

Program EVALPLOT
(Version 2018-1)

by

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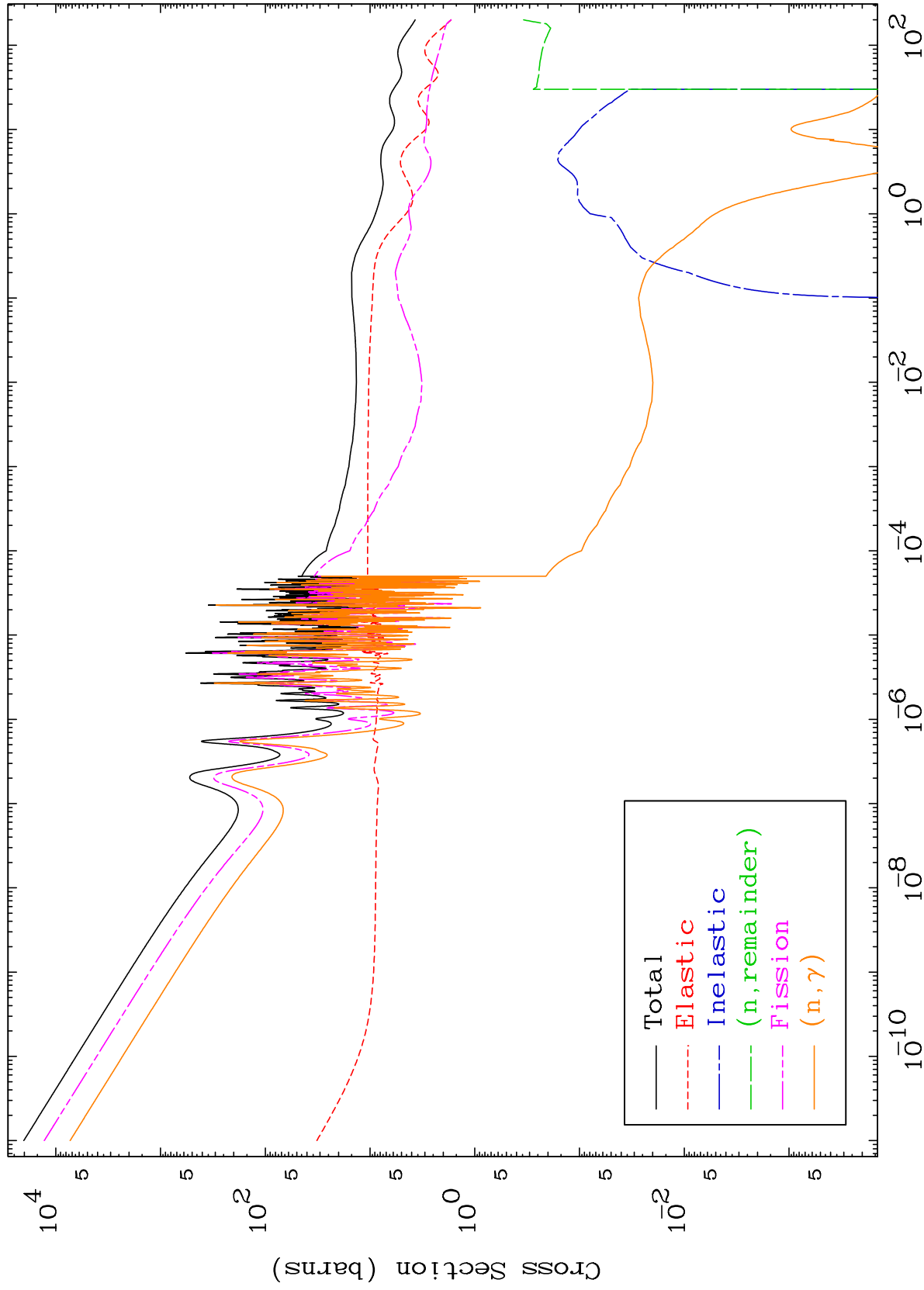
Press Mouse Button to Start

