

Program EVALPLOT
(Version 2018-1)

by

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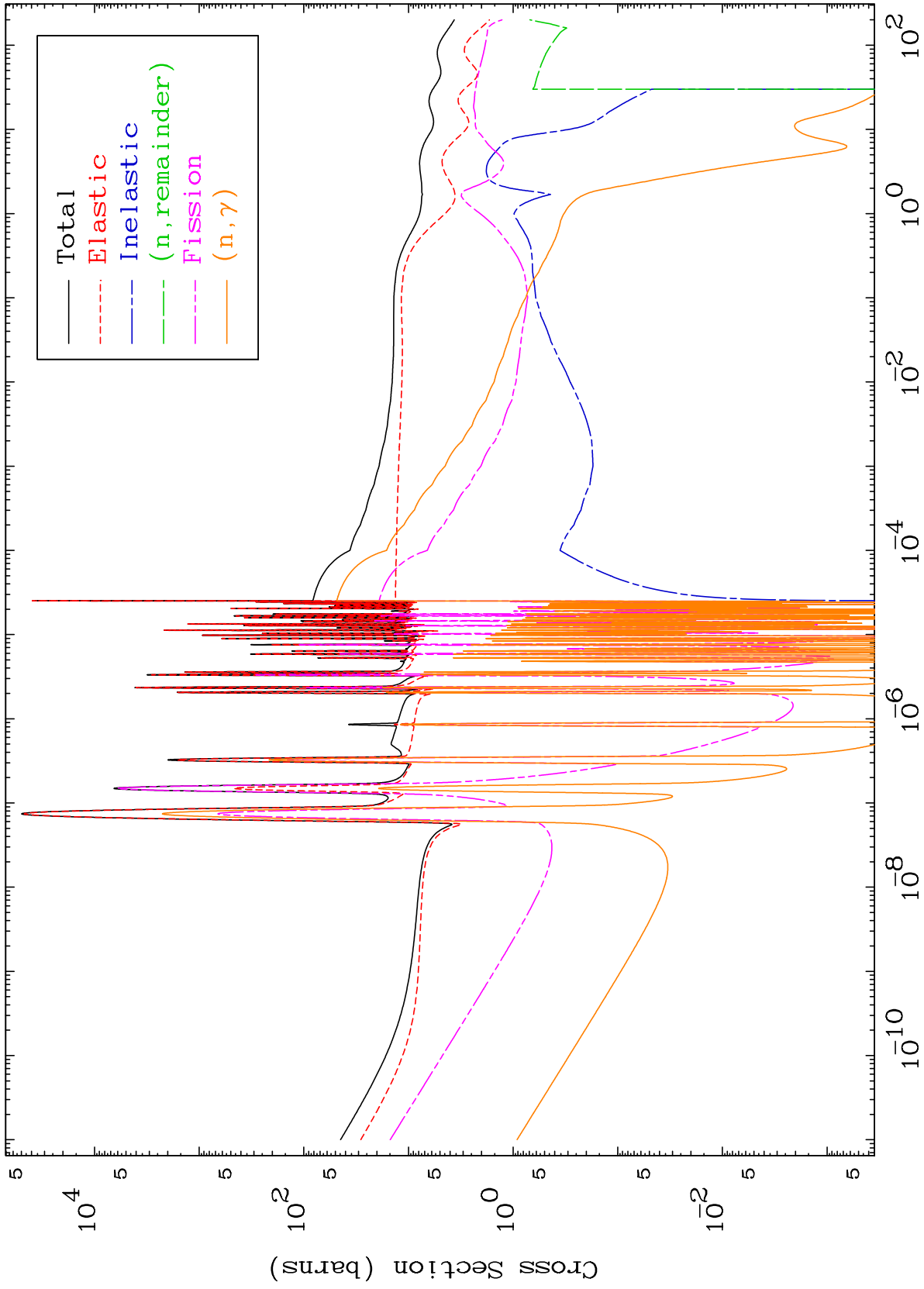
Press Mouse Button to Start

MAT 9974

Major

293 Kelvin Cross Sections

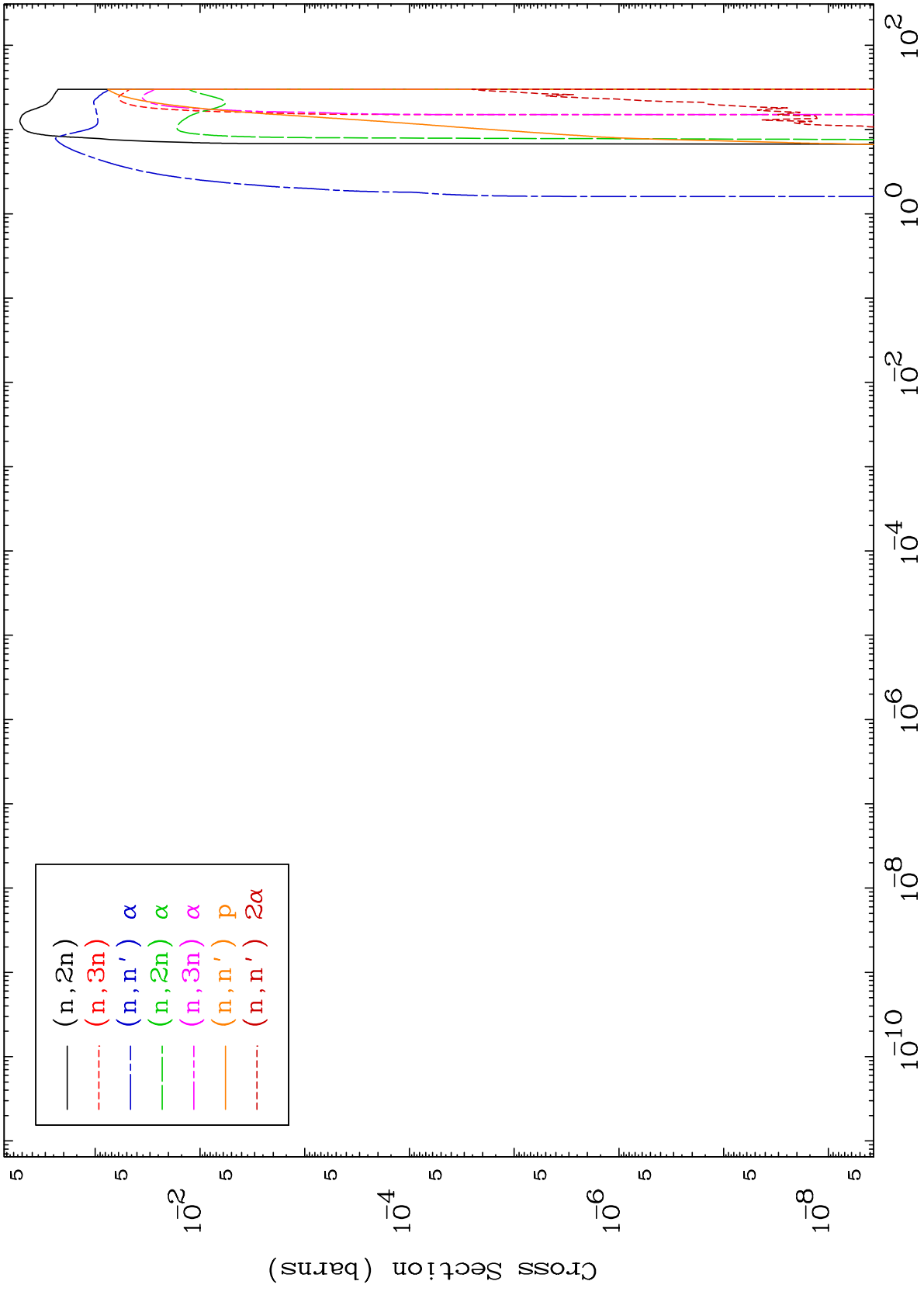
102-No-251



MAT 9974

Neutron Production
293 Kelvin Cross Sections

102-No-251

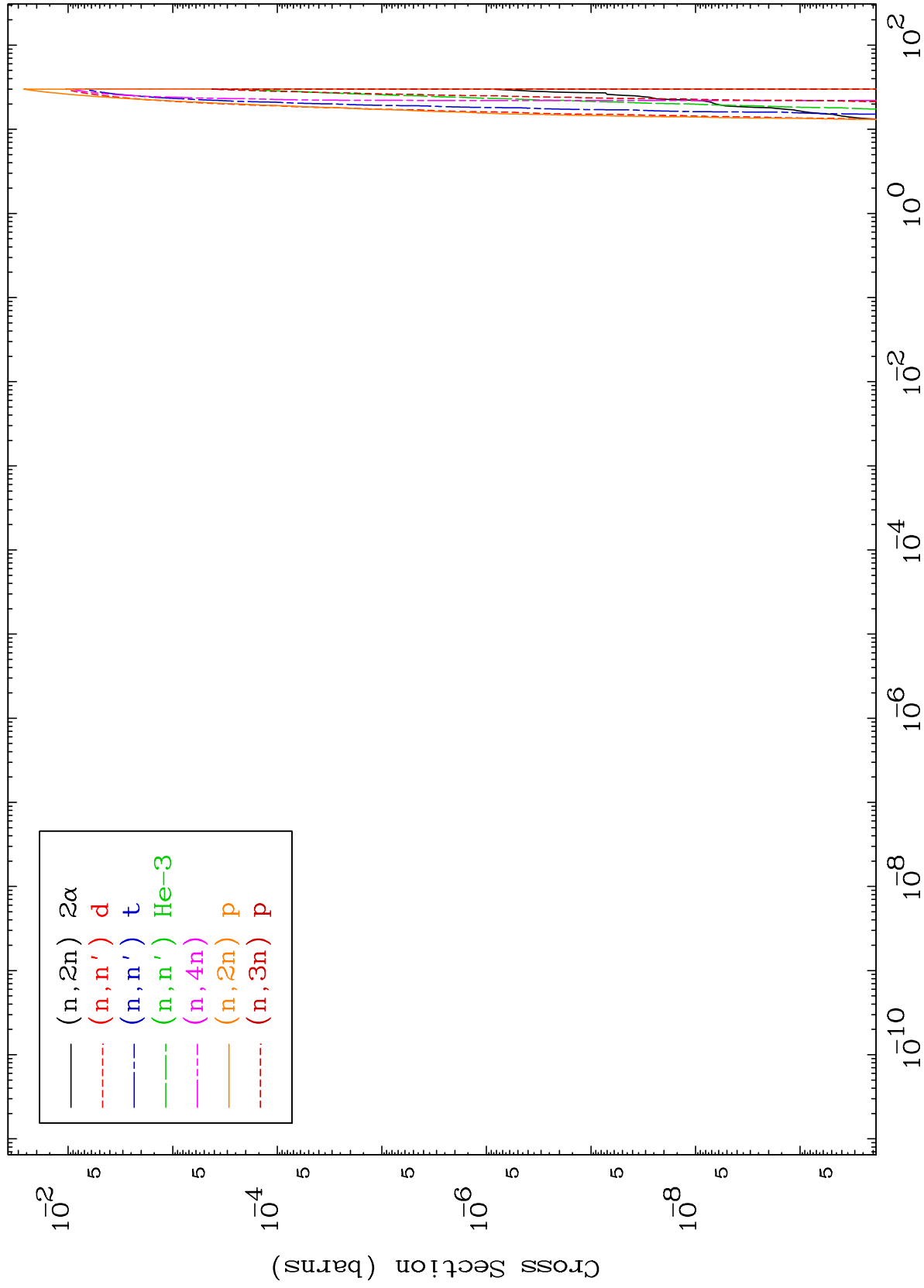


102-No-251

MAT 9974

Neutron Production
293 Kelvin Cross Sections

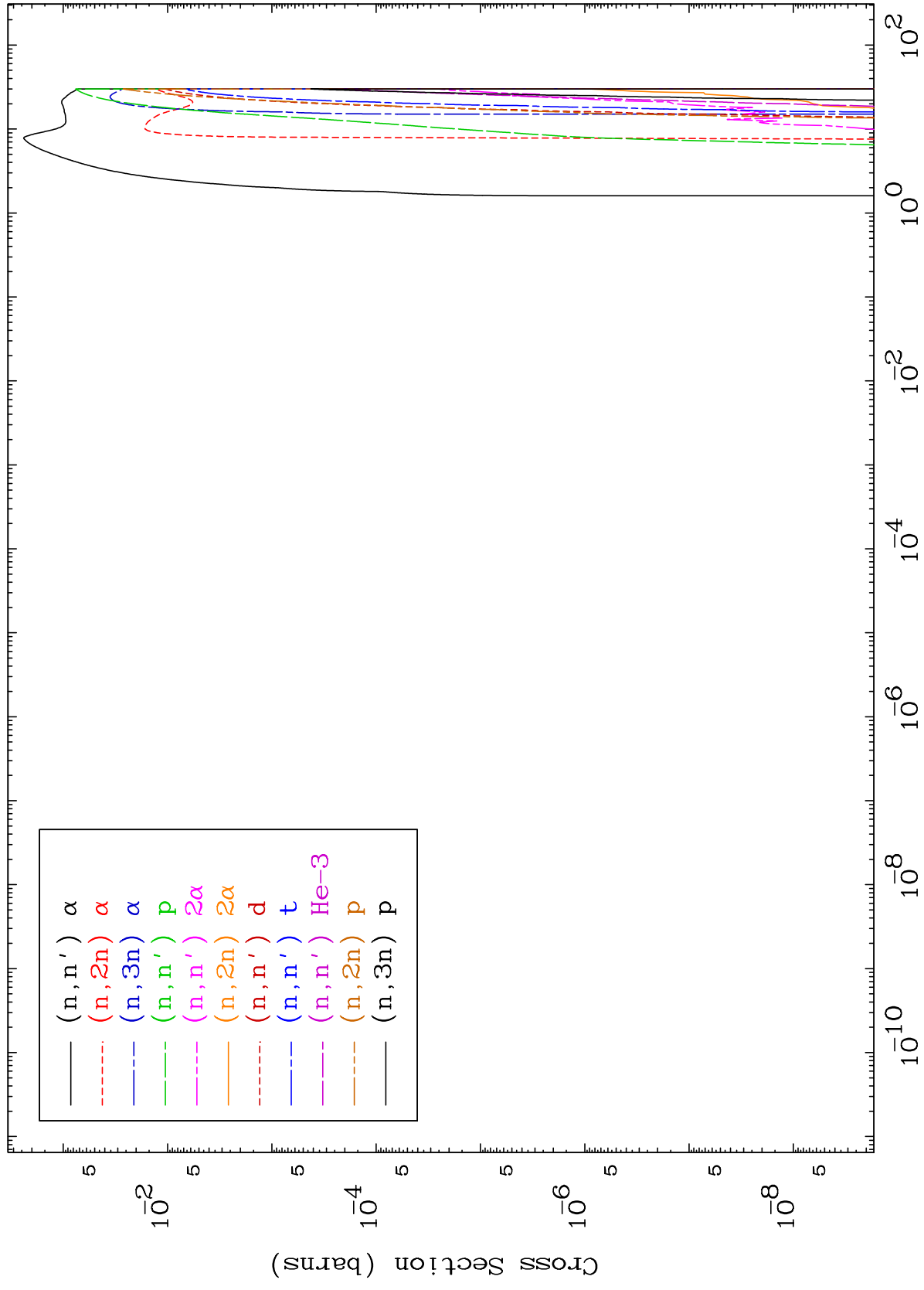
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MAT 9974

