

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

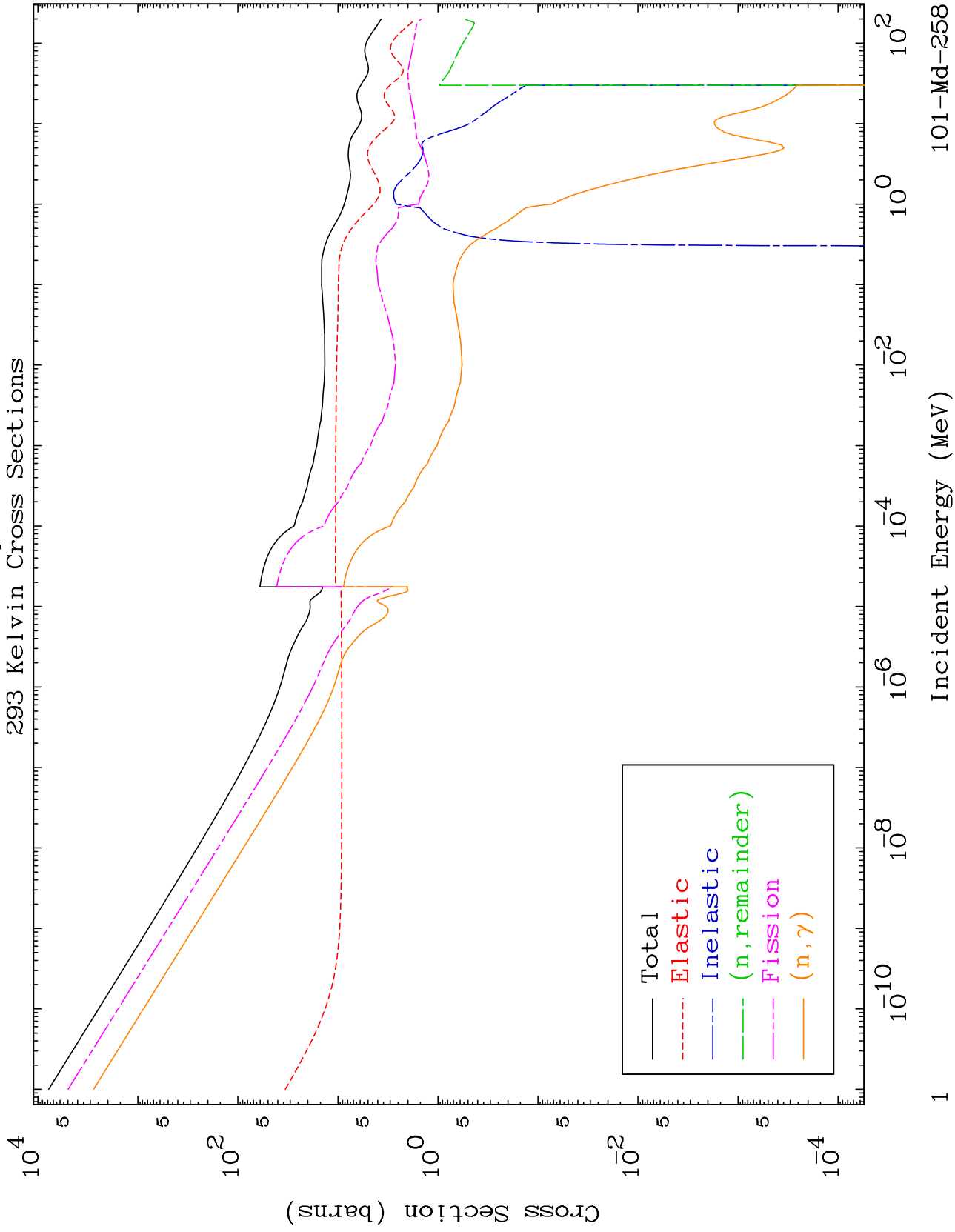
Press Mouse Button to Start

MAT 9959

Major

293 Kelvin Cross Sections

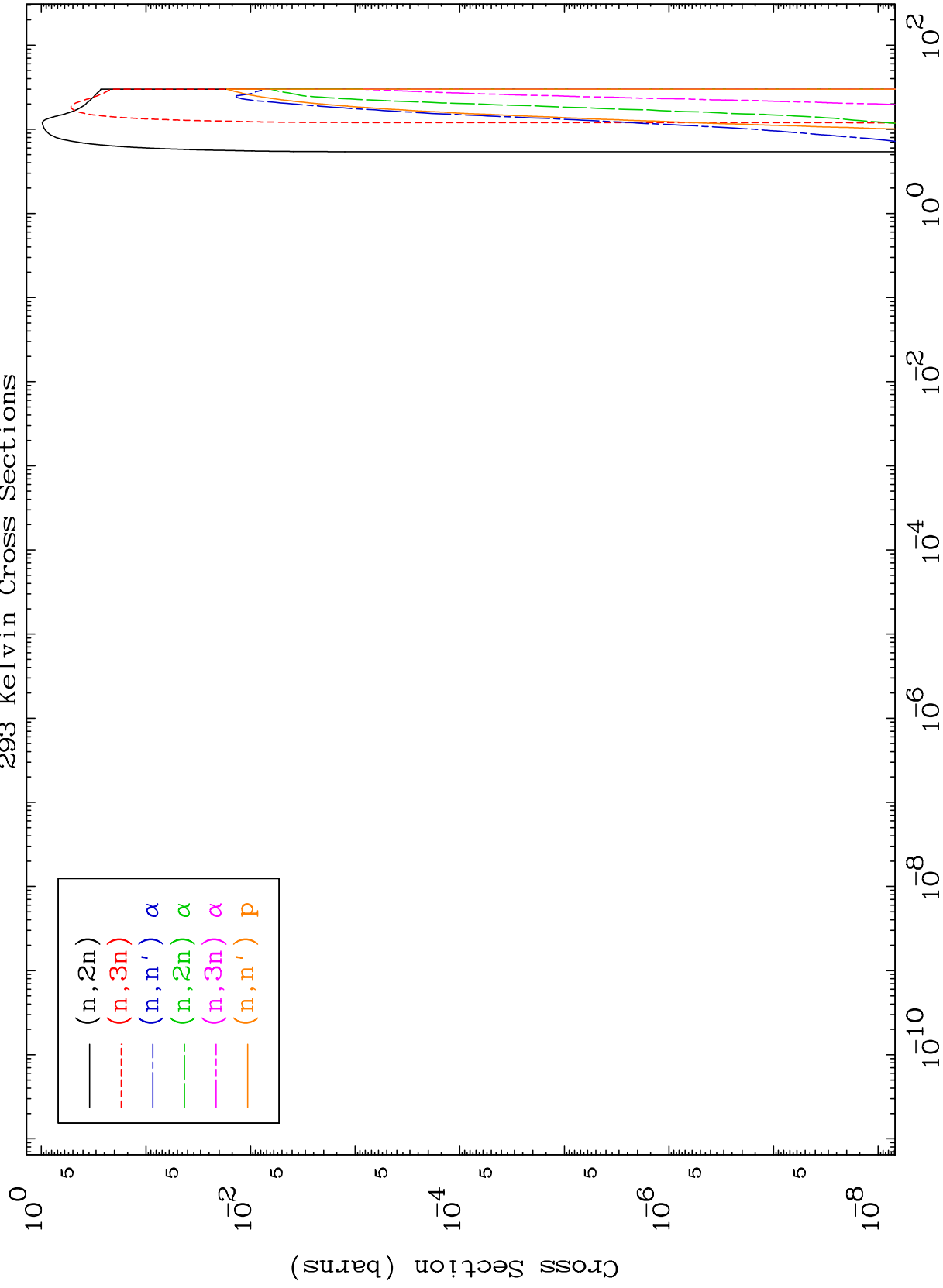
101-Md-258



MAT 9959

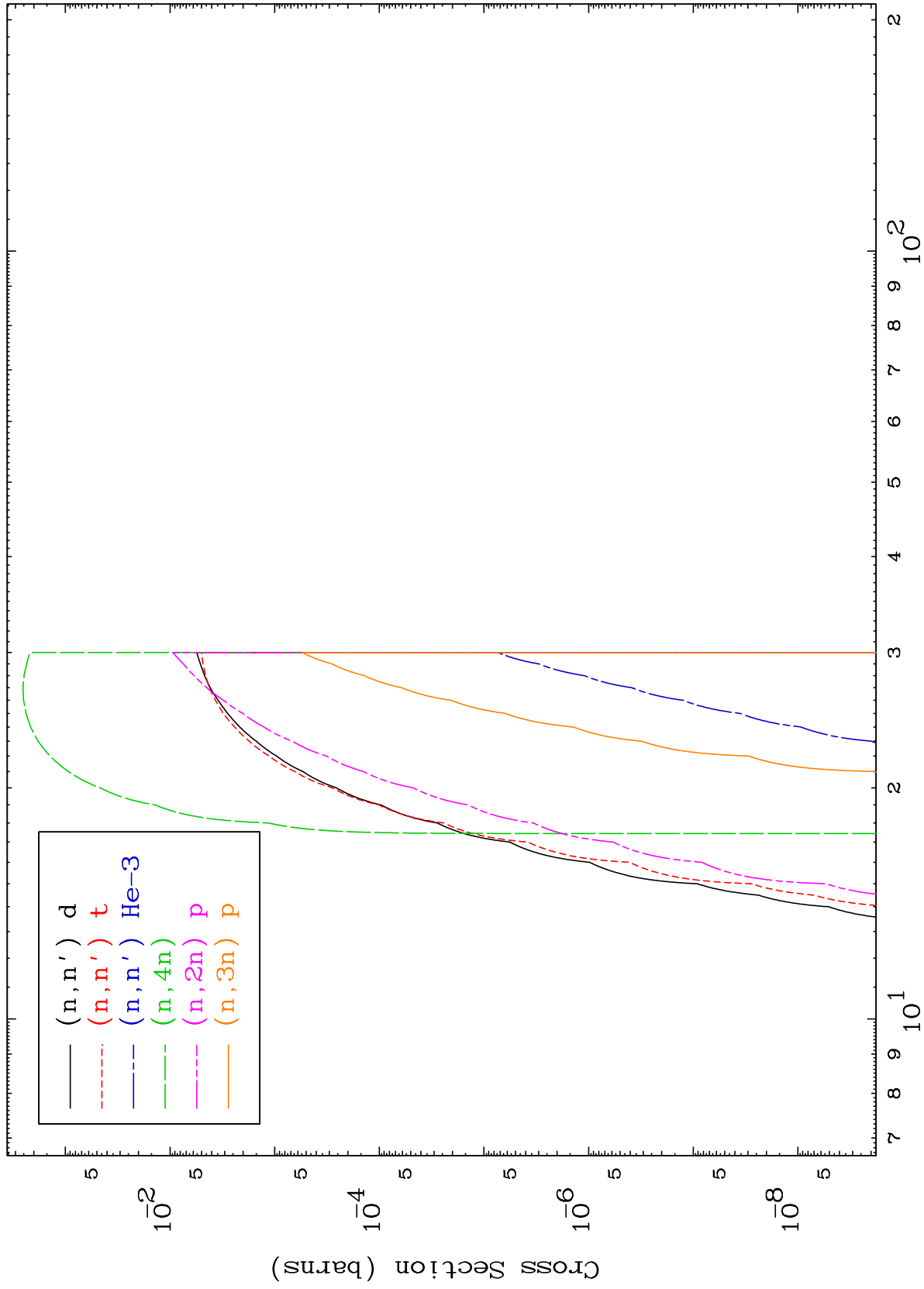
Neutron Production
293 Kelvin Cross Sections