MAT 9962

Major

293 Kelvin Cross Sections

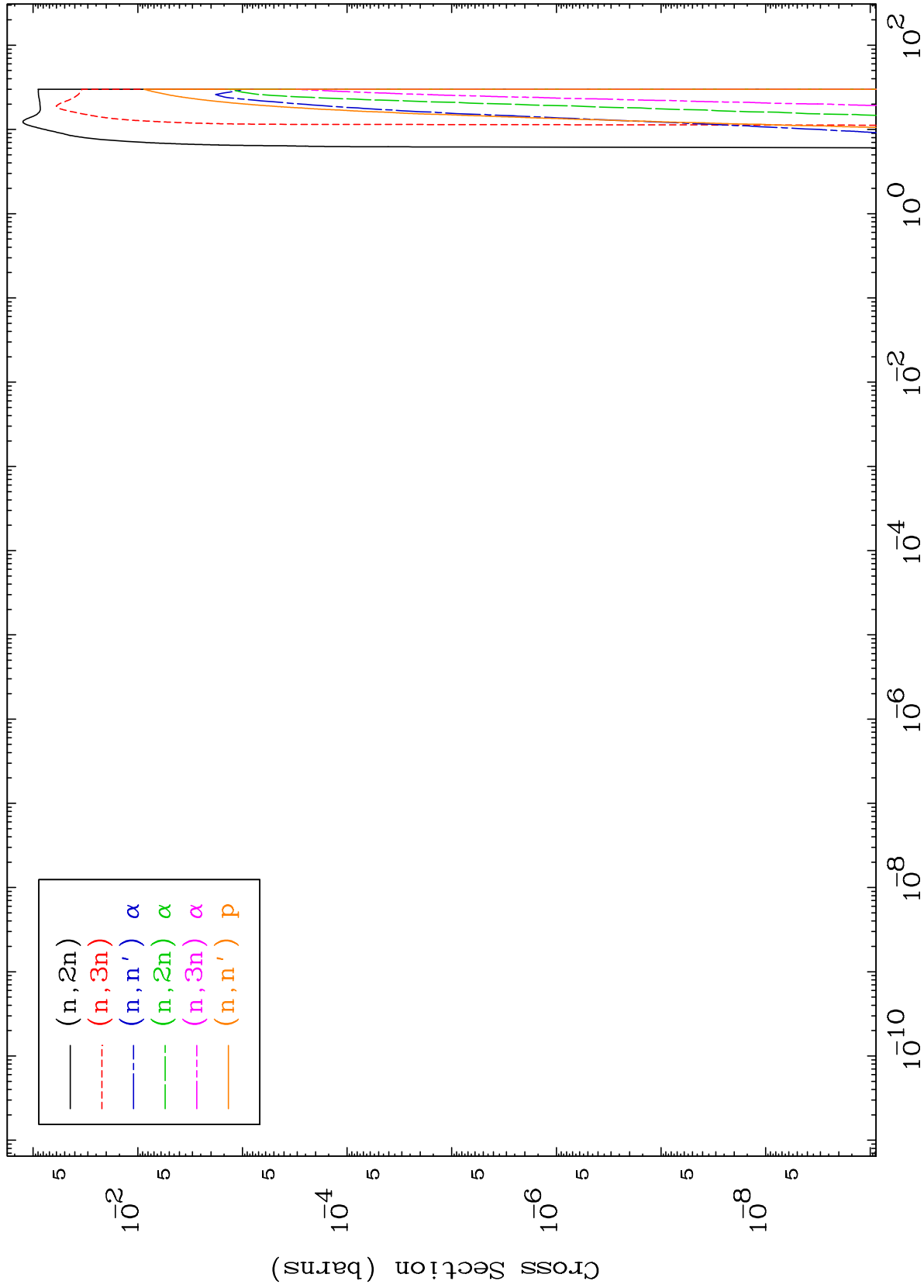
101-Md-261



MAT 9962

Neutron Production
293 Kelvin Cross Sections

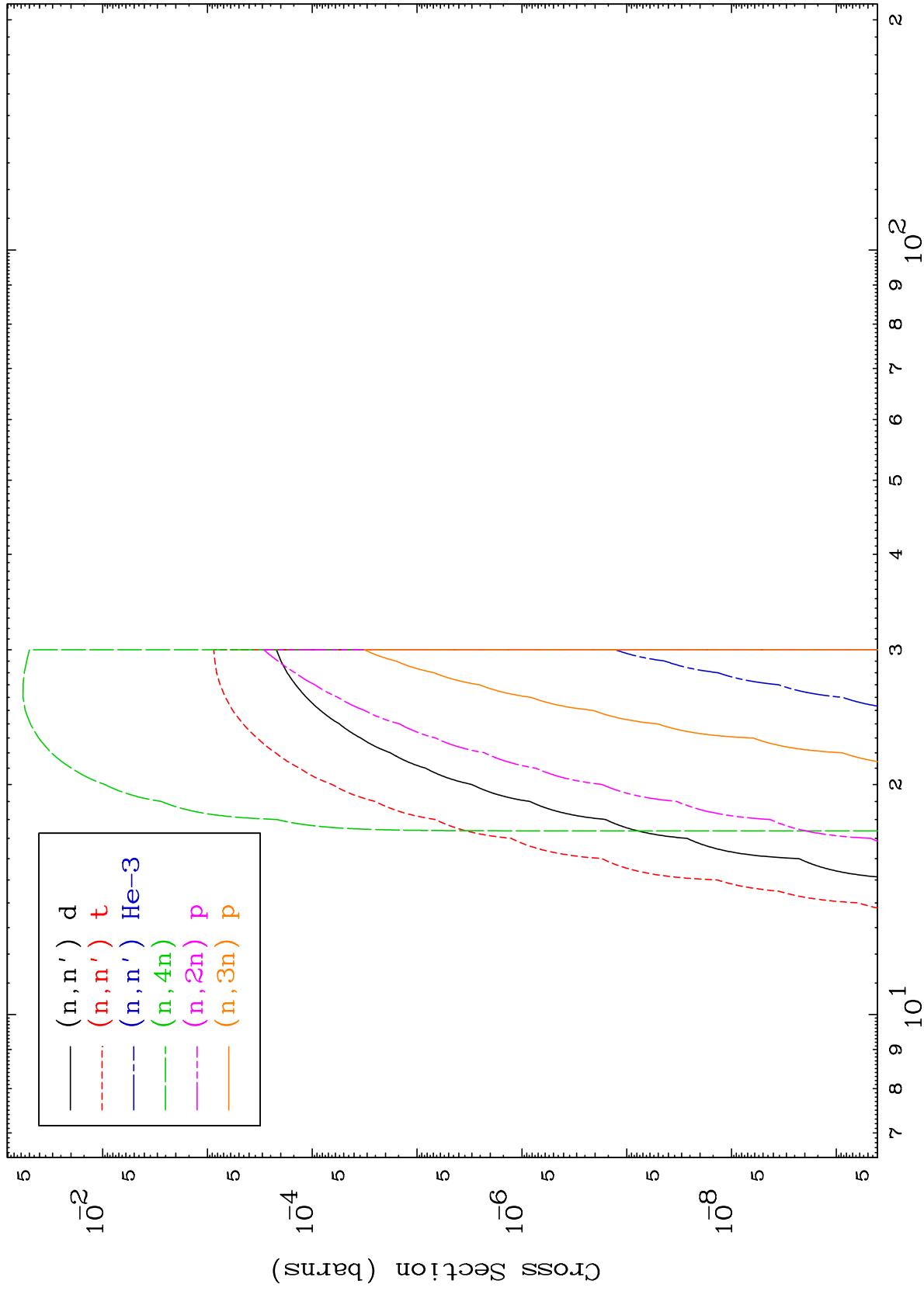
101-Md-261



MAT 9962

Neutron Production
293 Kelvin Cross Sections

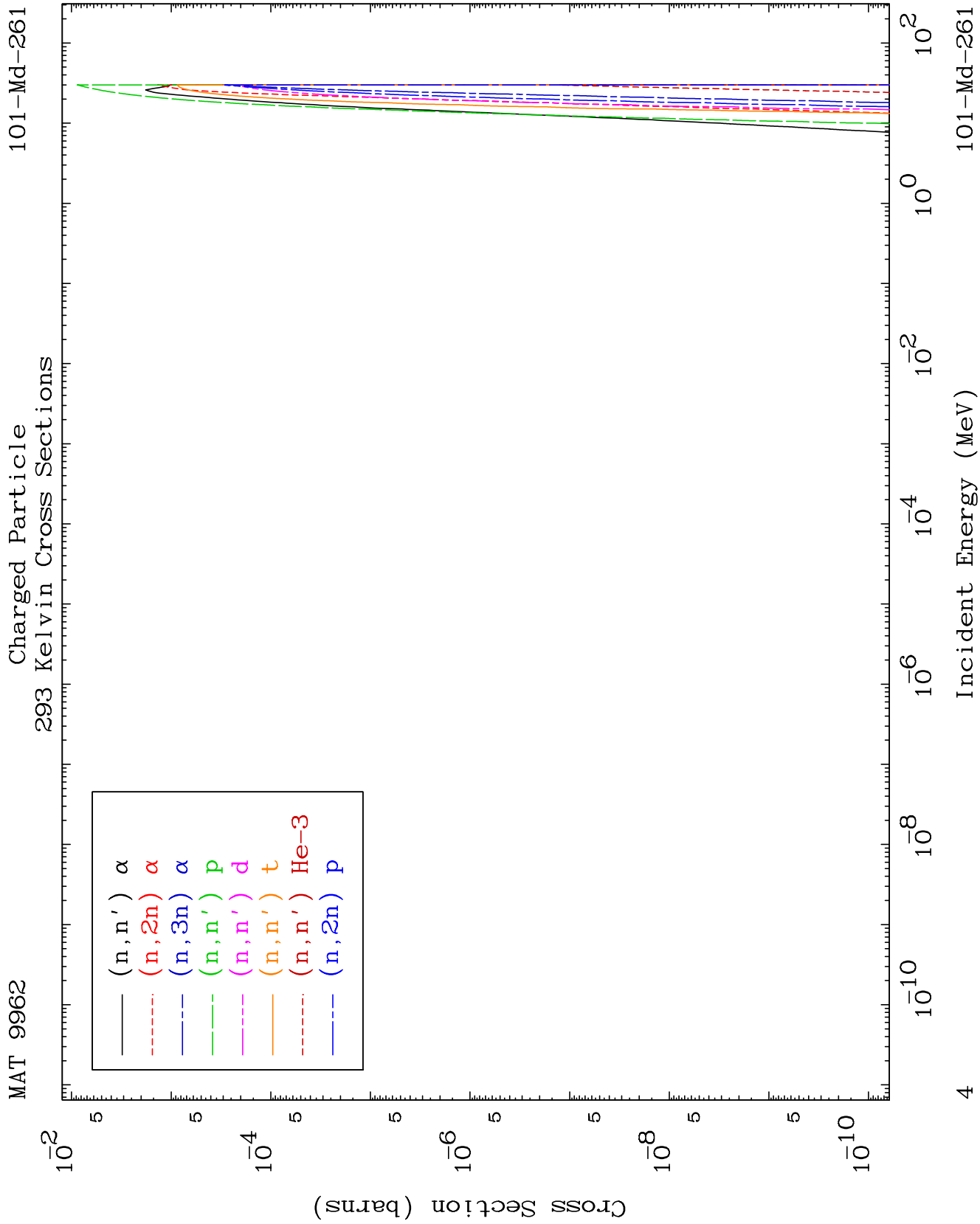
101-Md-261



3

Incident Energy (MeV)

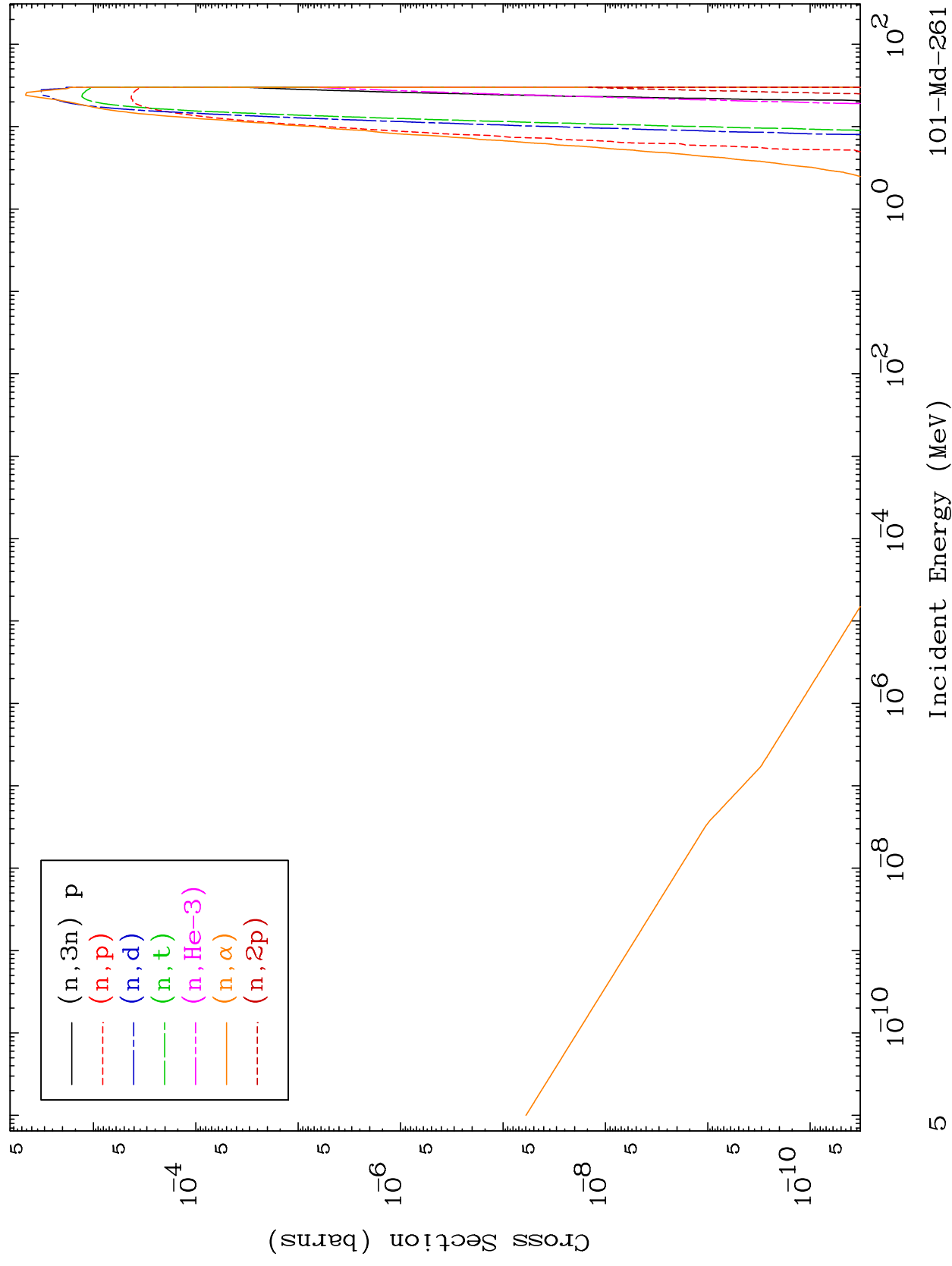
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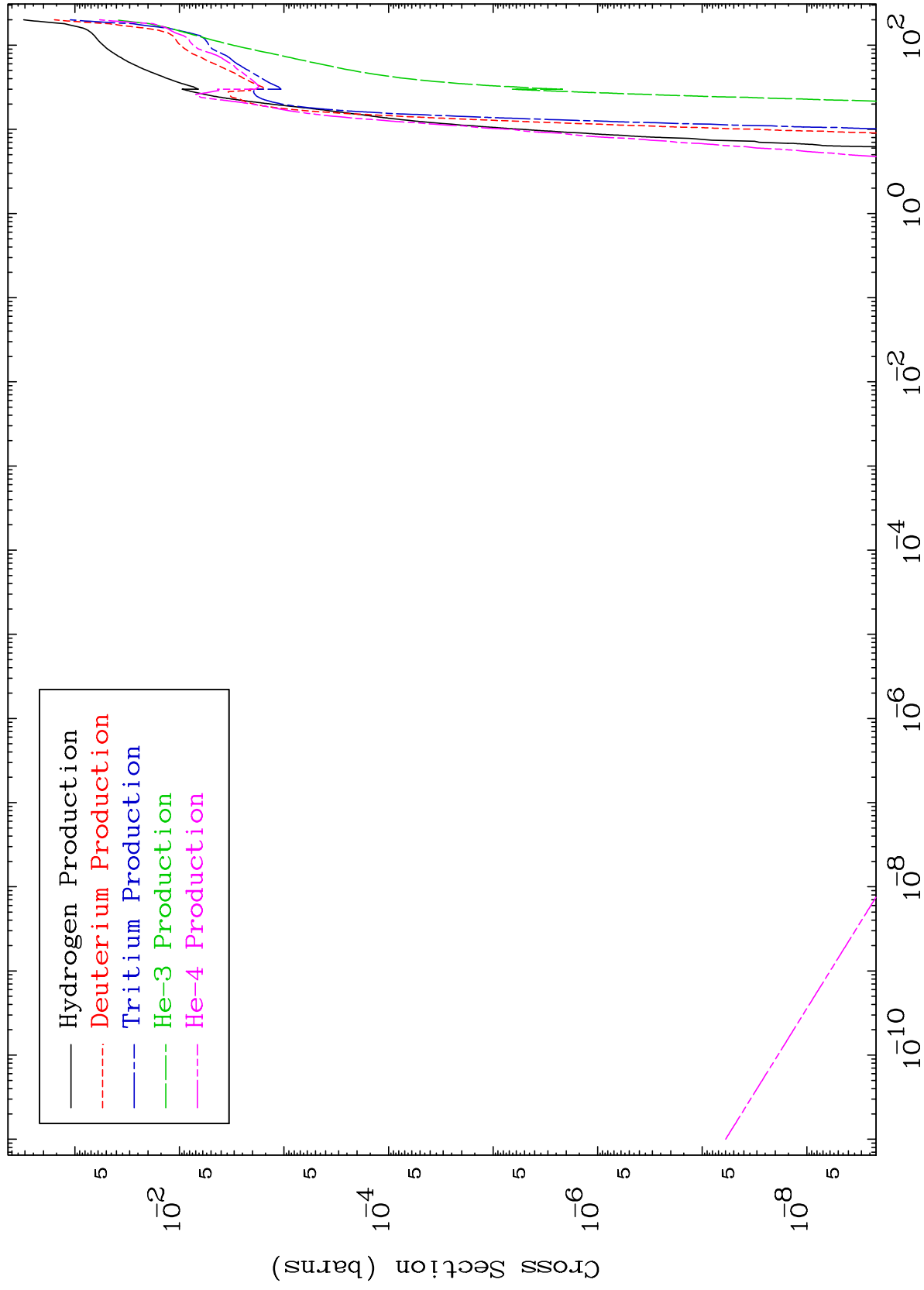


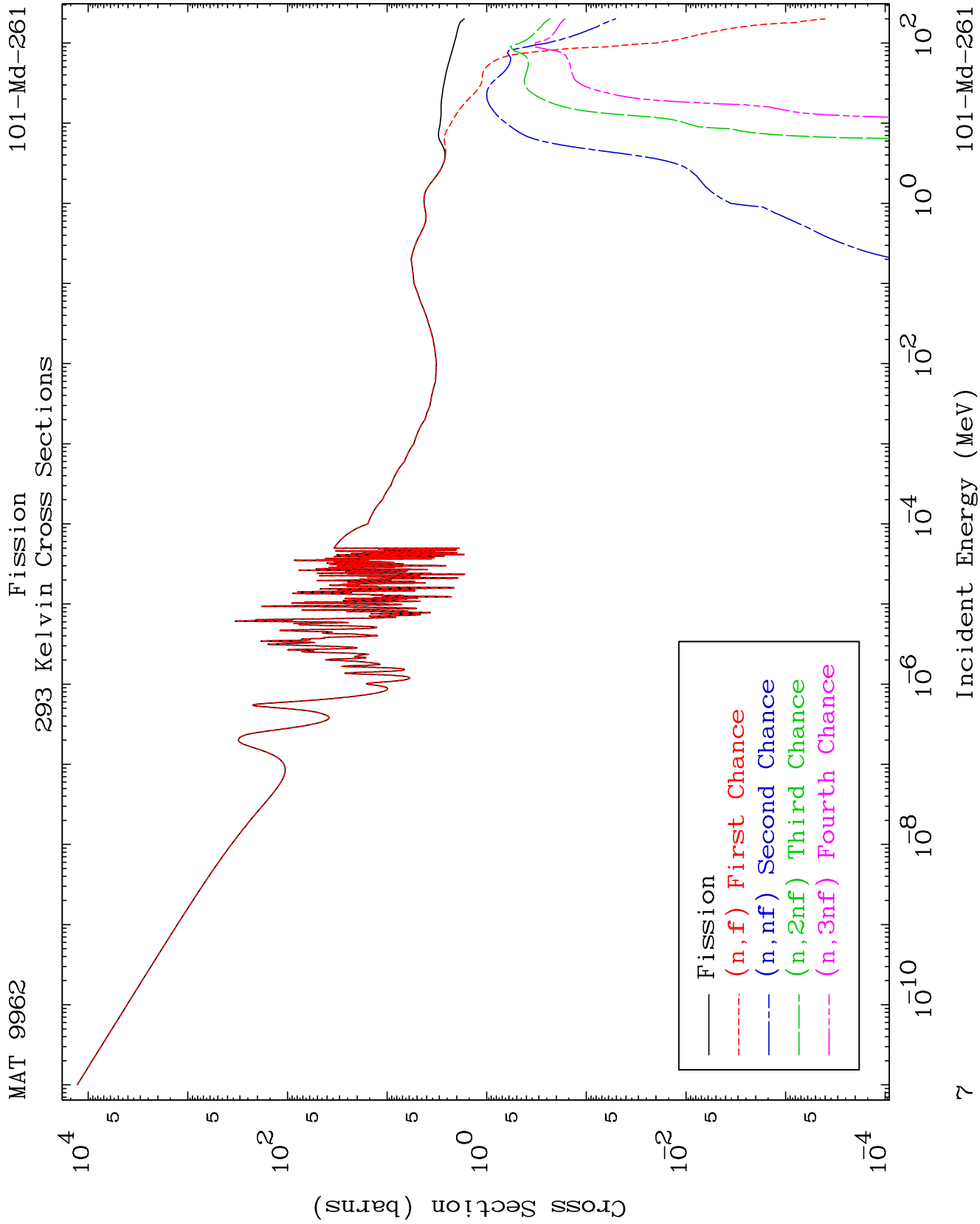
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Charged Particle 293 Kelvin Cross Sections