Charged Particle
293 Kelvin Cross Sections

102-No-251

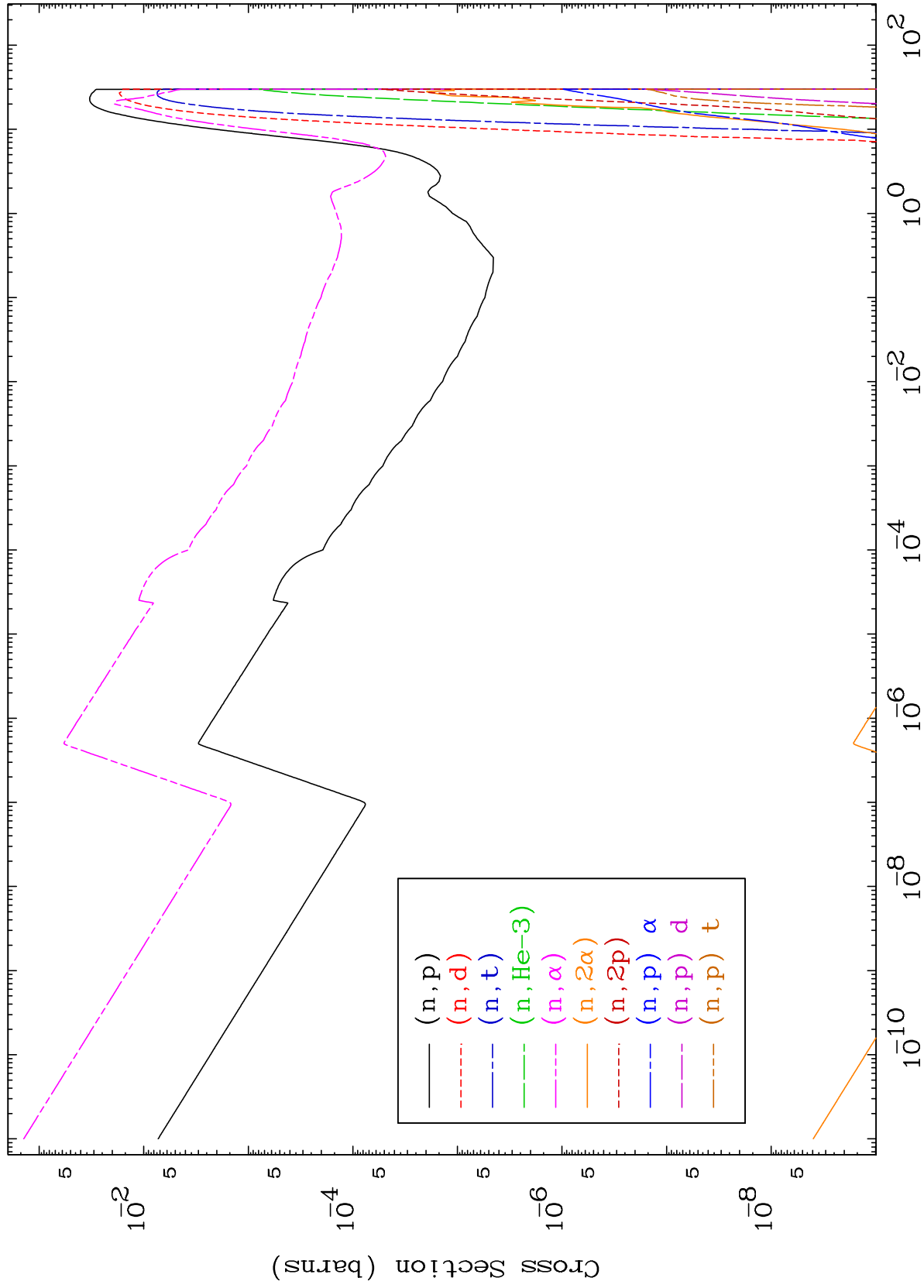


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Charged Particle
293 Kelvin Cross Sections

102-No-251



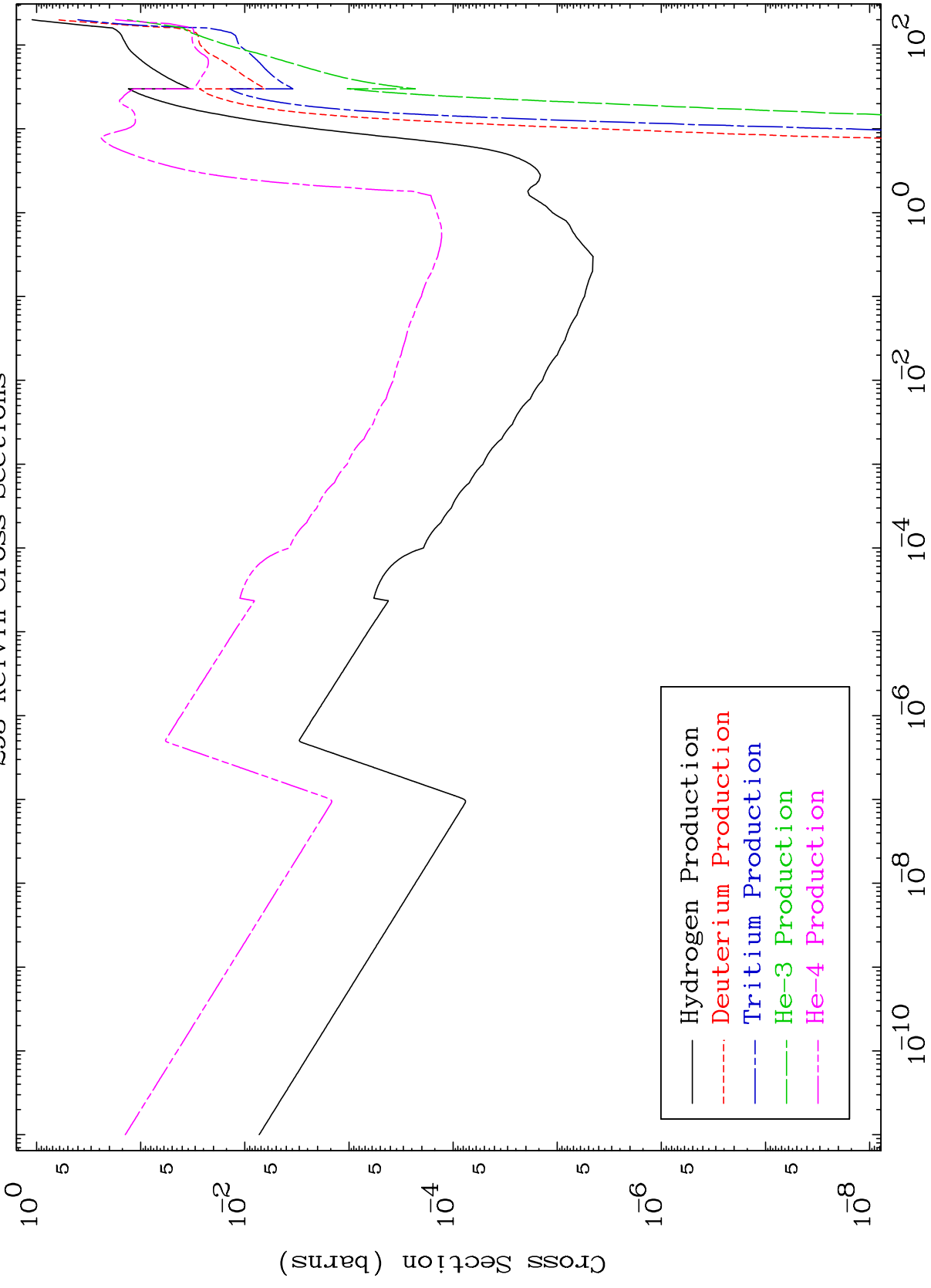
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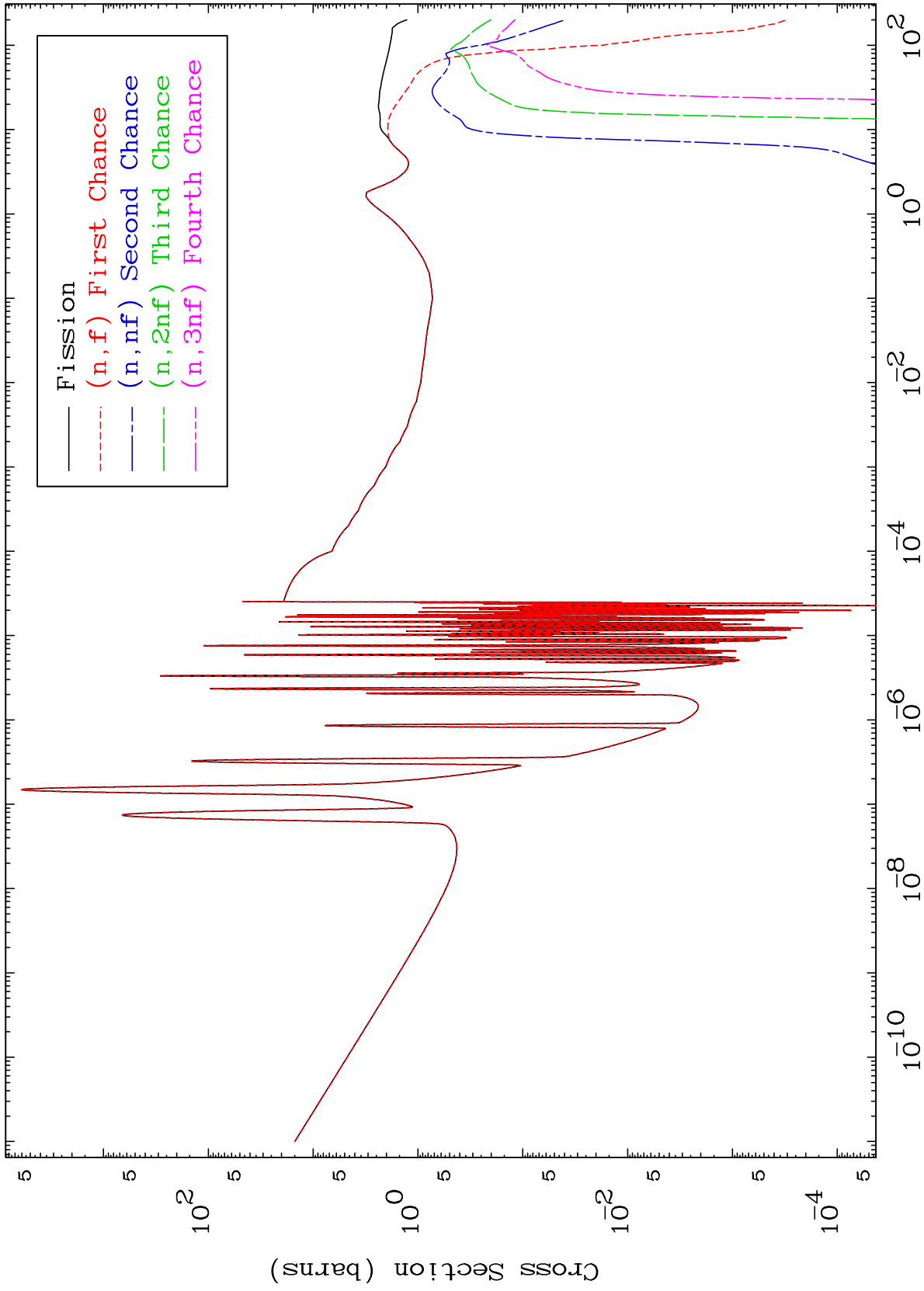
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Particle Production
293 Kelvin Cross Sections

