

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

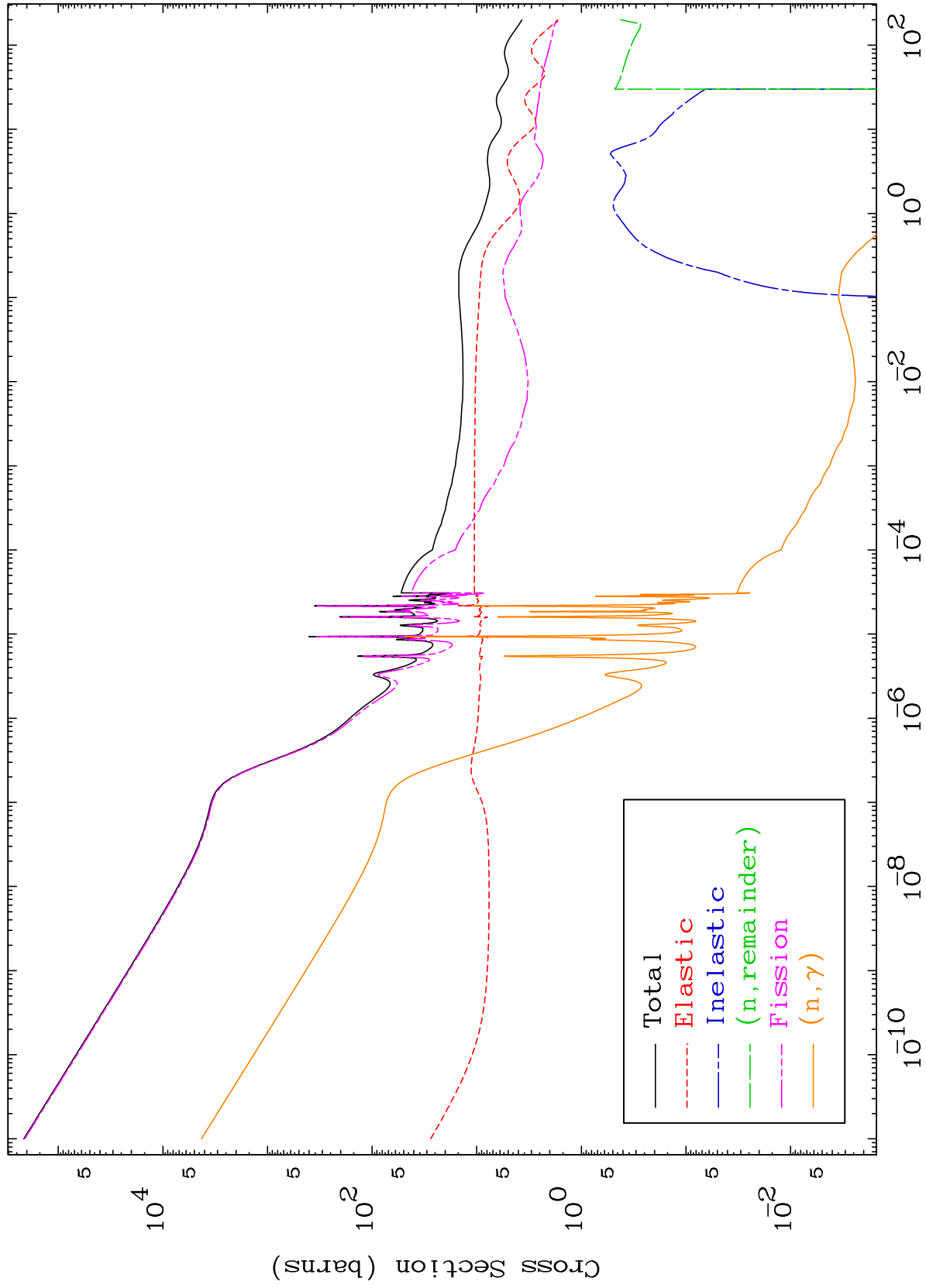
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9961