101-Md-261



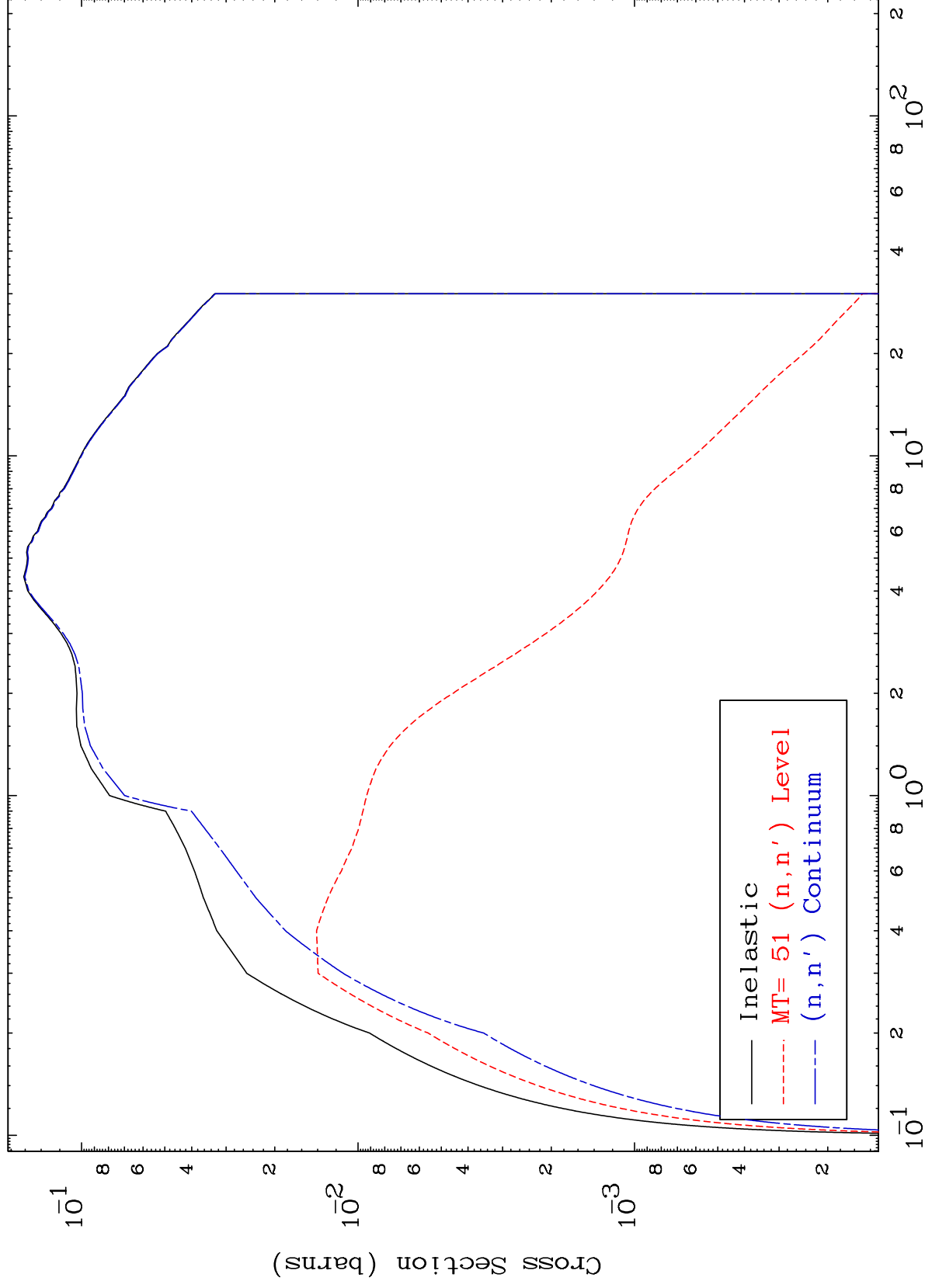




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(n,n') Level
293 Kelvin Cross Sections

101-Md-261



Incident Energy (MeV)

