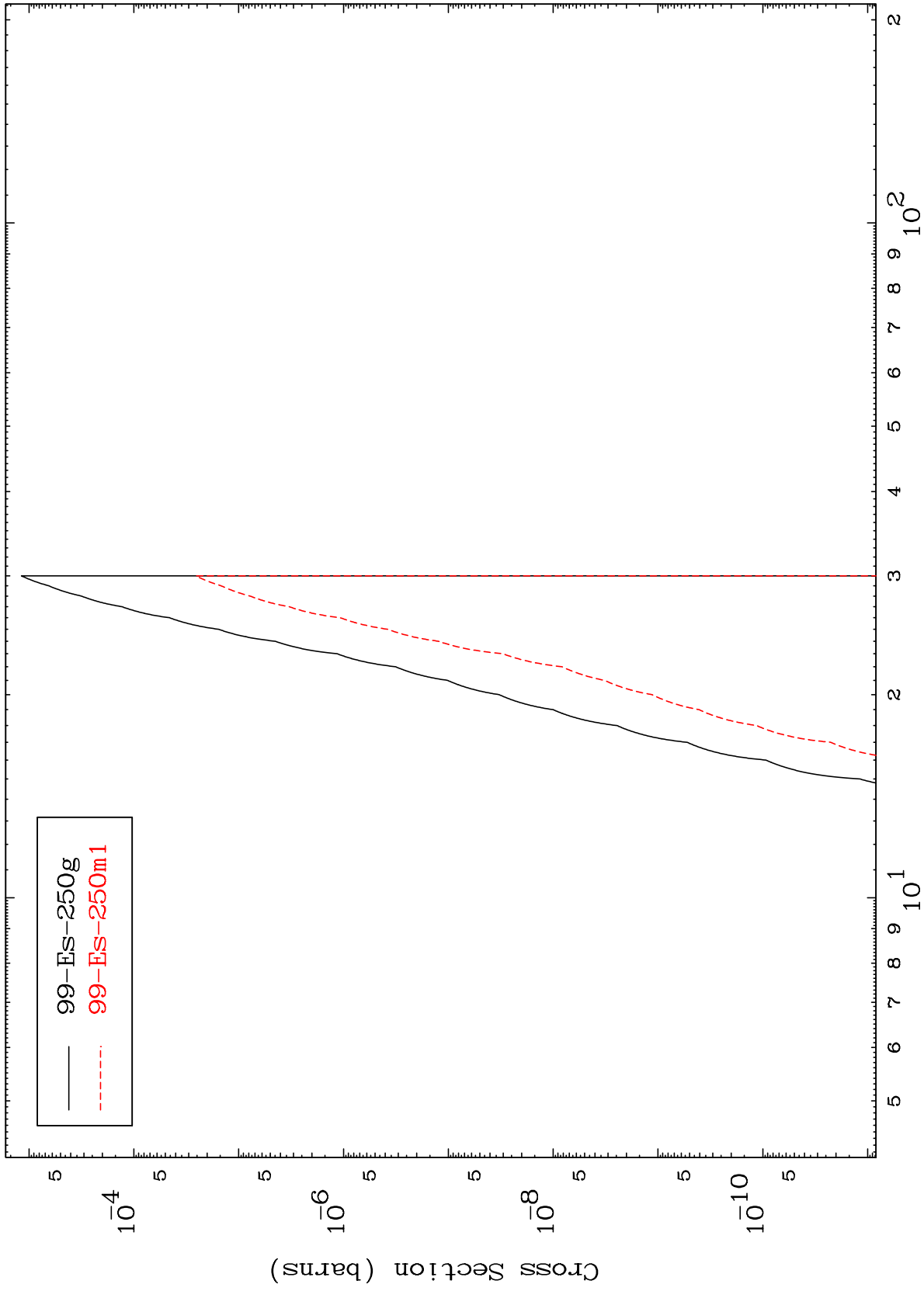
101-Md-256

MAT 9957

(n, 3n) α

101-Md-256

Radionuclide Production Cross Section



28