102-No-251

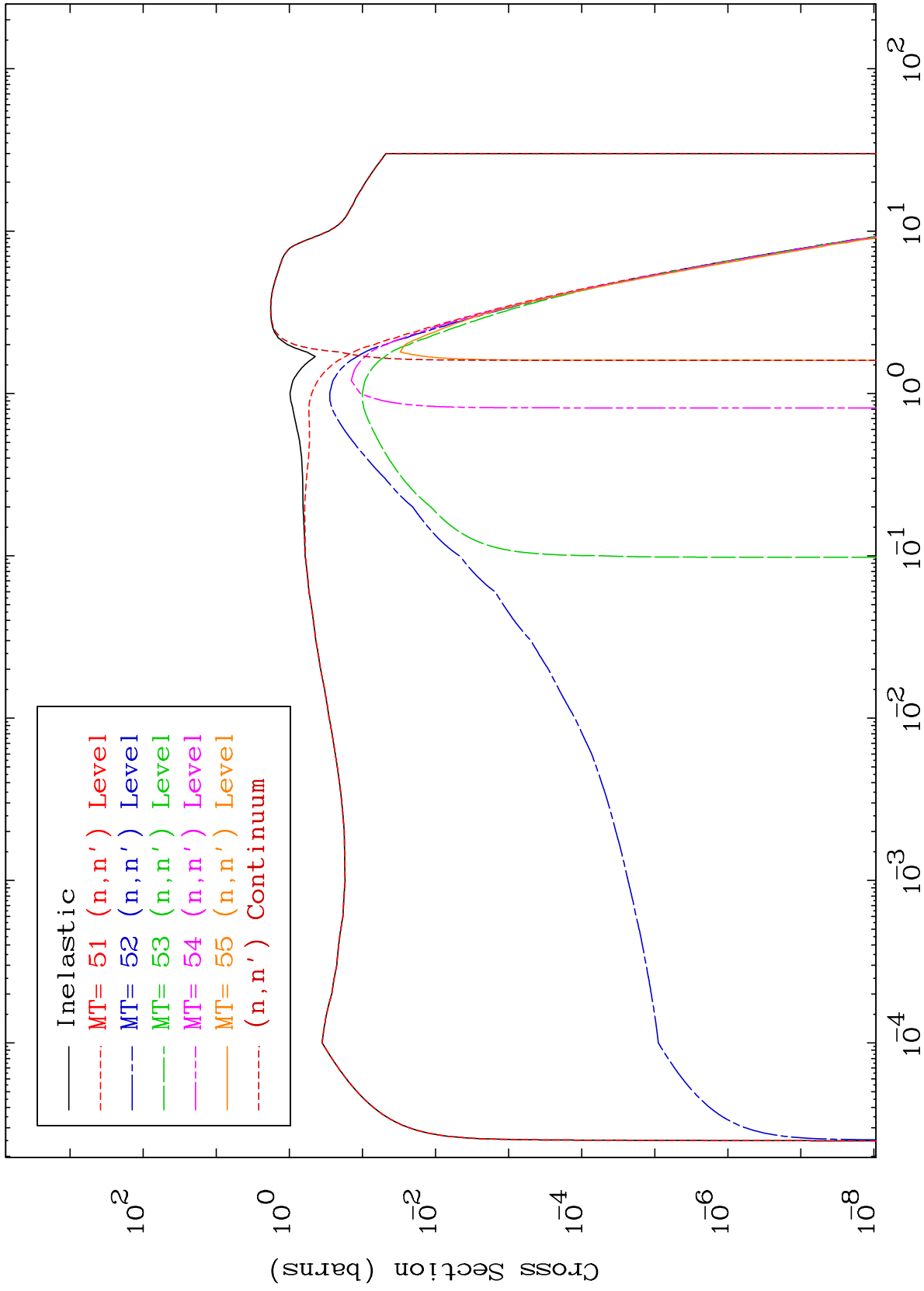




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293 Kelvin Cross Sections

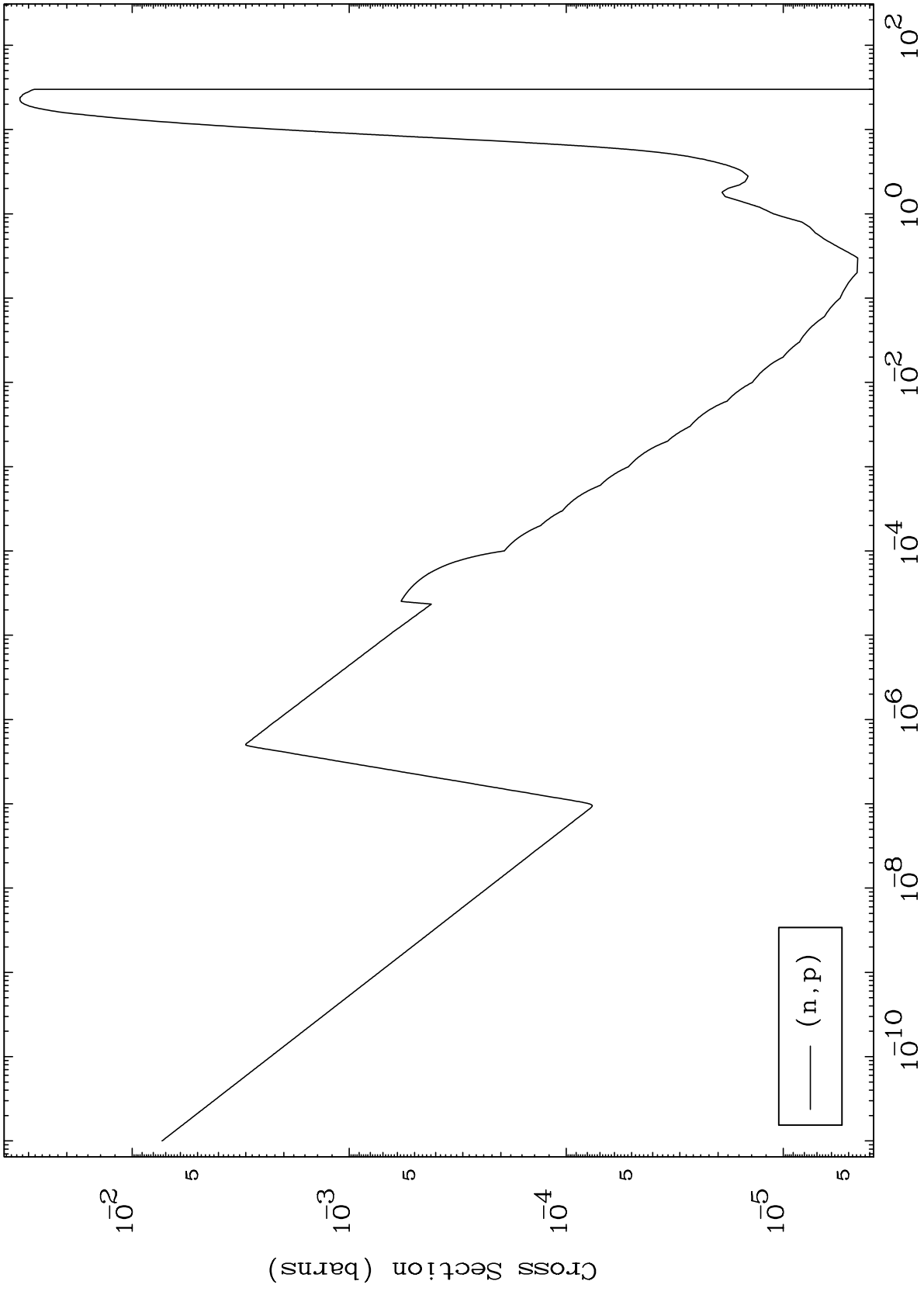
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(n,p) Levels
293 Kelvin Cross Sections

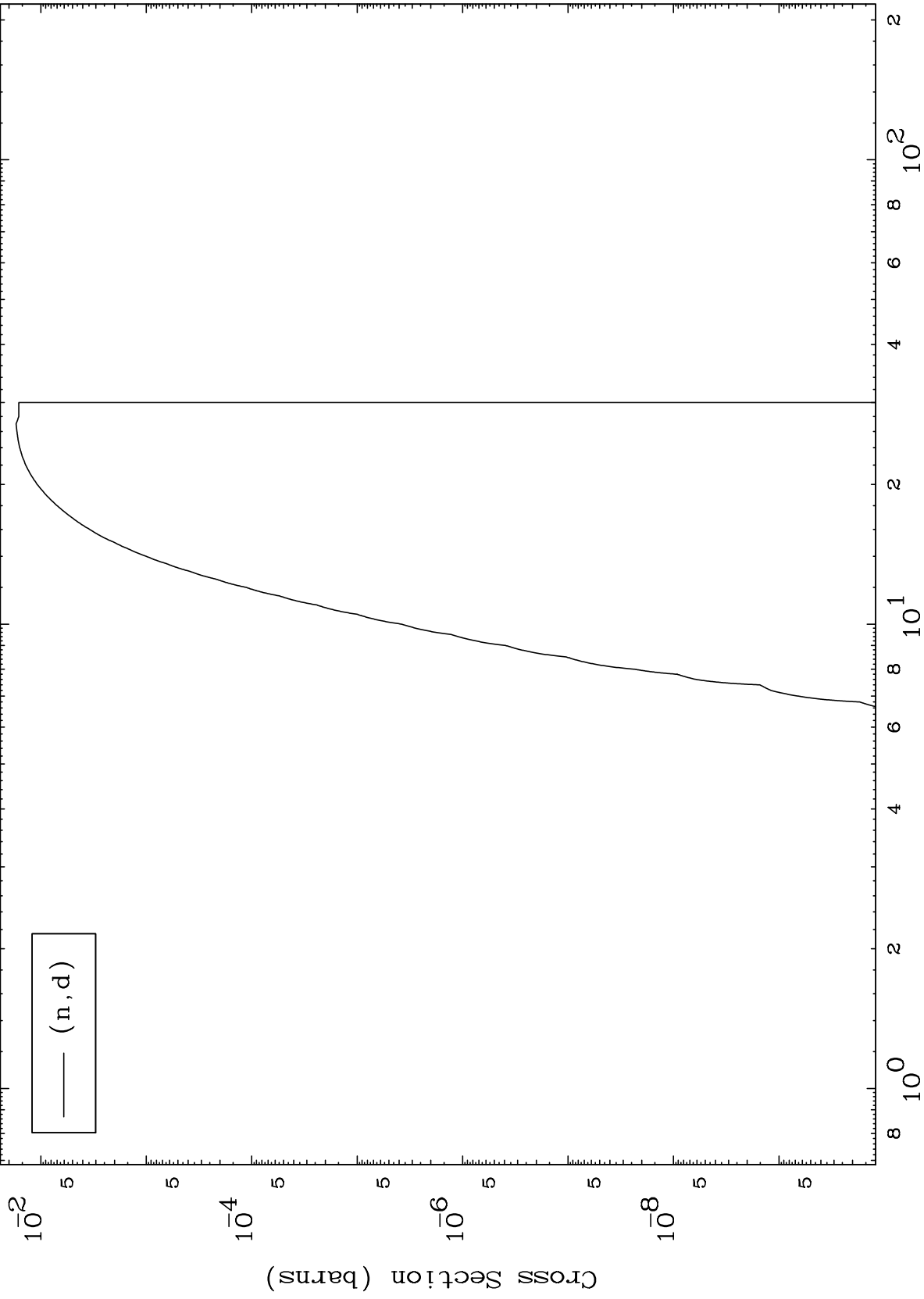
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(n,d) Levels
293 Kelvin Cross Sections