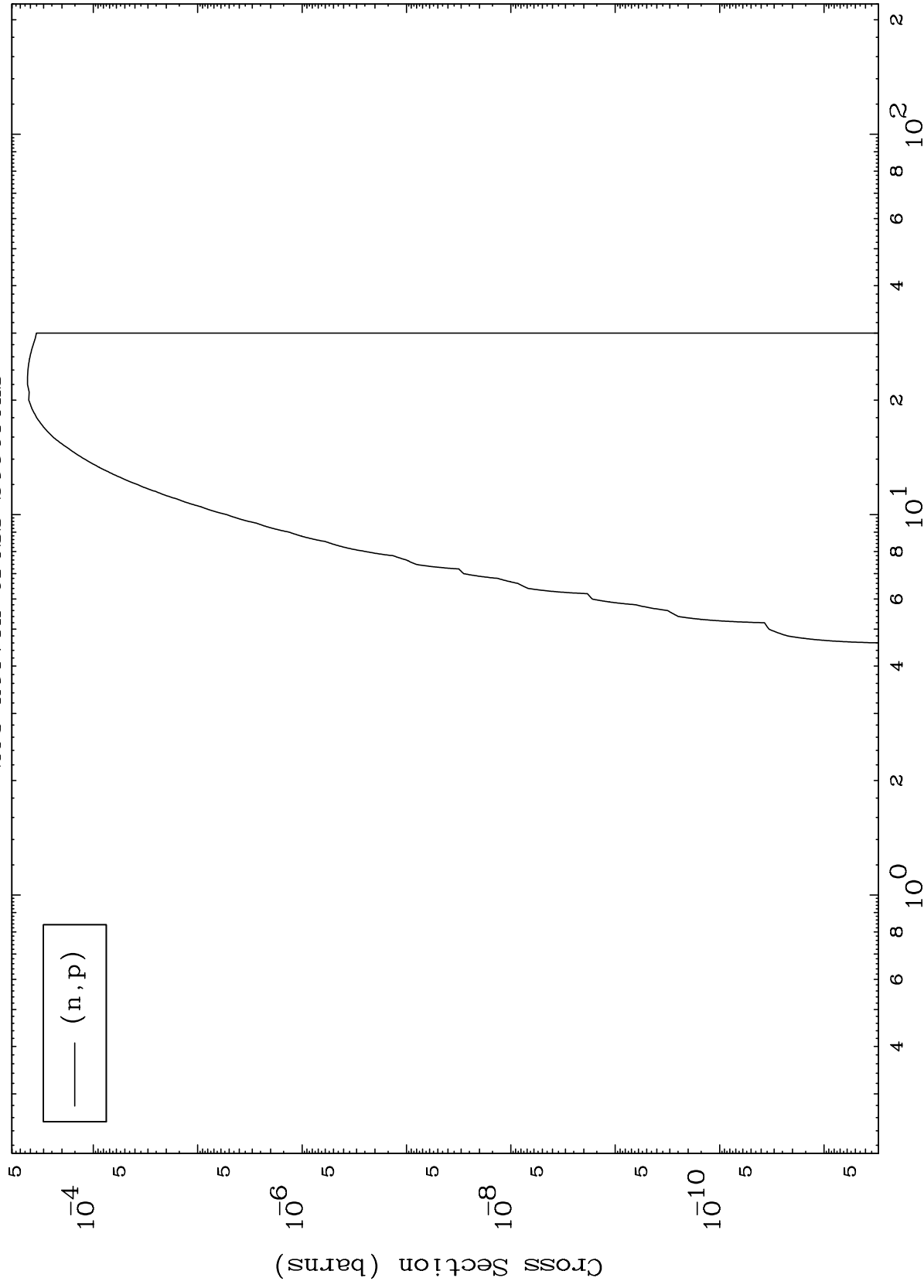
101-Md-261

MAT 9962

(n,p) Levels

101-Md-261

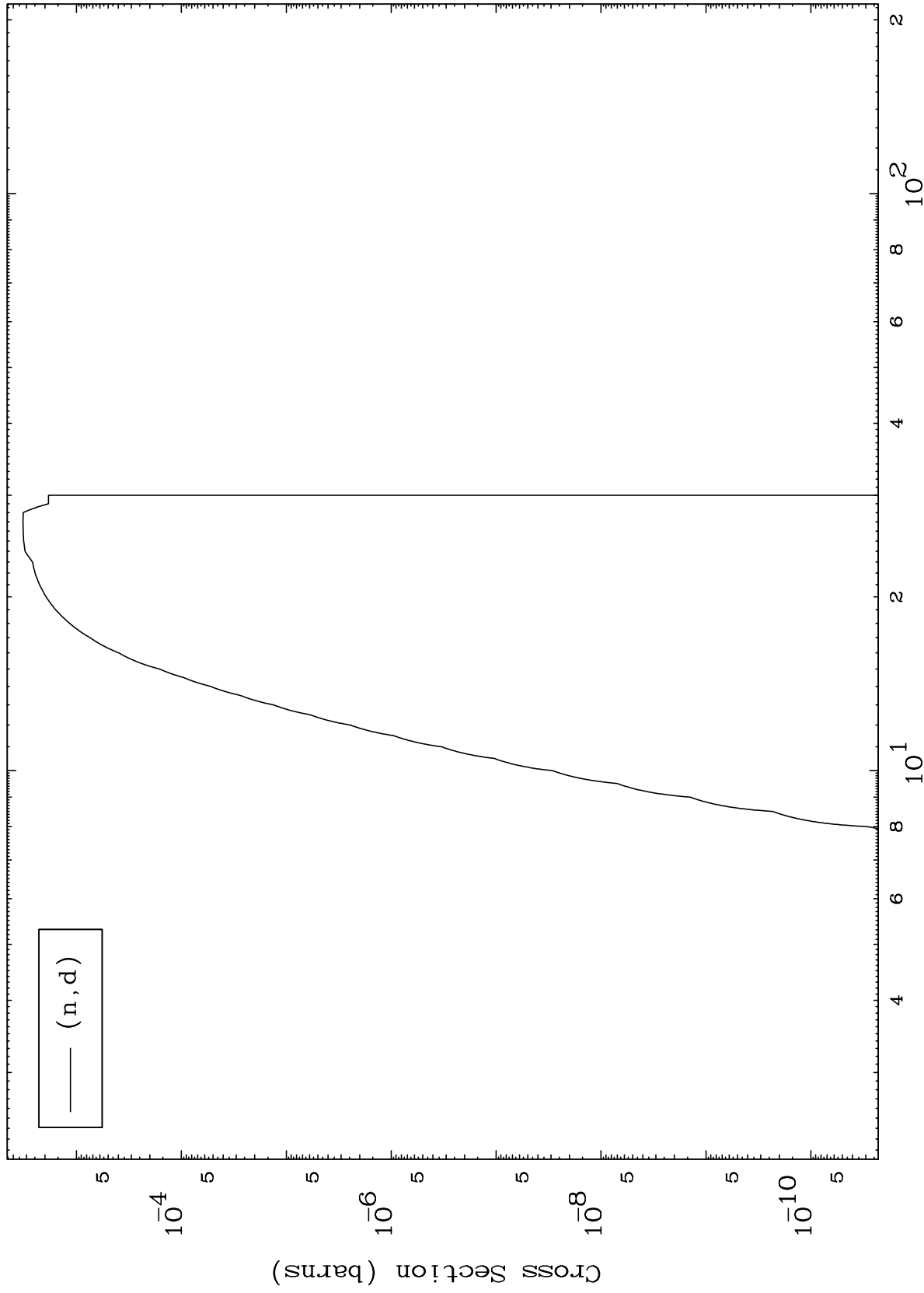
293 Kelvin Cross Sections



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-261



10

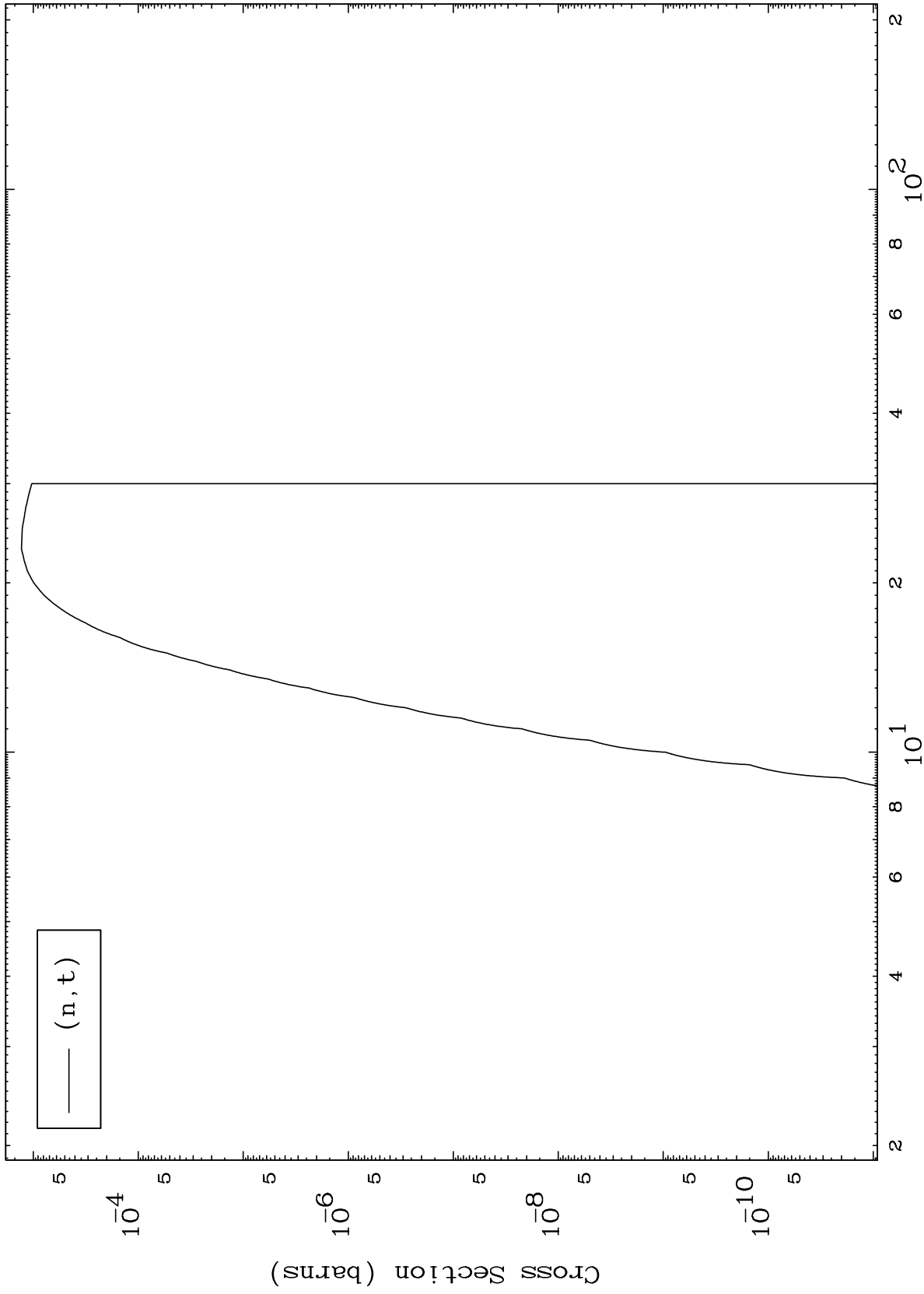
Incident Energy (MeV)

101-Md-261

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(n,t) Levels
293 Kelvin Cross Sections

101-Md-261



11

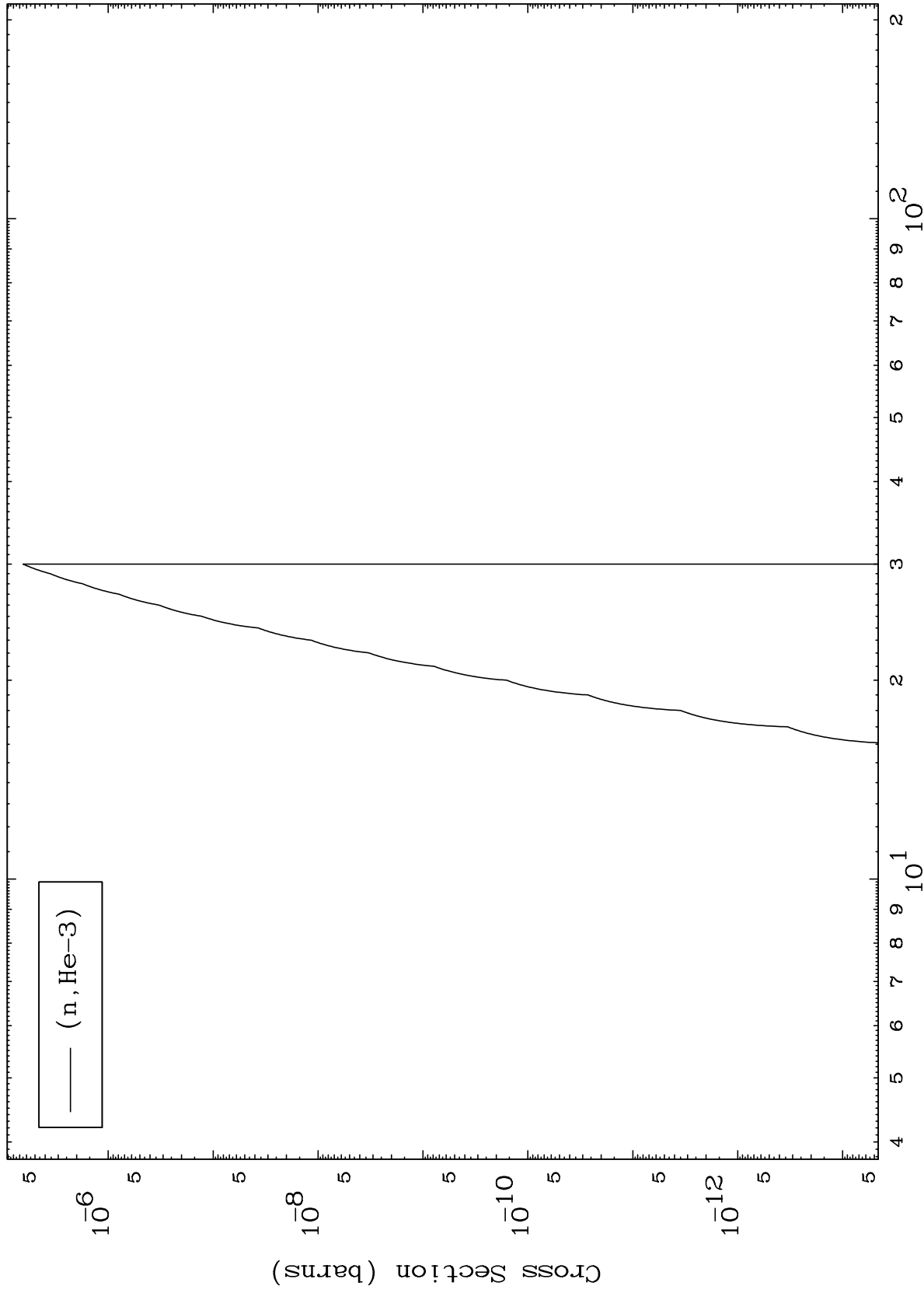
Incident Energy (MeV)

101-Md-261

MAT 9962

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-261



12

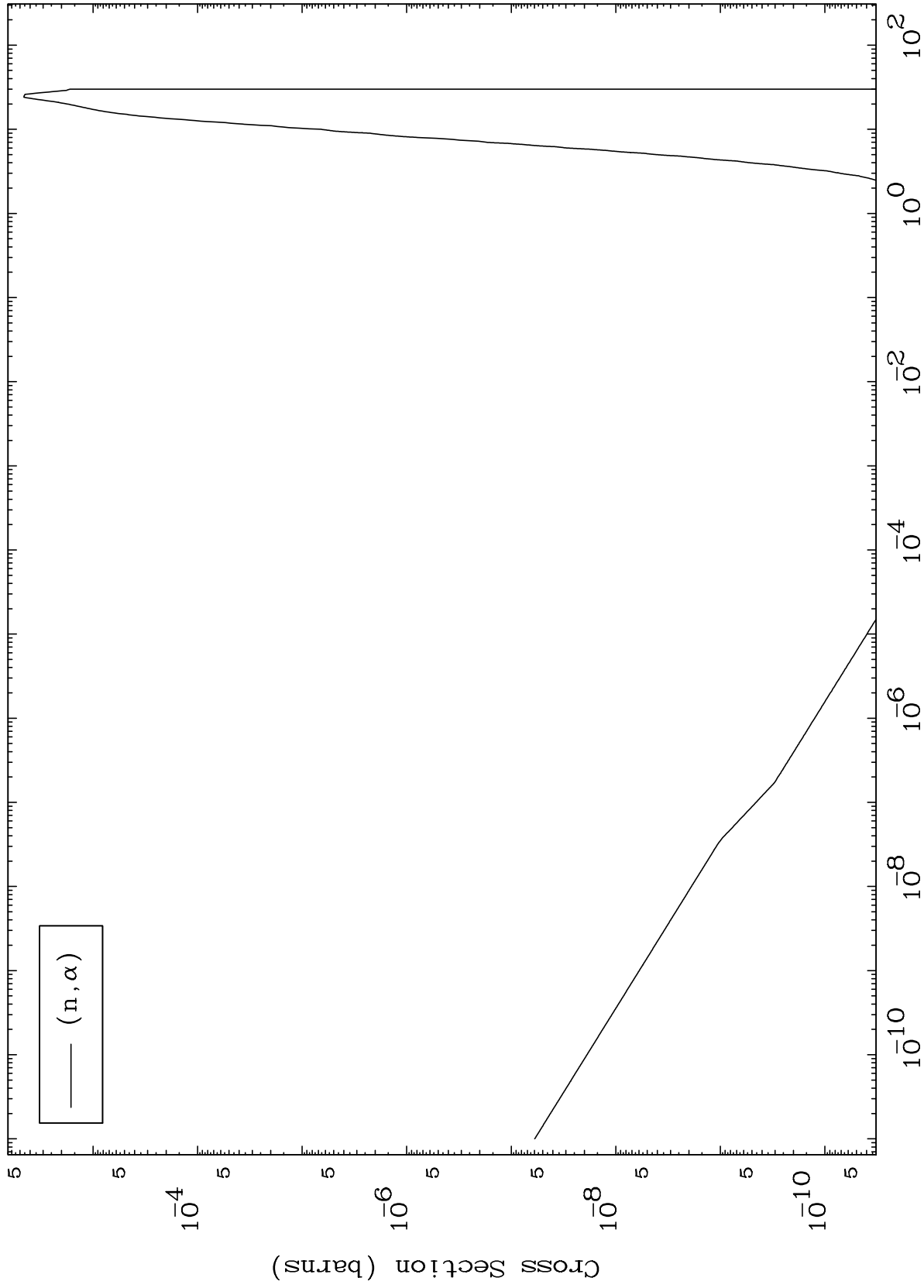
Incident Energy (MeV)