101-Md-258



2

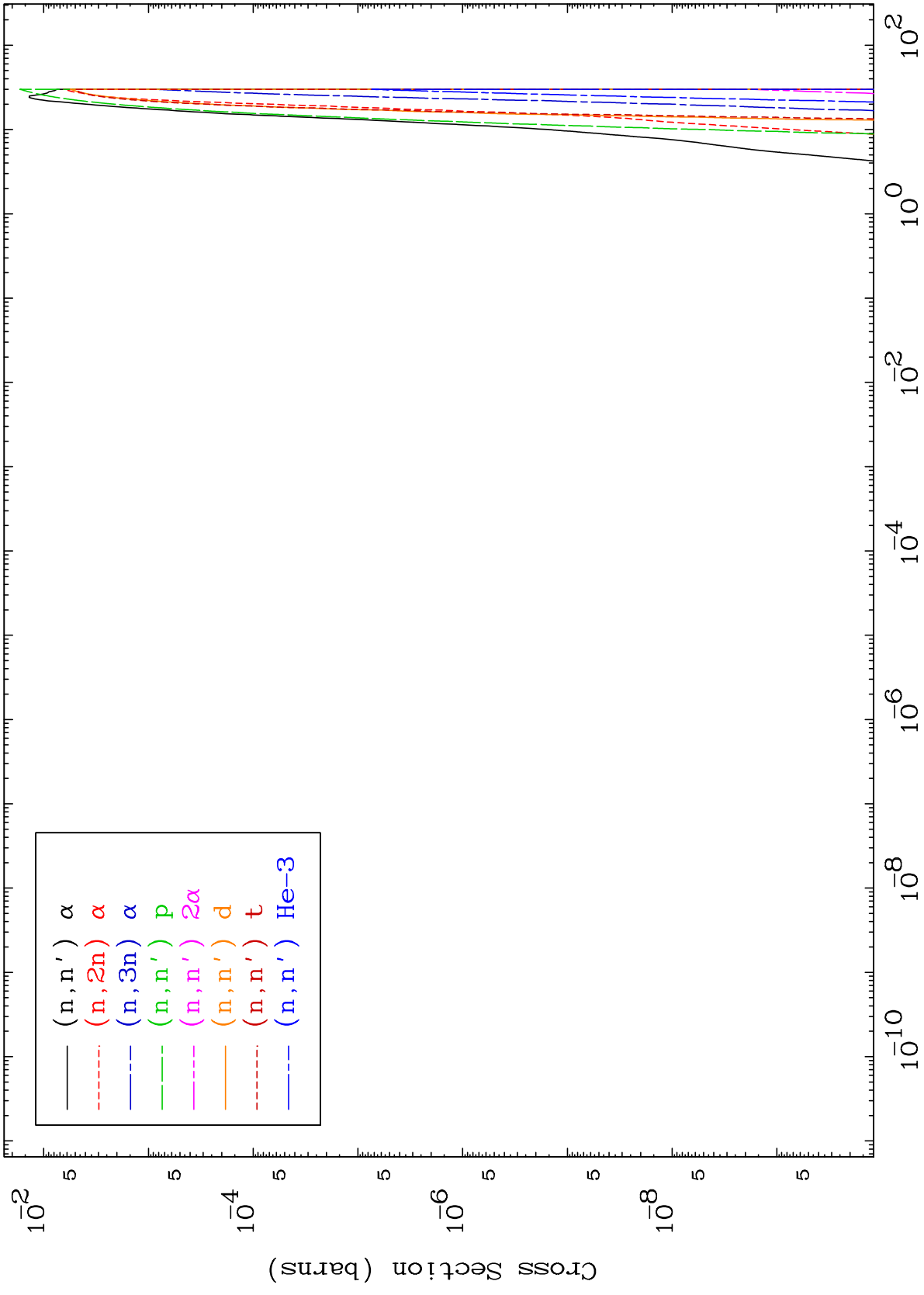
101-Md-258



MAT 9959

Charged Particle
293 Kelvin Cross Sections

101-Md-258

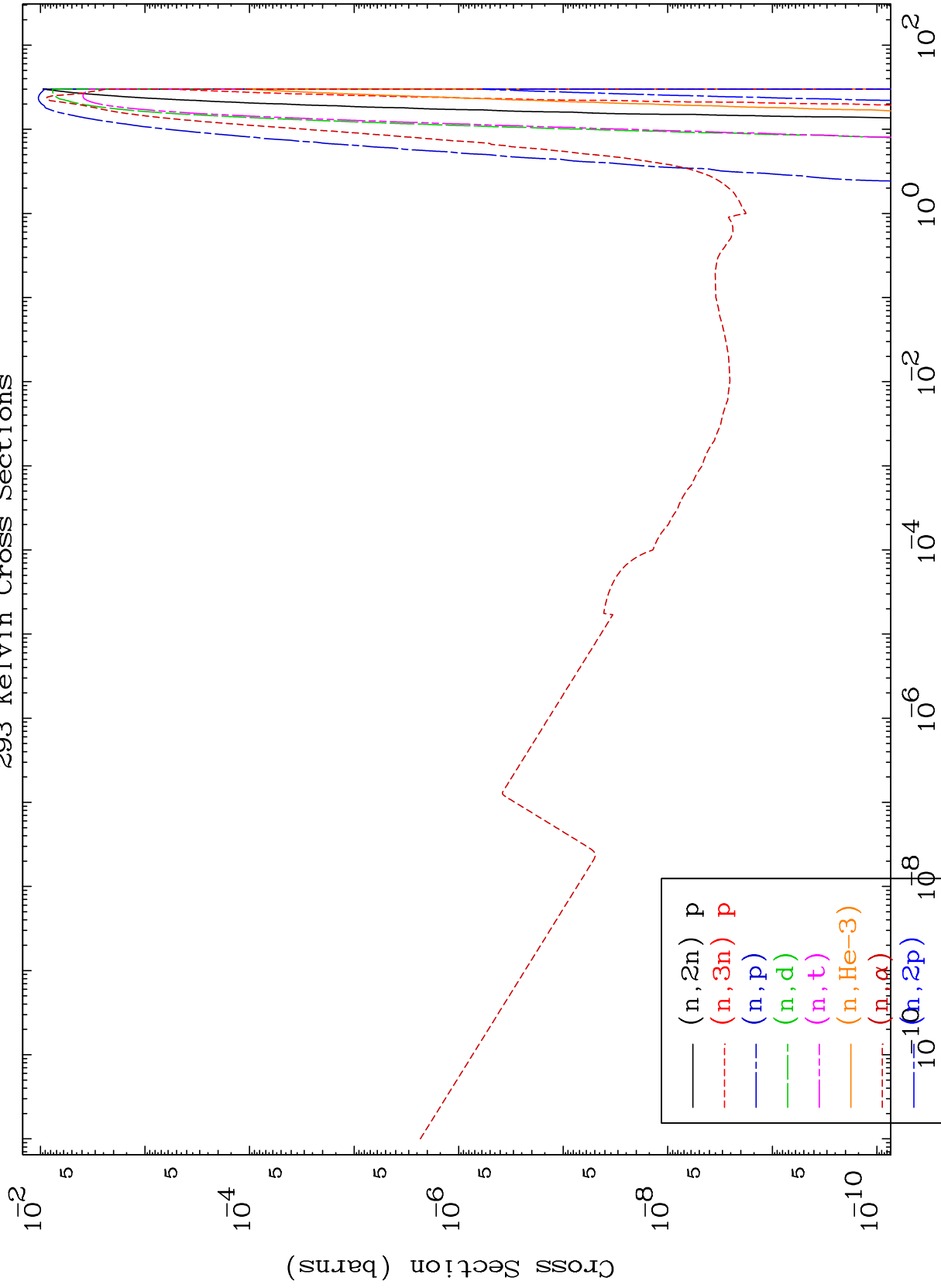


101-Md-258

MAT 9959

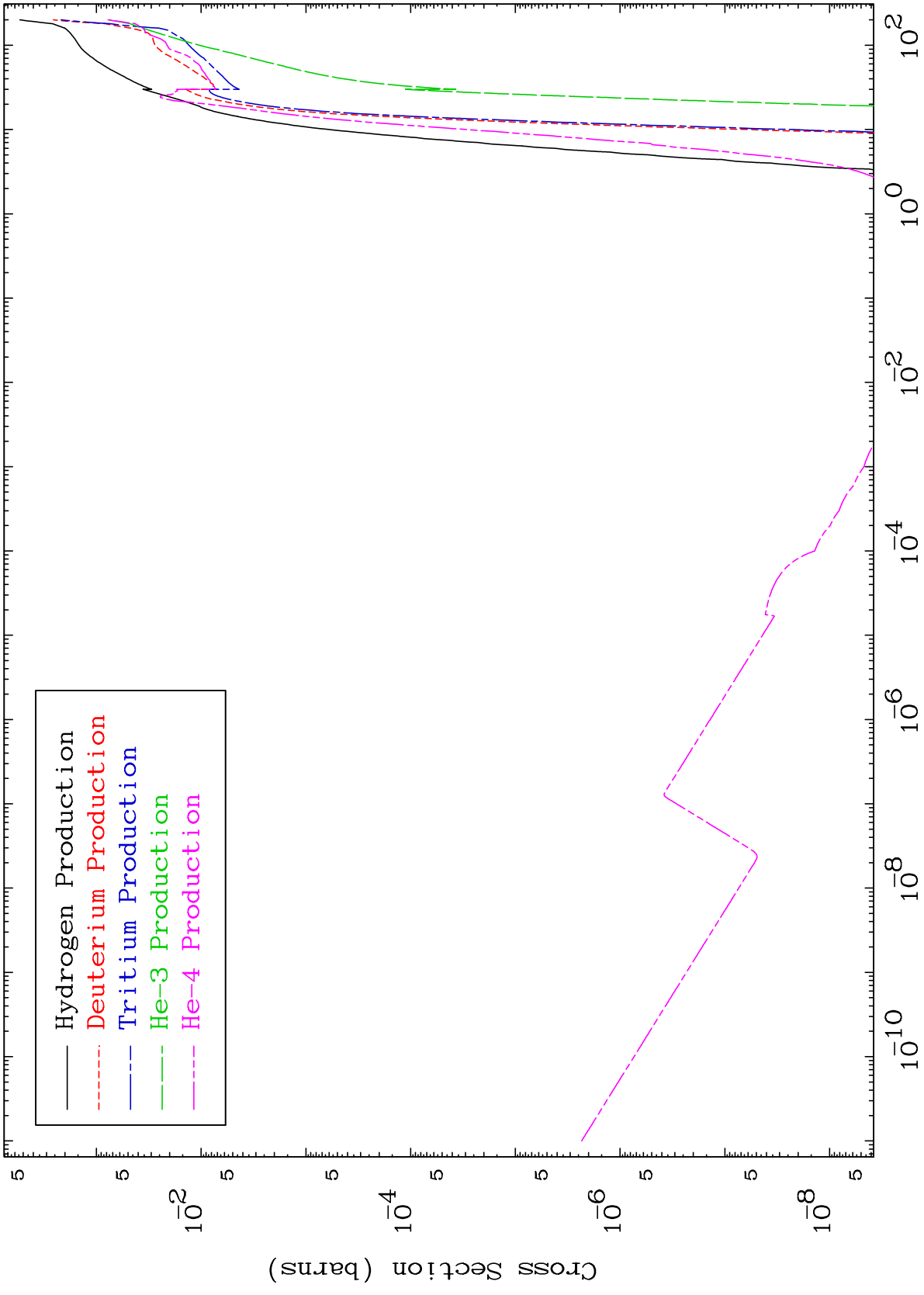
293 Kelvin Cross Sections

101-Md-258



5

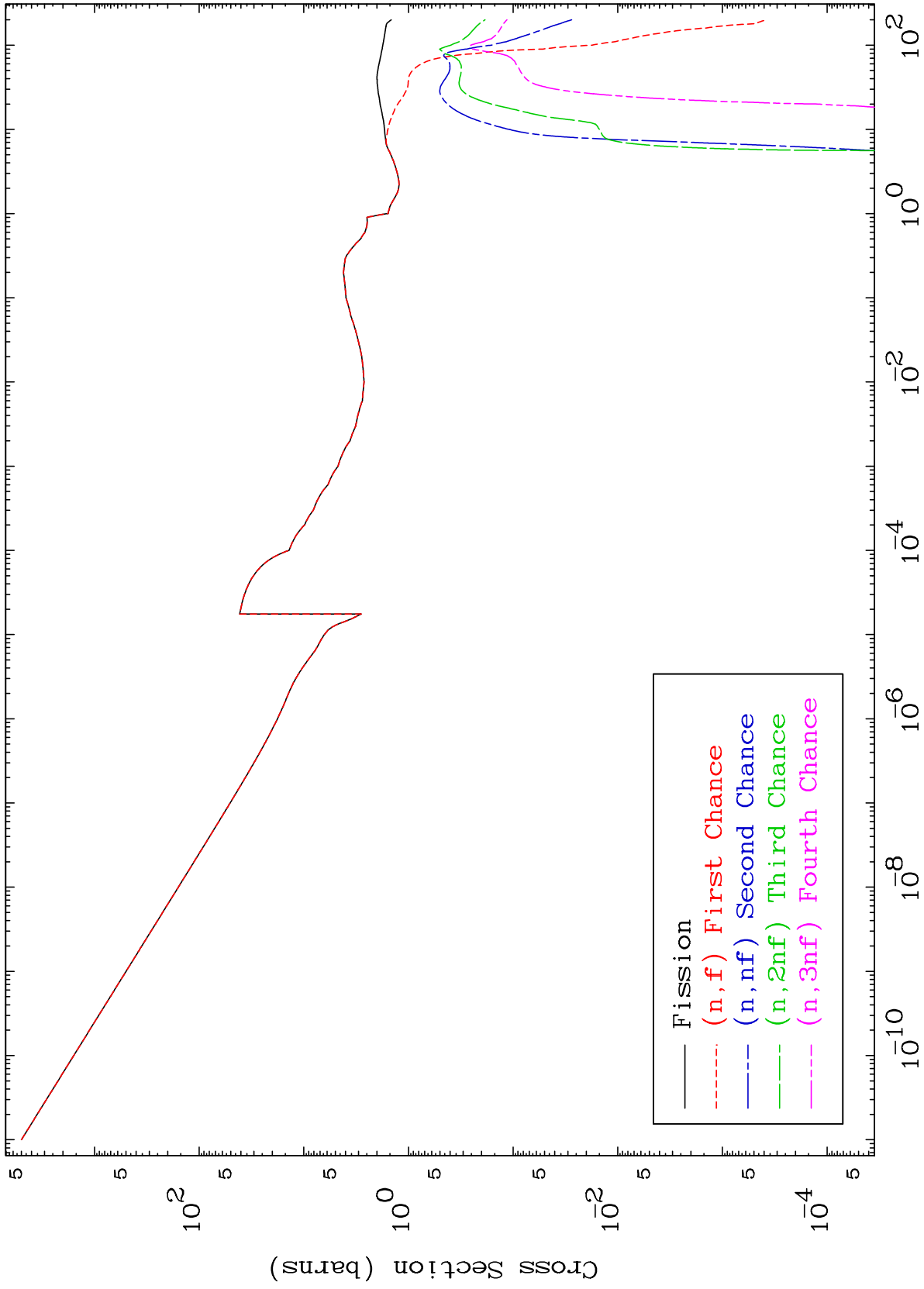
101-Md-258



MAT 9959

Fission
293 Kelvin Cross Sections