102-No-251

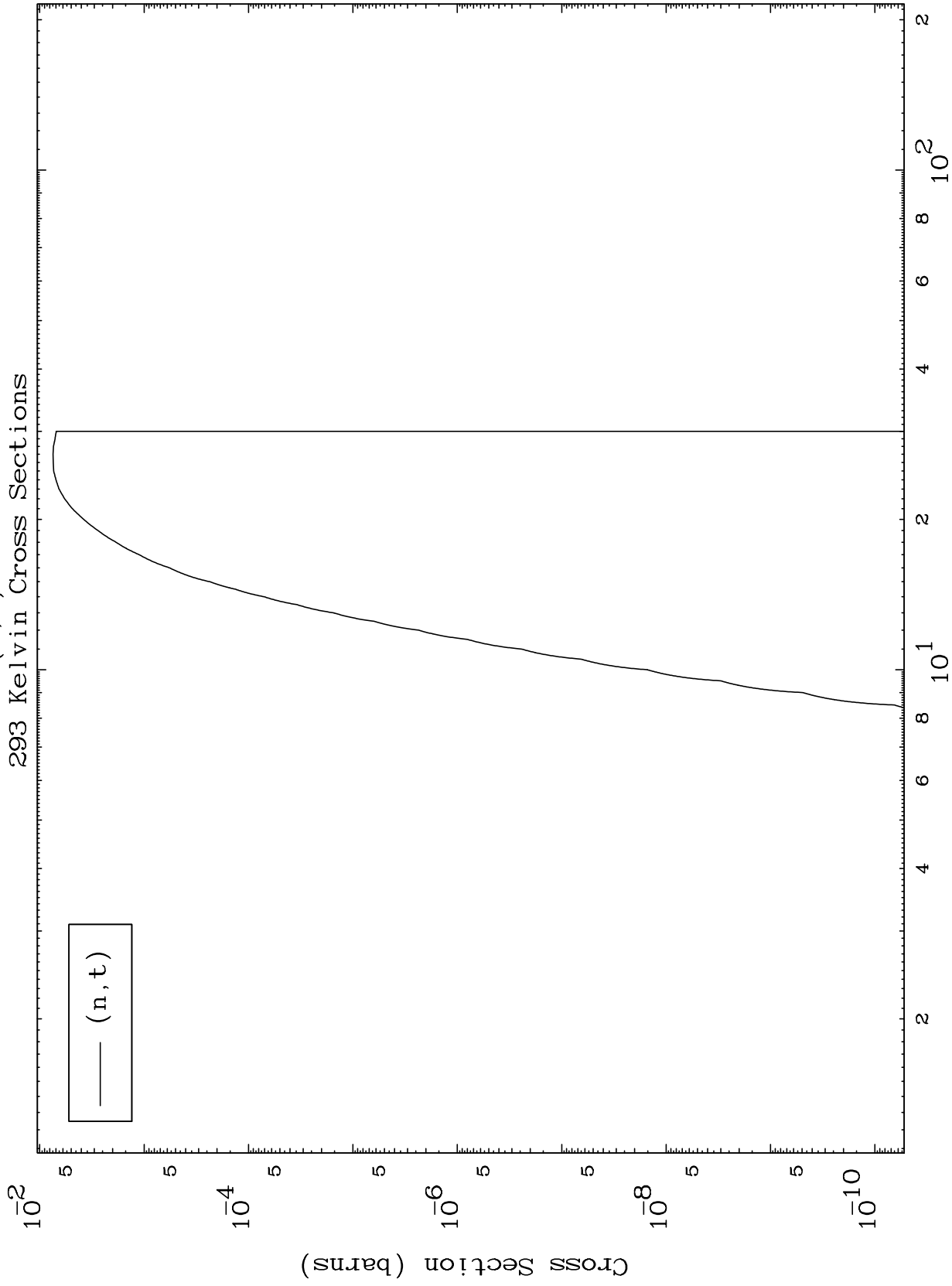


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MAT 9974

(n,t) Levels
293 Kelvin Cross Sections

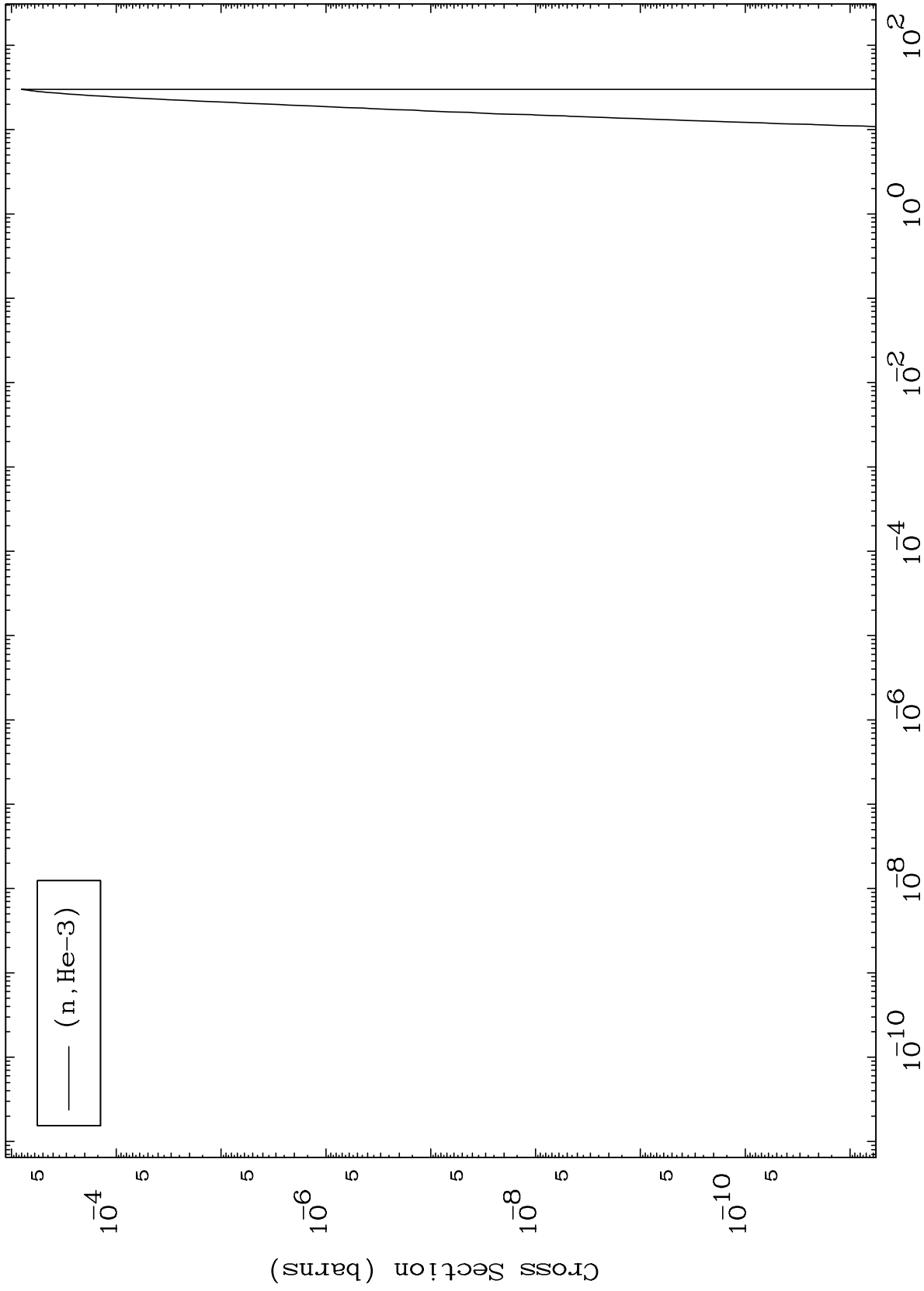
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MAT 9974

(n,He3) Levels
293 Kelvin Cross Sections

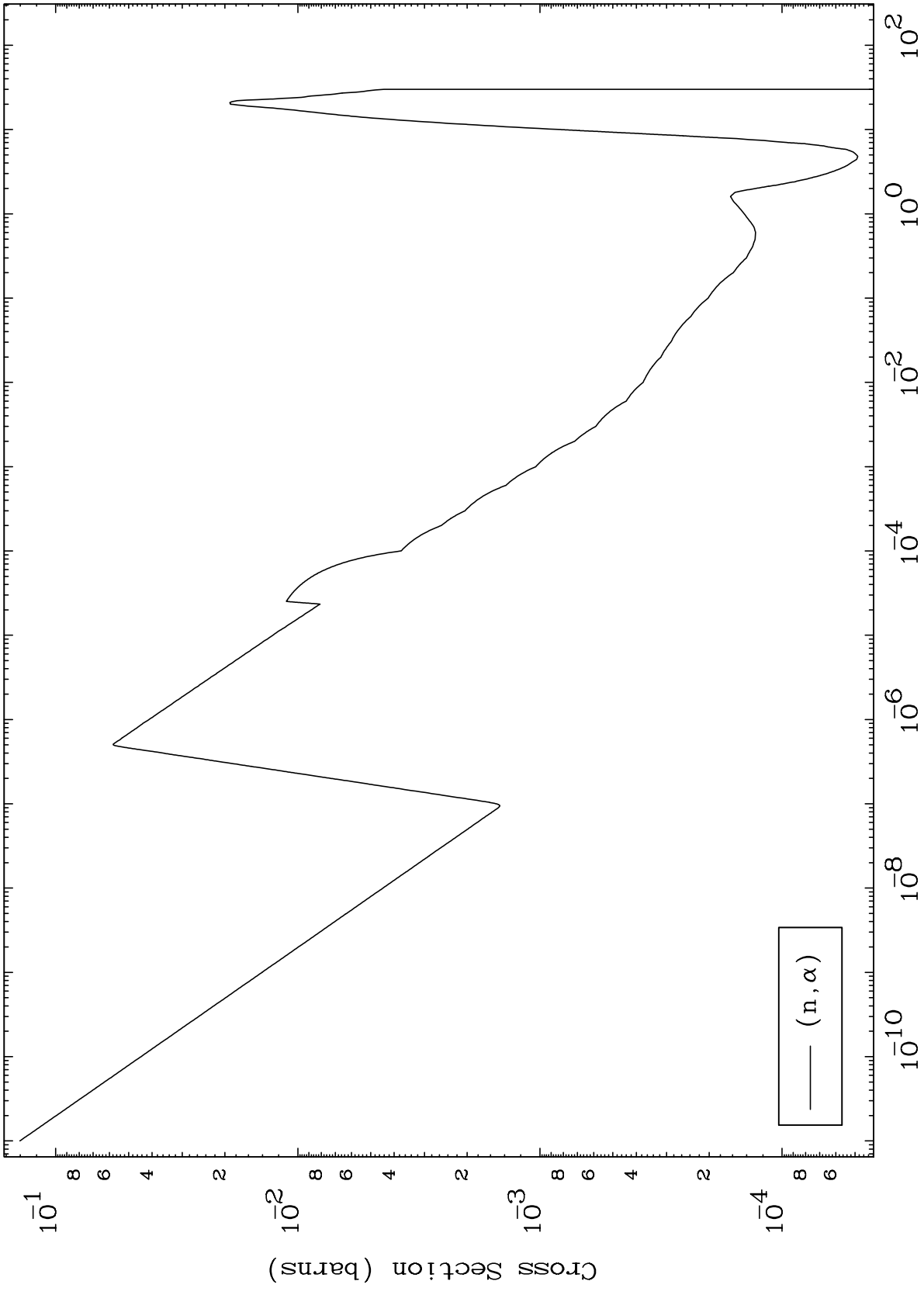
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(n,α) Levels
293 Kelvin Cross Sections

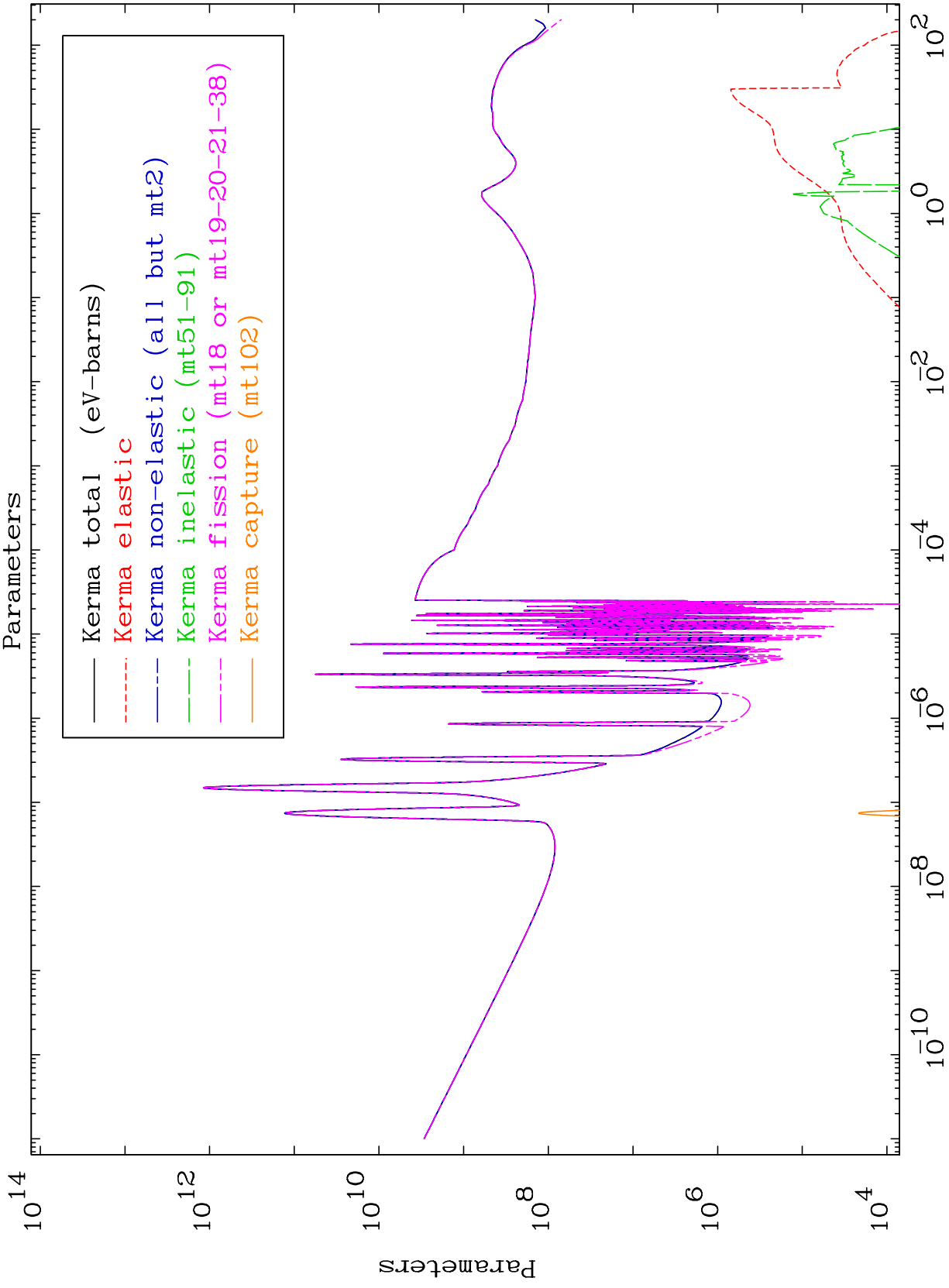
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Energy Release
Parameters