101-Md-261

MAT 9962

(n, α) Levels
293 Kelvin Cross Sections

101-Md-261



13

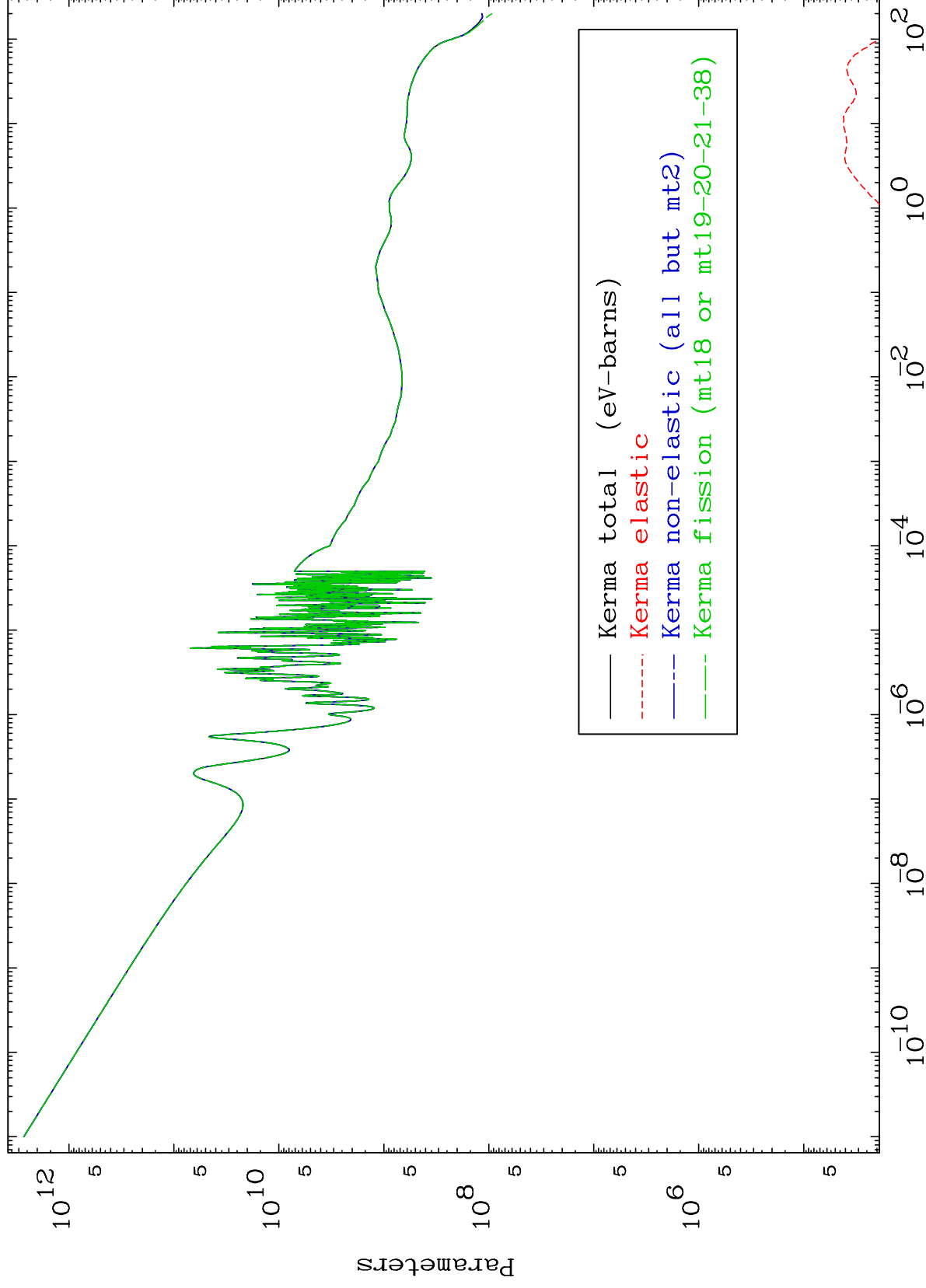
Incident Energy (MeV)

101-Md-261

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Energy Release
Parameters

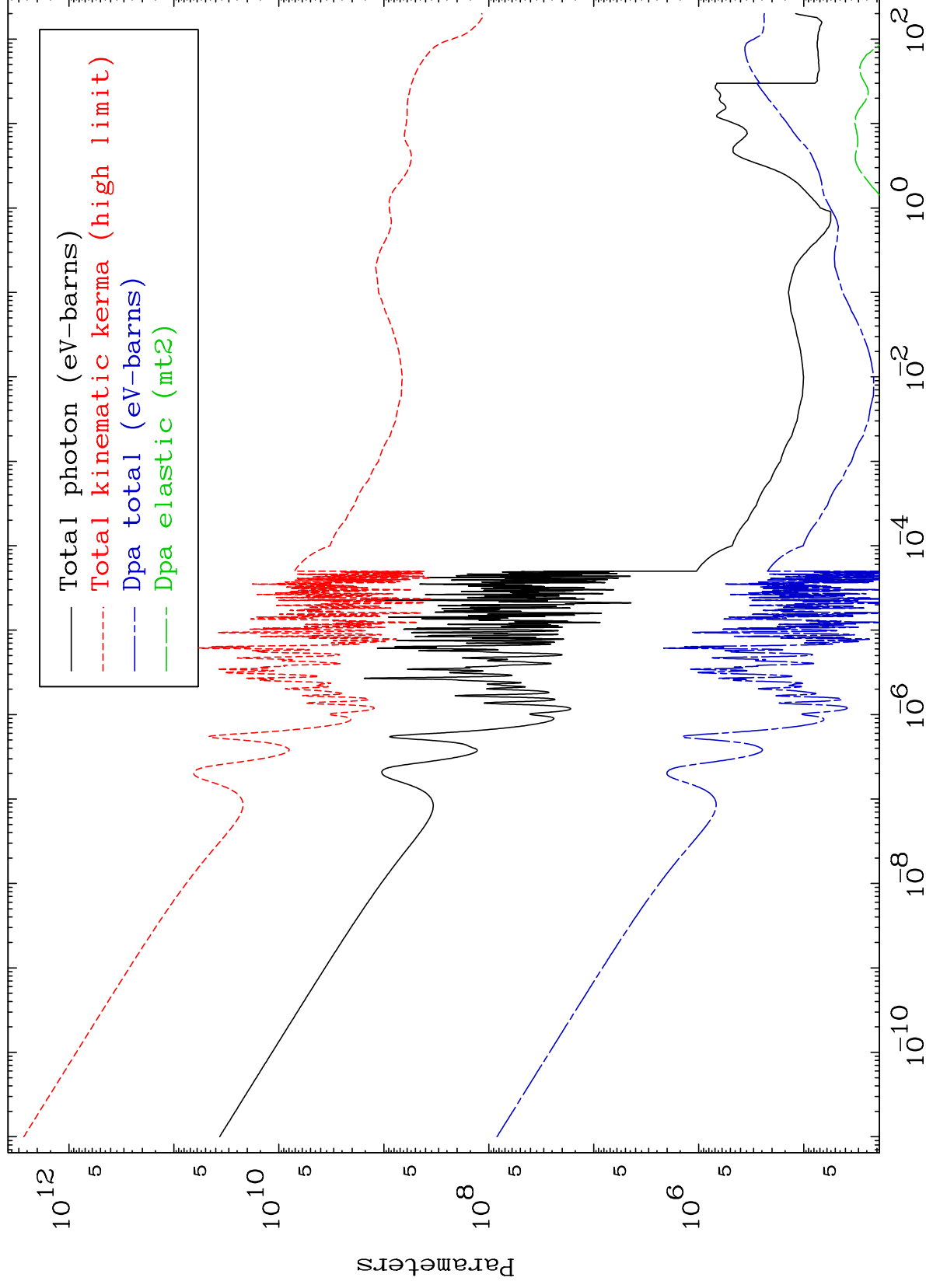
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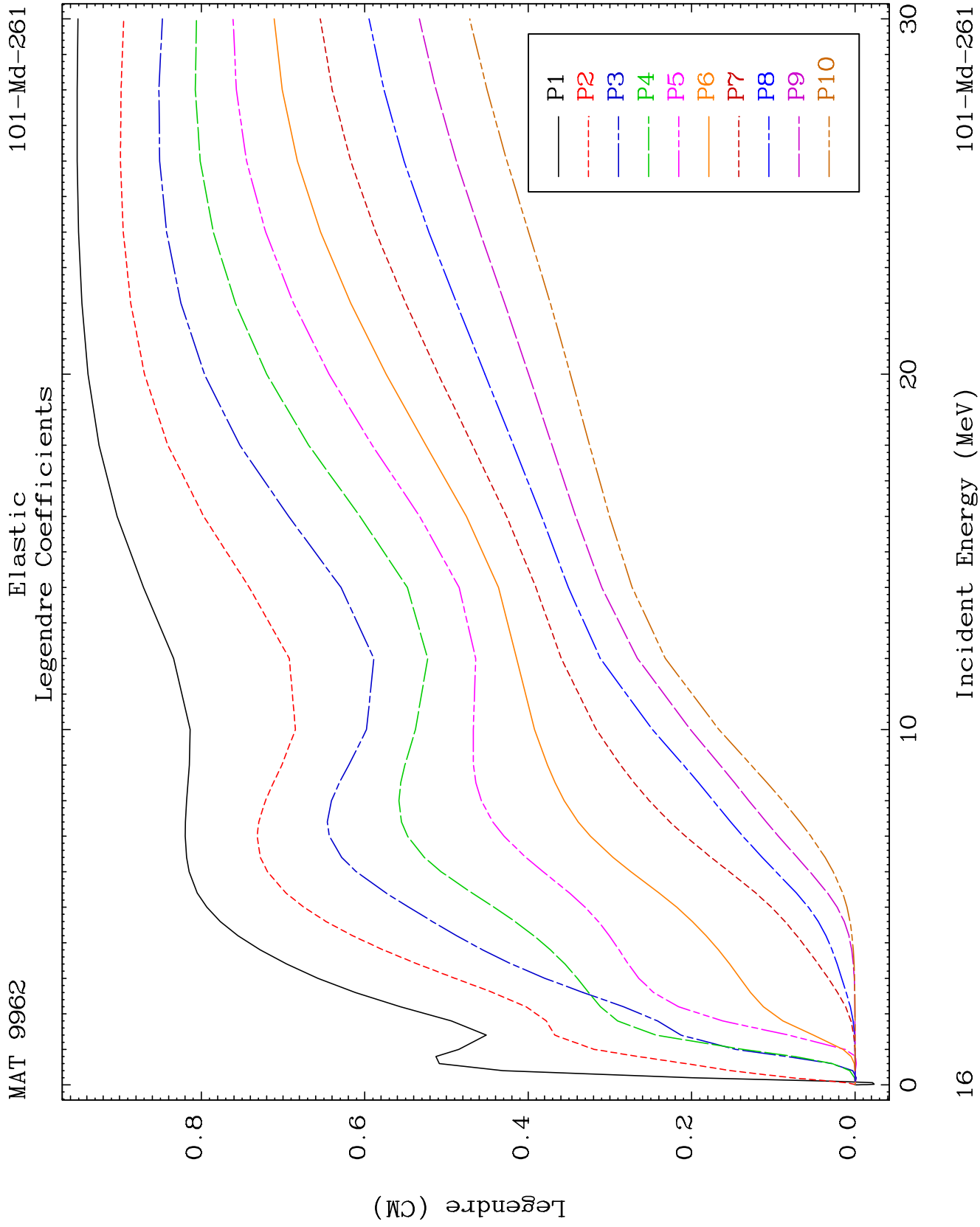