101-Md-258

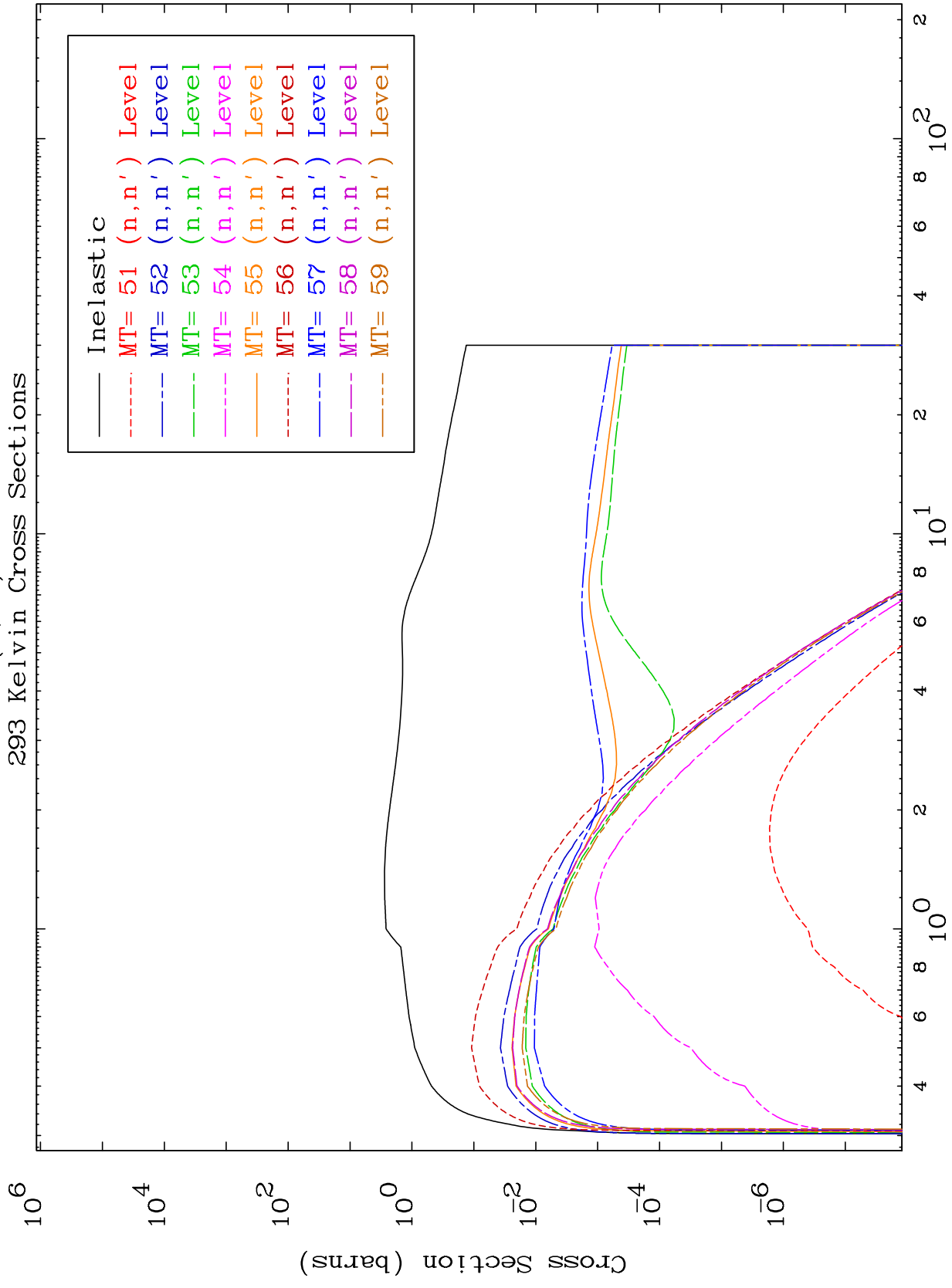


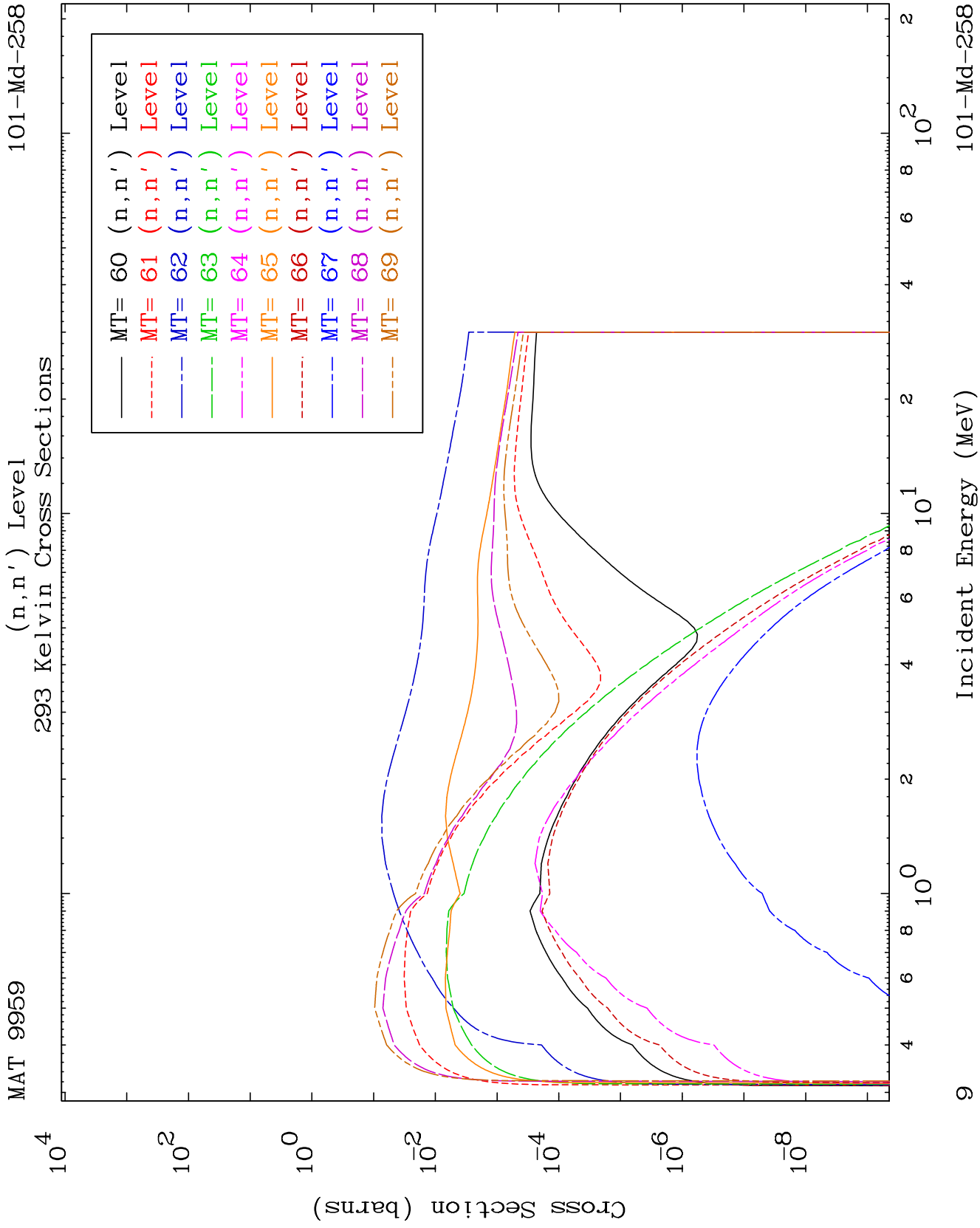
101-Md-258

MAT 9959

(n,n') Level

101-Md-258

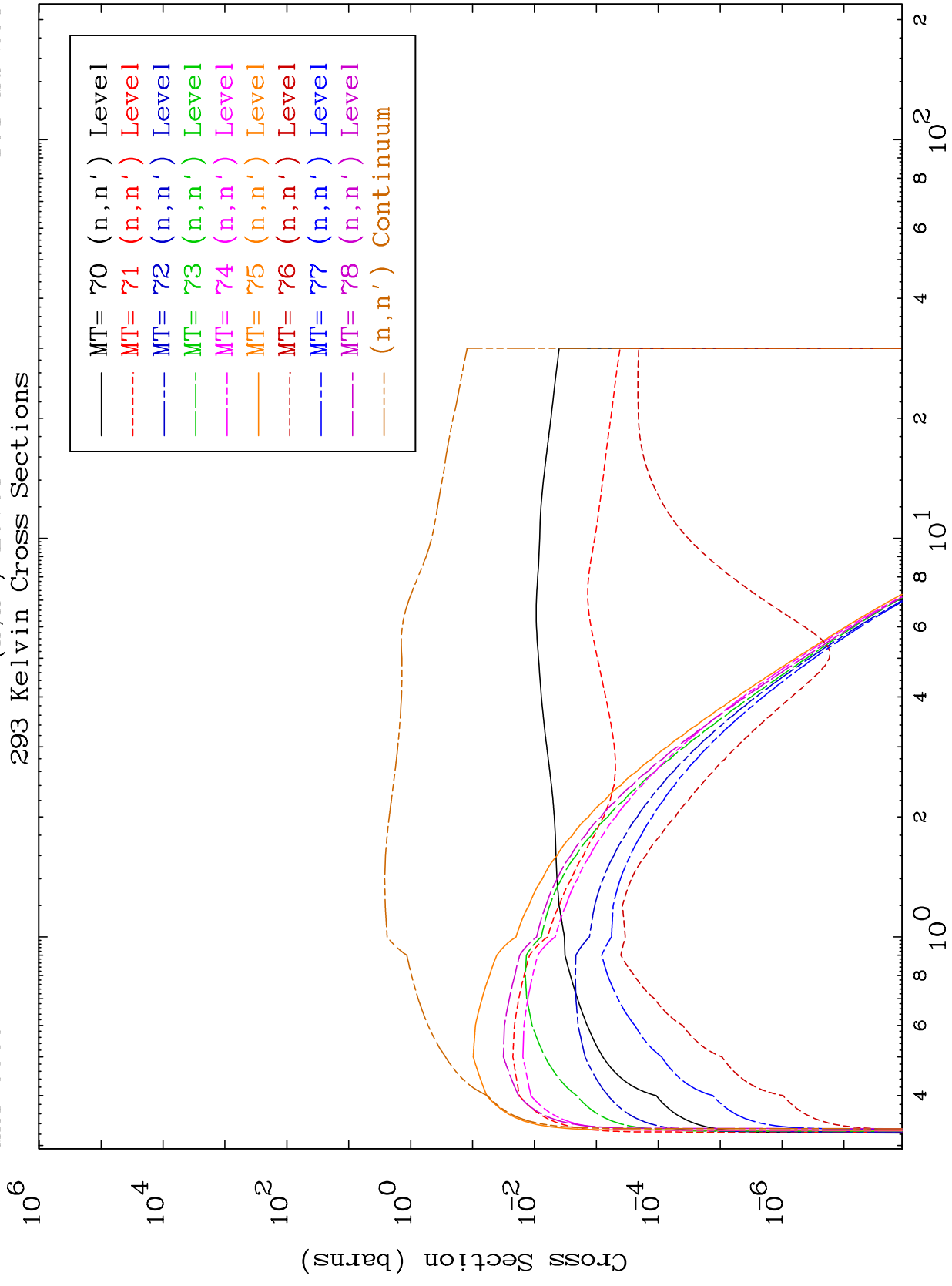




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(n,n') Level
293 Kelvin Cross Sections

101-Md-258



10

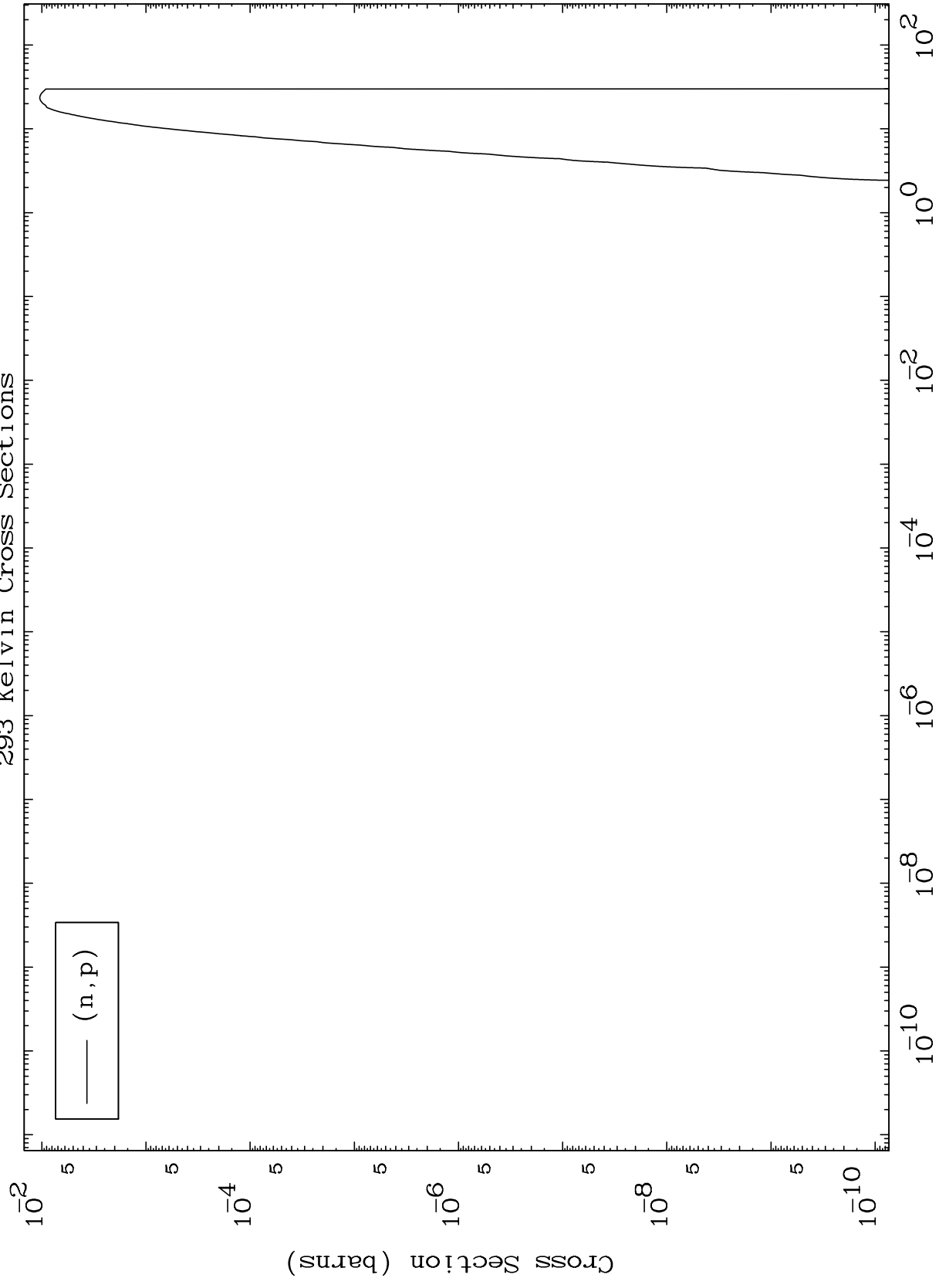
Incident Energy (MeV)