Major  
293 Kelvin Cross Sections

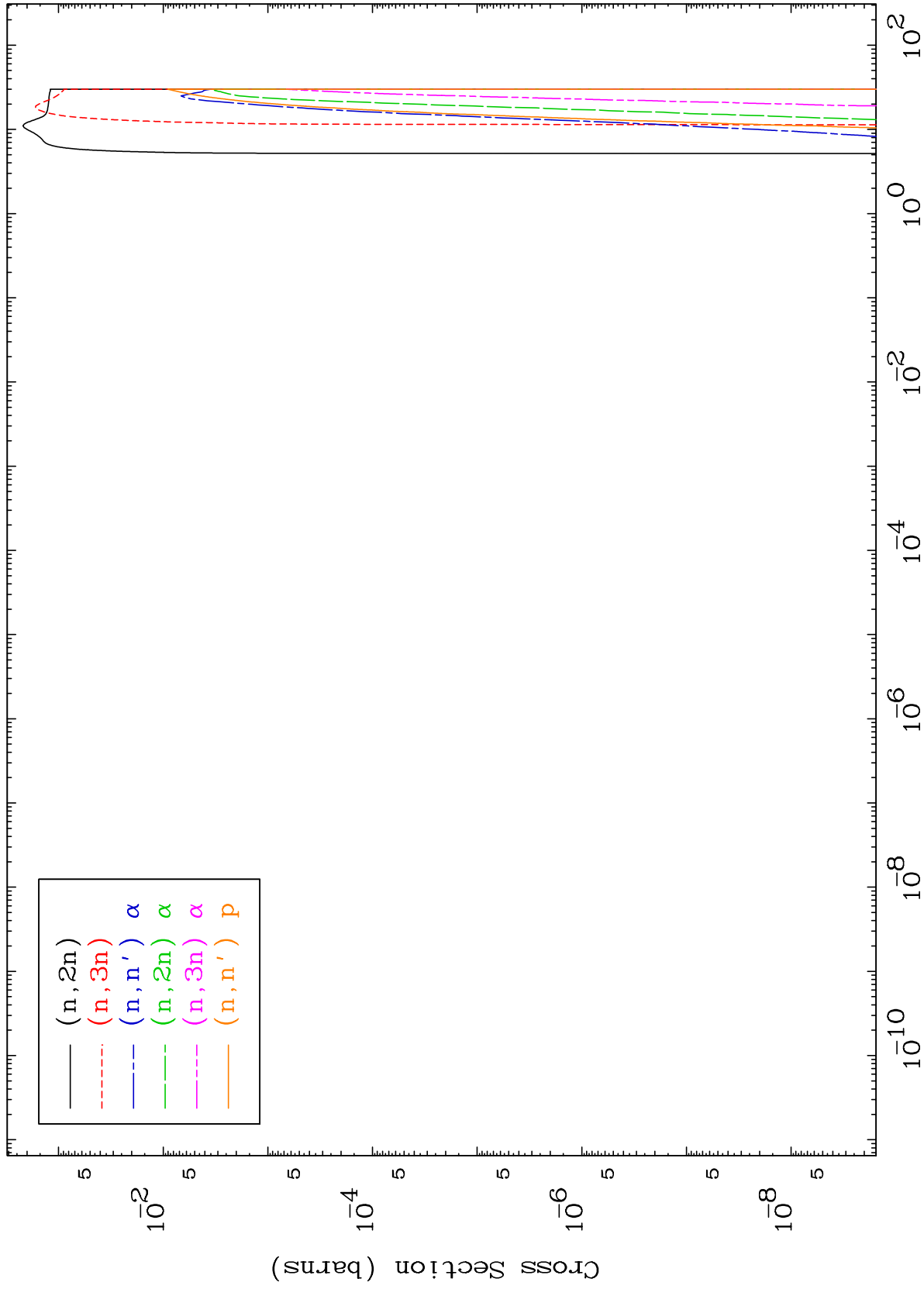
101-Md-260



MAT 9961

Neutron Production  
293 Kelvin Cross Sections

101-Md-260



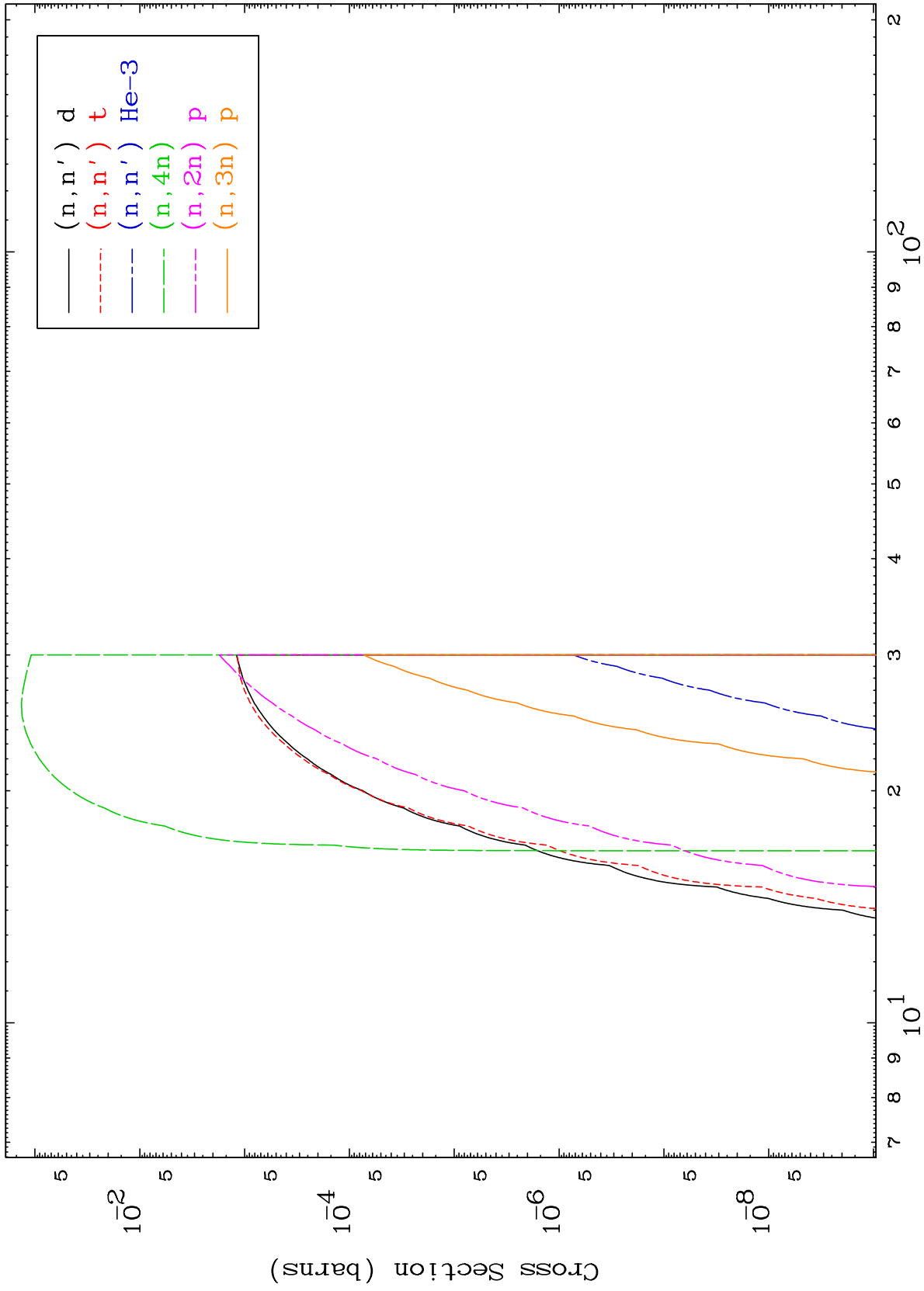
2

101-Md-260

MAT 9961

Neutron Production  
293 Kelvin Cross Sections

101-Md-260



3

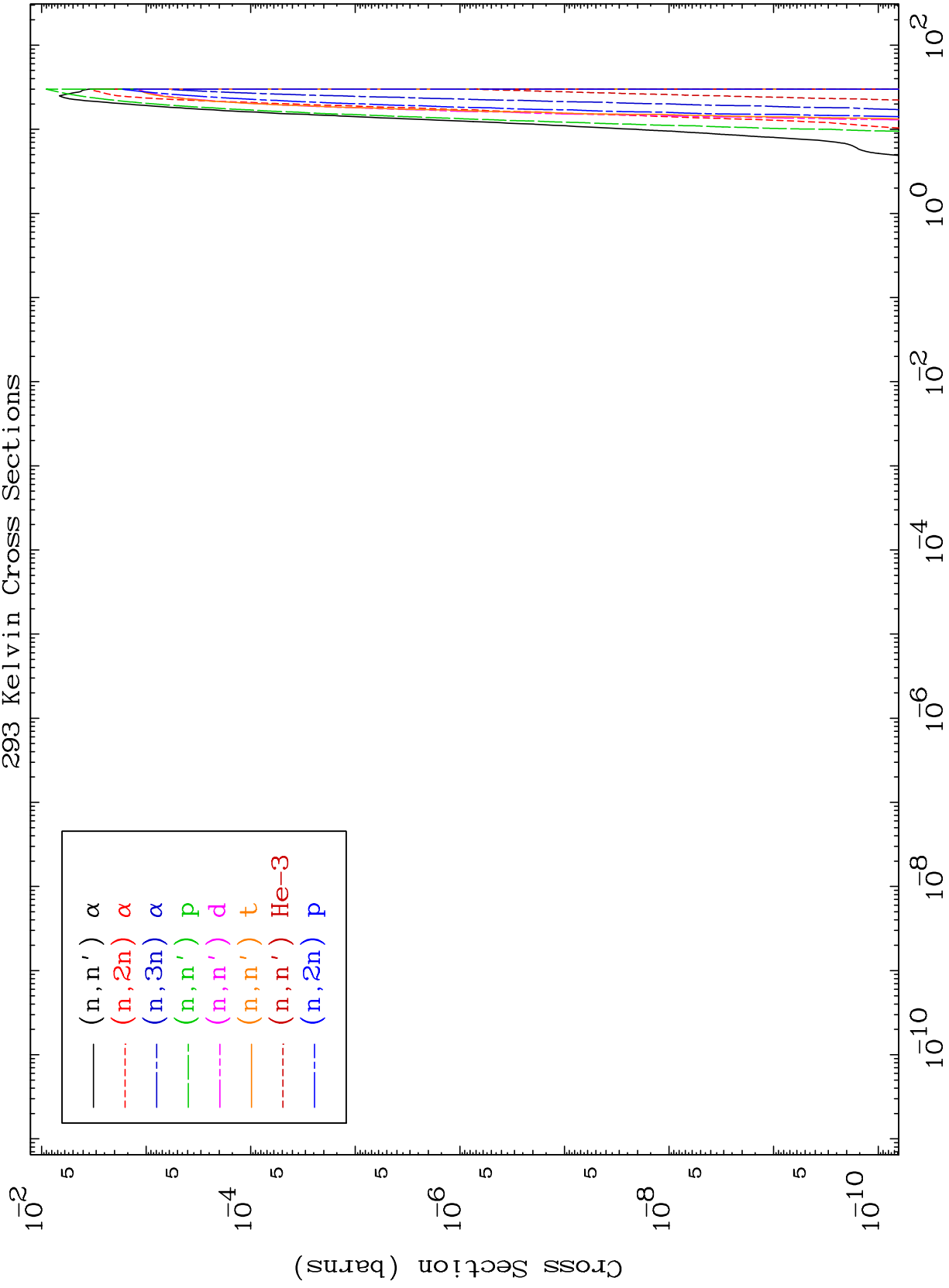
Incident Energy (MeV)