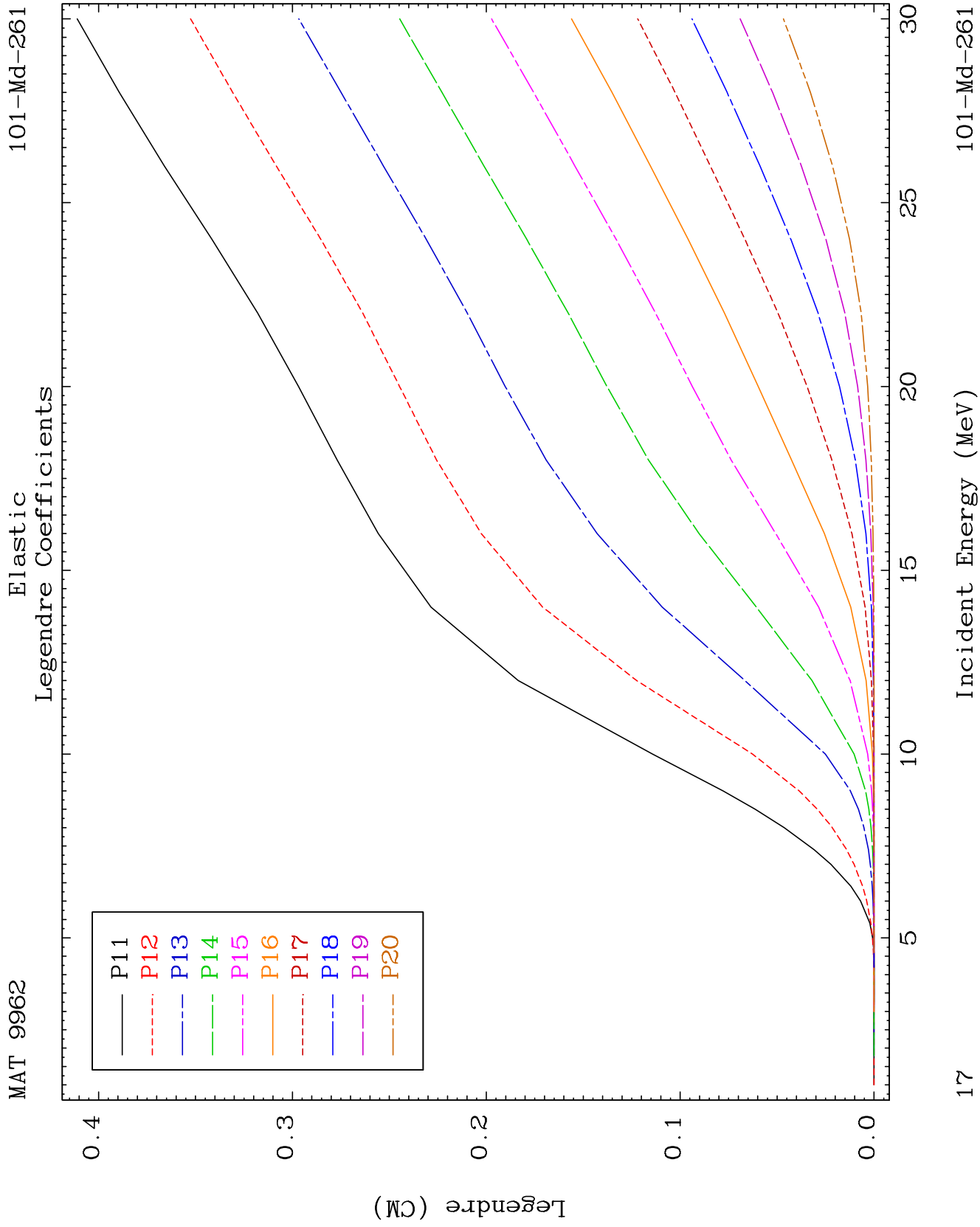
14

Incident Energy (MeV)

101-Md-261



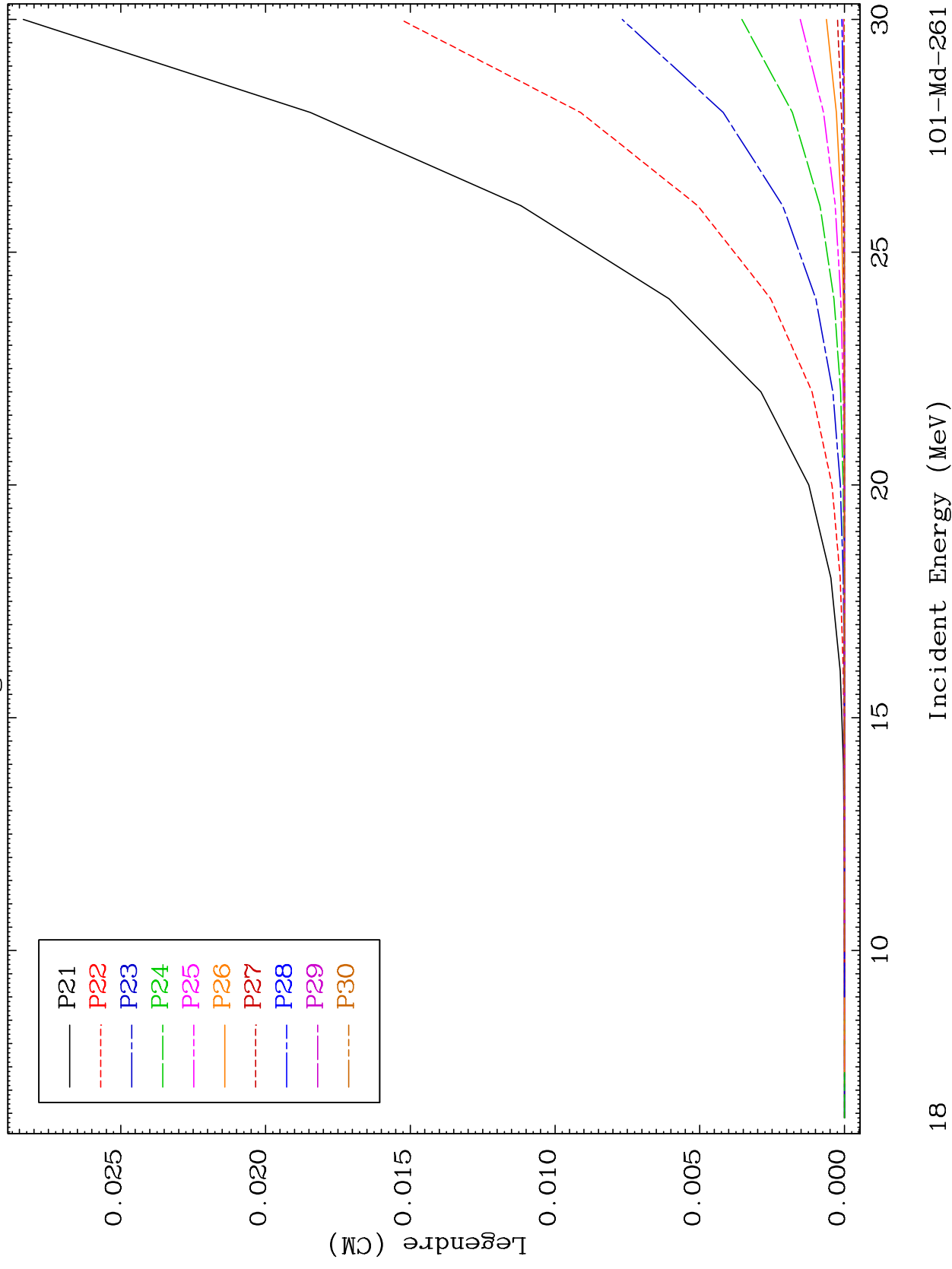


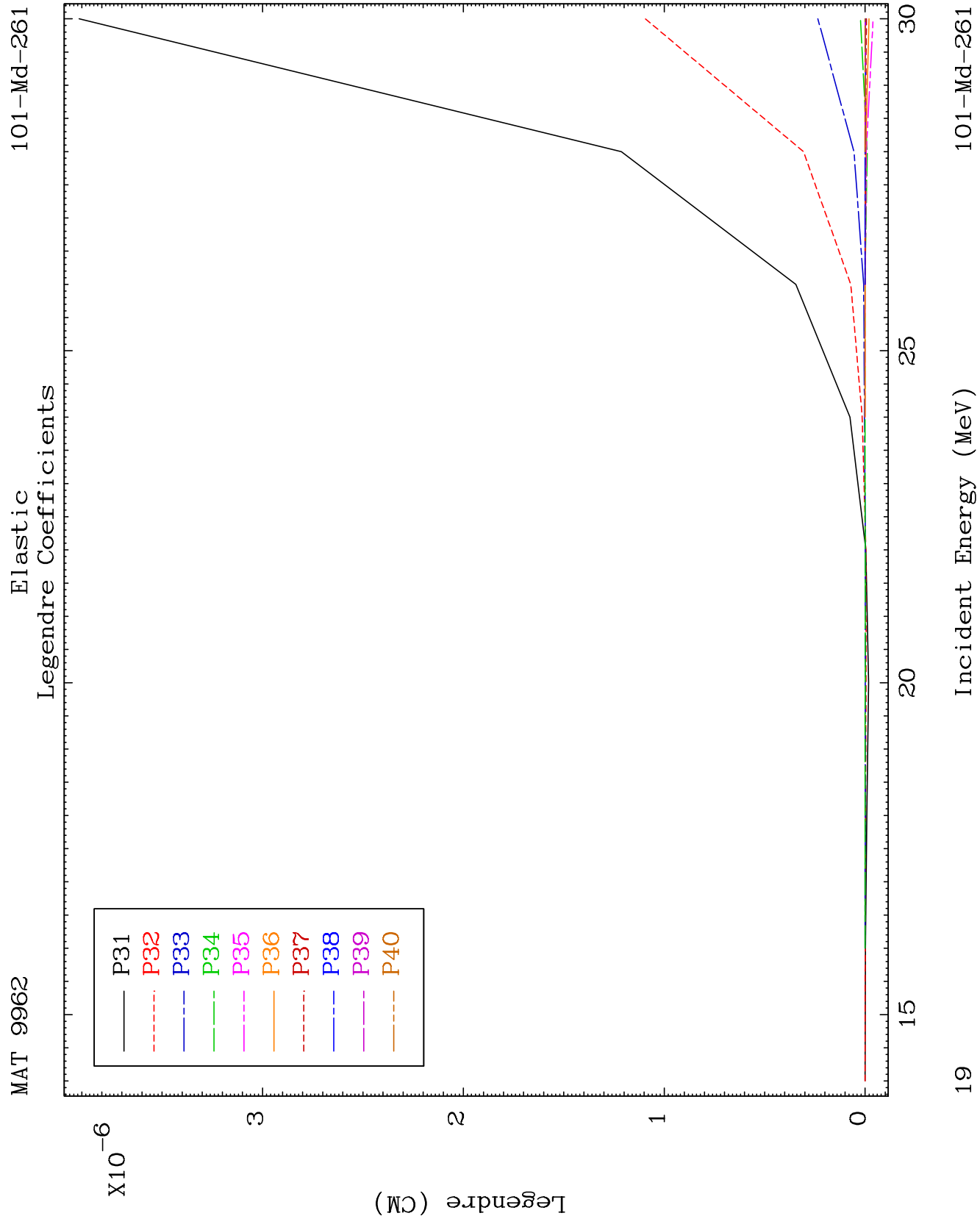


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Elastic Legendre Coefficients

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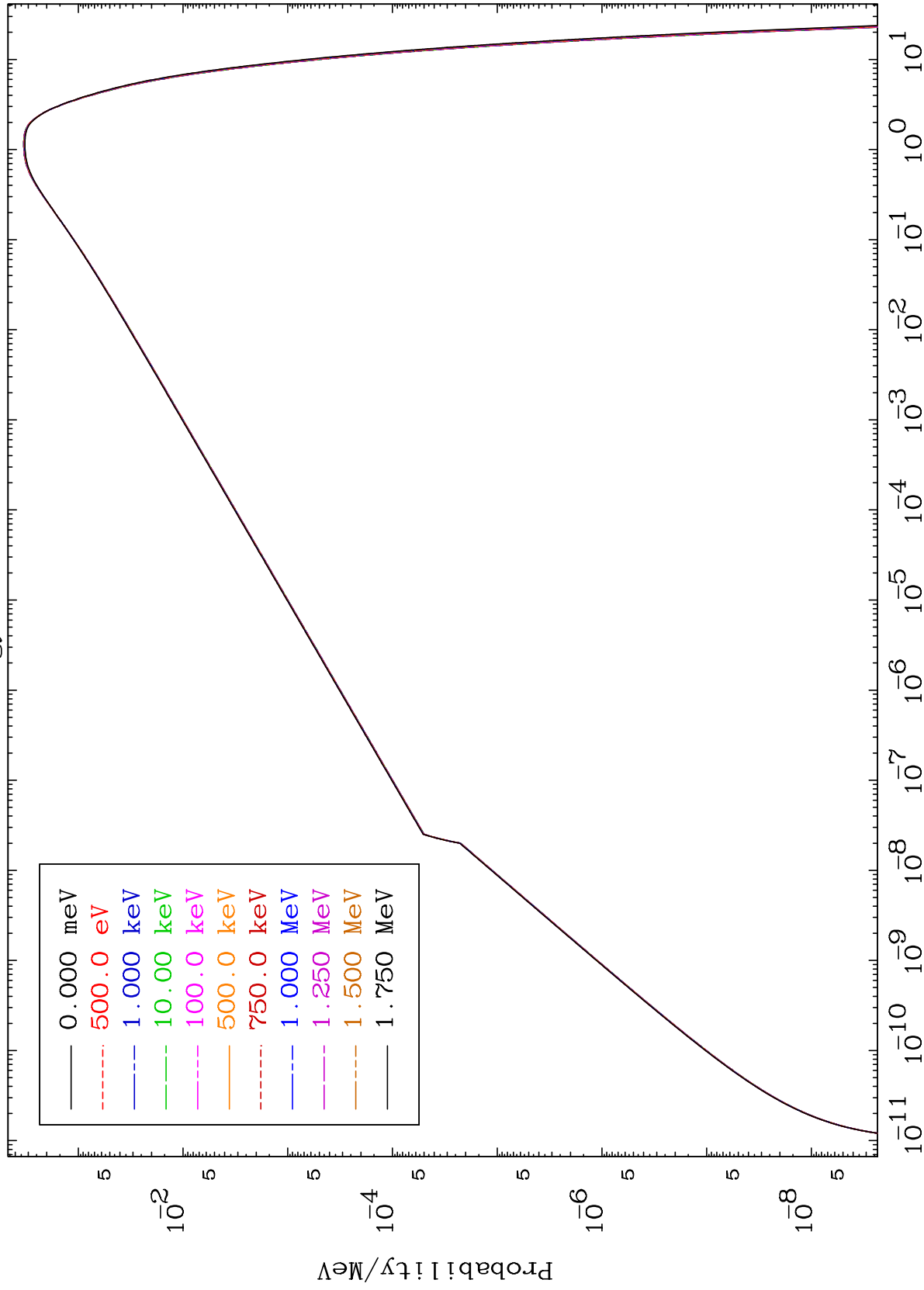