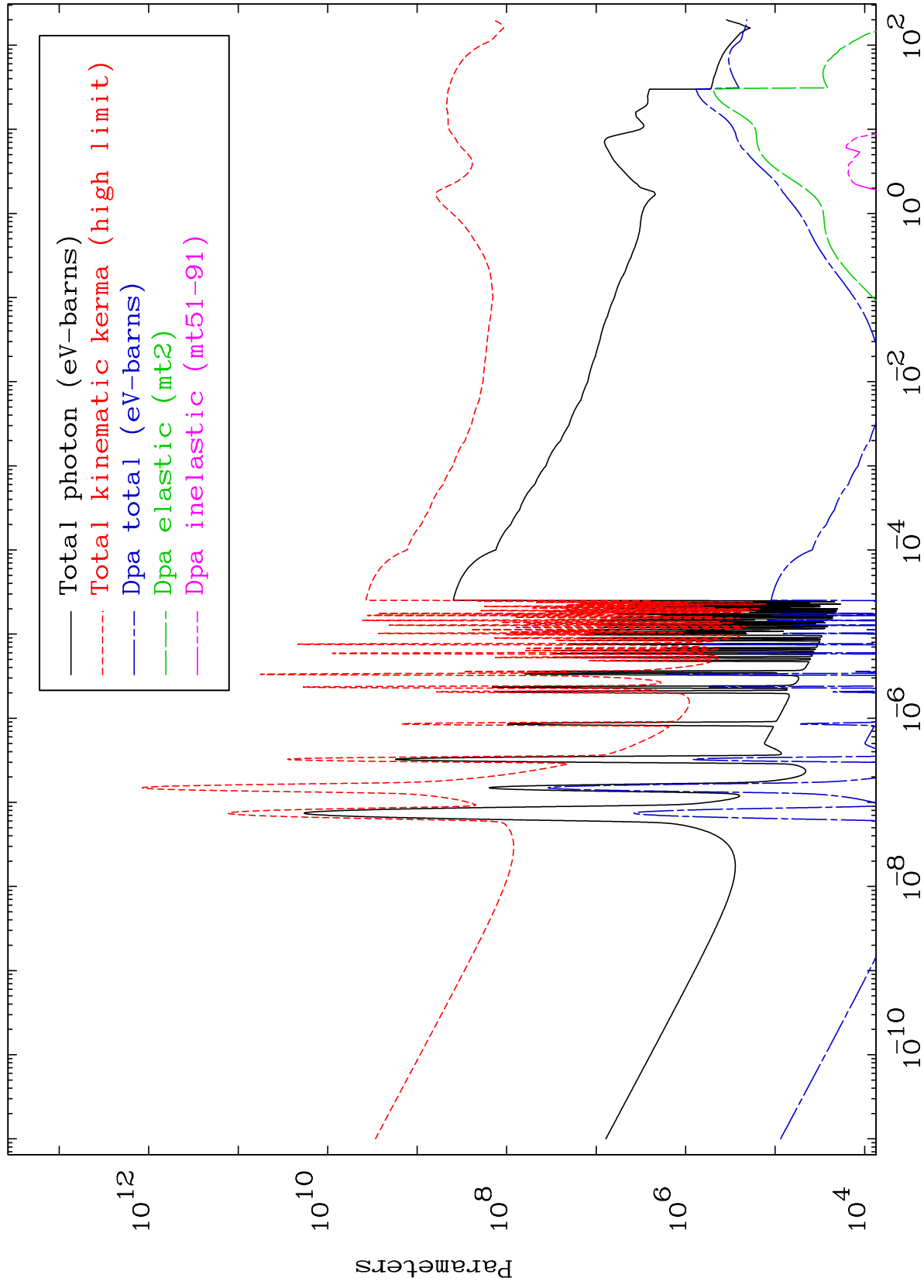
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Incident Energy (MeV)

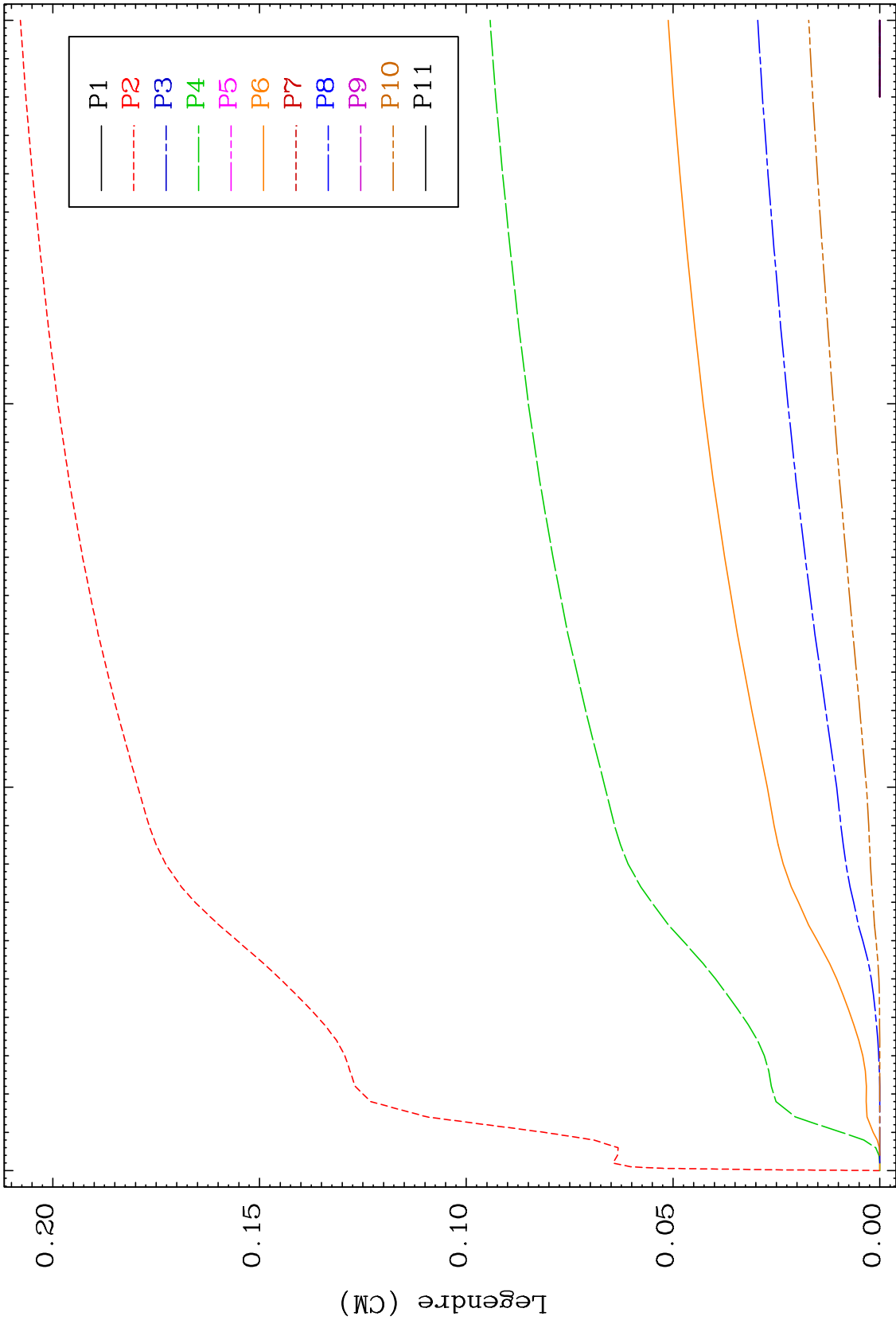
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Elastic Legendre Coefficients

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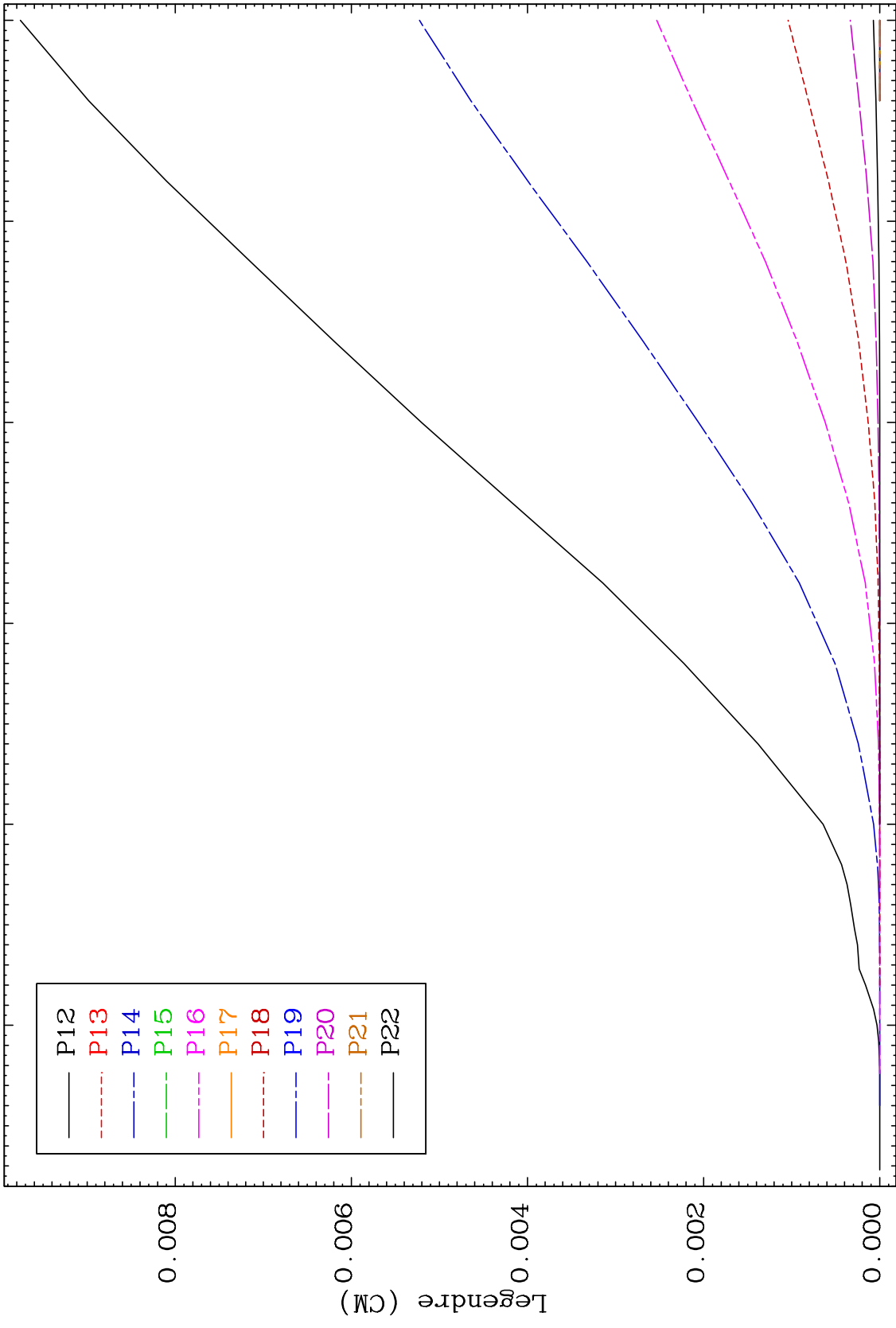
Incident Energy (MeV)

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MAT 9974

Elastic Legendre Coefficients

102-No-251



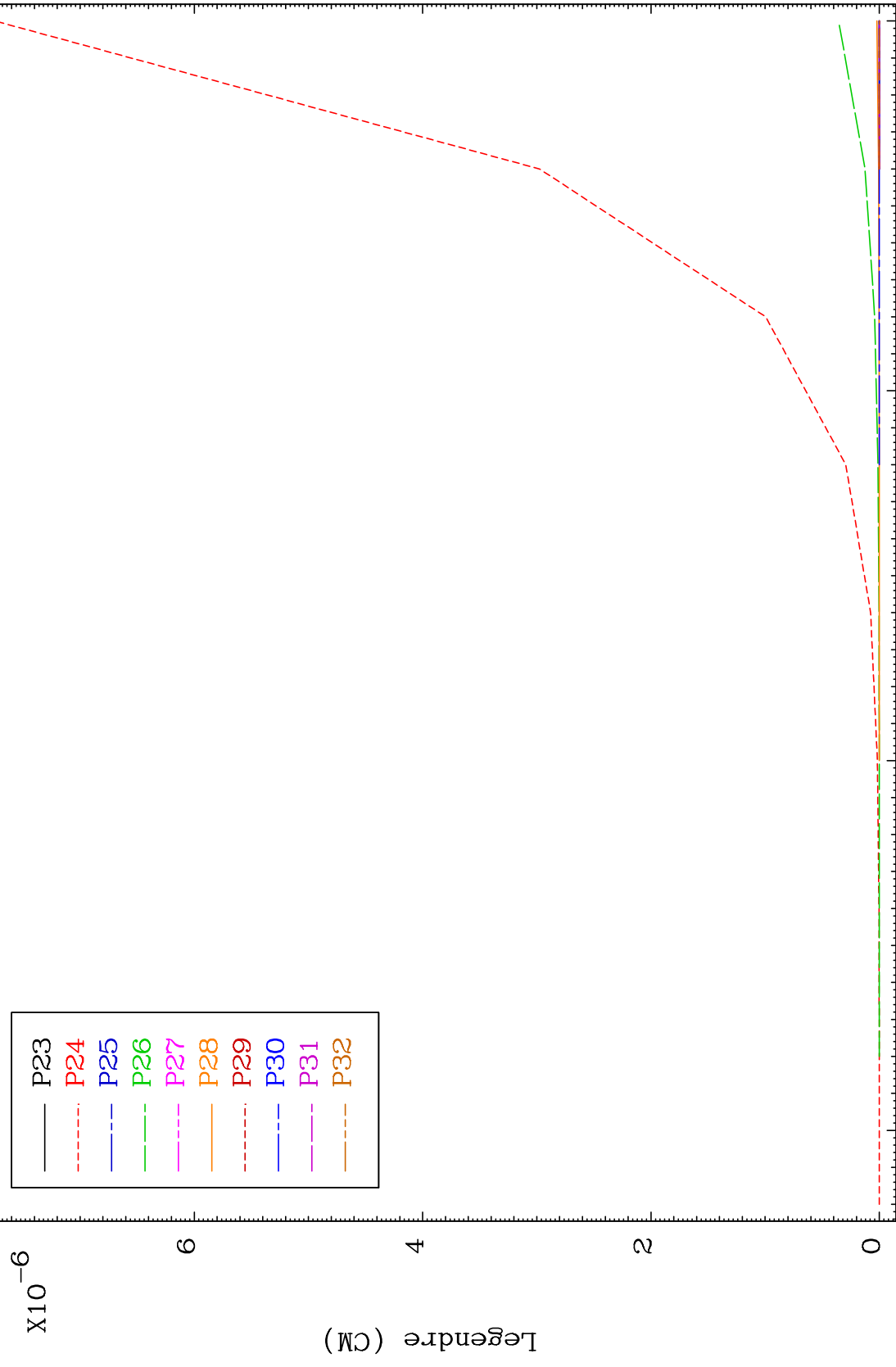
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Elastic Legendre Coefficients