101-Md-260

MAT 9961

Charged Particle  
293 Kelvin Cross Sections

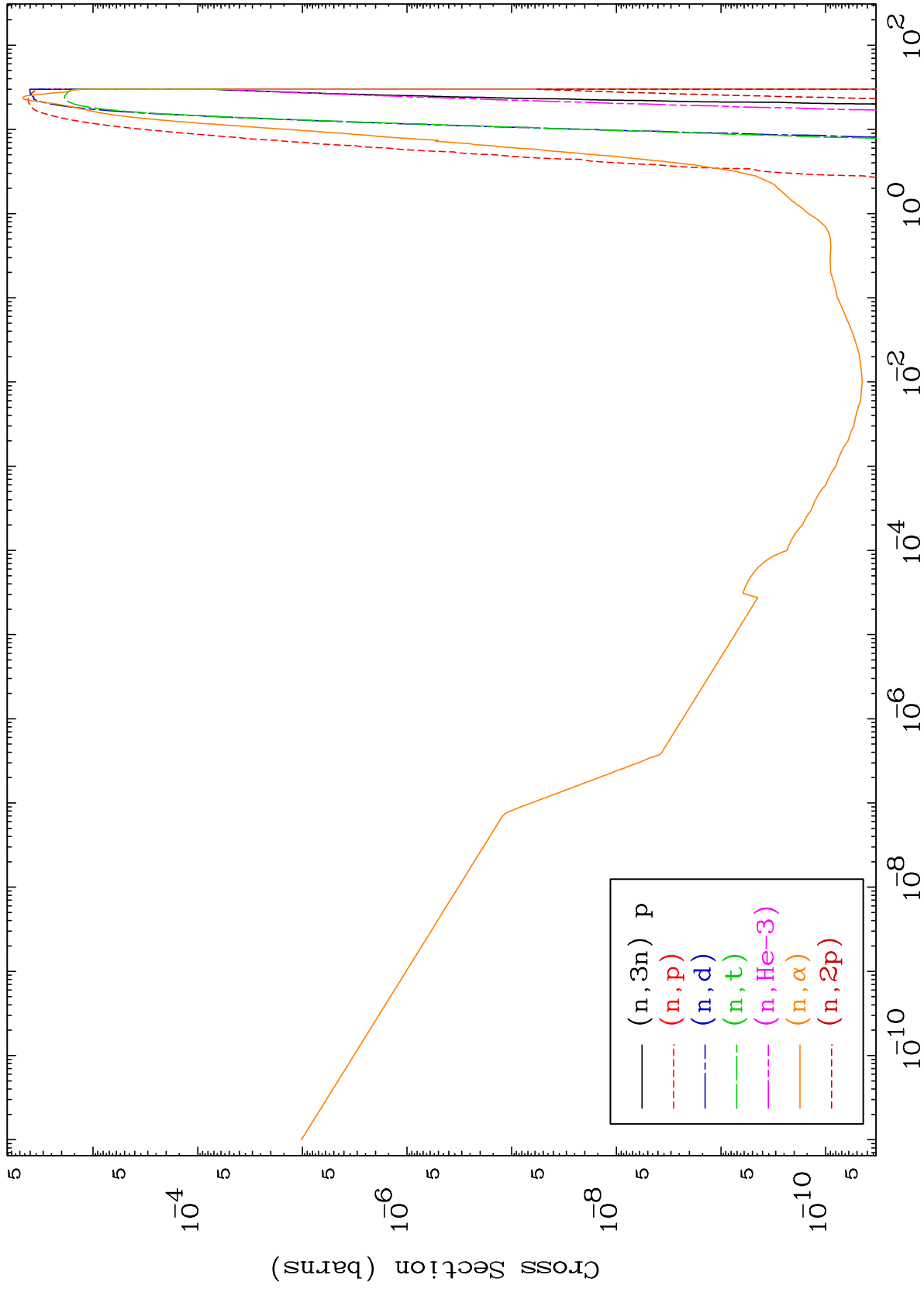
101-Md-260



MAT 9961

Charged Particle  
293 Kelvin Cross Sections

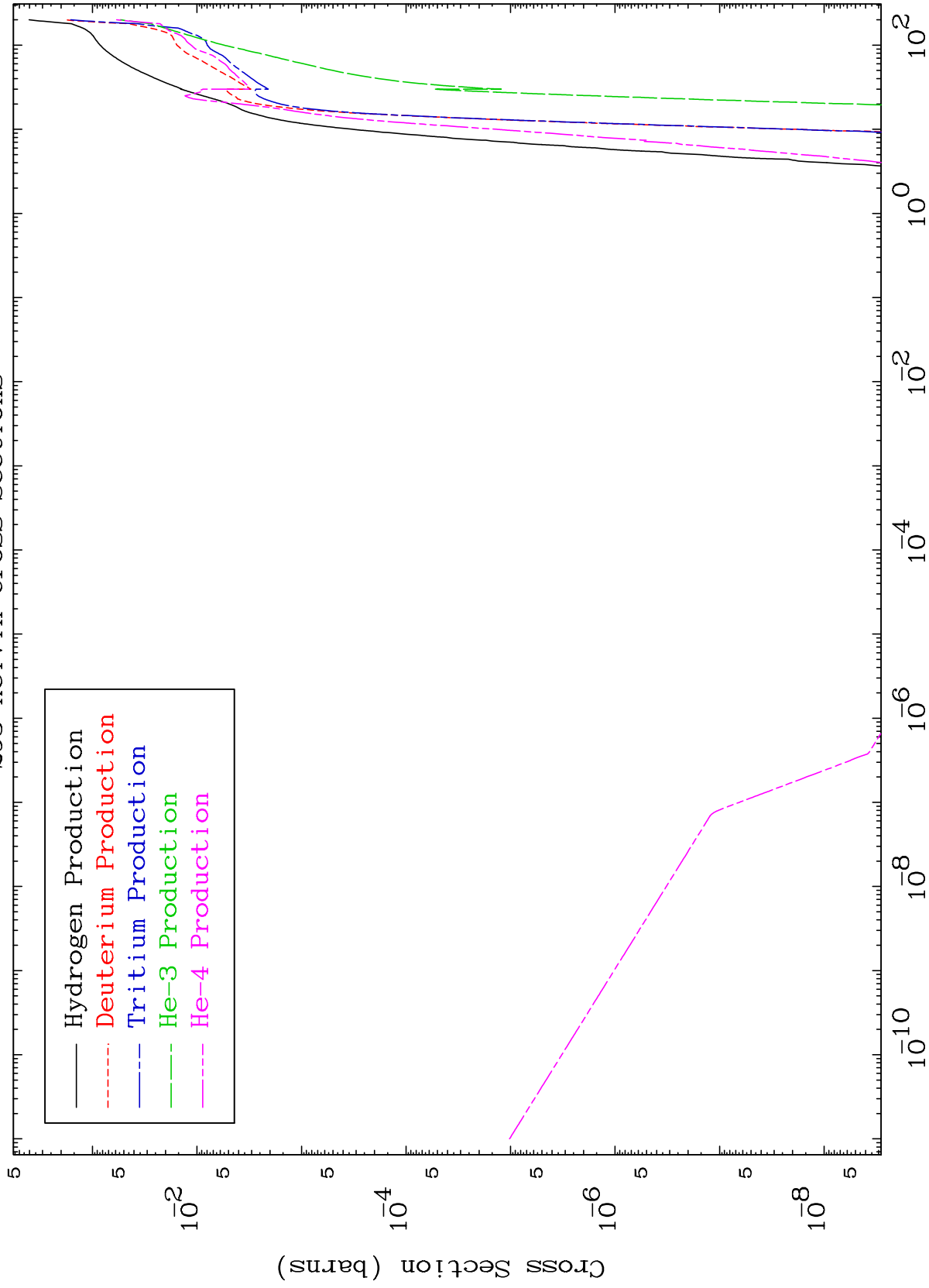
101-Md-260



MAT 9961

Particle Production  
293 Kelvin Cross Sections

101-Md-260



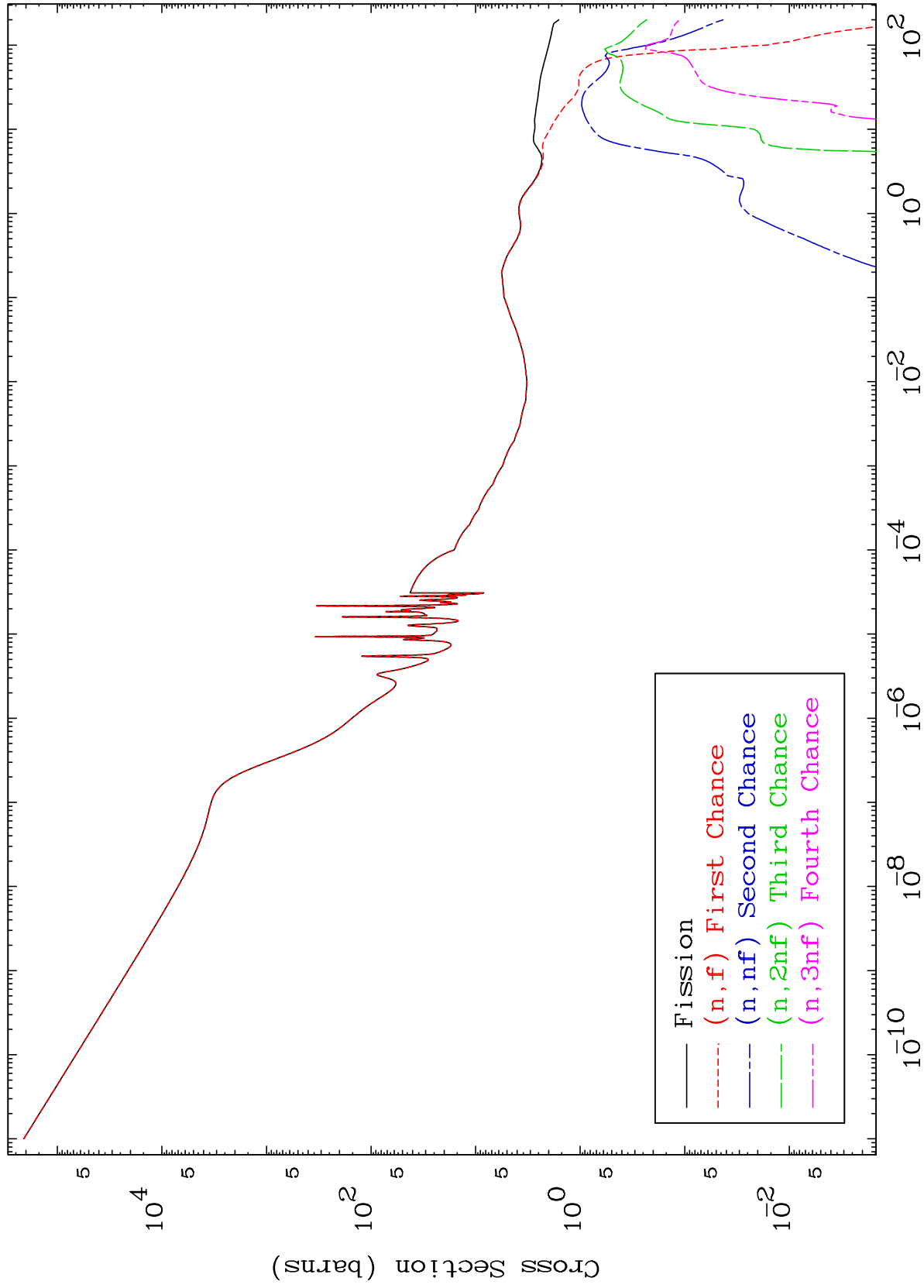
6

101-Md-260

MAT 9961

Fission  
293 Kelvin Cross Sections

101-Md-260

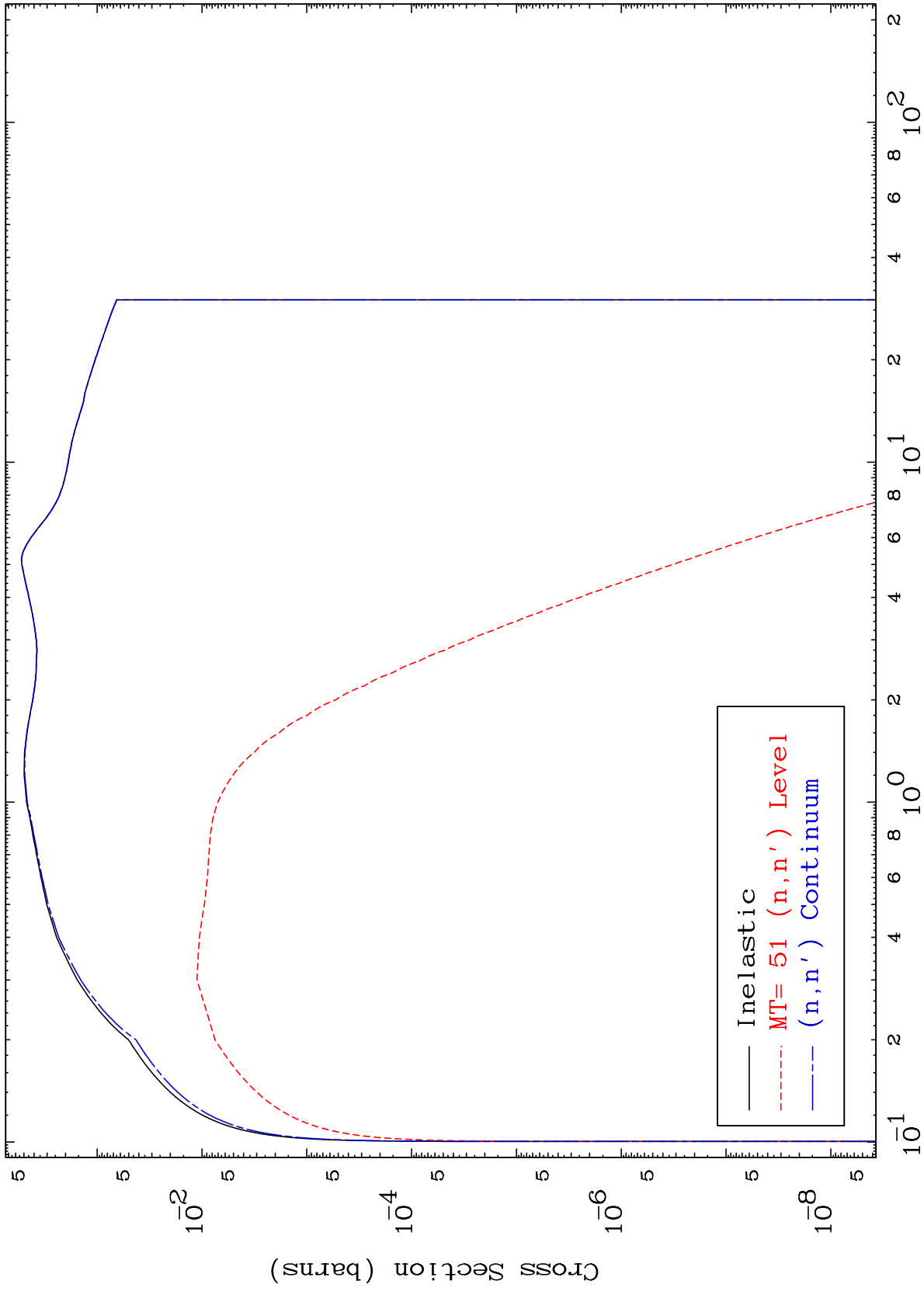




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101-Md-260

(n,n') Level  
293 Kelvin Cross Sections



8

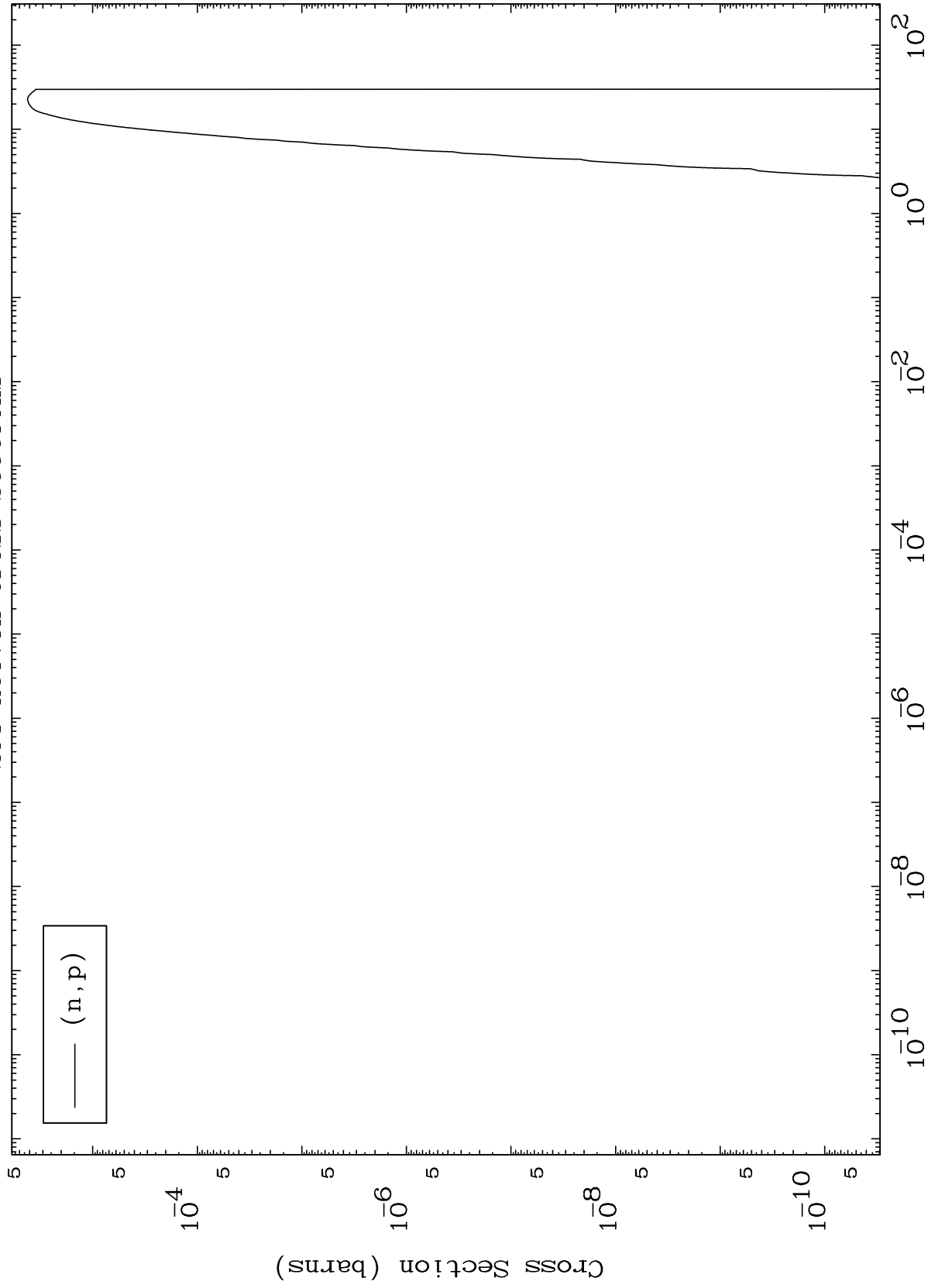
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,p) Levels  
293 Kelvin Cross Sections