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Fission
Energy Distribution

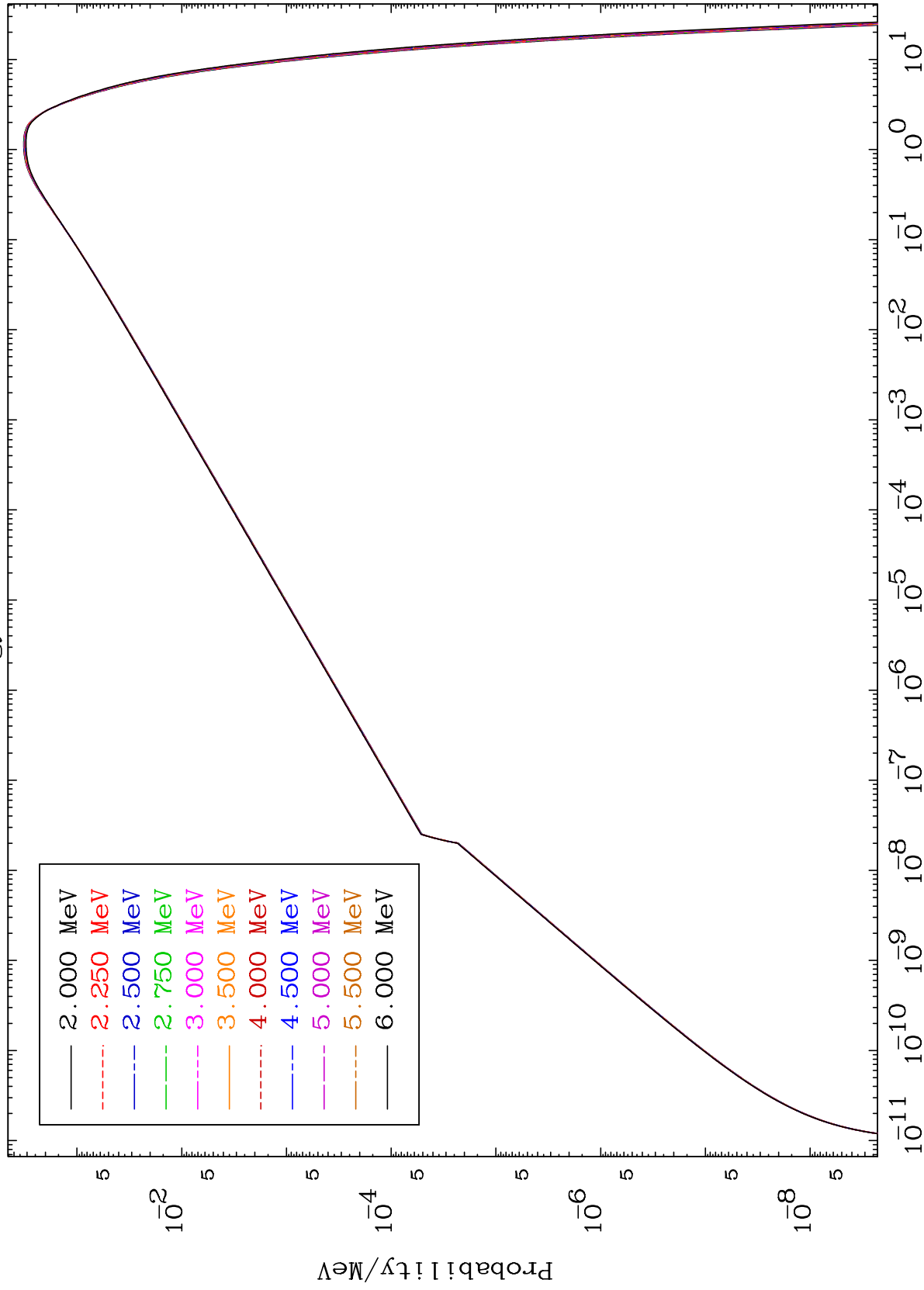
101-Md-261



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Fission
Energy Distribution

101-Md-261



21

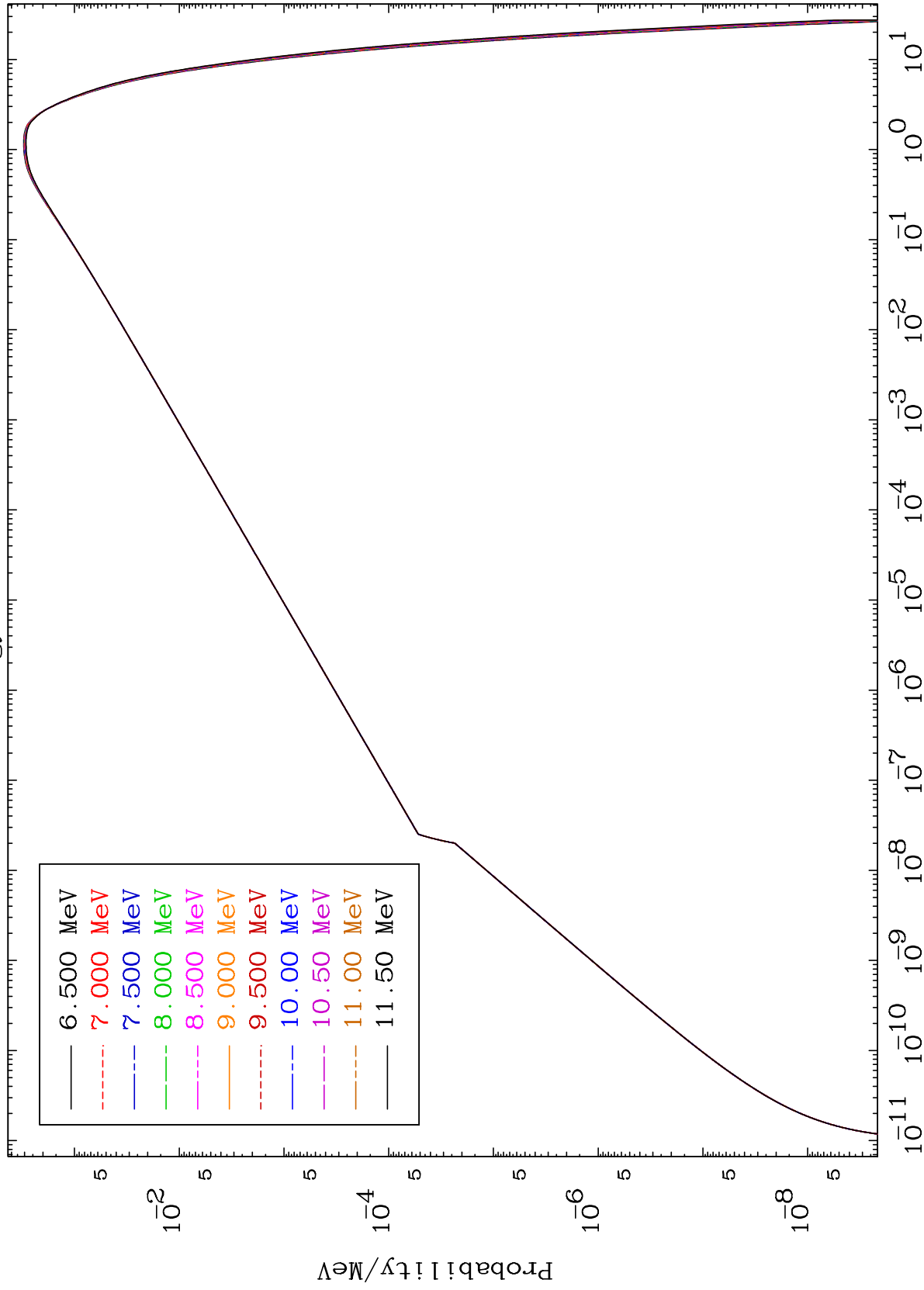
Secondary Energy (MeV)

101-Md-261

MAT 9962

Fission
Energy Distribution

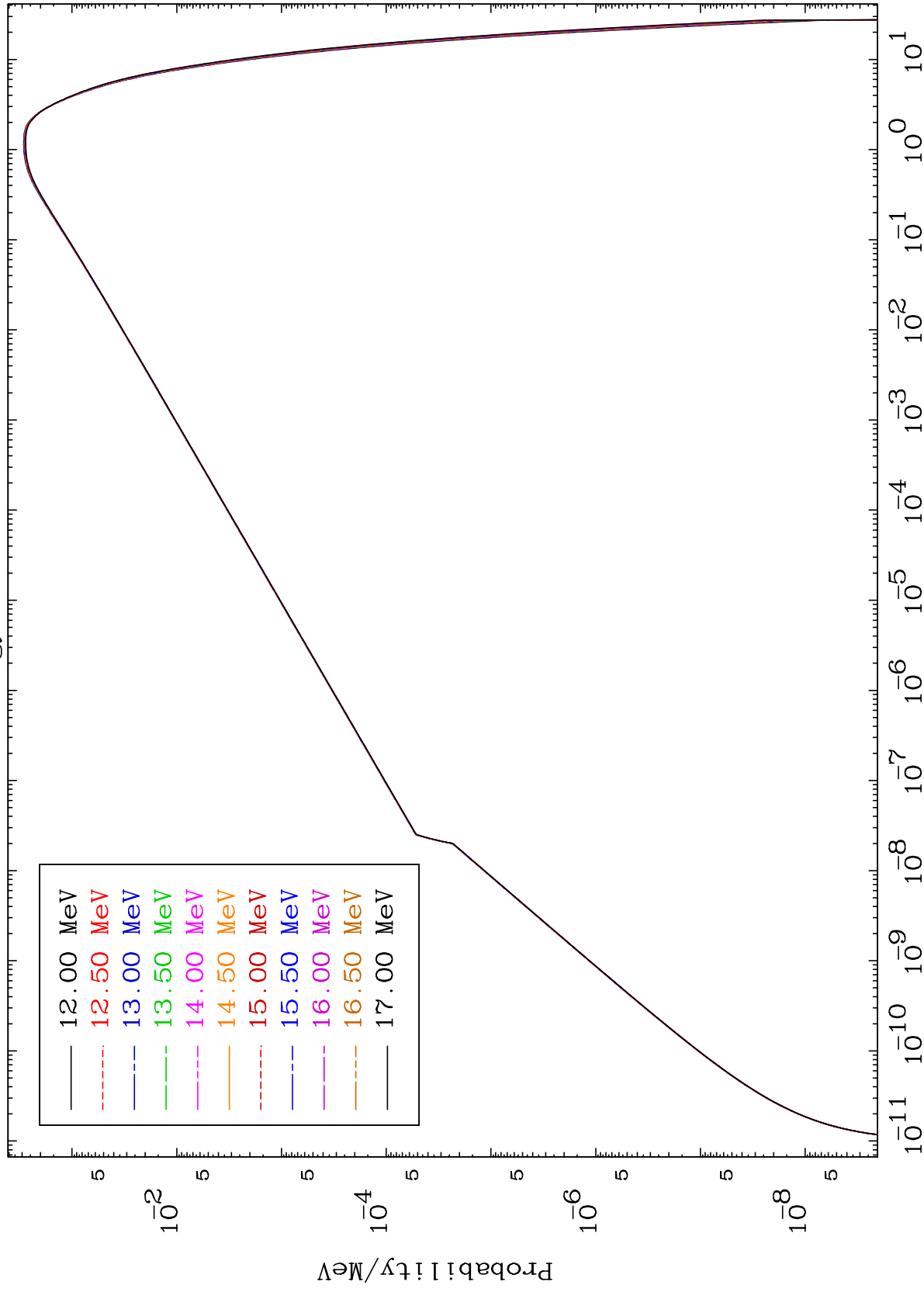
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MAT 9962