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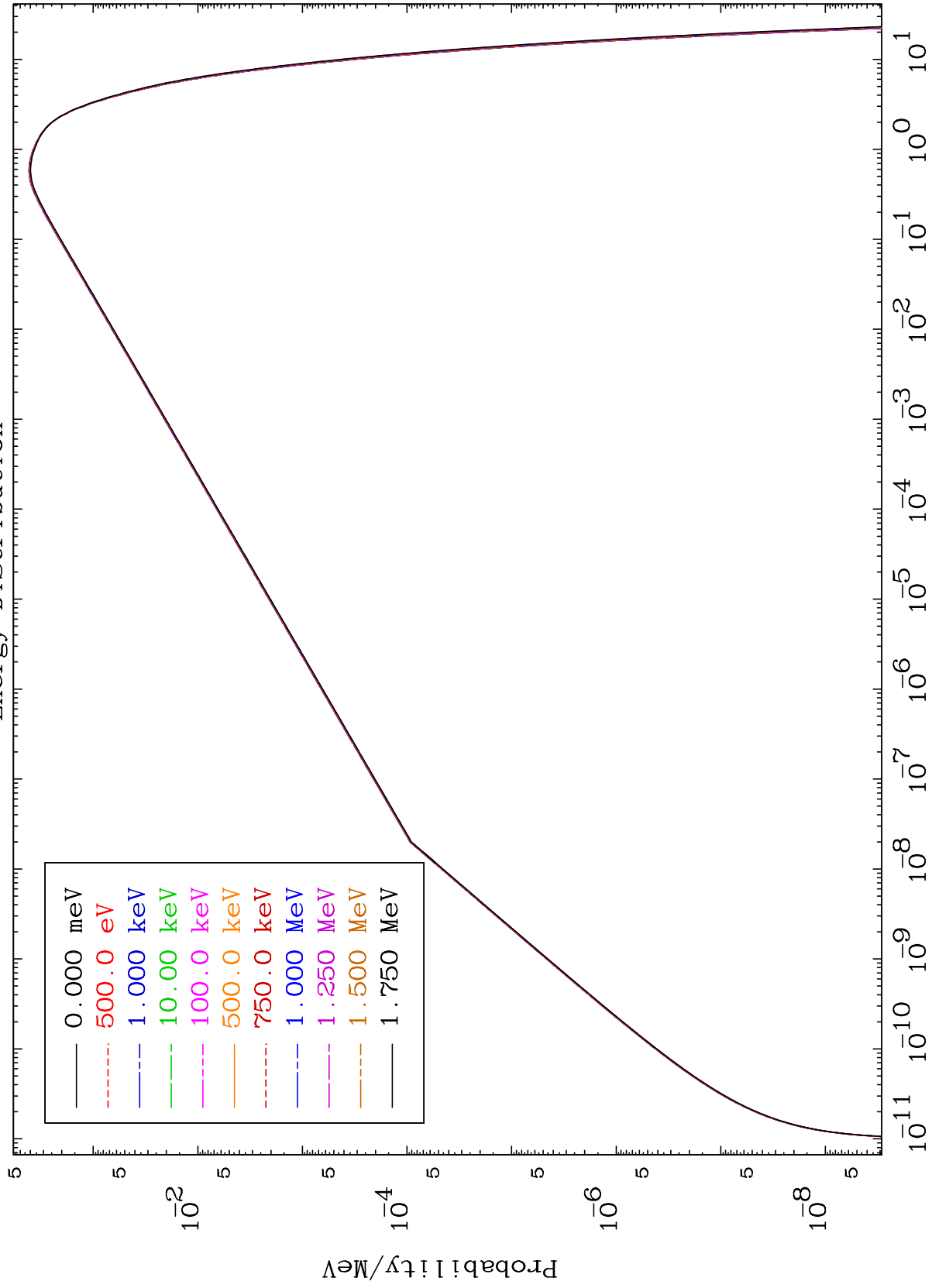
Incident Energy (MeV)

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Fission Energy Distribution

102-No-251

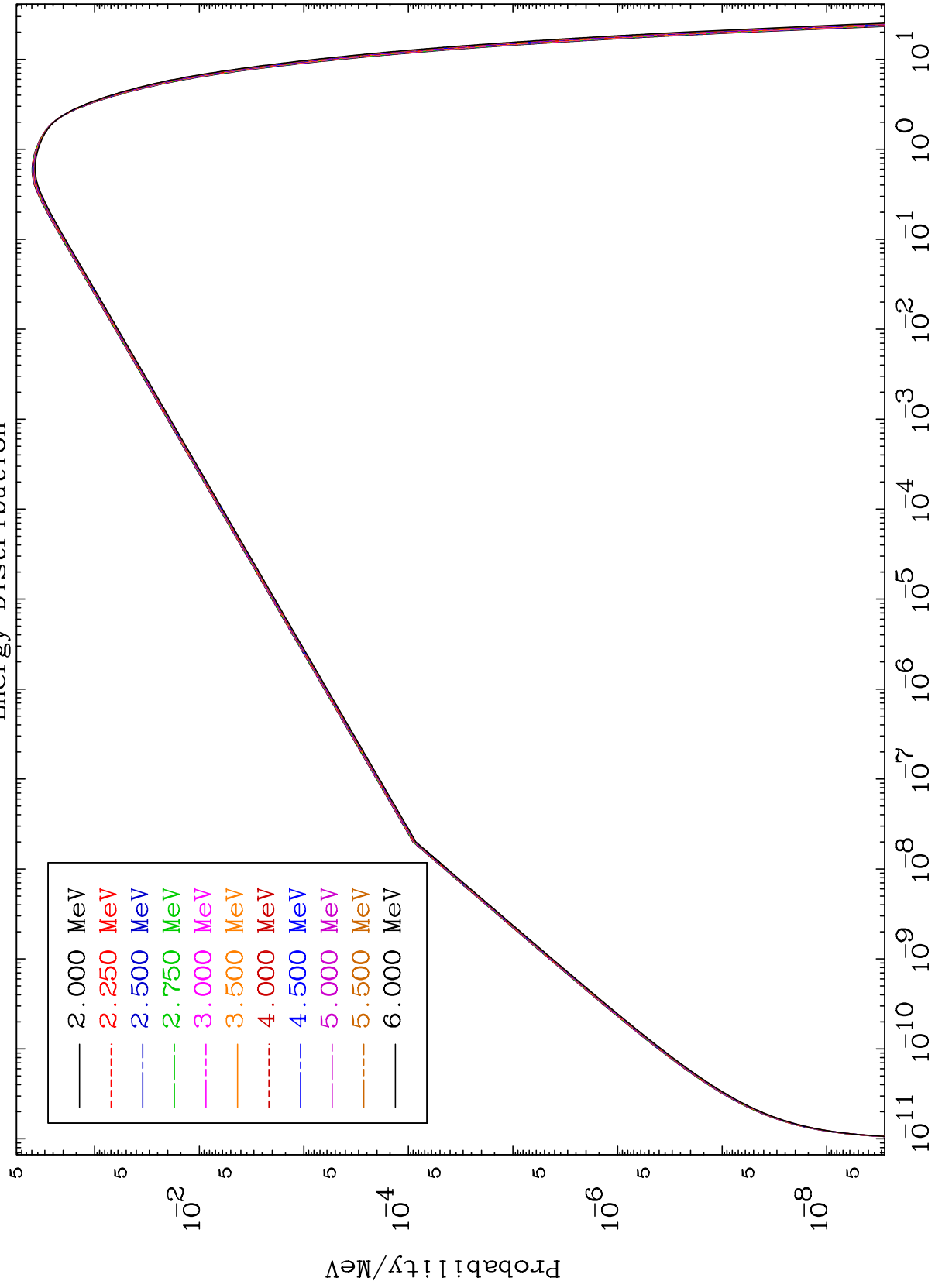


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Fission Energy Distribution

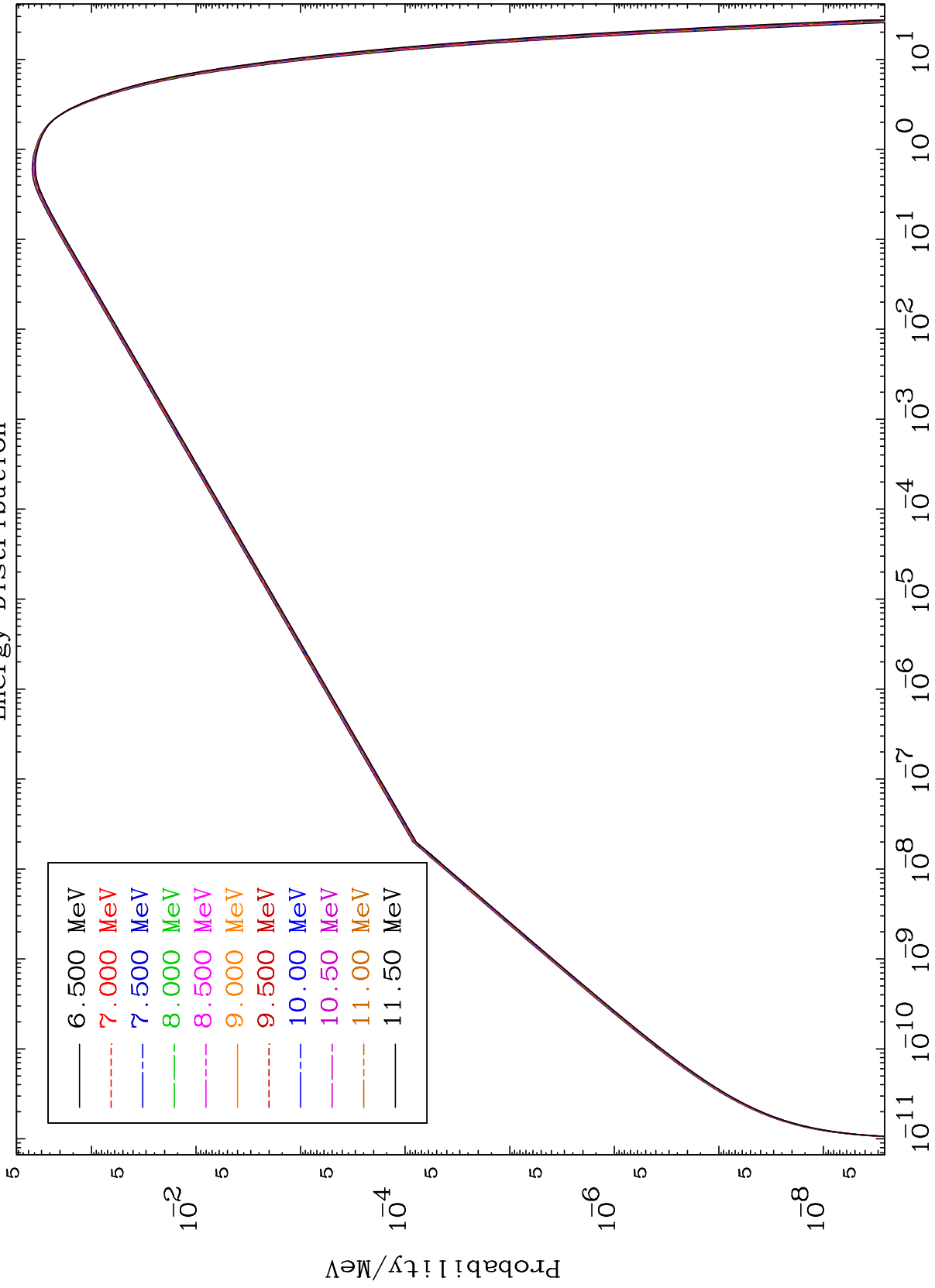
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Fission Energy Distribution