101-Md-258

MAT 9959

(n,p) Levels
293 Kelvin Cross Sections

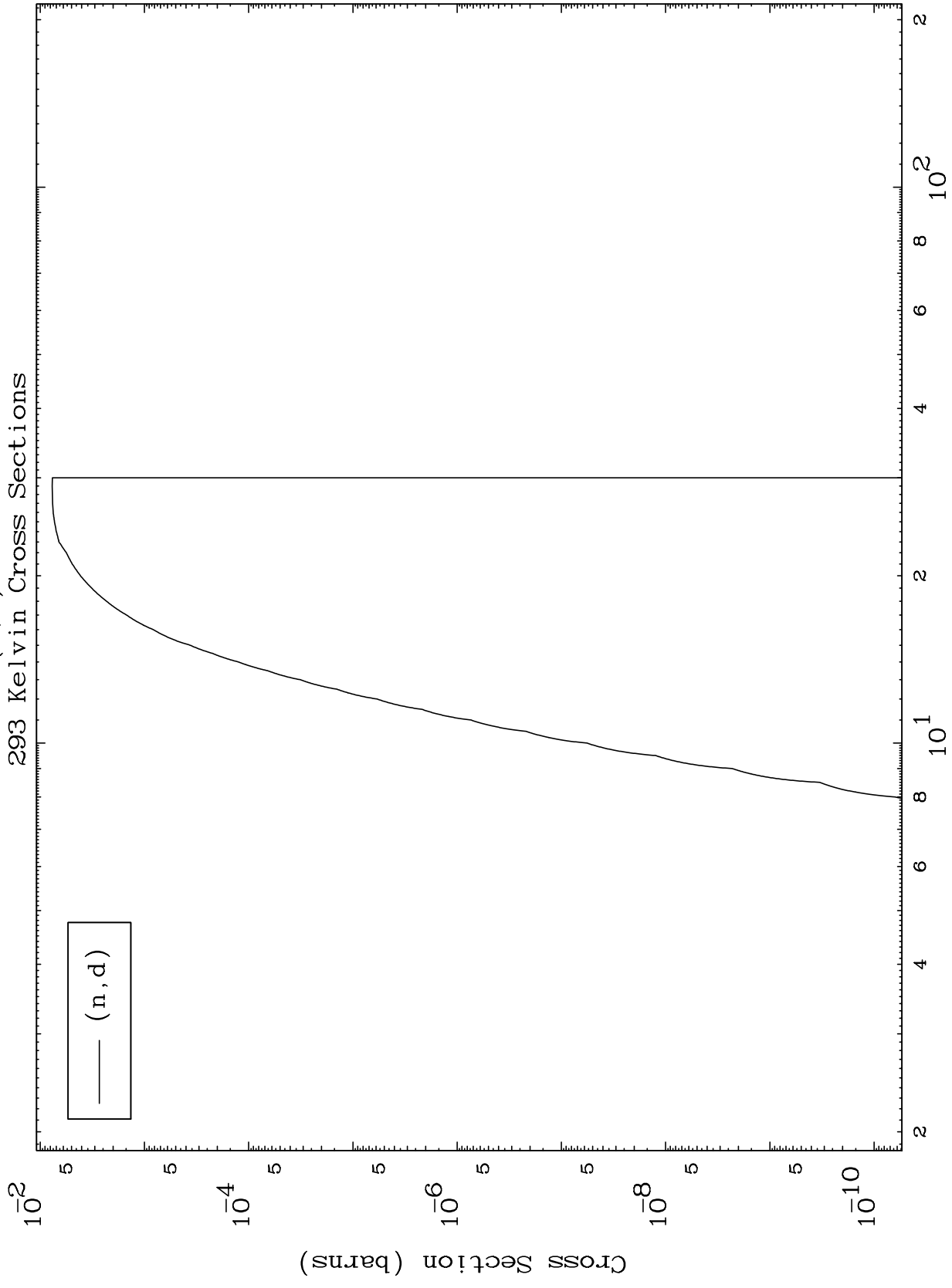
101-Md-258



MAT 9959

(n,d) Levels
293 Kelvin Cross Sections

101-Md-258



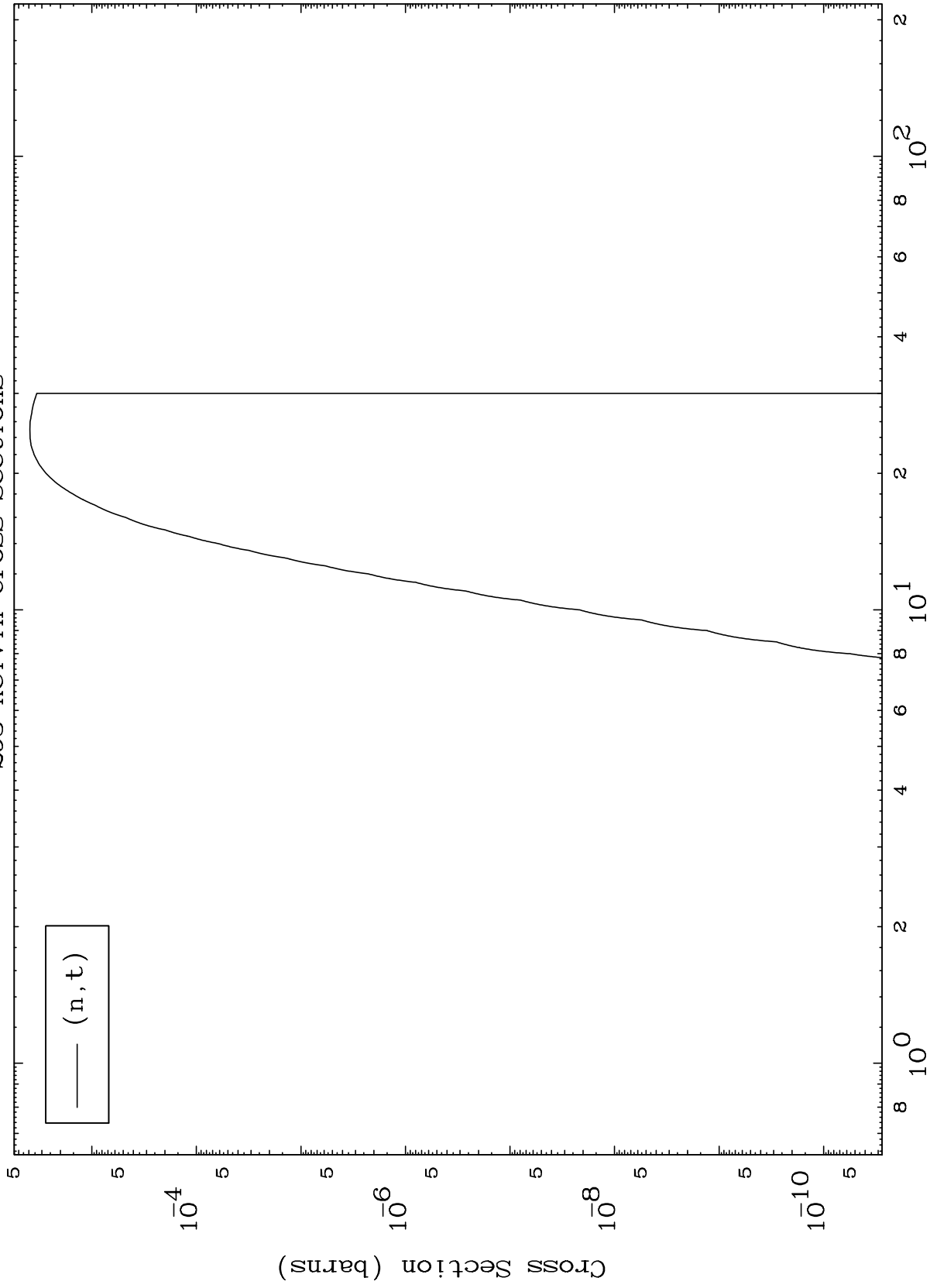
12

101-Md-258

MAT 9959

(n,t) Levels
293 Kelvin Cross Sections

101-Md-258



13

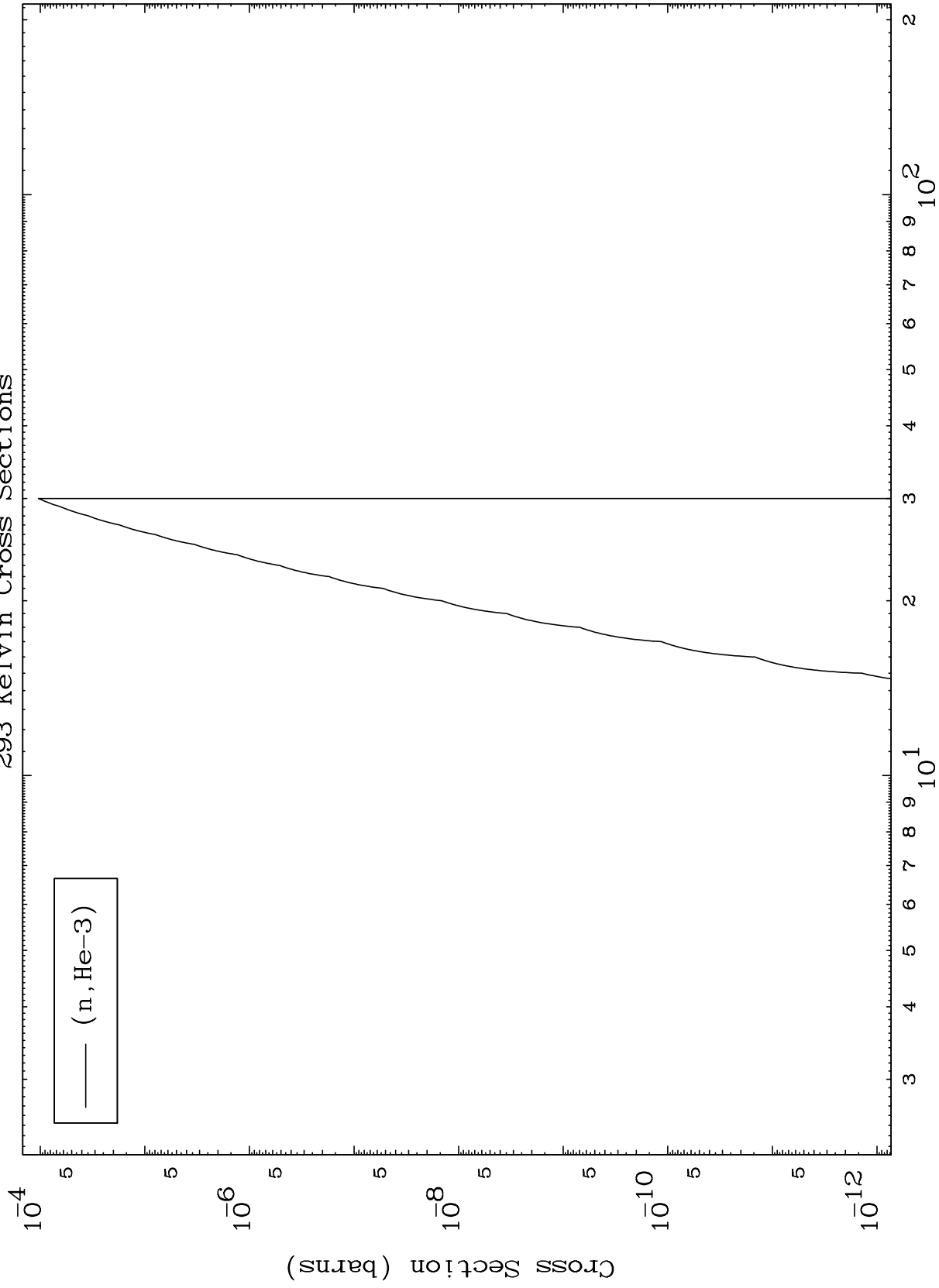
Incident Energy (MeV)

101-Md-258

MAT 9959

(n,He3) Levels
293 Kelvin Cross Sections

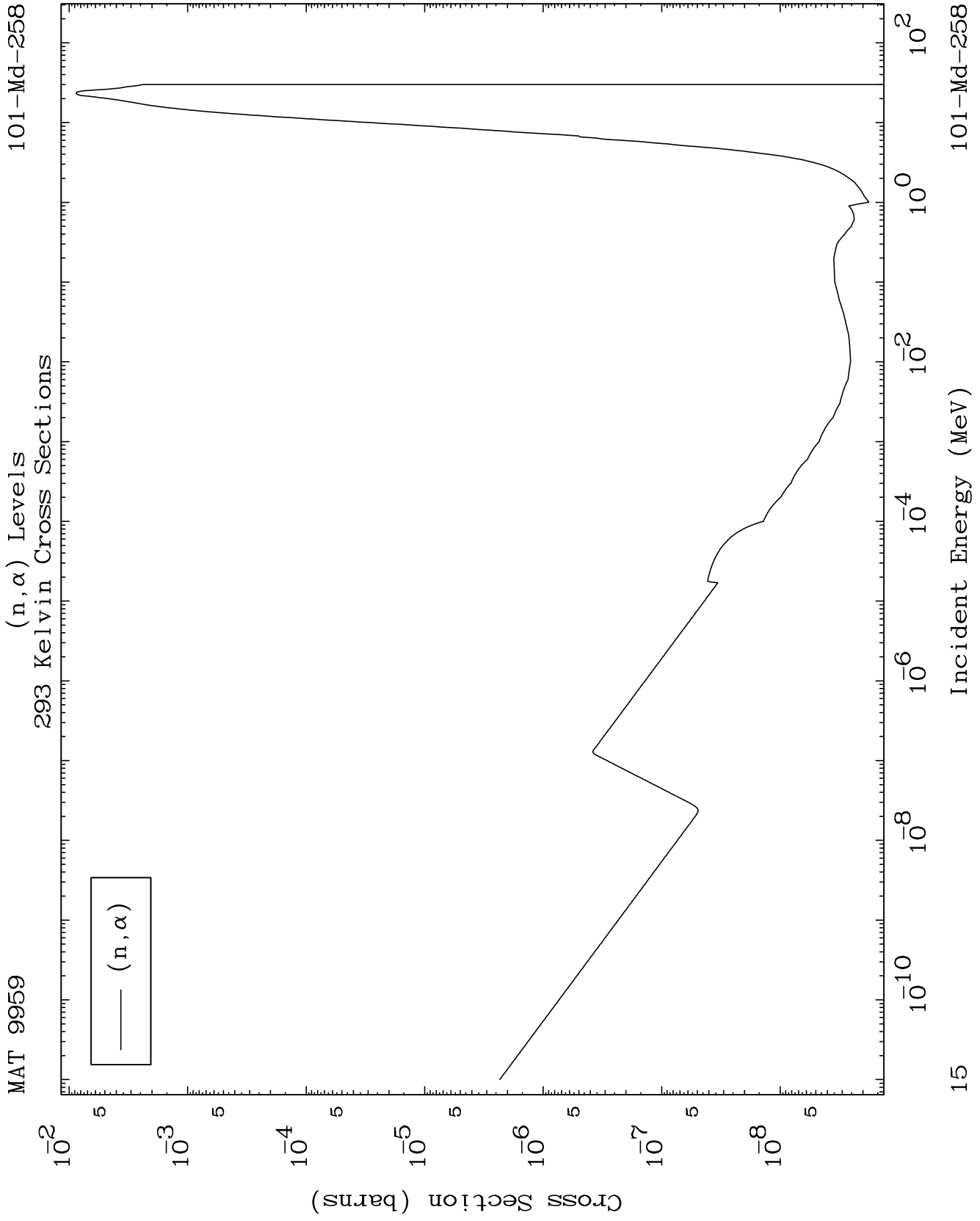
101-Md-258



14

Incident Energy (MeV)