Fission
Energy Distribution

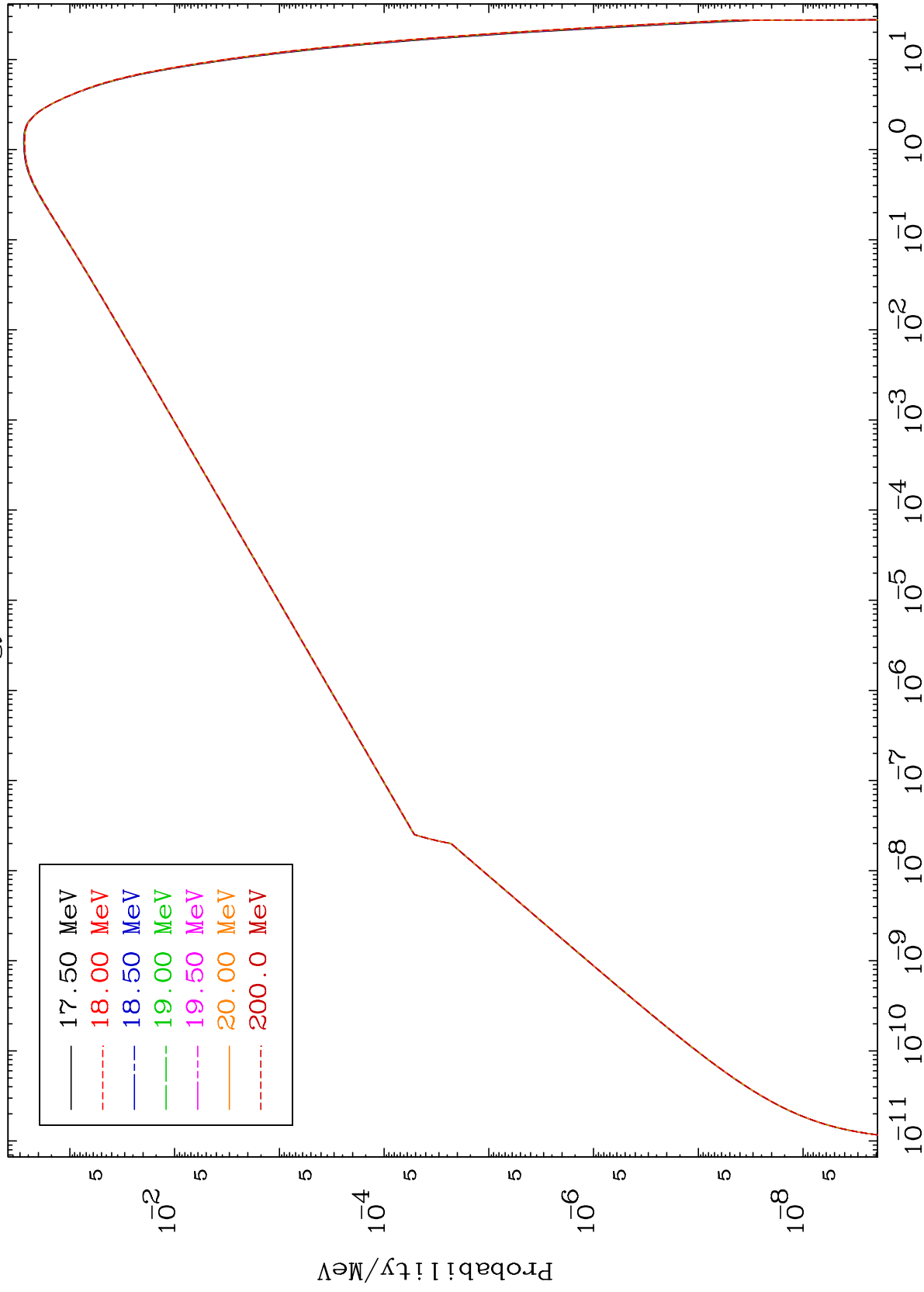
101-Md-261



MAT 9962

Fission
Energy Distribution

101-Md-261

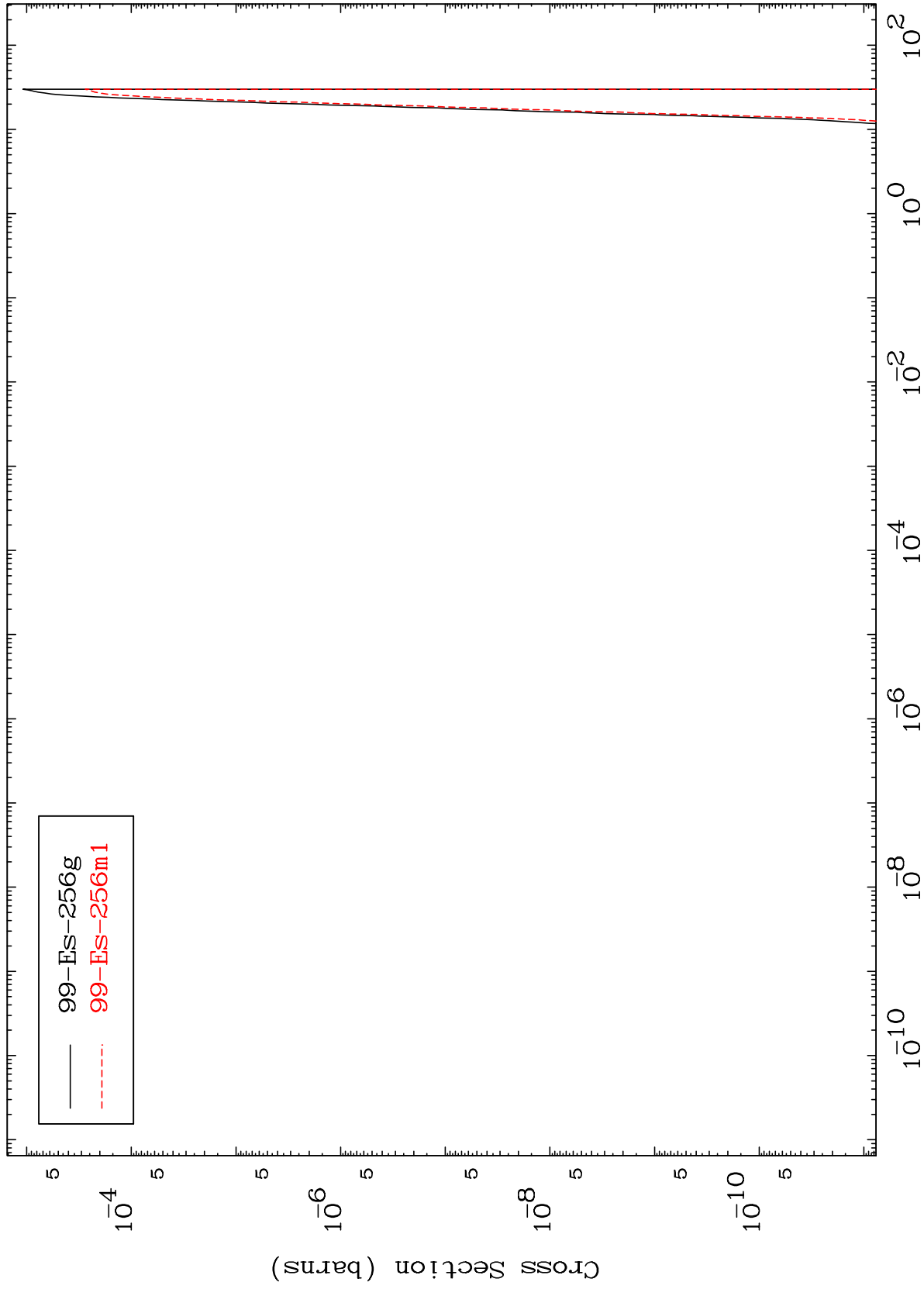


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(n,2n) α

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Radionuclide Production Cross Section



25

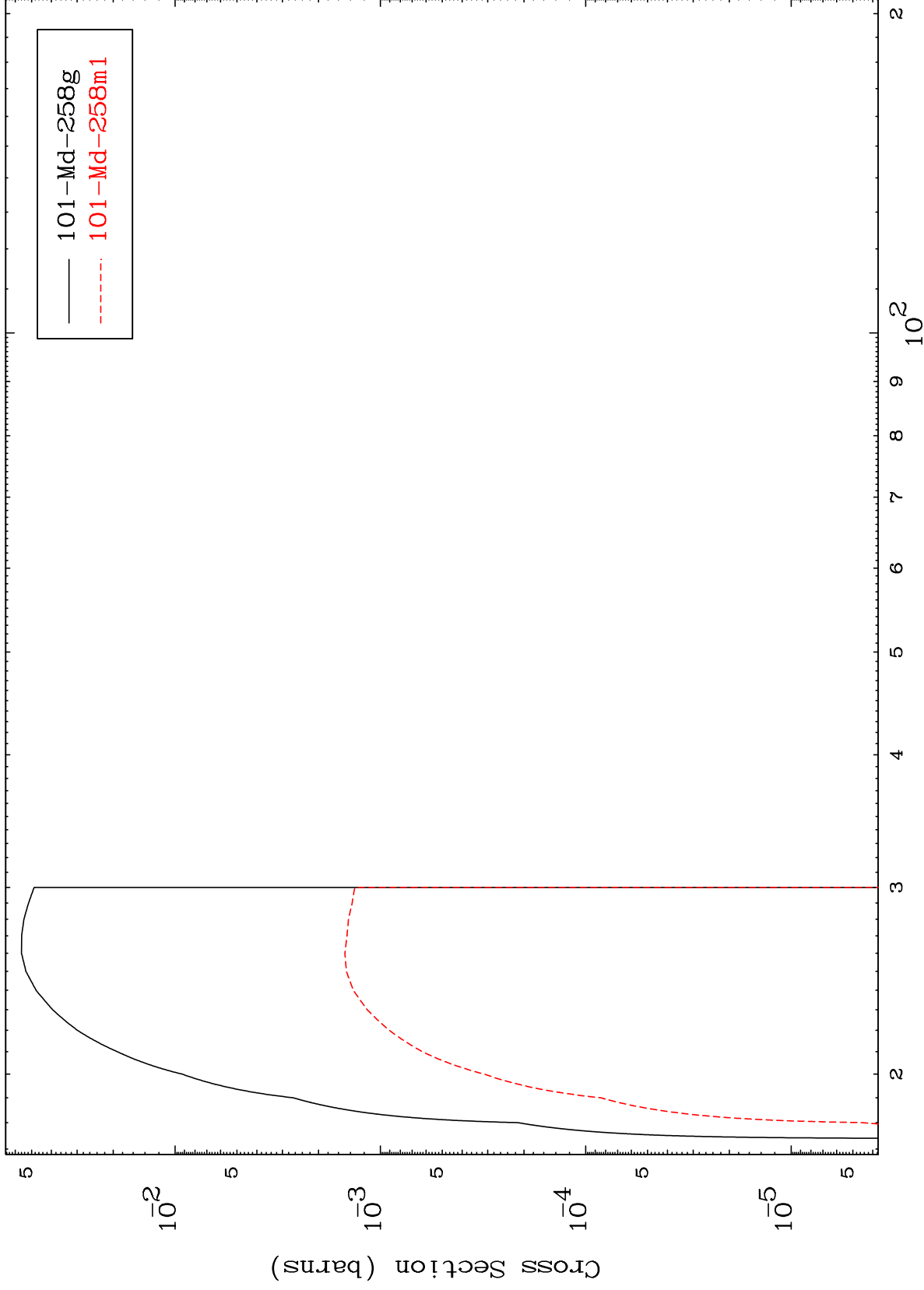
Incident Energy (MeV)

101-Md-261

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101-Md-261

(n,4n)
Radionuclide Production Cross Section



26

Incident Energy (MeV)

101-Md-261