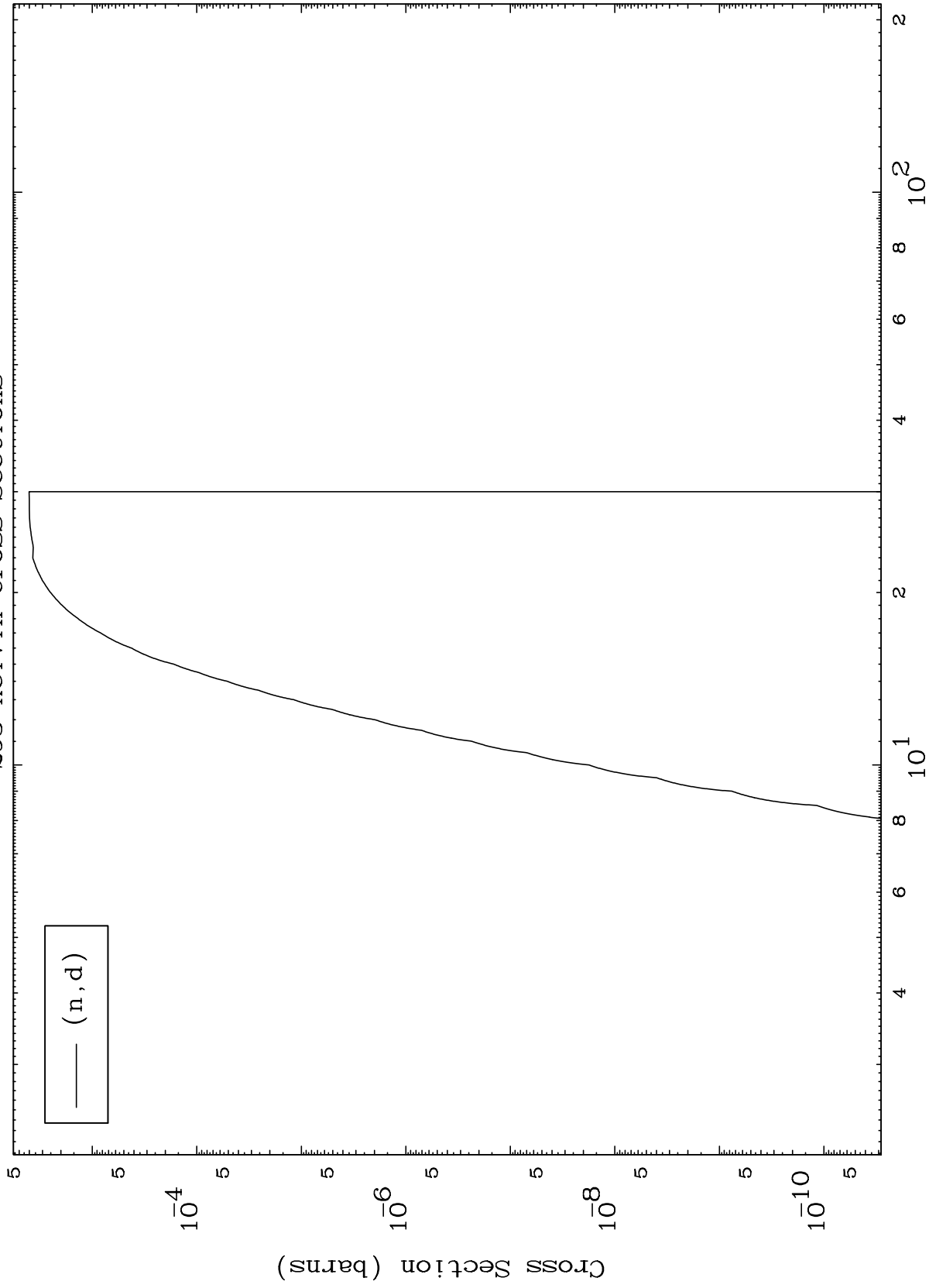
101-Md-260



MAT 9961

(n,d) Levels  
293 Kelvin Cross Sections

101-Md-260



10

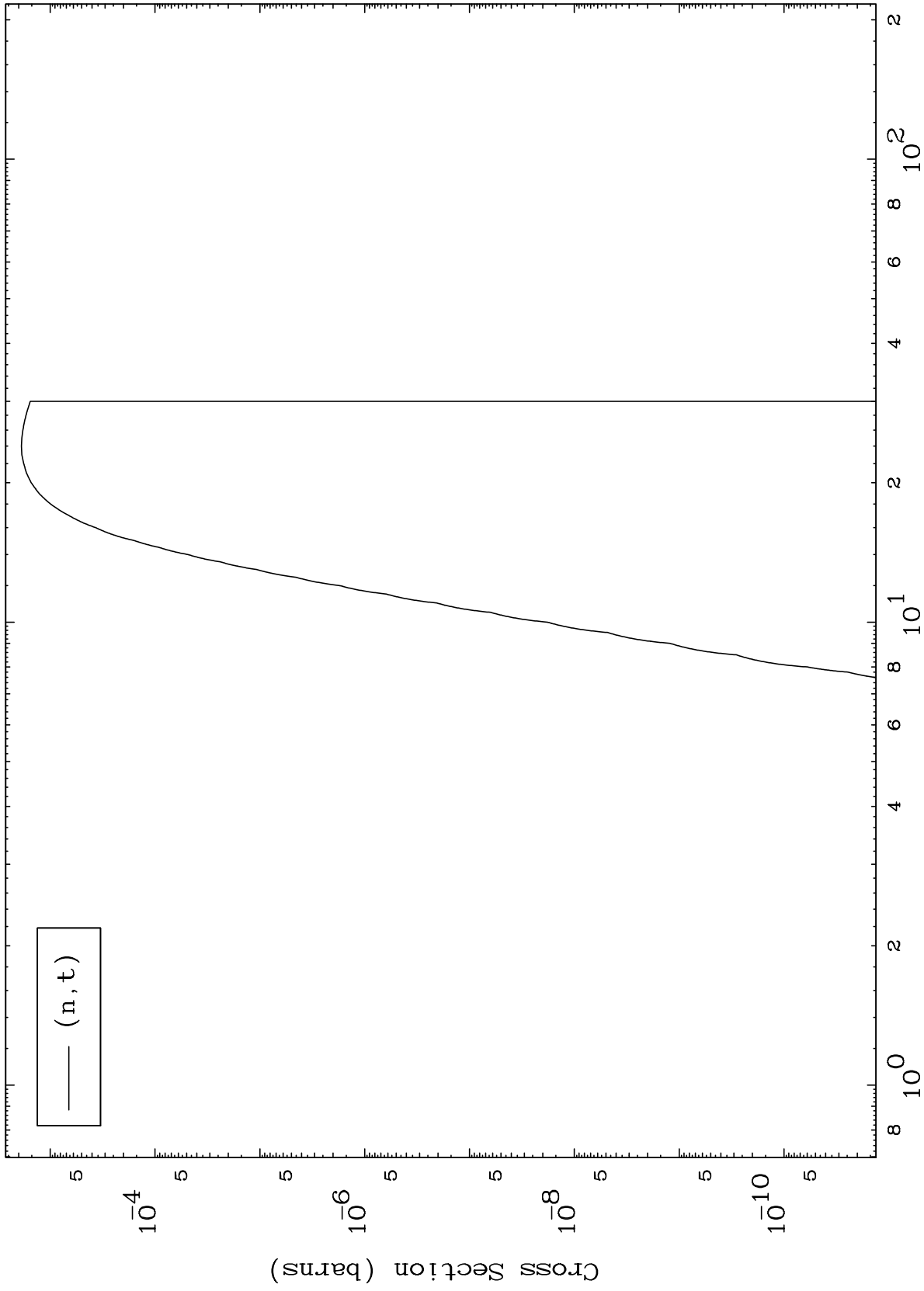
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,t) Levels  
293 Kelvin Cross Sections

101-Md-260



11

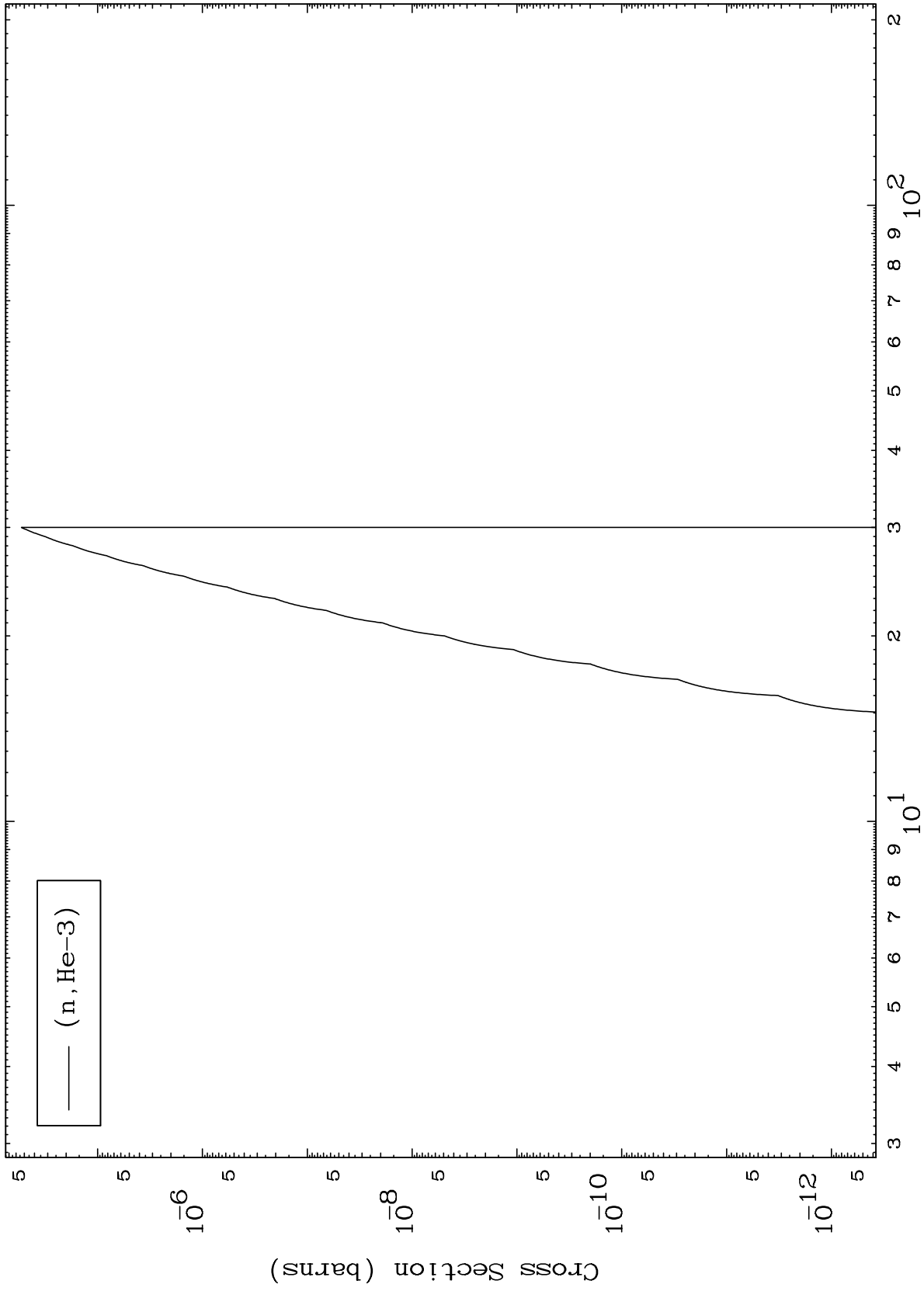
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,He3) Levels  
293 Kelvin Cross Sections

101-Md-260



12

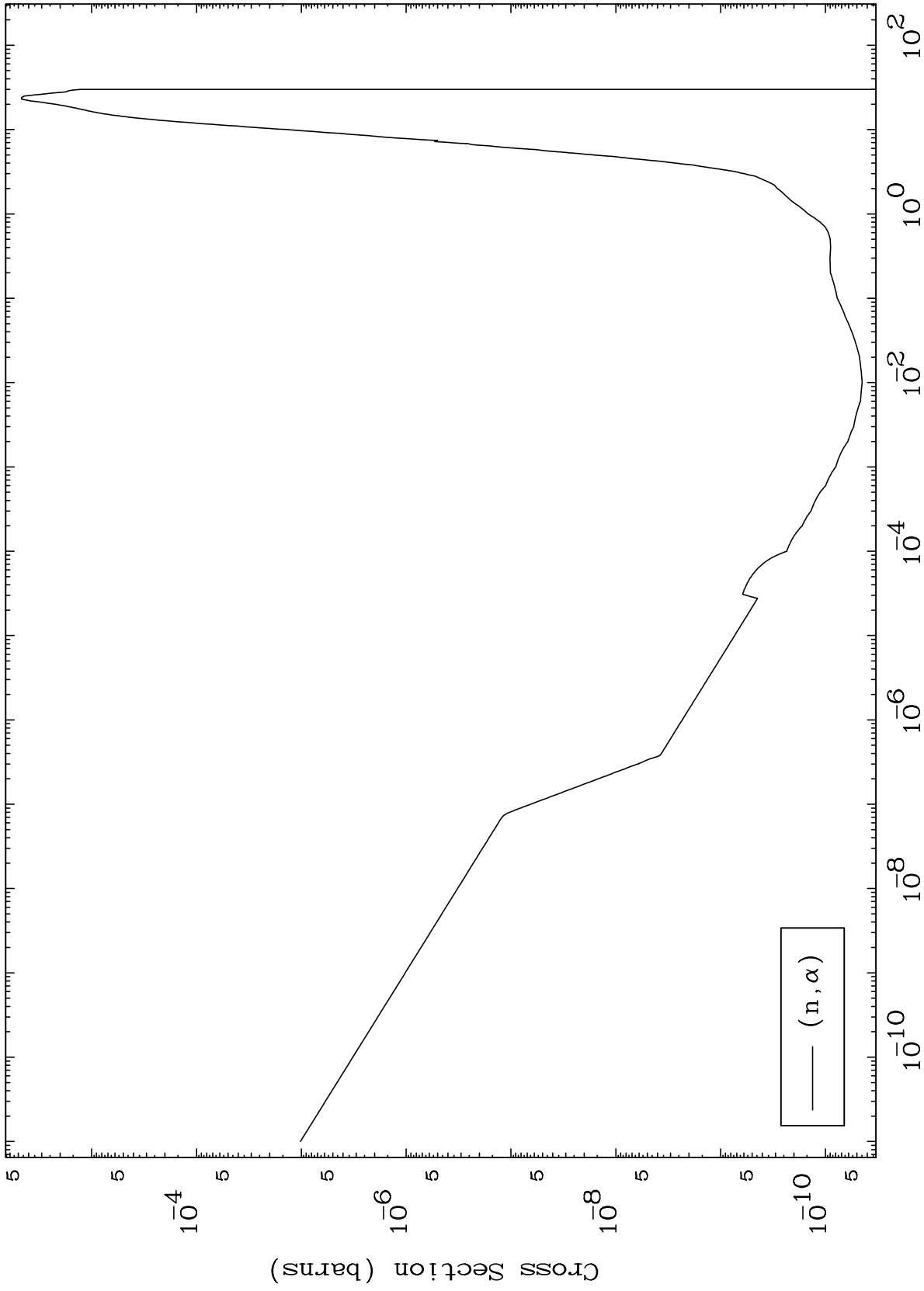
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,α) Levels  
293 Kelvin Cross Sections