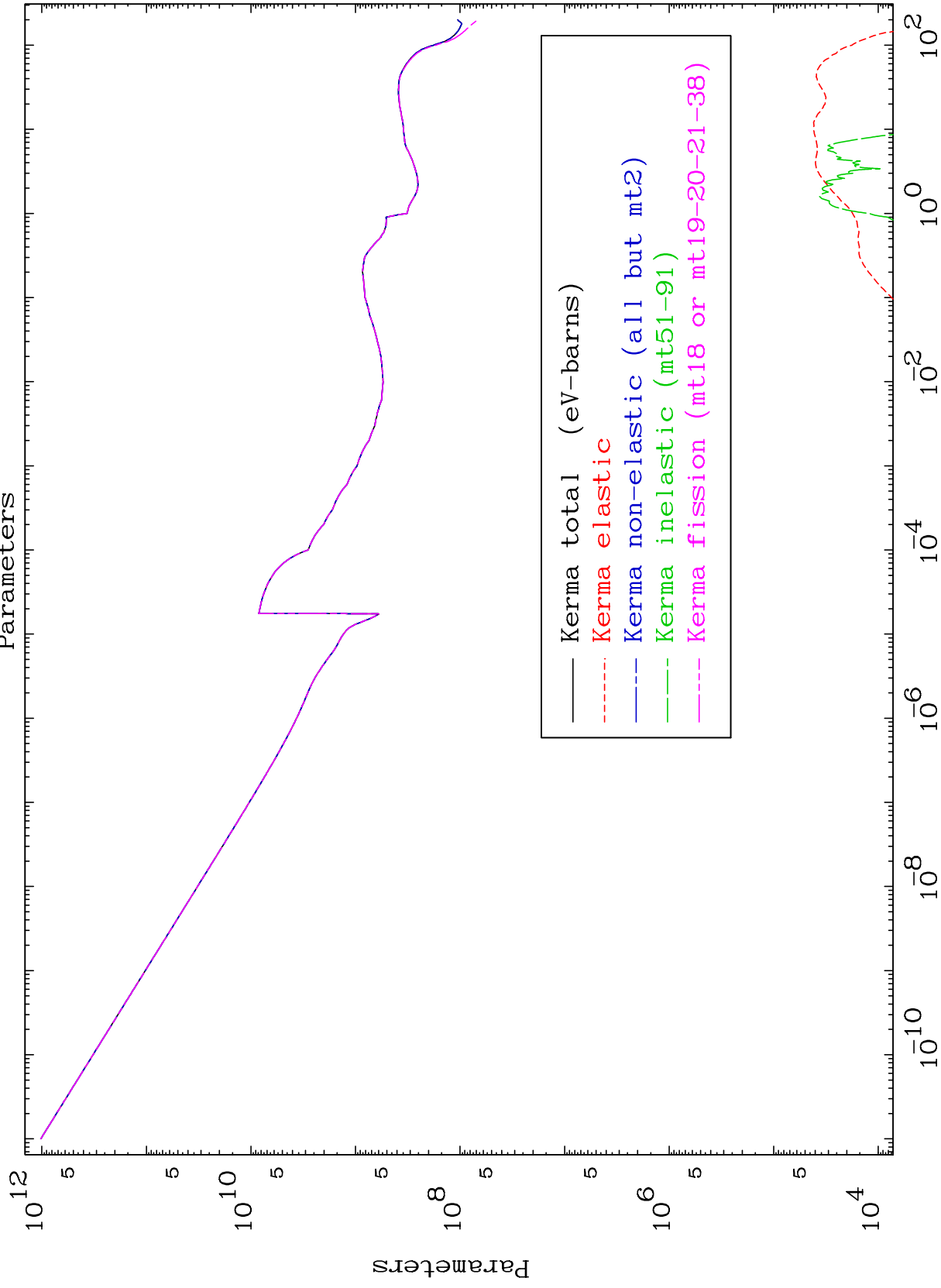
101-Md-258



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Energy Release
Parameters

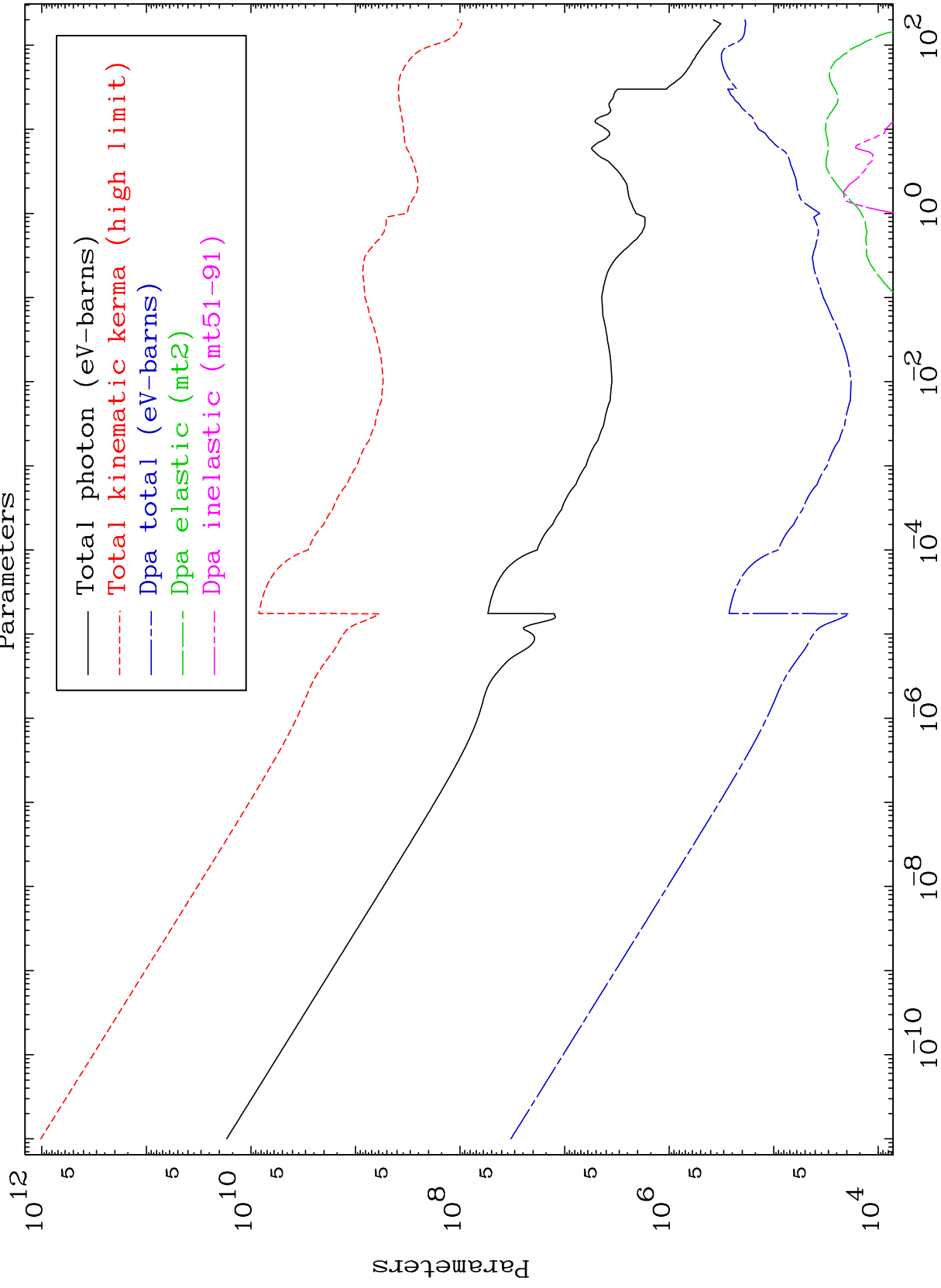
101-Md-258



16

Incident Energy (MeV)

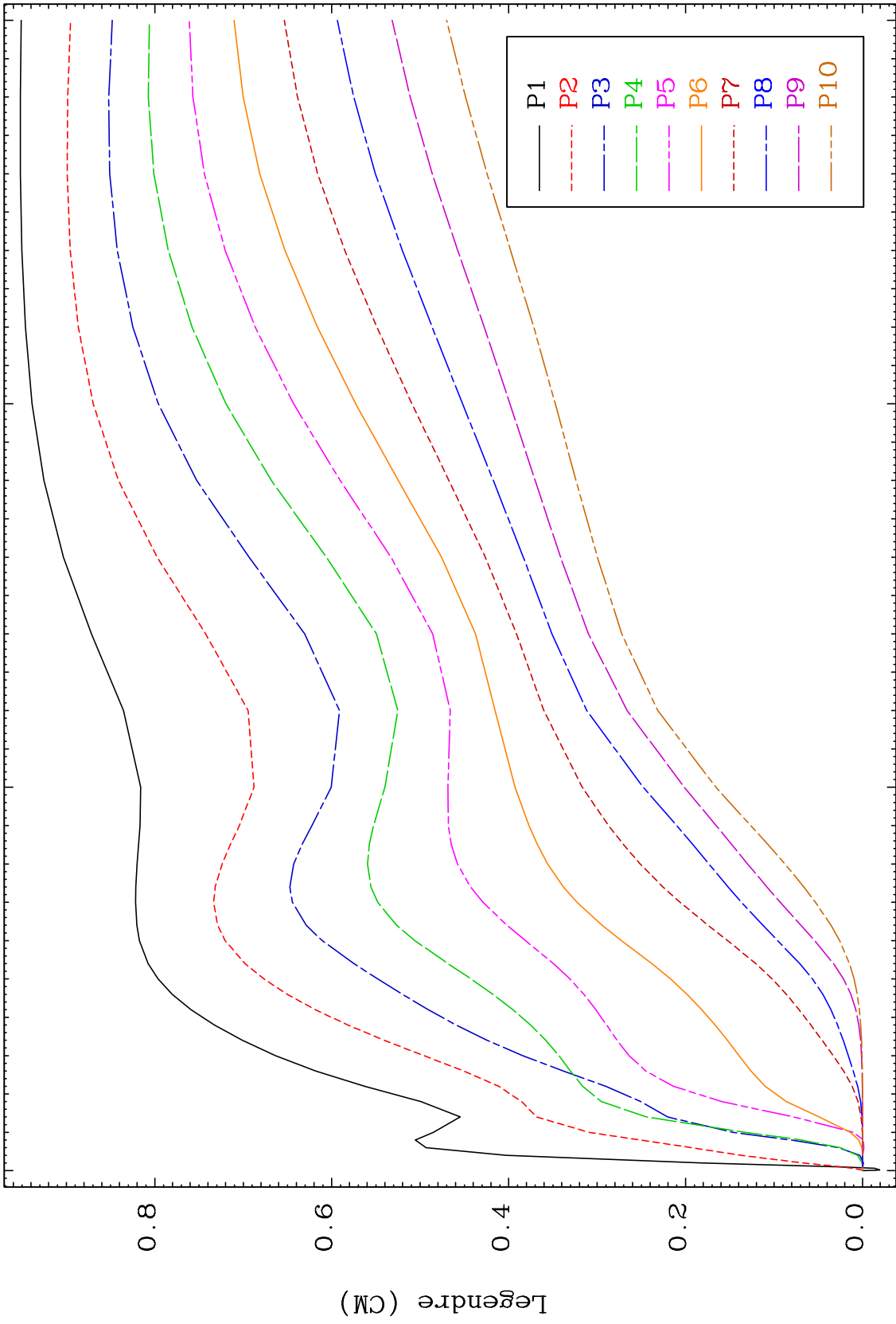
101-Md-258



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Elastic Legendre Coefficients

101-Md-258



18

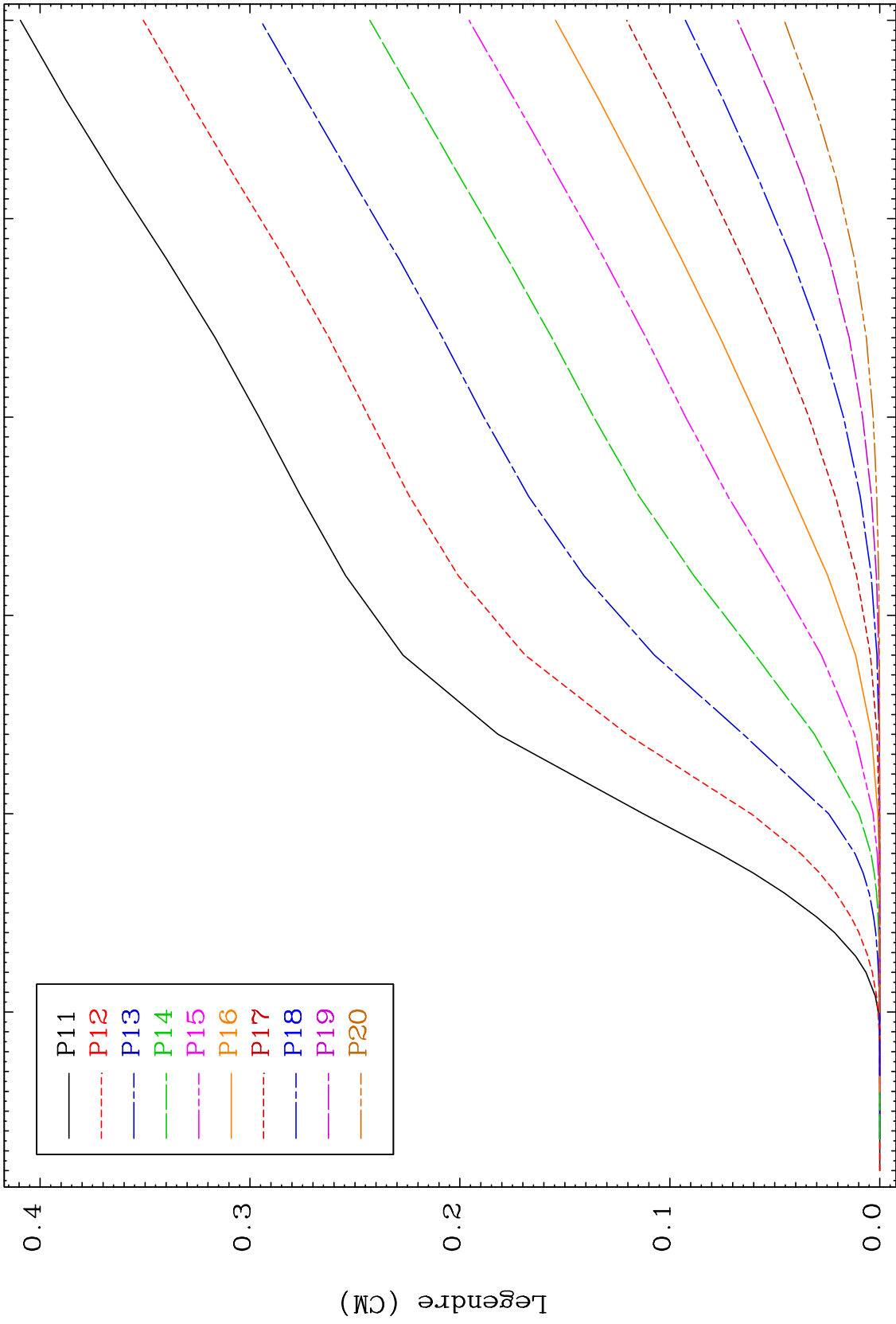
Incident Energy (MeV)