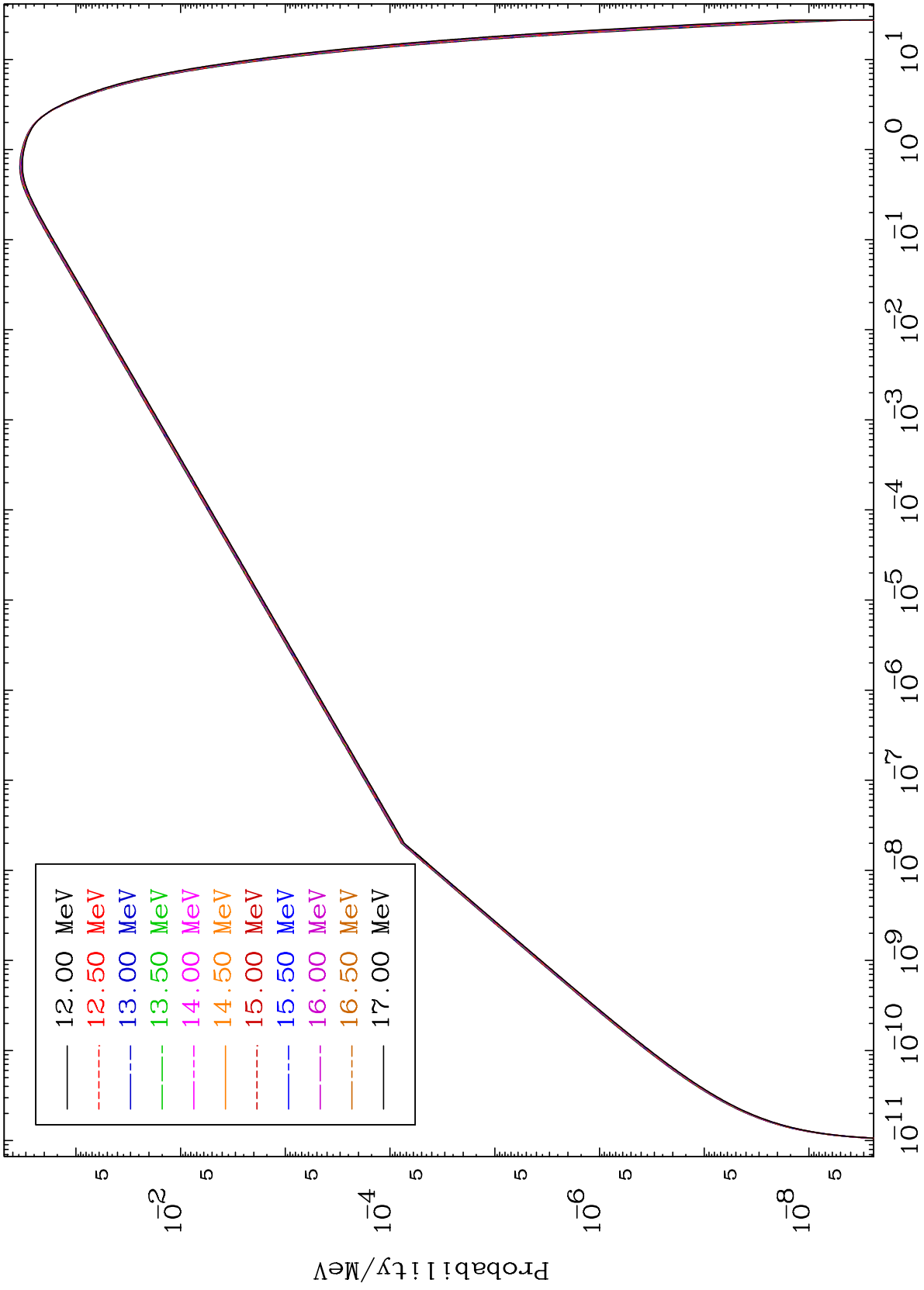
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Fission Energy Distribution

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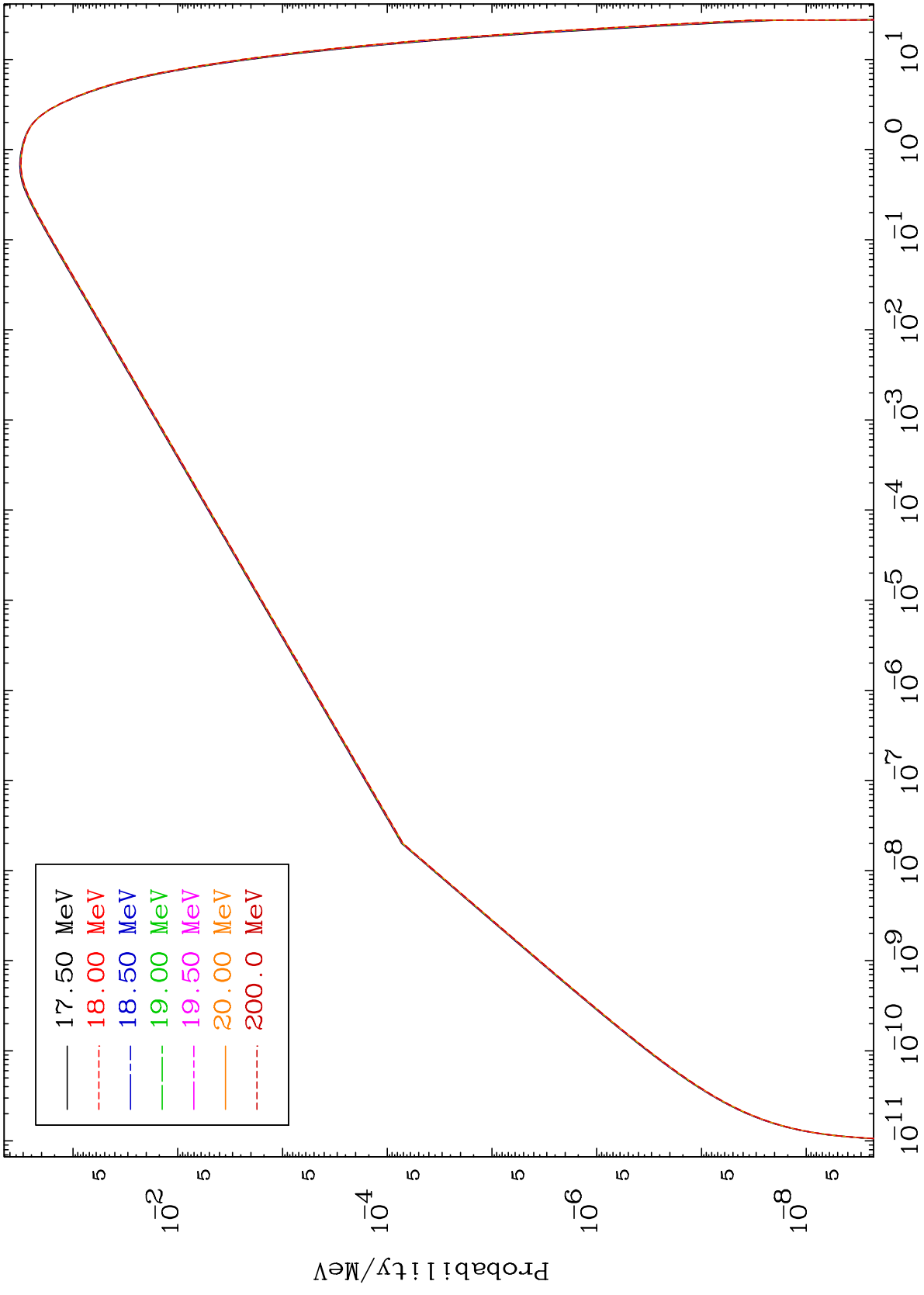


102-No-251

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Fission Energy Distribution

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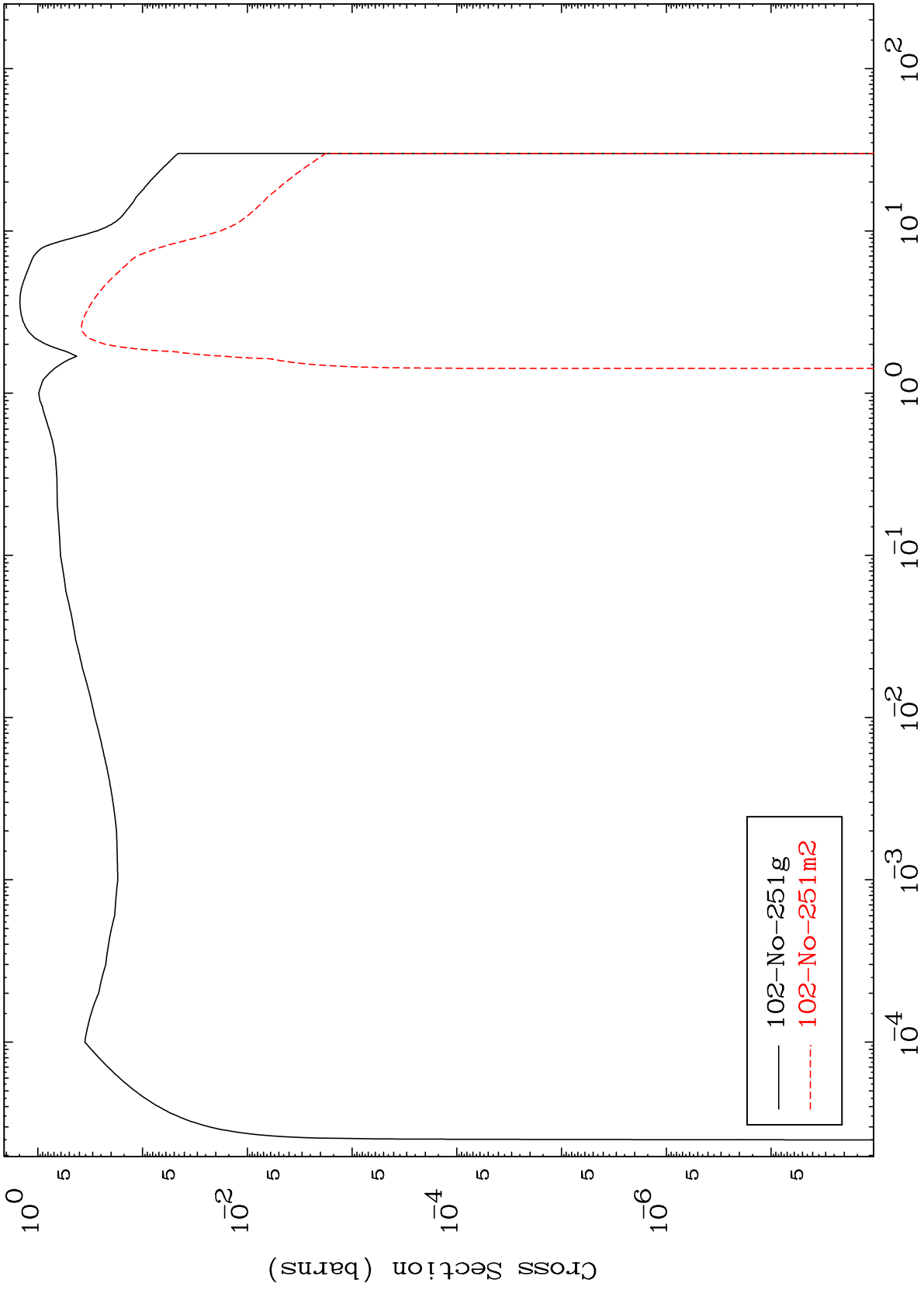


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Inelastic
Radionuclide Production Cross Section

102-No-251

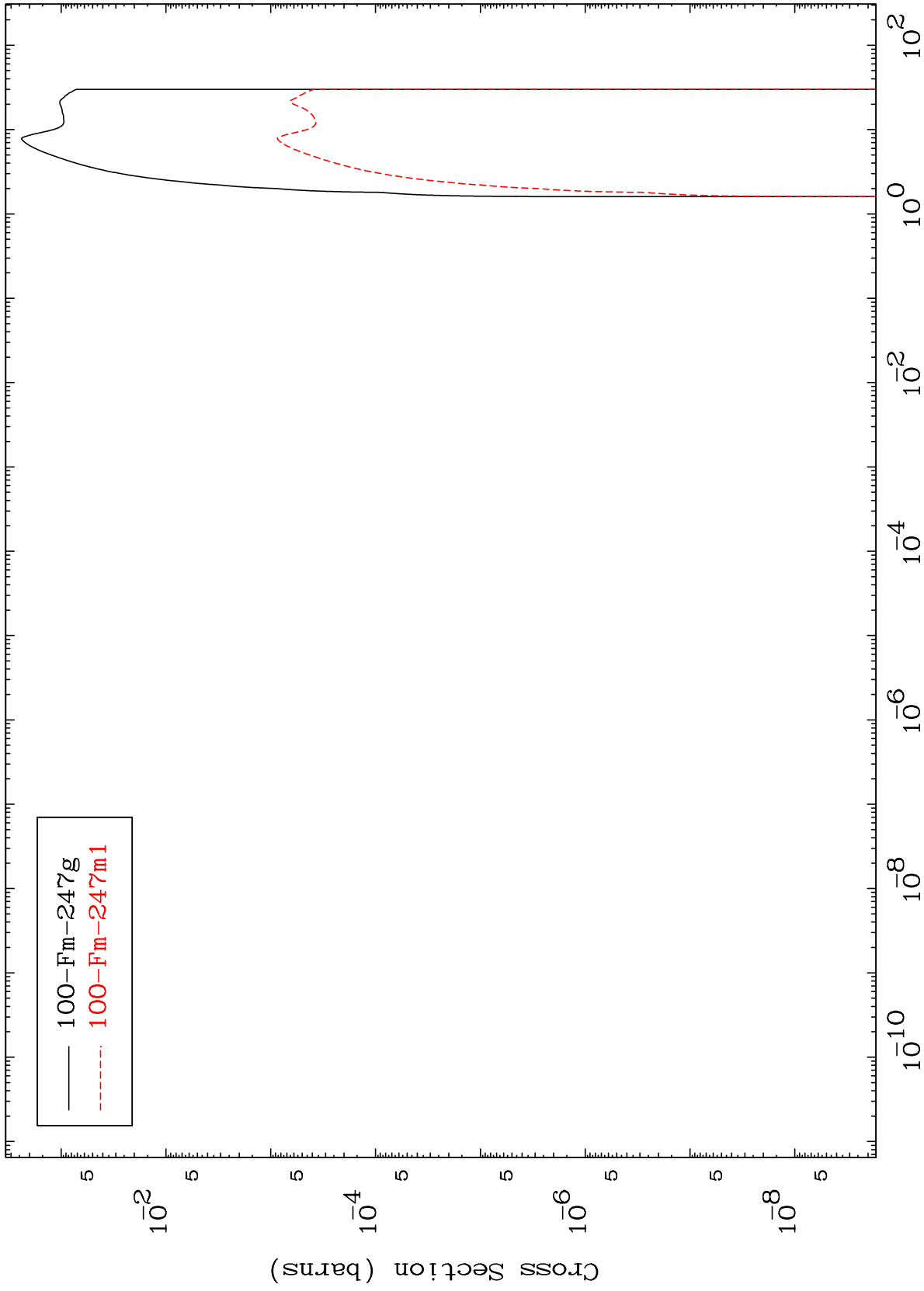


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(n, n') α

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Radionuclide Production Cross Section