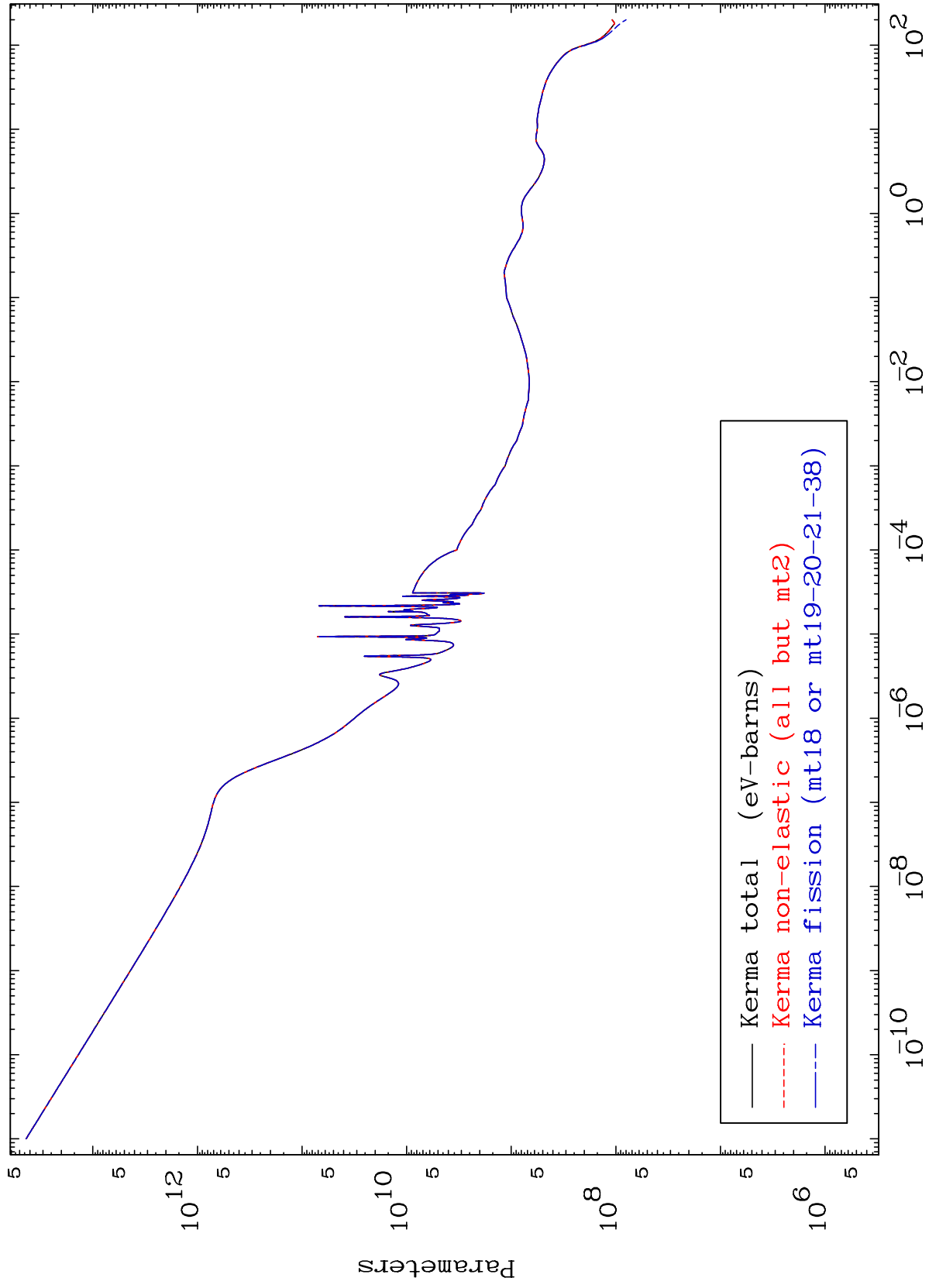
101-Md-260

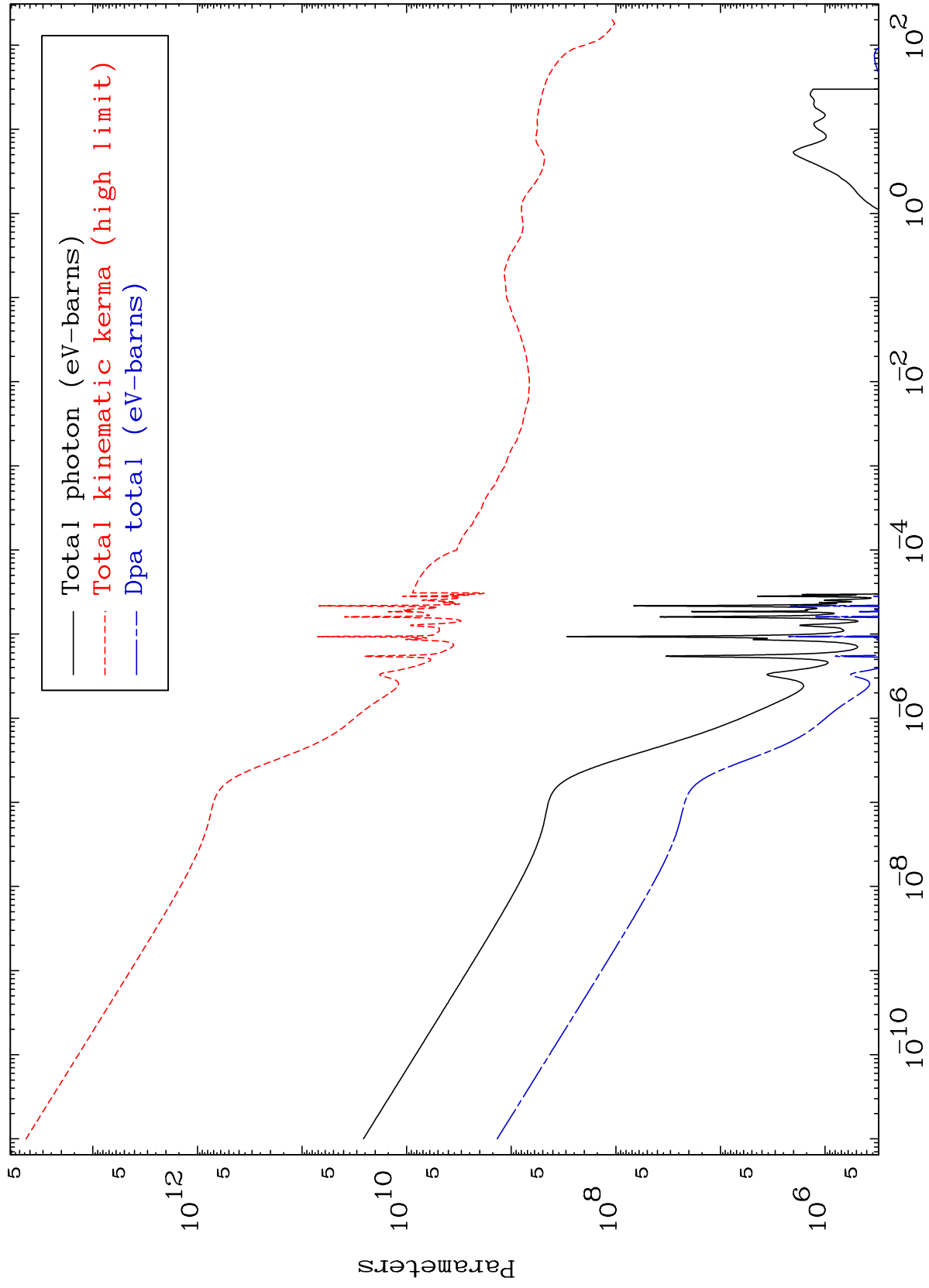


MAT 9961

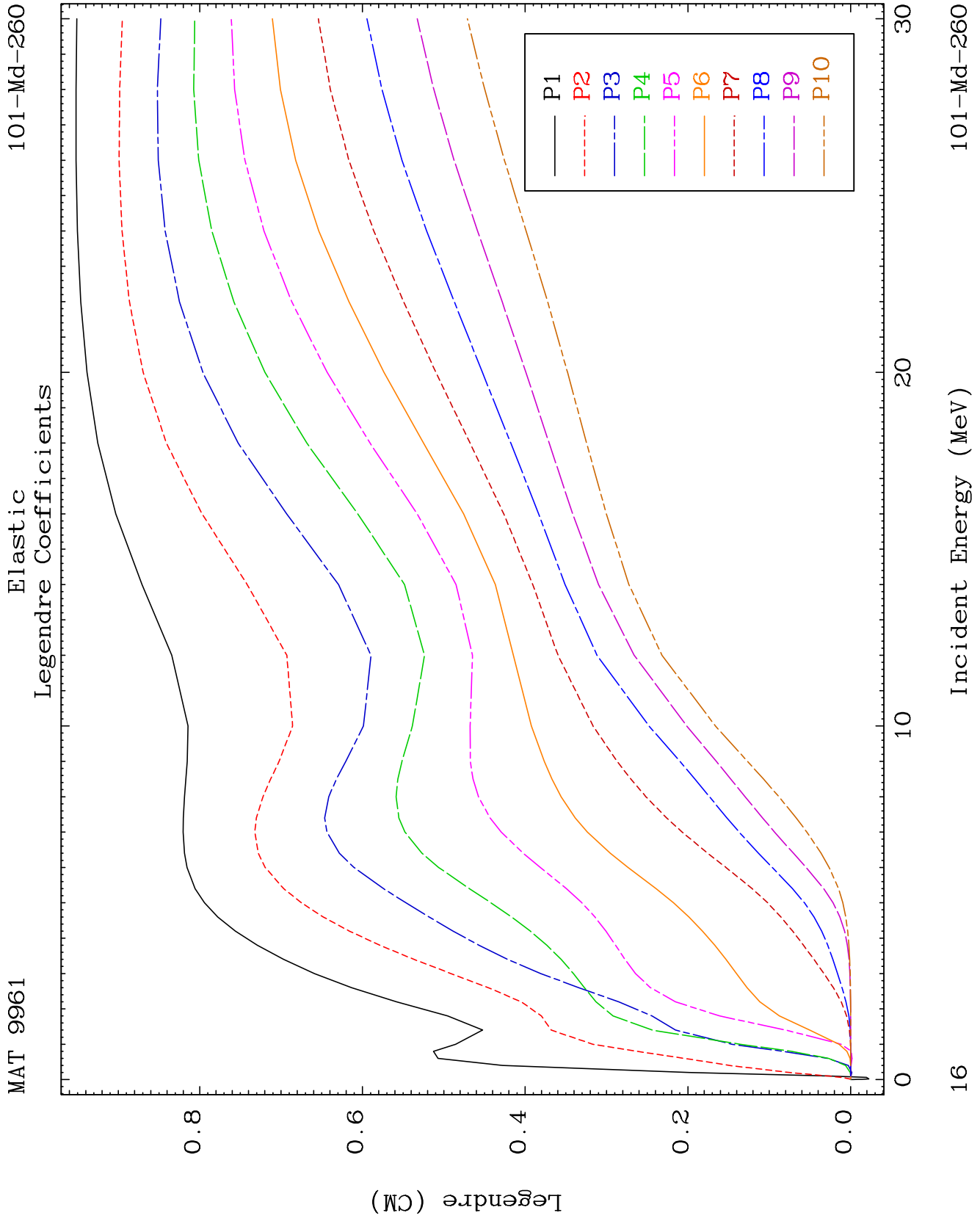
Energy Release  
Parameters

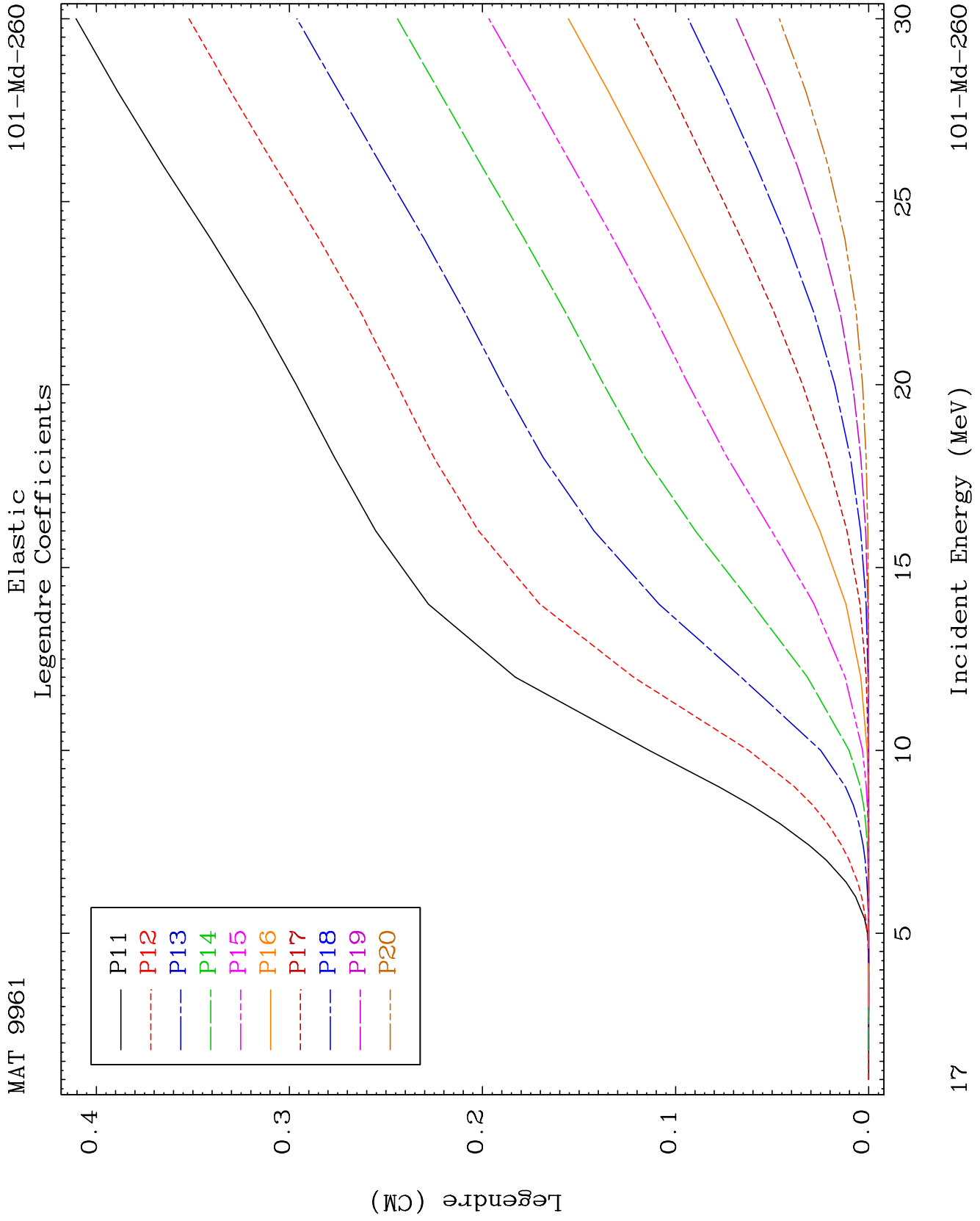
101-Md-260







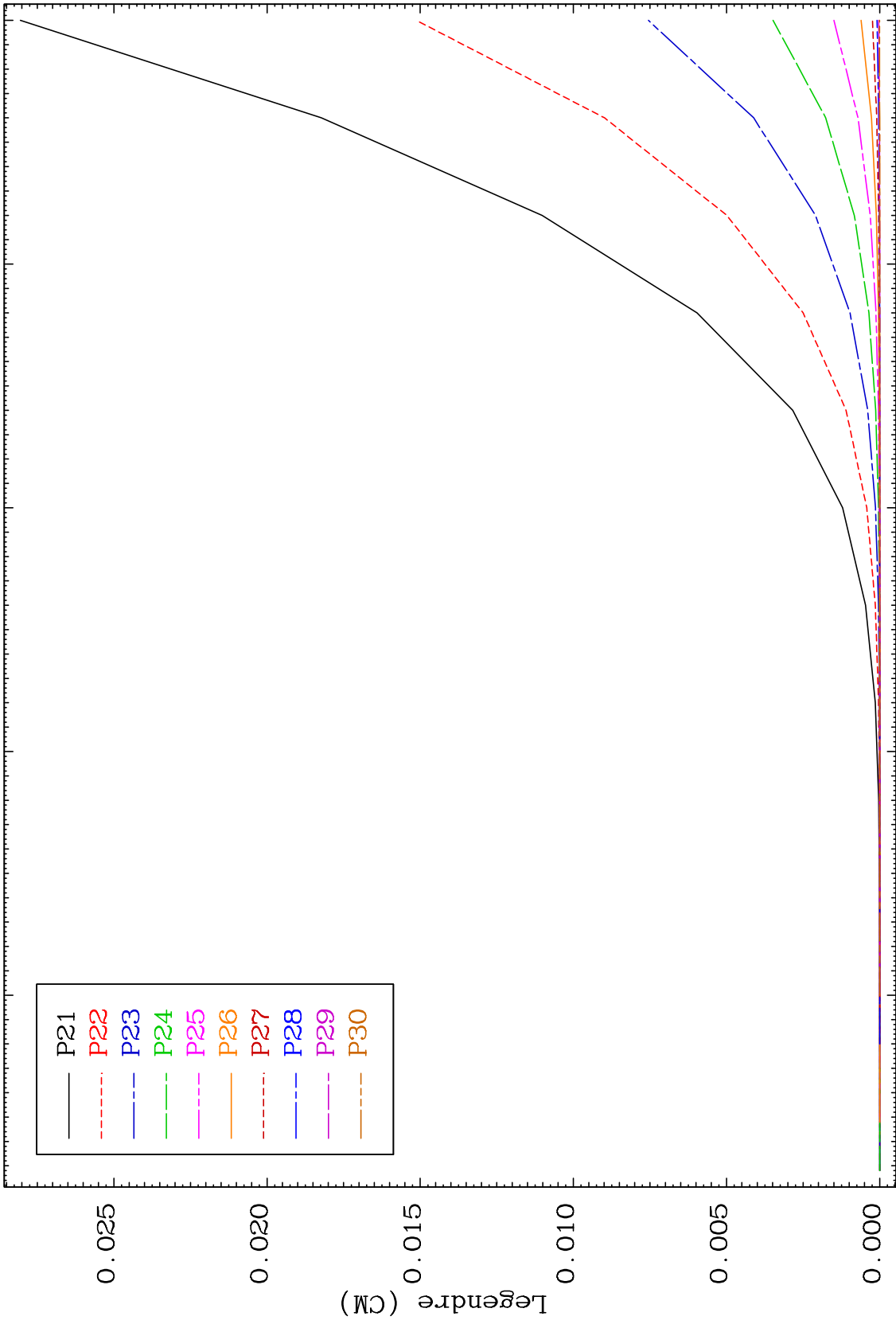




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### Elastic Legendre Coefficients

101-Md-260