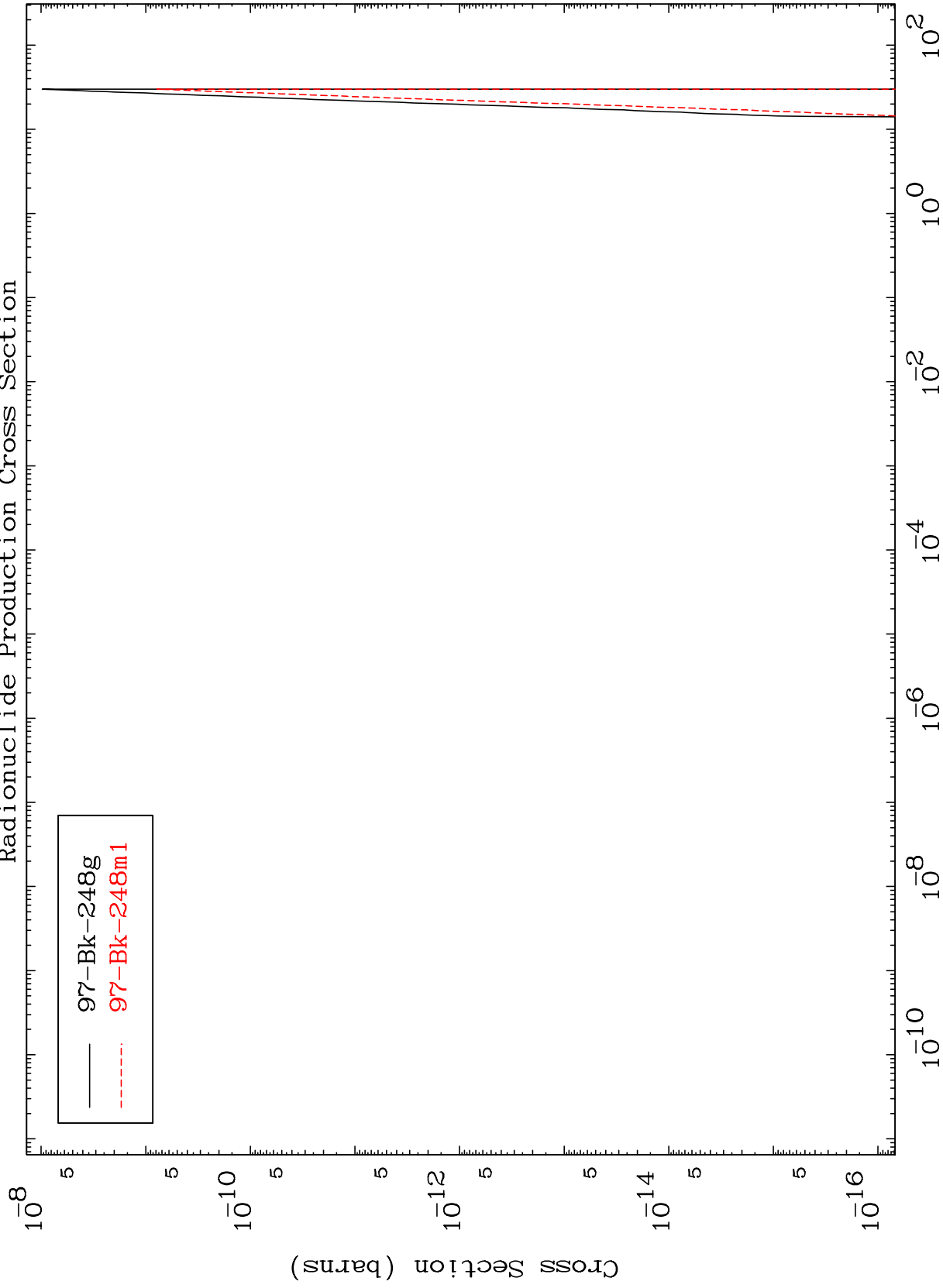
101-Md-256

MAT 9957

(n,n') 2 α

101-Md-256

Radionuclide Production Cross Section



29

Incident Energy (MeV)