101-Md-258

MAT 9959

Elastic Legendre Coefficients

101-Md-258



19

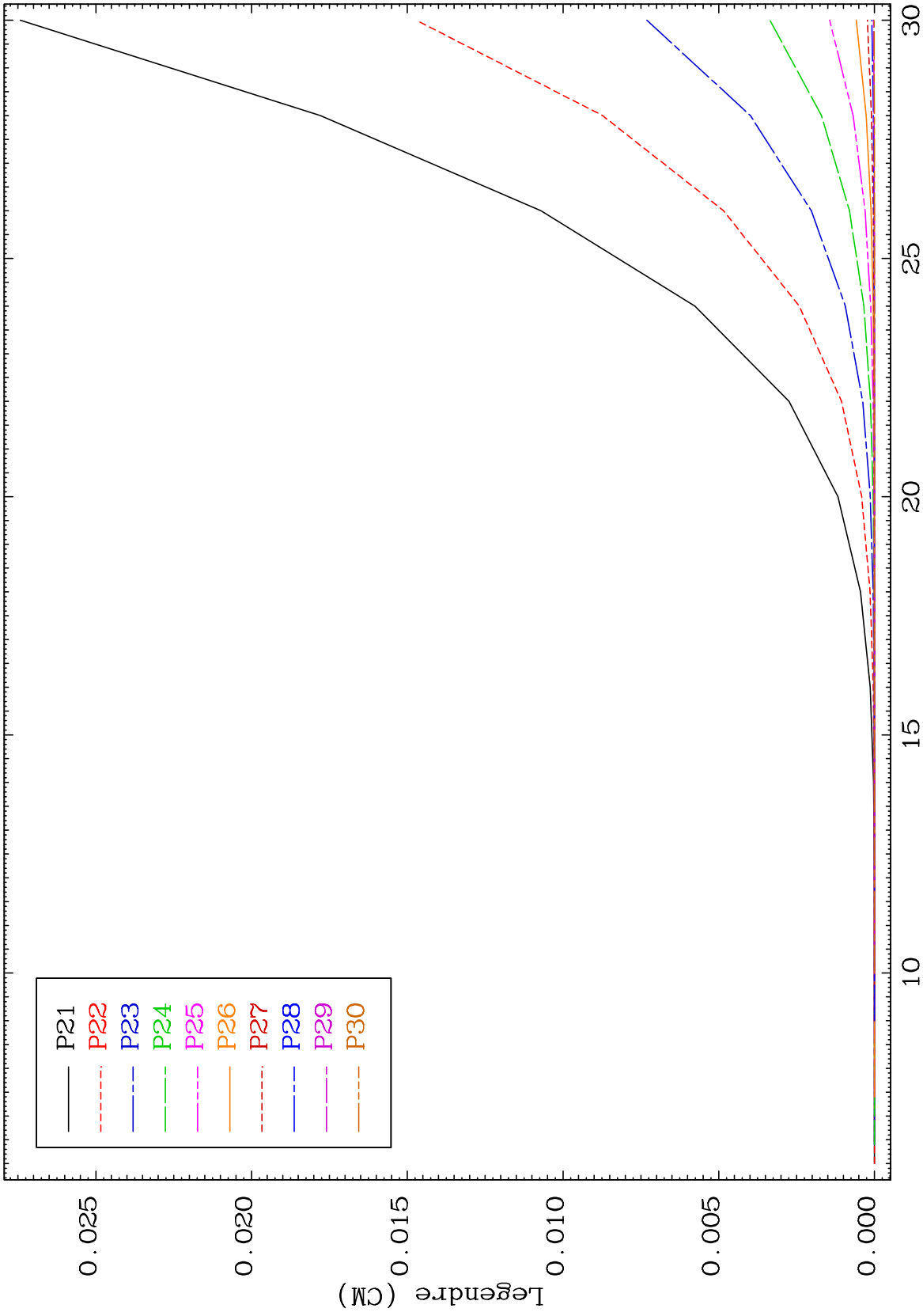
Incident Energy (MeV)

101-Md-258

MAT 9959

Elastic Legendre Coefficients

101-Md-258



101-Md-258

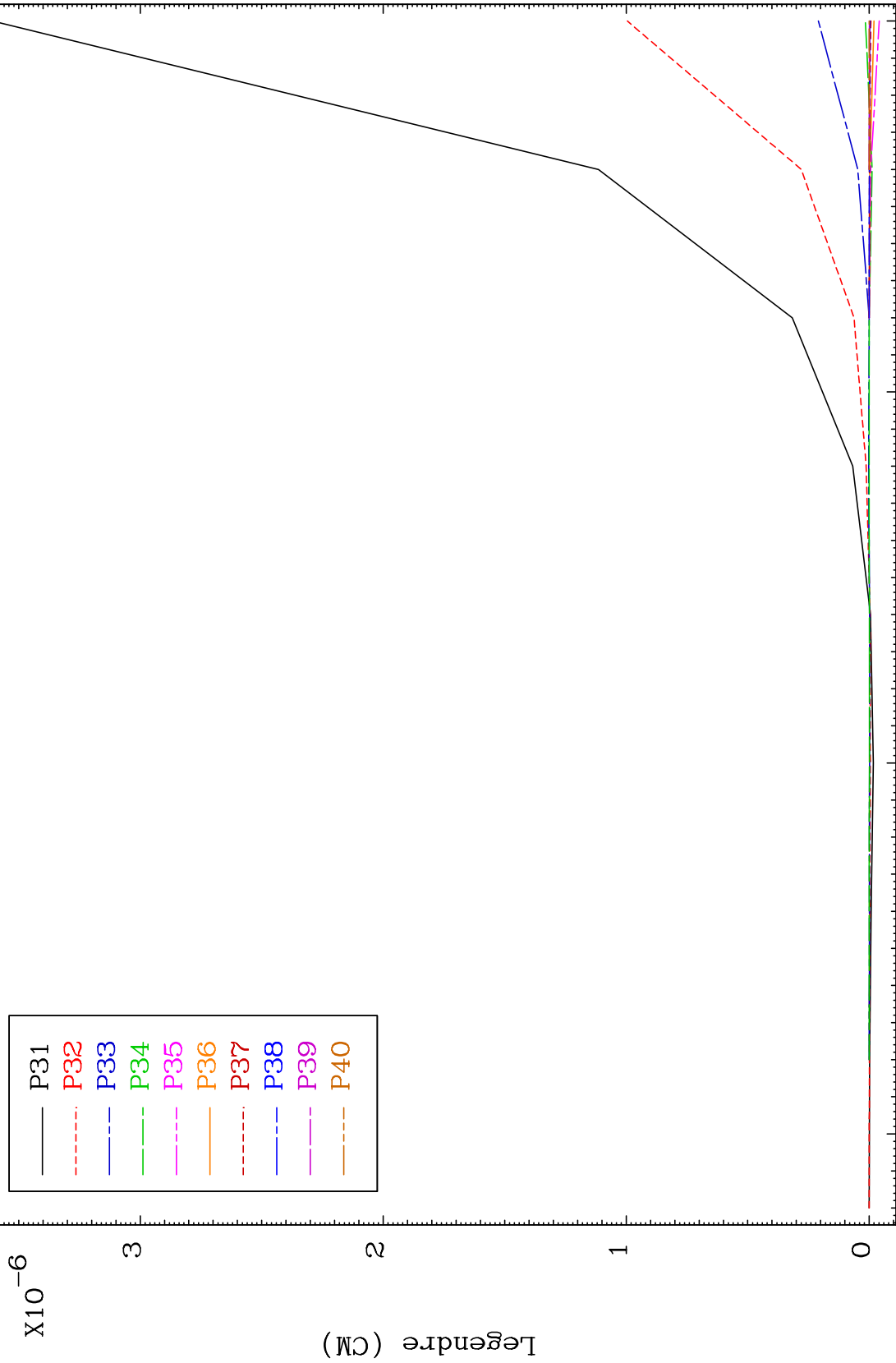
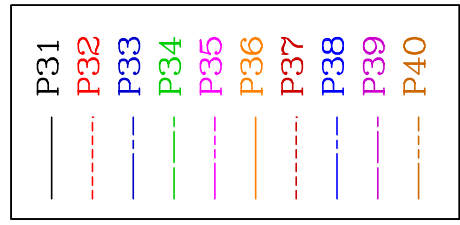
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

101-Md-258



21

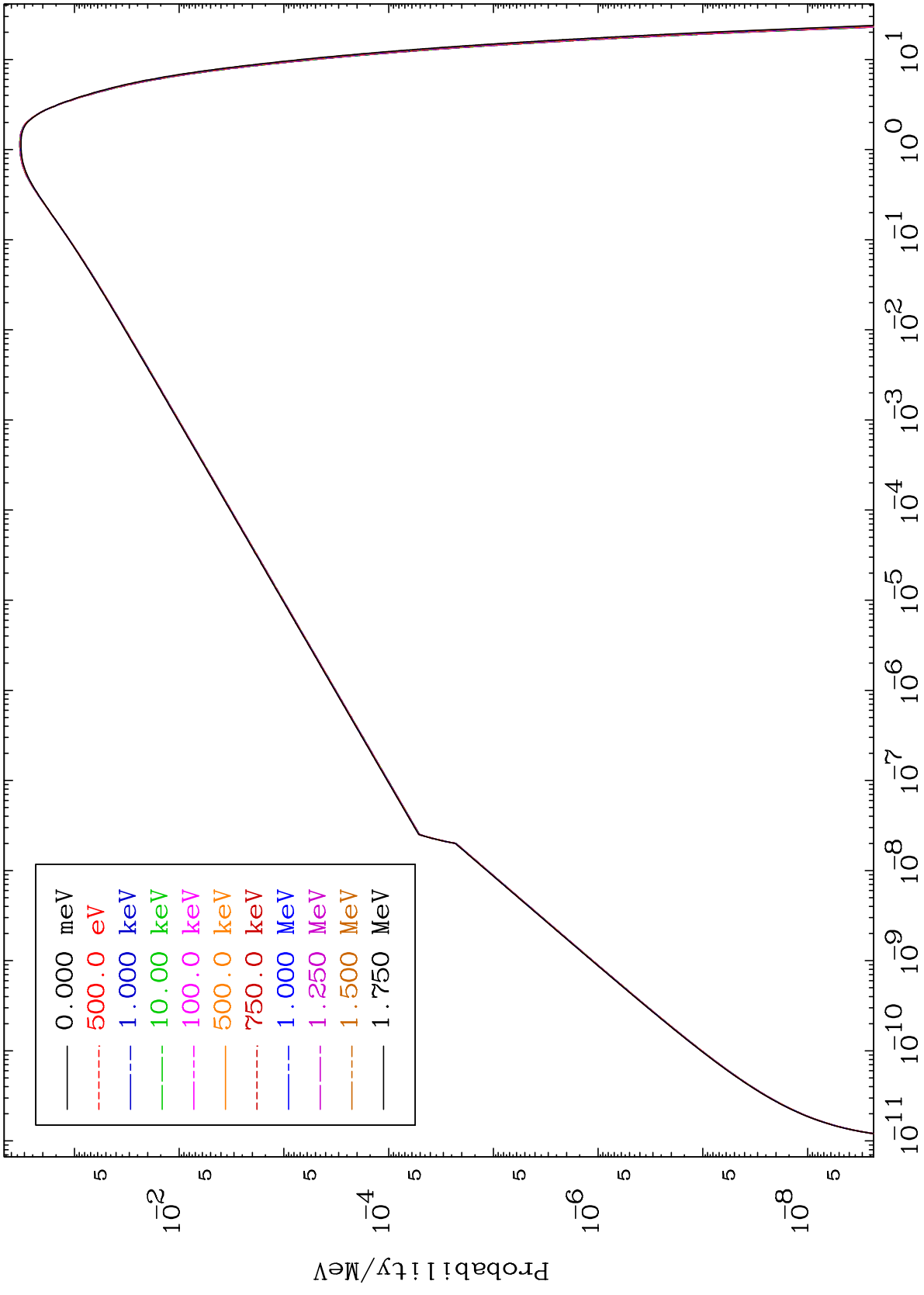
Incident Energy (MeV)