18

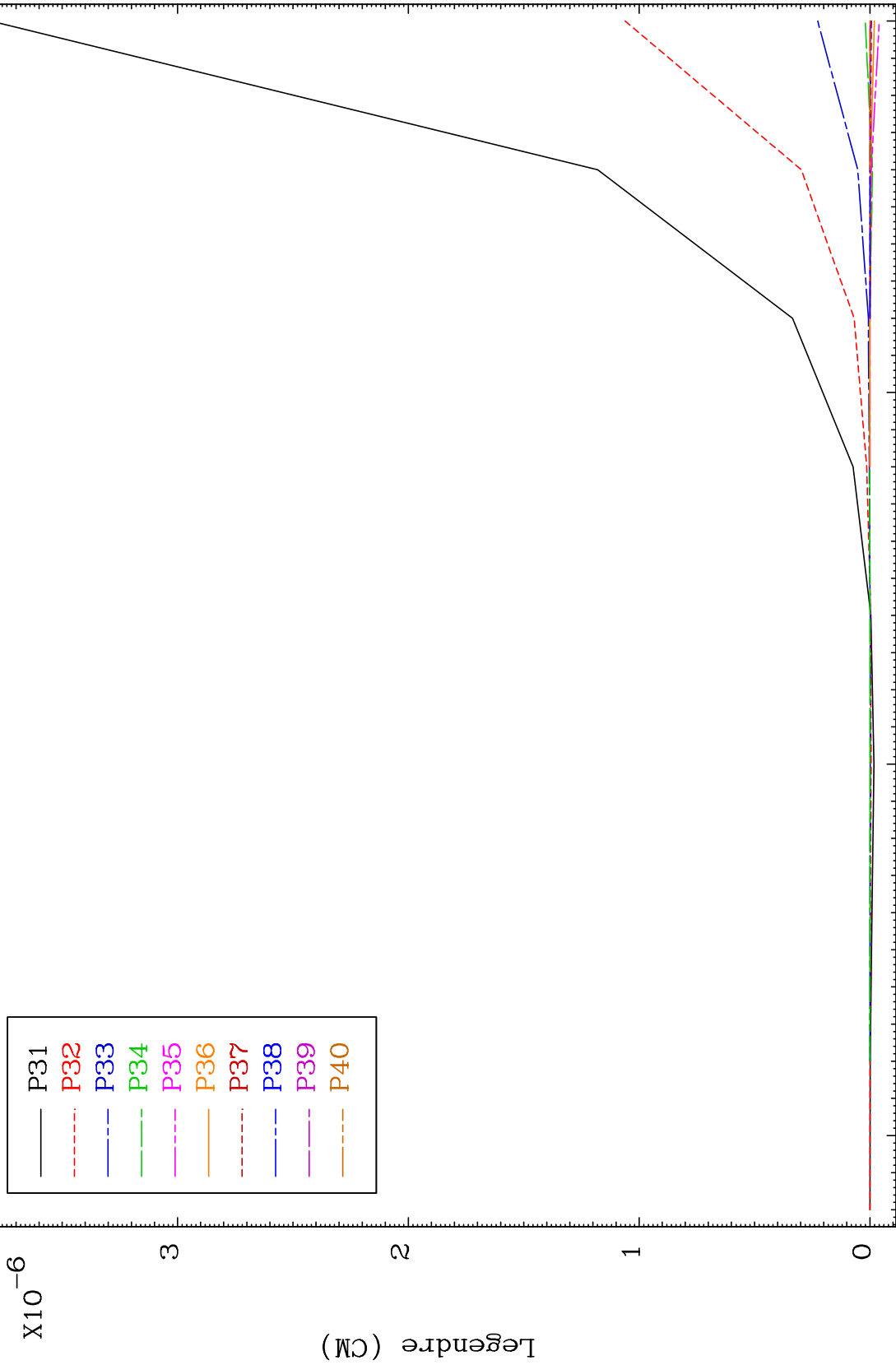
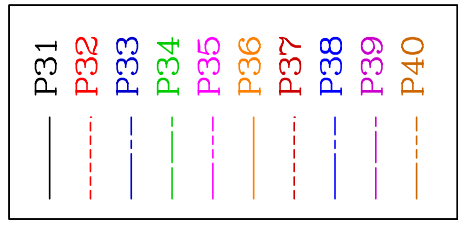
Incident Energy (MeV)

101-Md-260

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Elastic Legendre Coefficients

101-Md-260



19

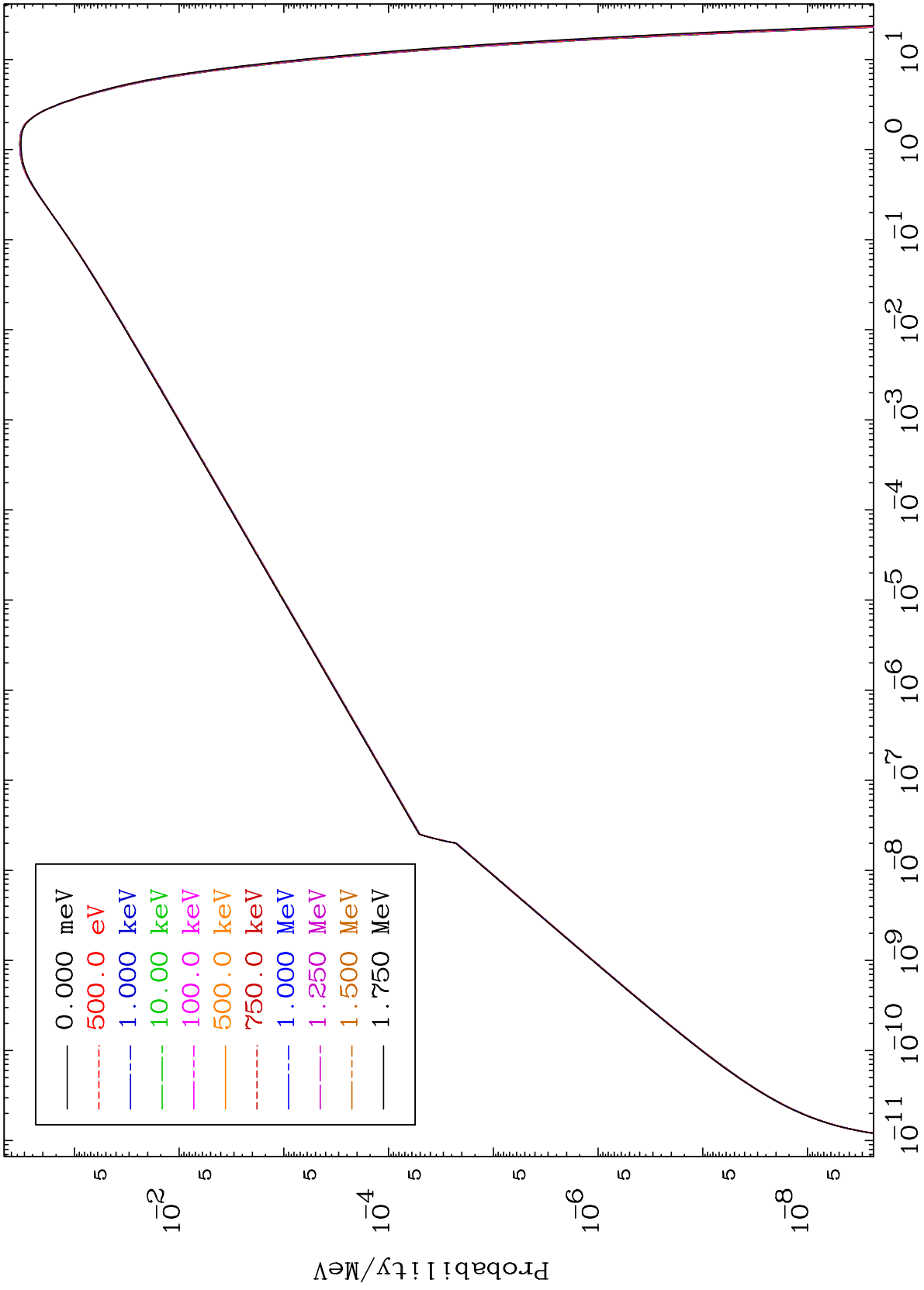
Incident Energy (MeV)

101-Md-260

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# Fission Energy Distribution