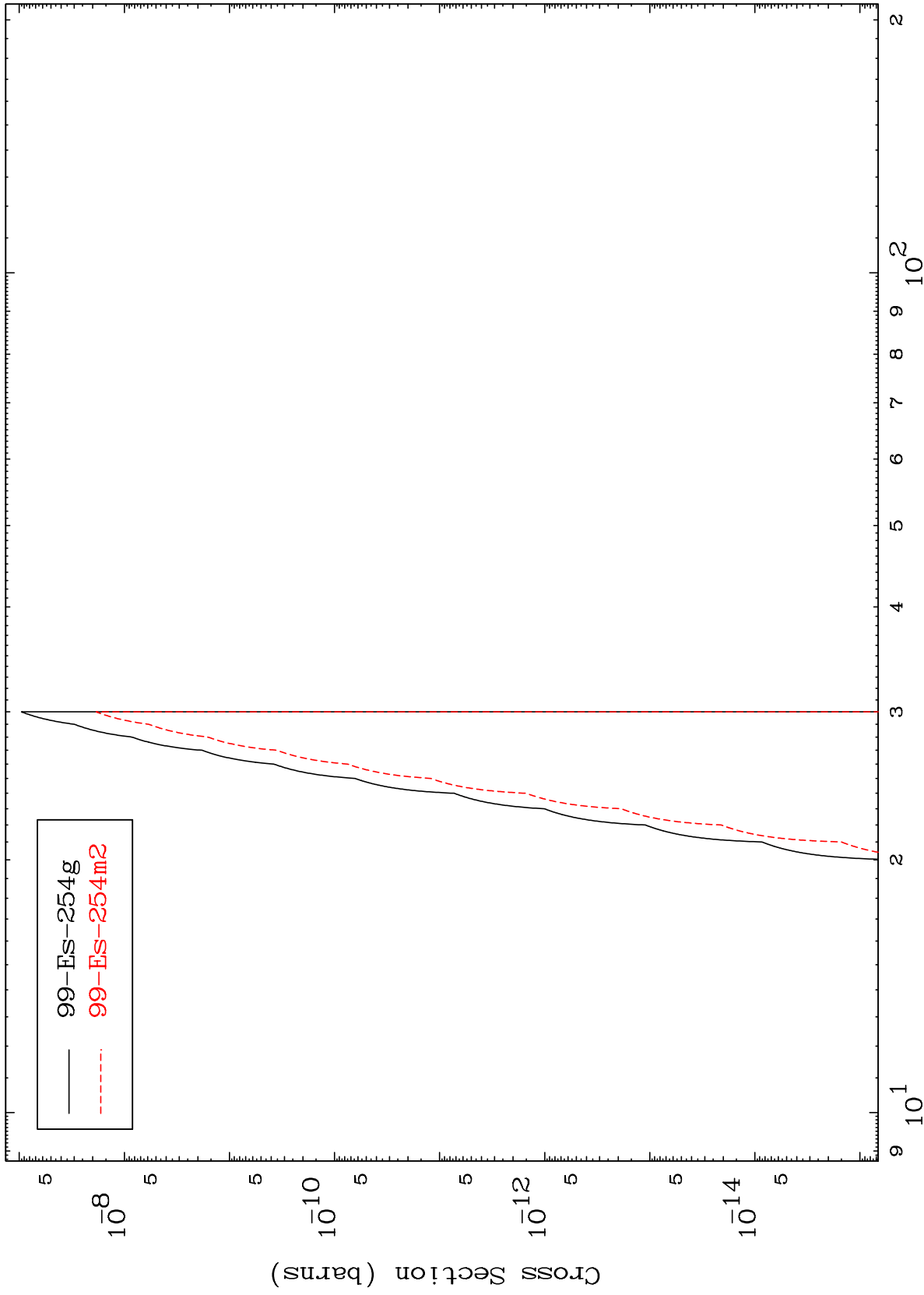
101-Md-256

MAT 9957

(n,2n) p

101-Md-256

Radionuclide Production Cross Section



30

Incident Energy (MeV)

101-Md-256

MAT 9957

(n, He-3)

101-Md-256

Radionuclide Production Cross Section