101-Md-258

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Fission Energy Distribution

101-Md-258

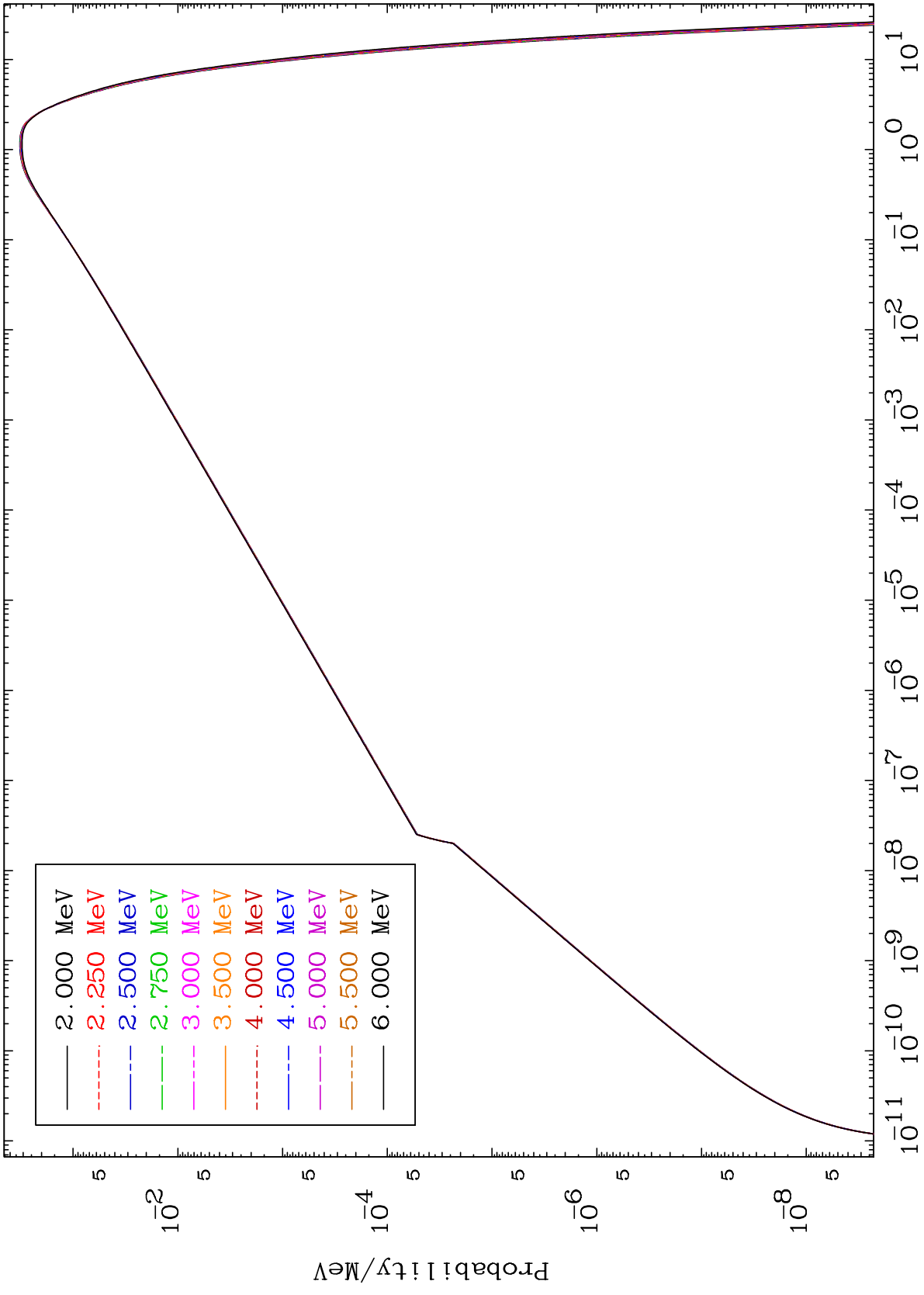


101-Md-258

MAT 9959

Fission Energy Distribution

101-Md-258

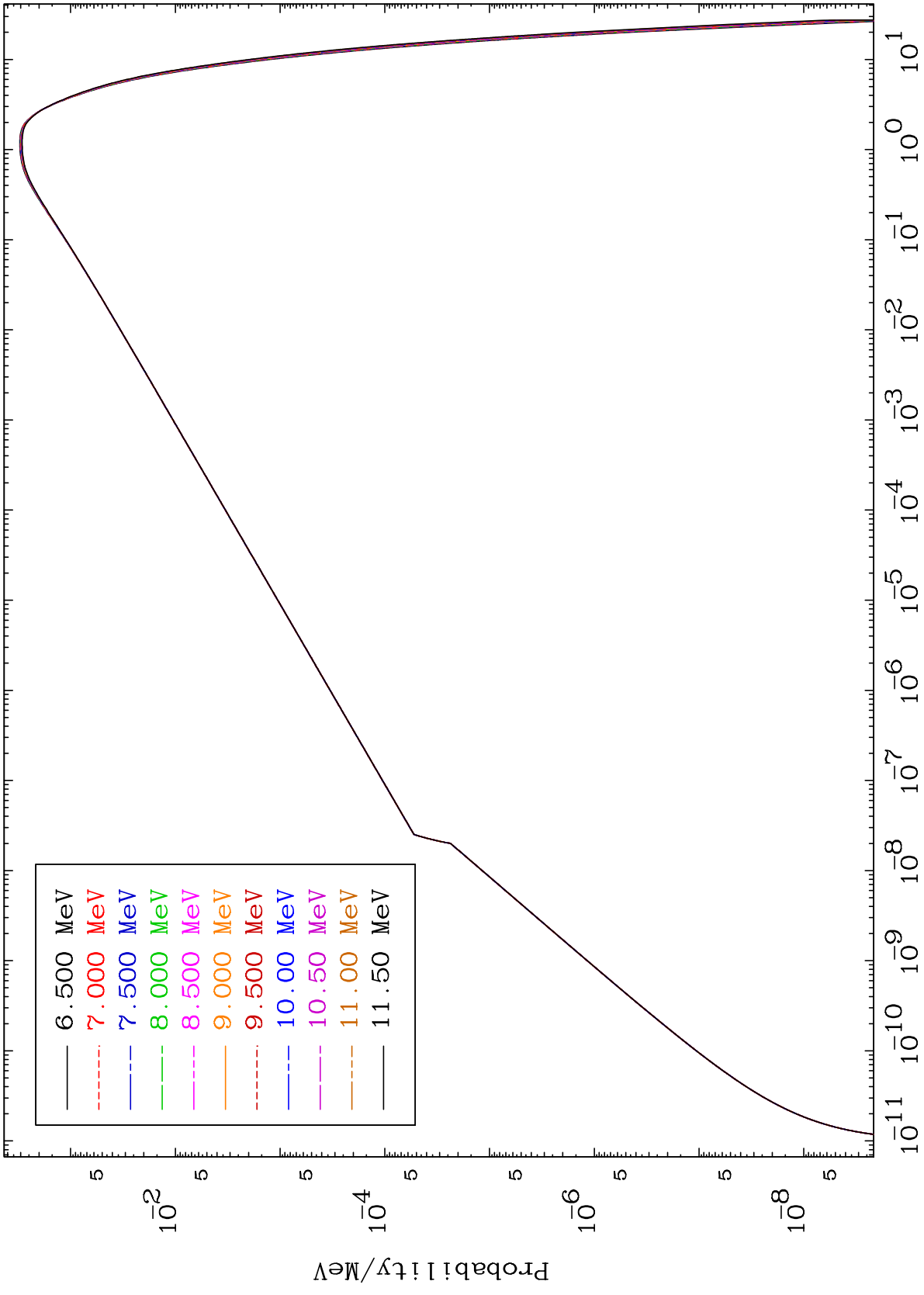


101-Md-258

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Fission Energy Distribution

101-Md-258

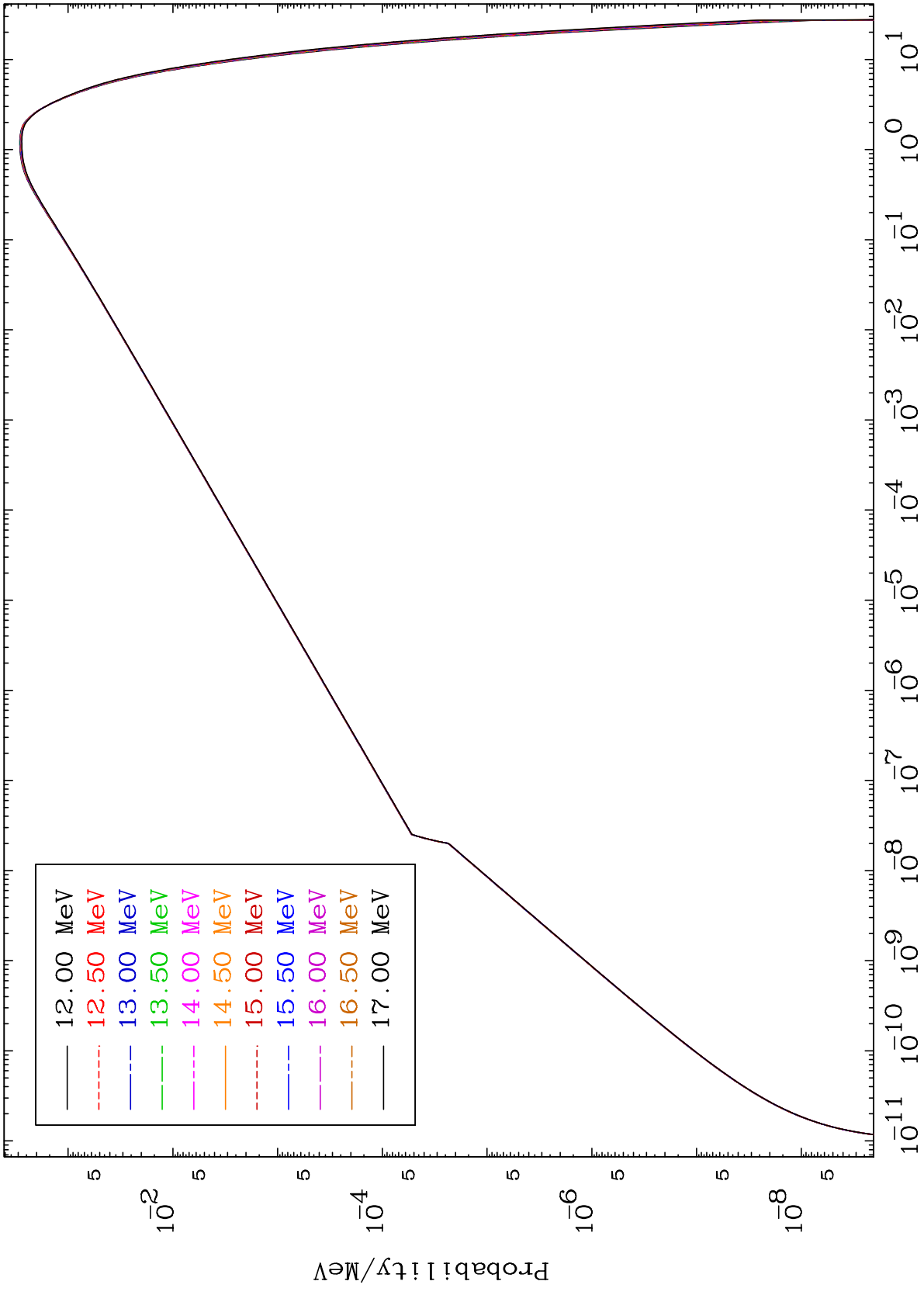


101-Md-258

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Fission Energy Distribution

101-Md-258

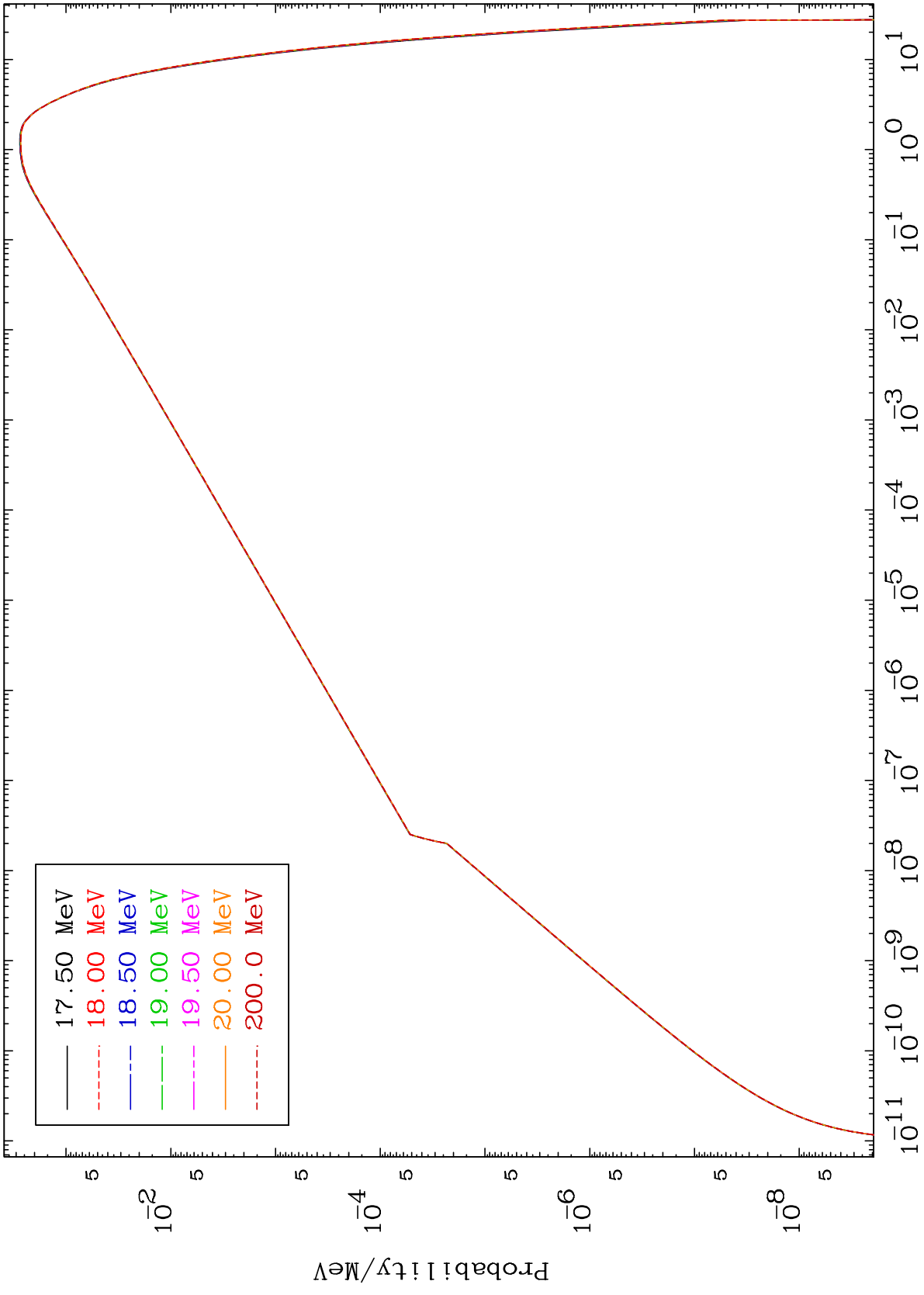


101-Md-258

MAT 9959

Fission Energy Distribution

101-Md-258



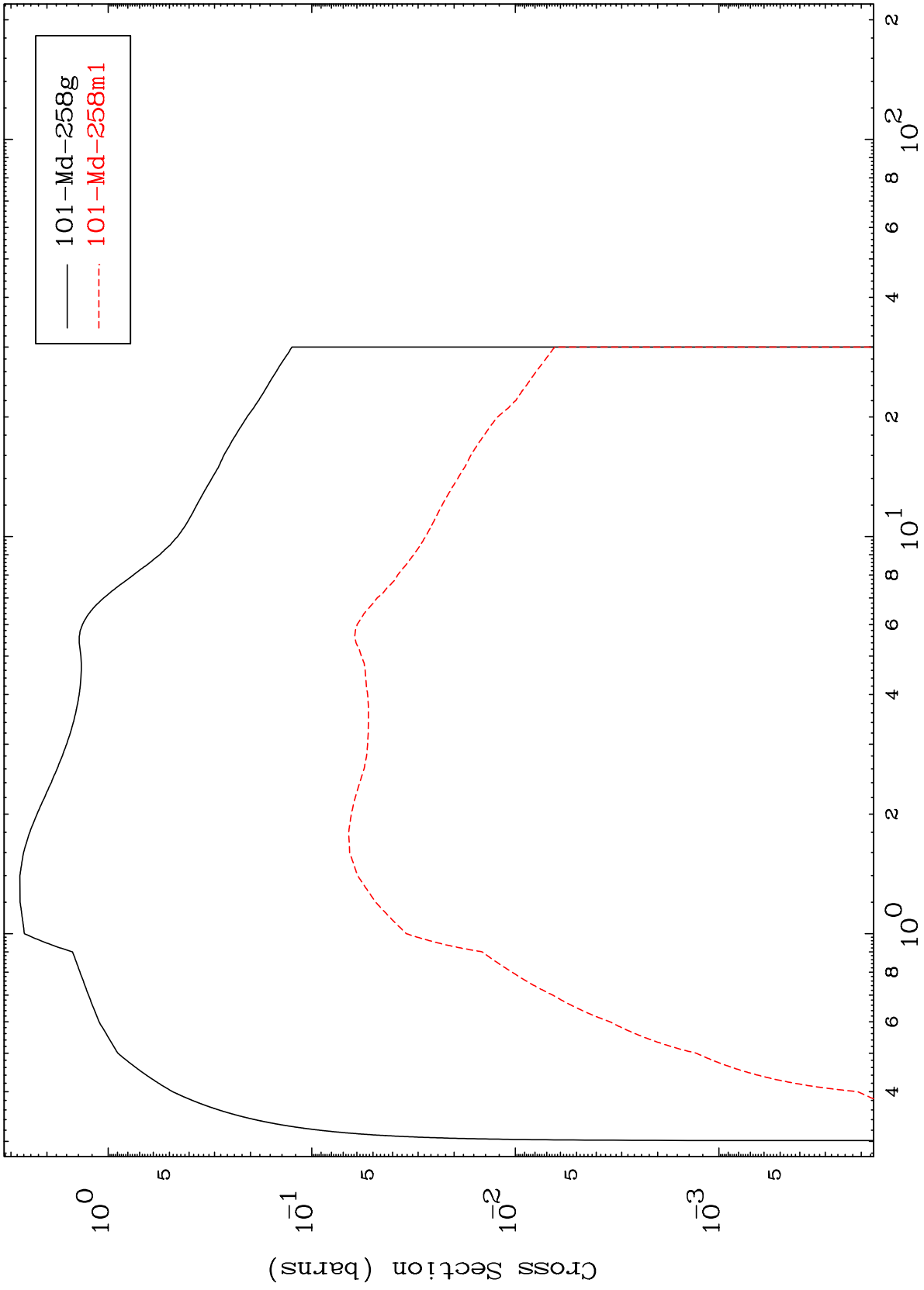
—	17.50	MeV
- - -	18.00	MeV
- · - ·	18.50	MeV
- - -	19.00	MeV
- · - ·	19.50	MeV
- - -	20.00	MeV
- - -	200.0	MeV