101-Md-260



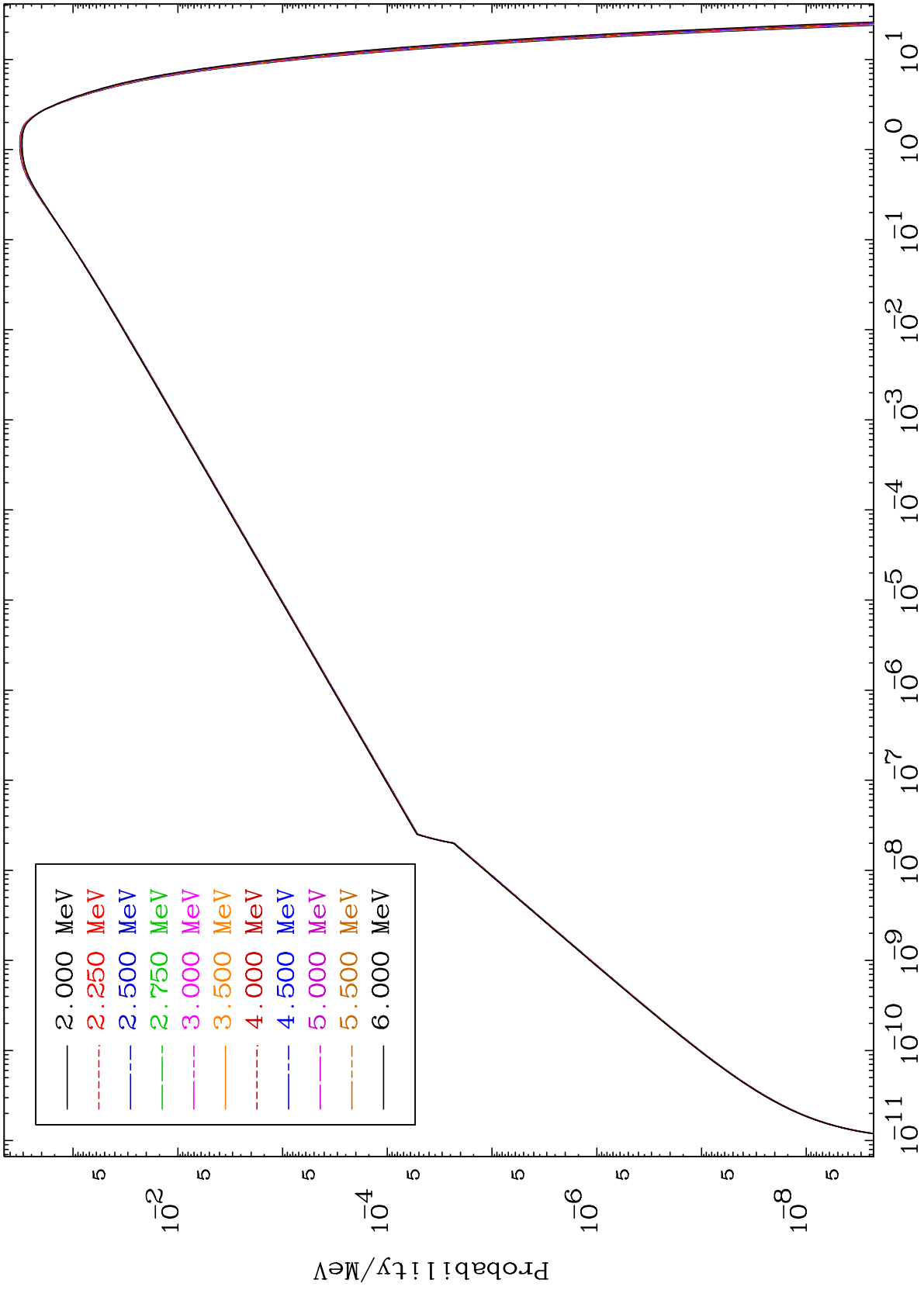
- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

101-Md-260

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# Fission Energy Distribution

101-Md-260

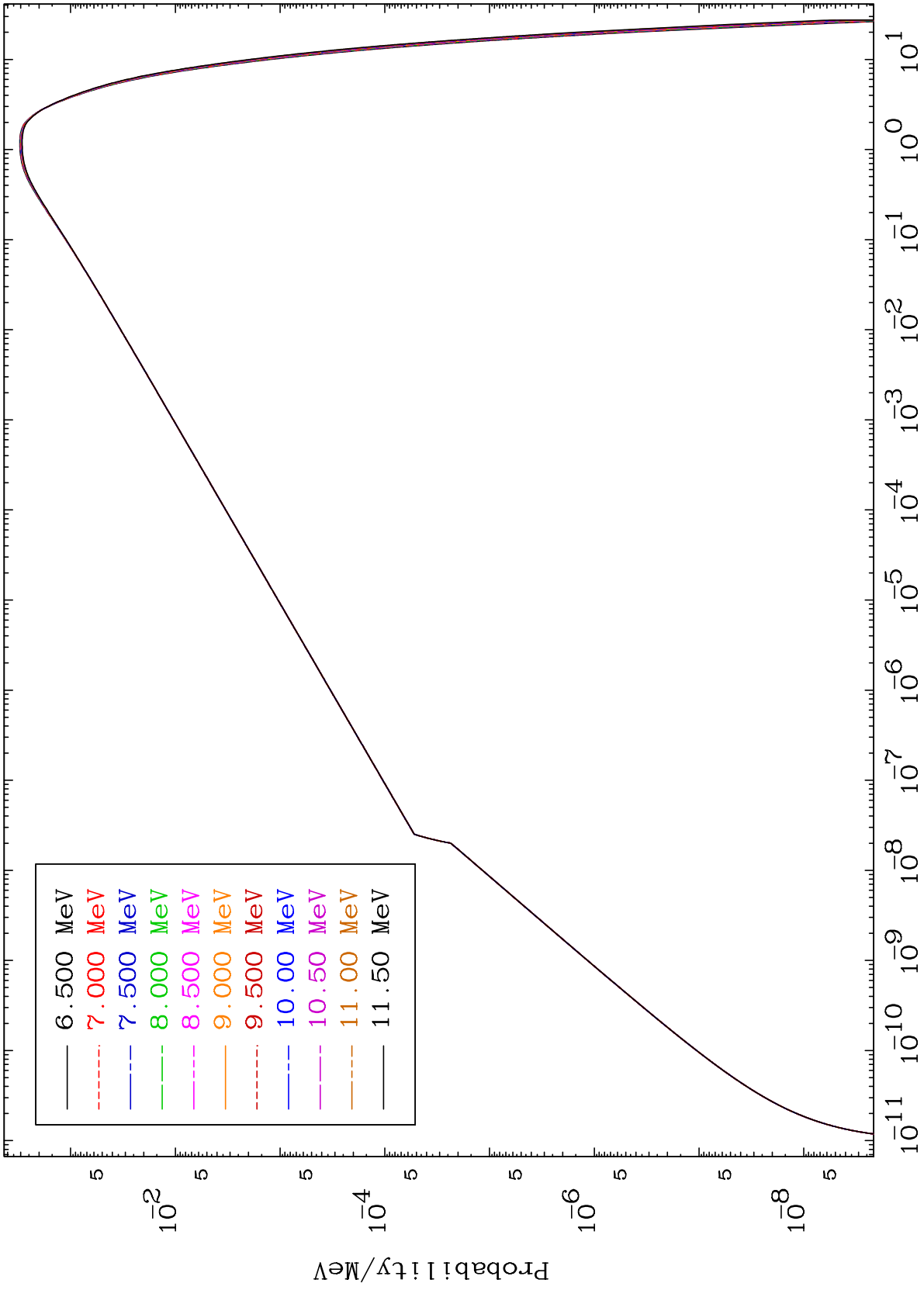


101-Md-260

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# Fission Energy Distribution

101-Md-260

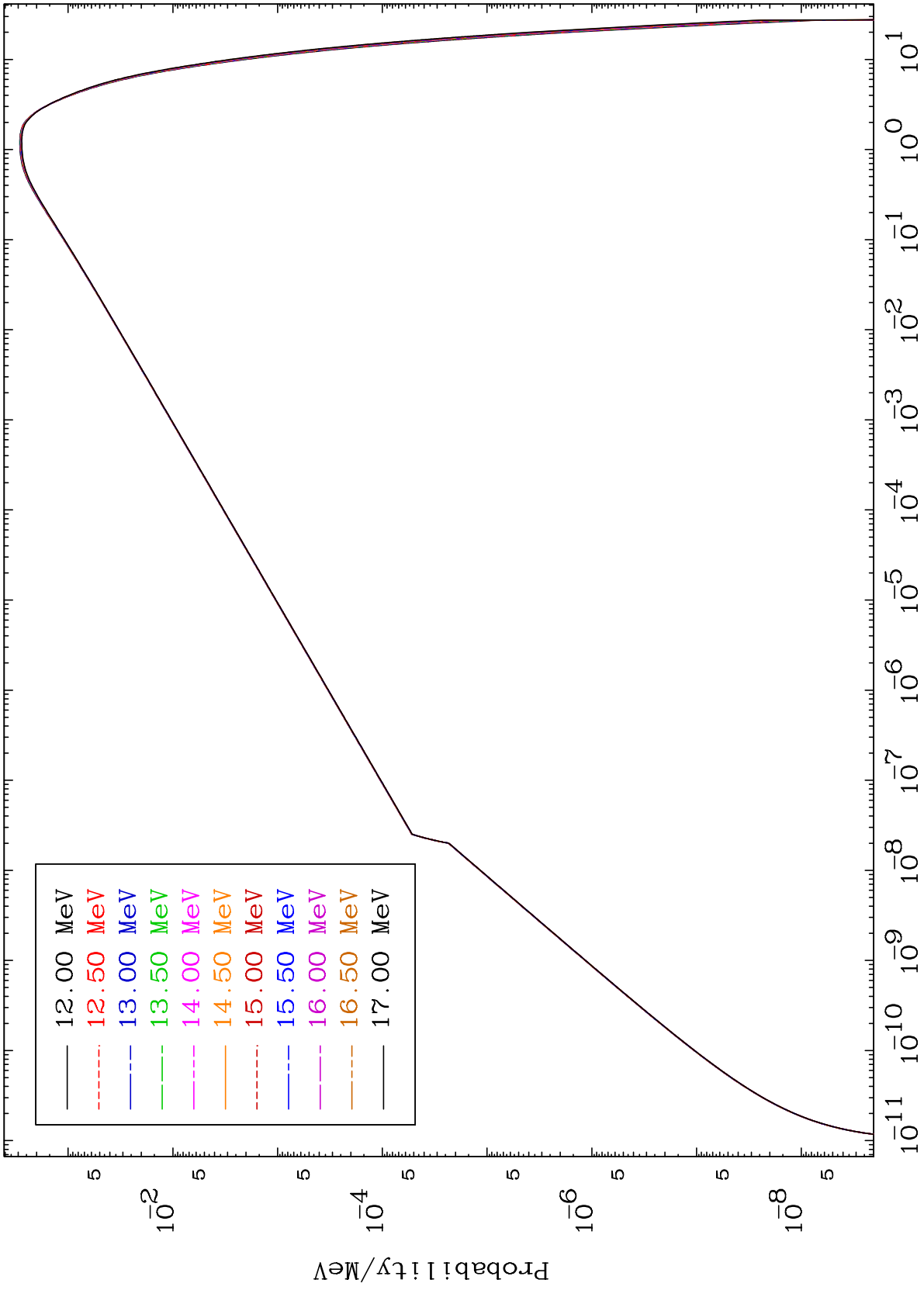


101-Md-260

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### Fission Energy Distribution

101-Md-260



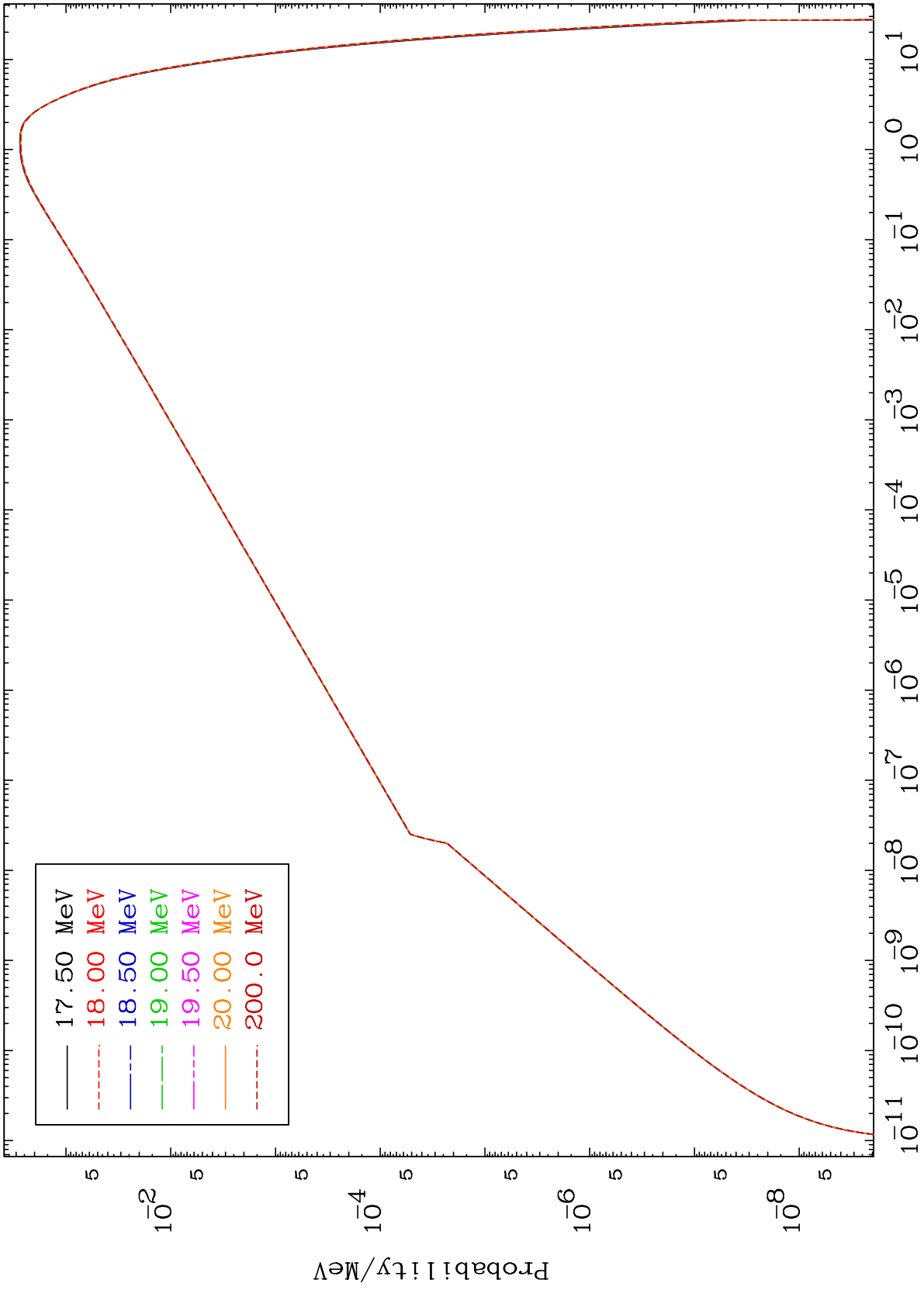
101-Md-260



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### Fission Energy Distribution