101-Md-258

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101-Md-258

Inelastic Radionuclide Production Cross Section



27

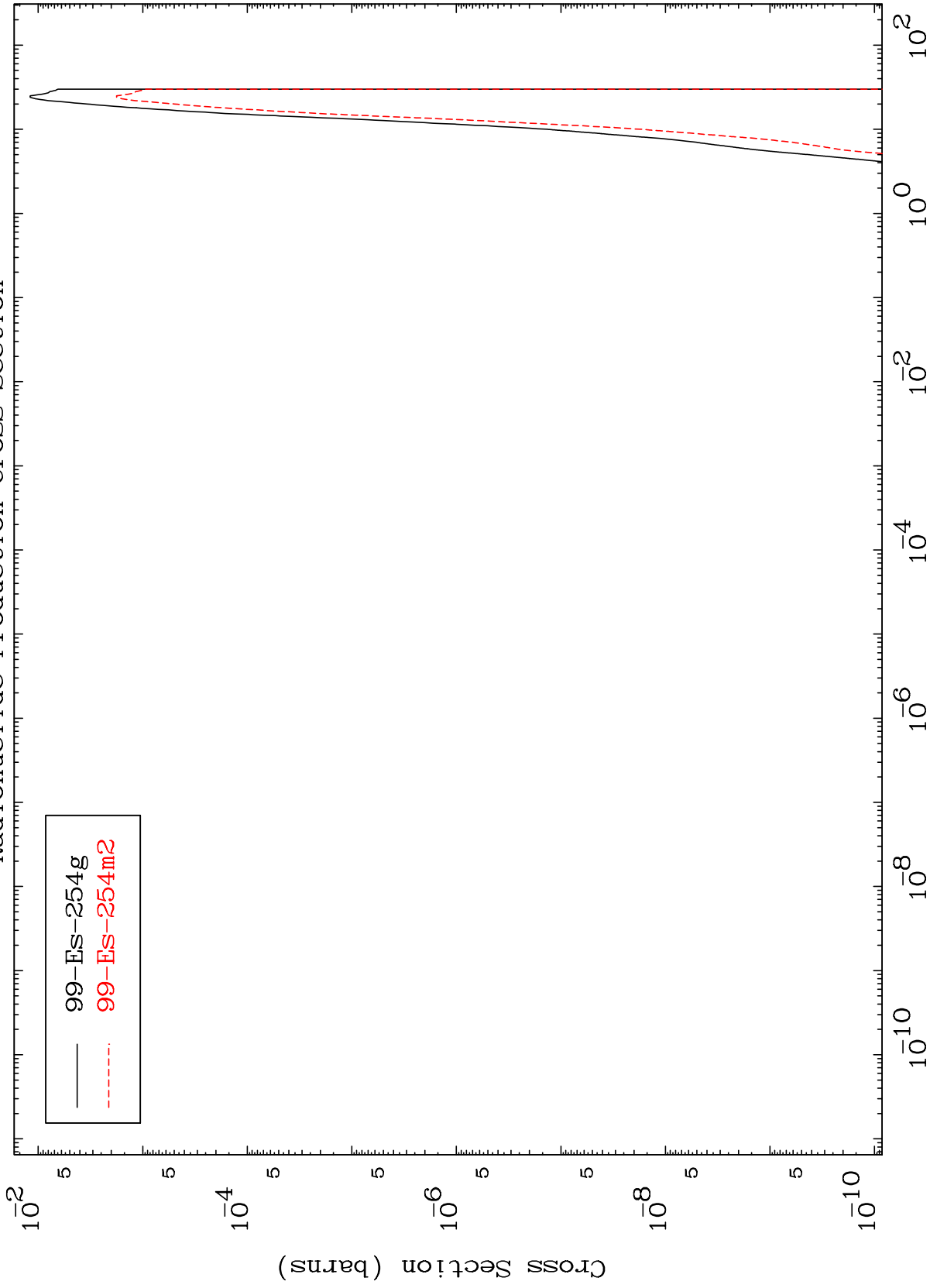
101-Md-258

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(n,n') α

101-Md-258

Radionuclide Production Cross Section



28

Incident Energy (MeV)

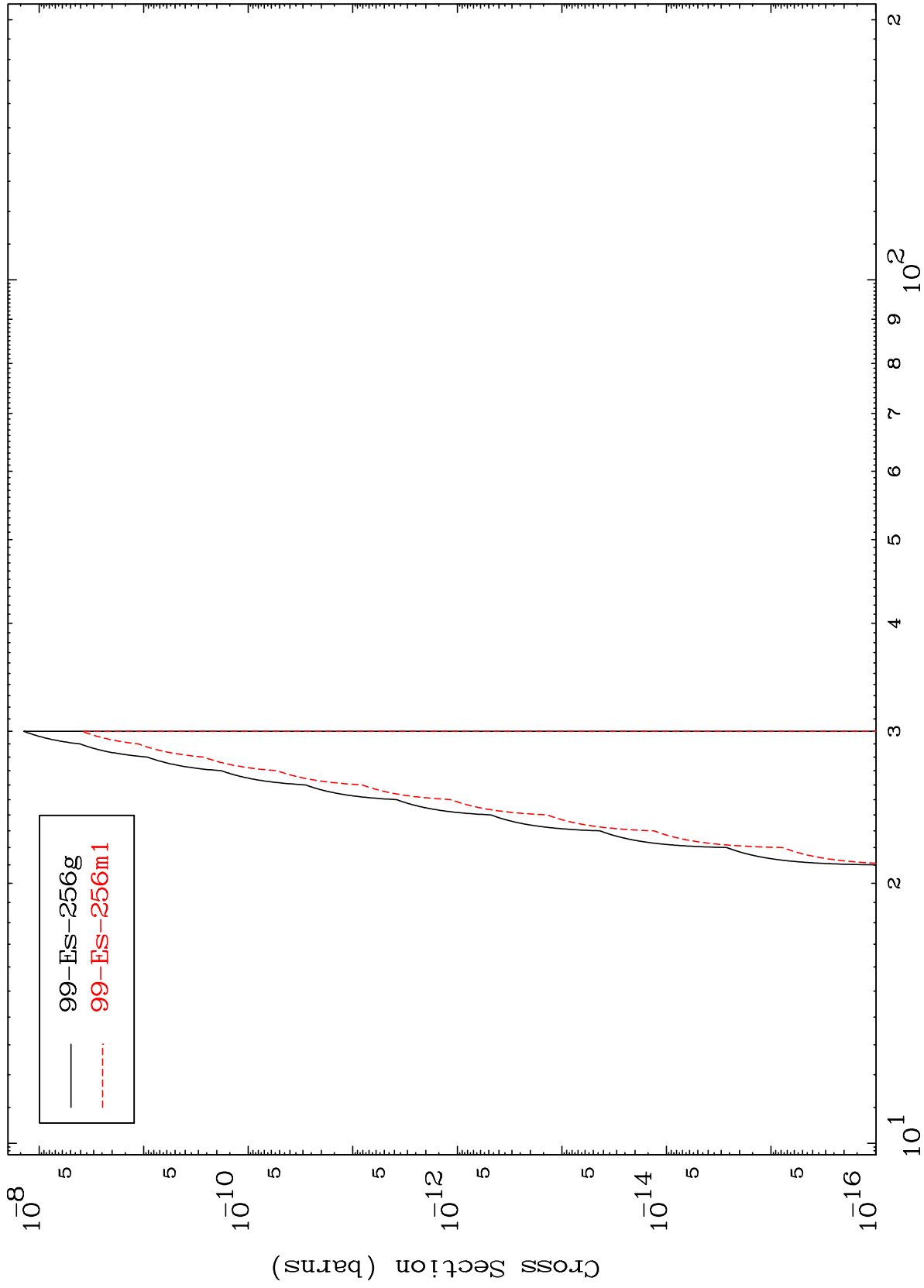
101-Md-258

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(n,2n) p

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Radionuclide Production Cross Section



99-Es-256g
99-Es-256m1

29

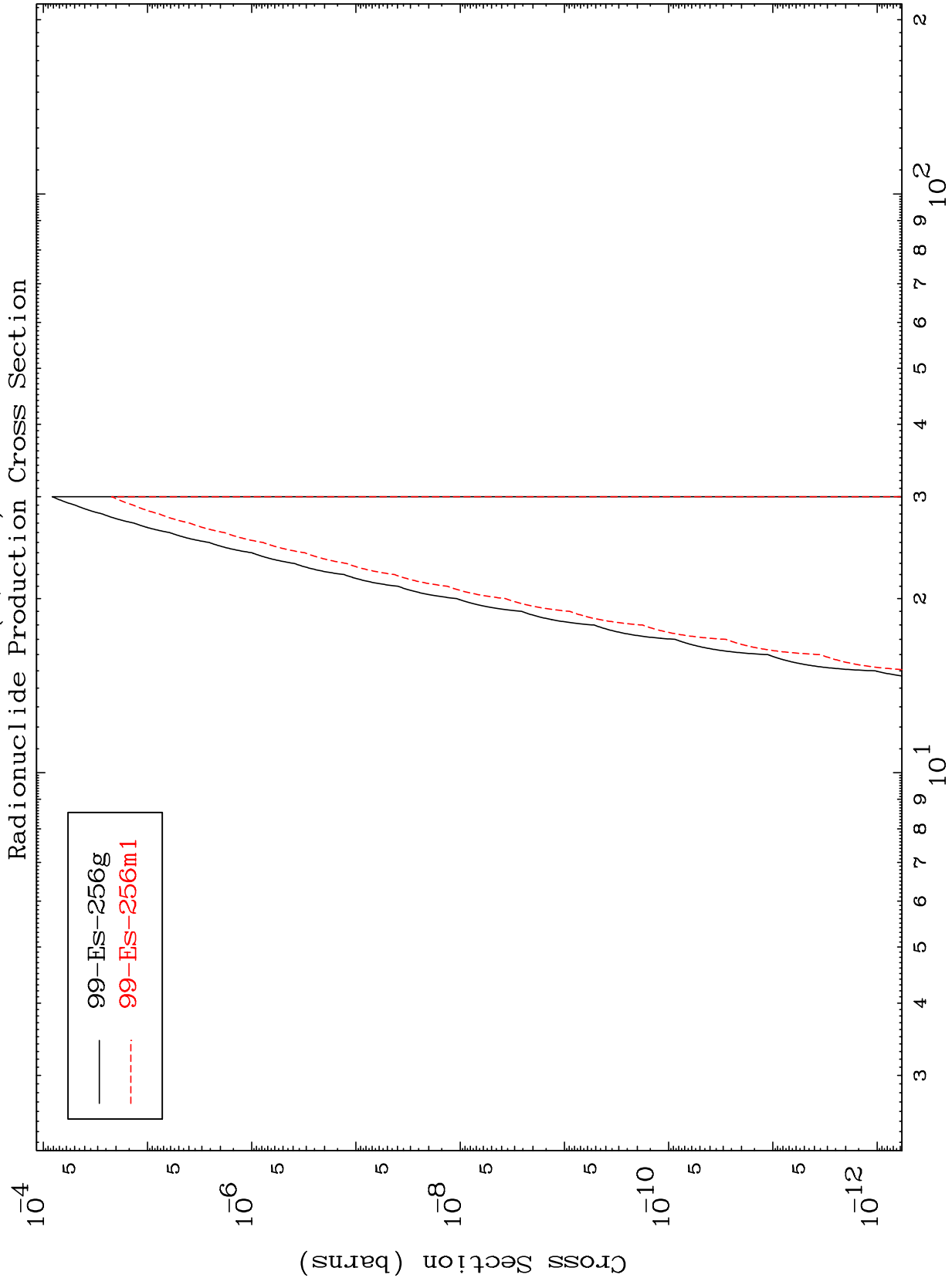
Incident Energy (MeV)

101-Md-258

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(n, He-3)

101-Md-258



30

Incident Energy (MeV)

101-Md-258