101-Md-260

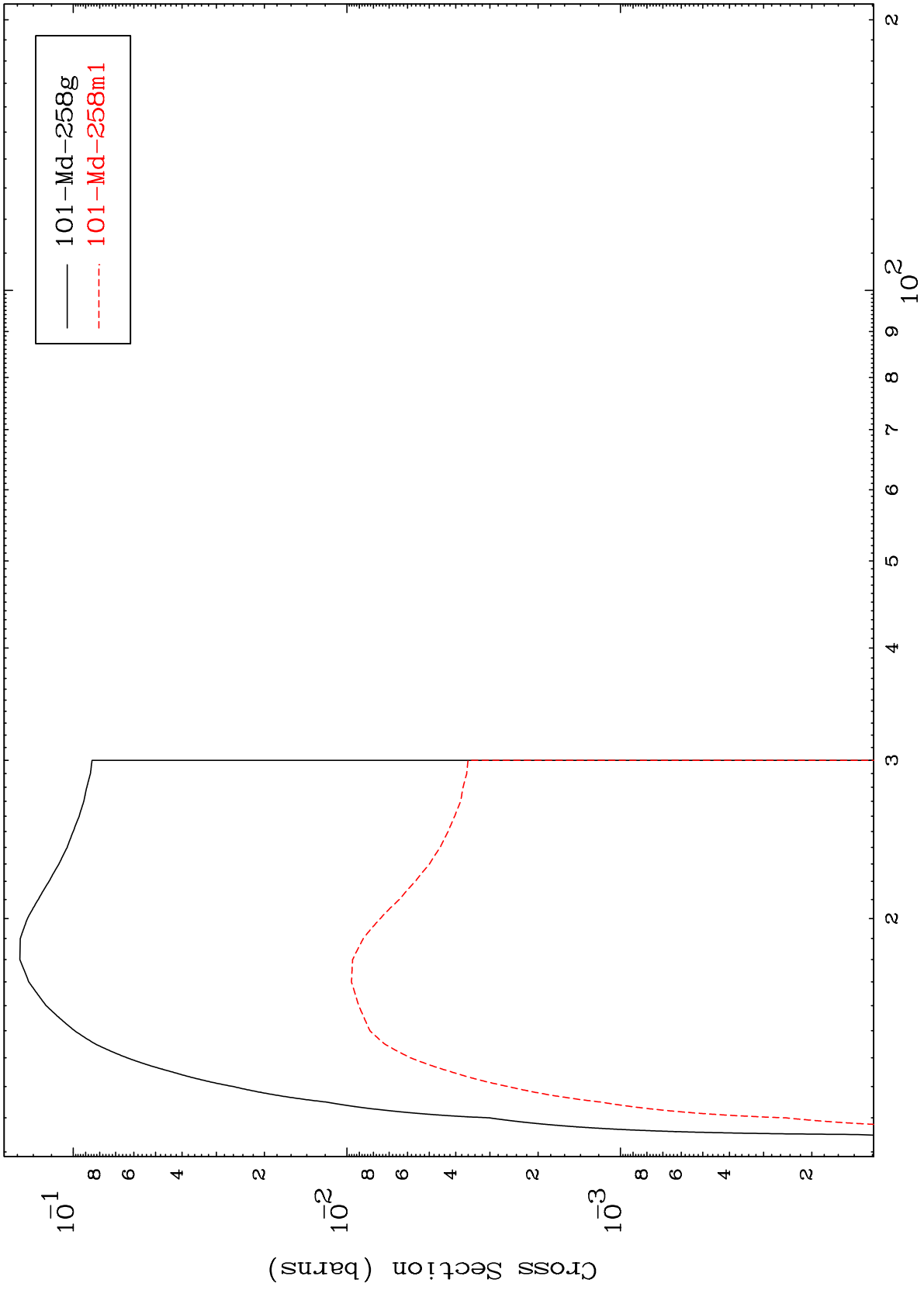


101-Md-260

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101-Md-260

(n,3n)  
Radionuclide Production Cross Section



25

Incident Energy (MeV)

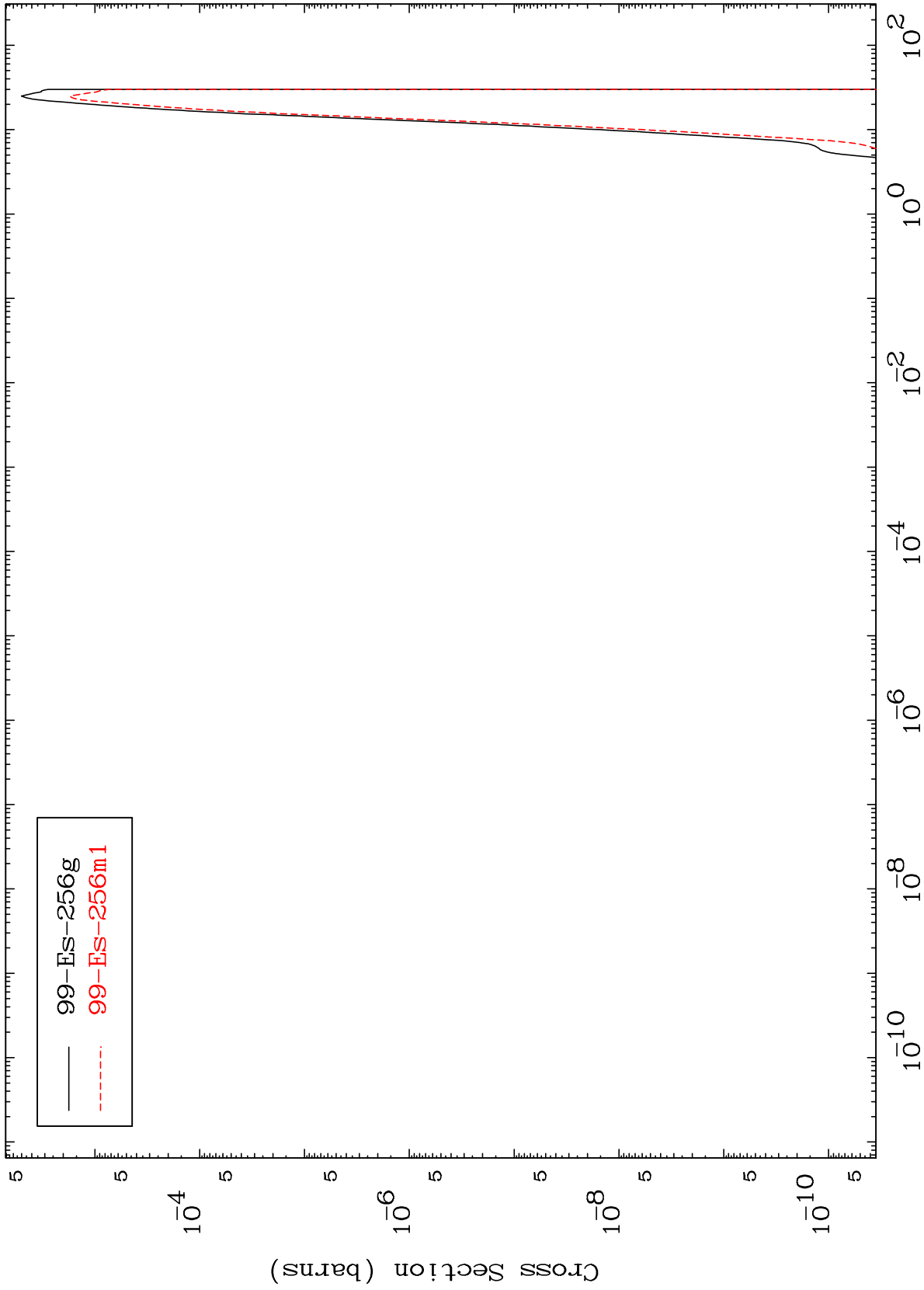
101-Md-260

MAT 9961

(n,n')  $\alpha$

101-Md-260

Radionuclide Production Cross Section



26

Incident Energy (MeV)

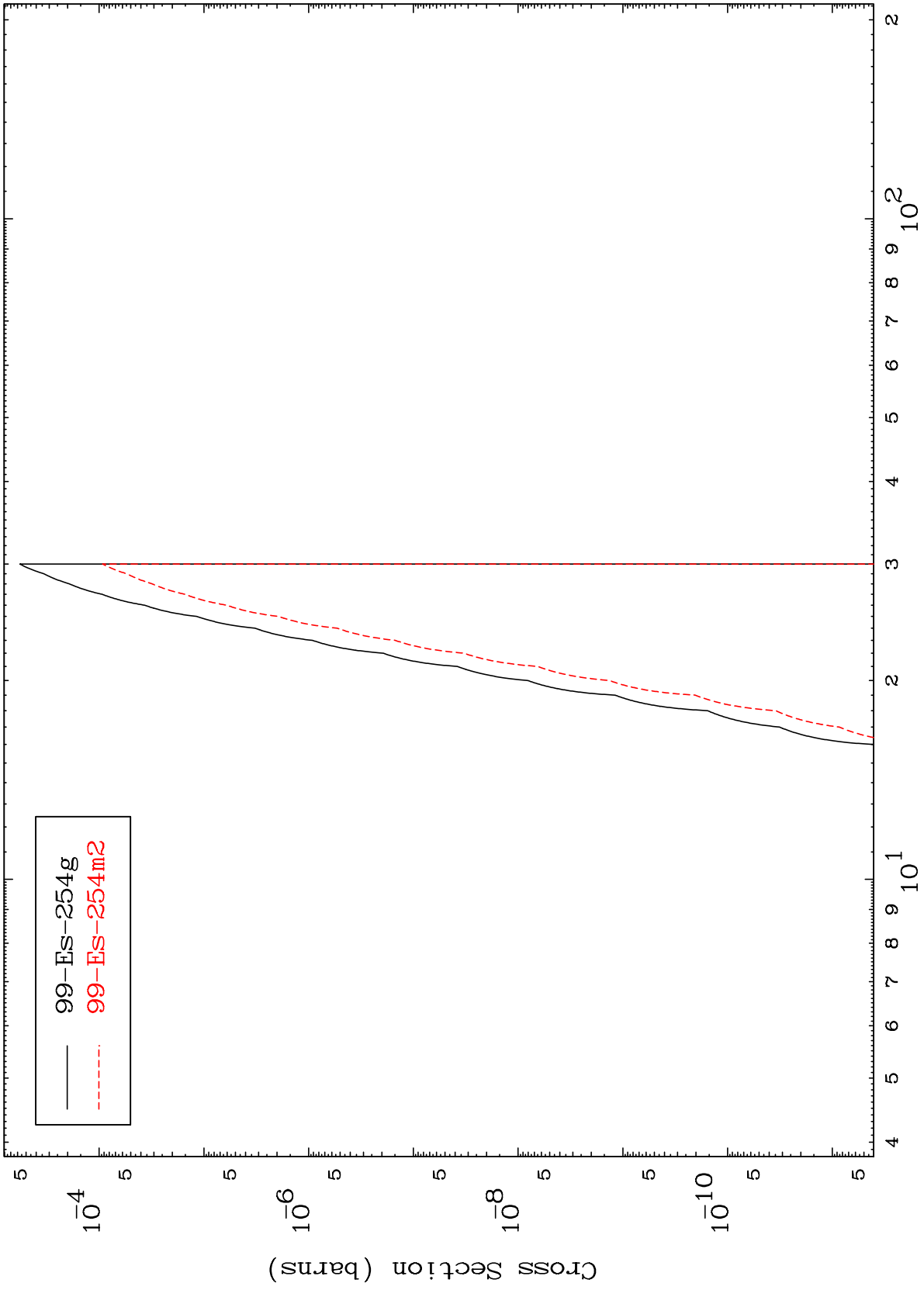
101-Md-260

MAT 9961

(n,3n)  $\alpha$

101-Md-260

Radionuclide Production Cross Section



27

Incident Energy (MeV)

101-Md-260