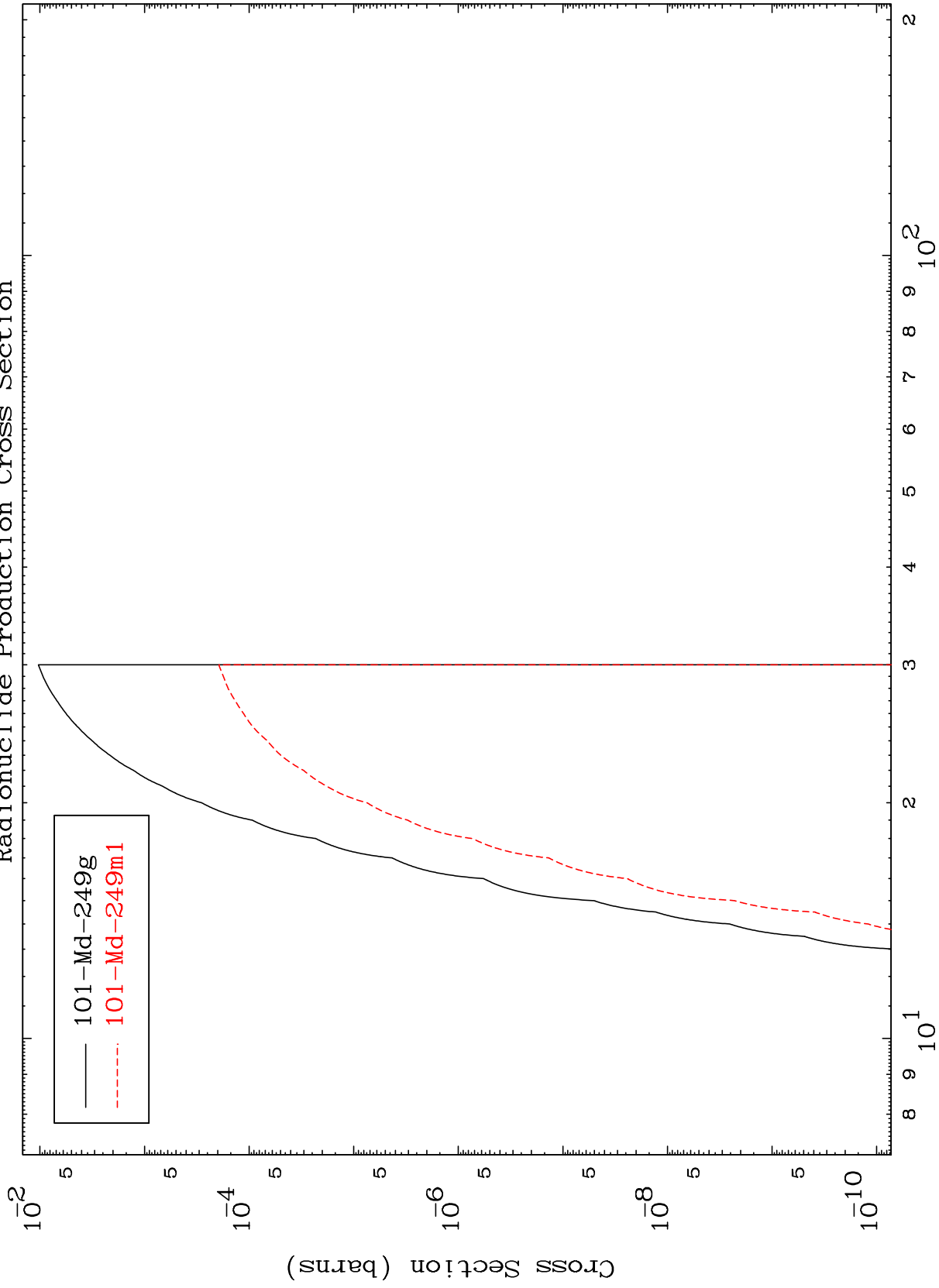


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(n,n') d

102-No-251

Radionuclide Production Cross Section



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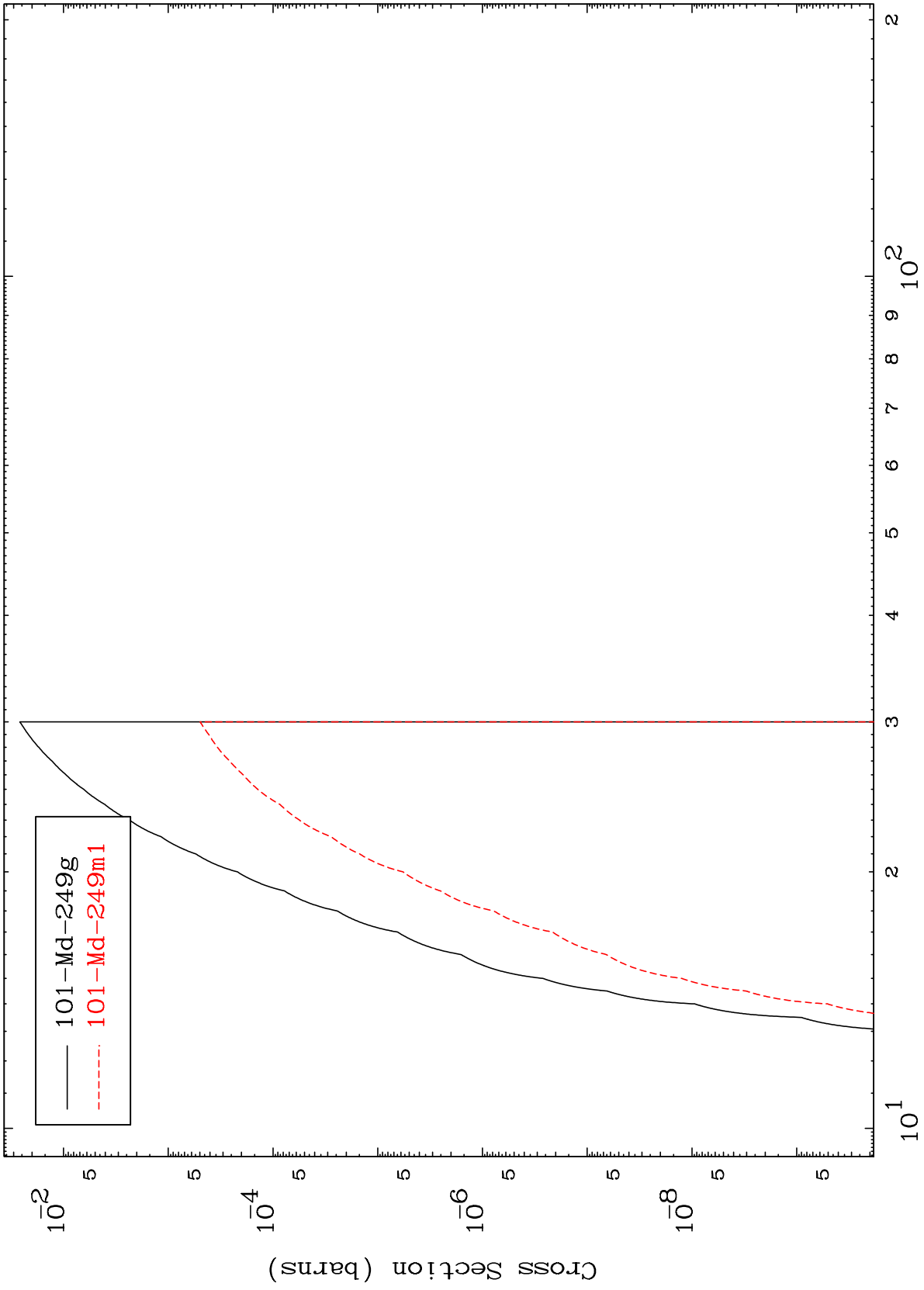
102-No-251

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(n,2n) p

102-No-251

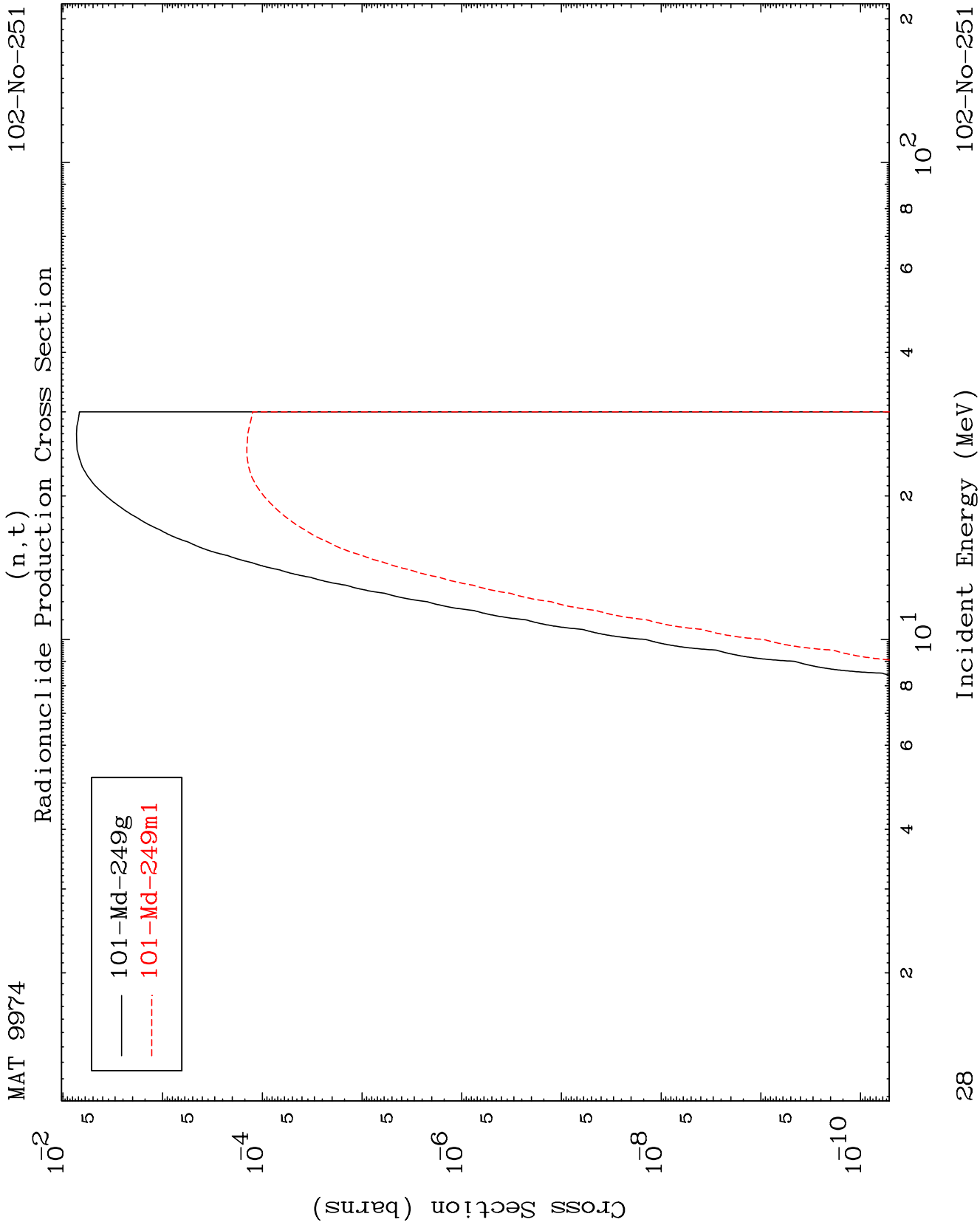
Radionuclide Production Cross Section



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Incident Energy (MeV)

102-No-251

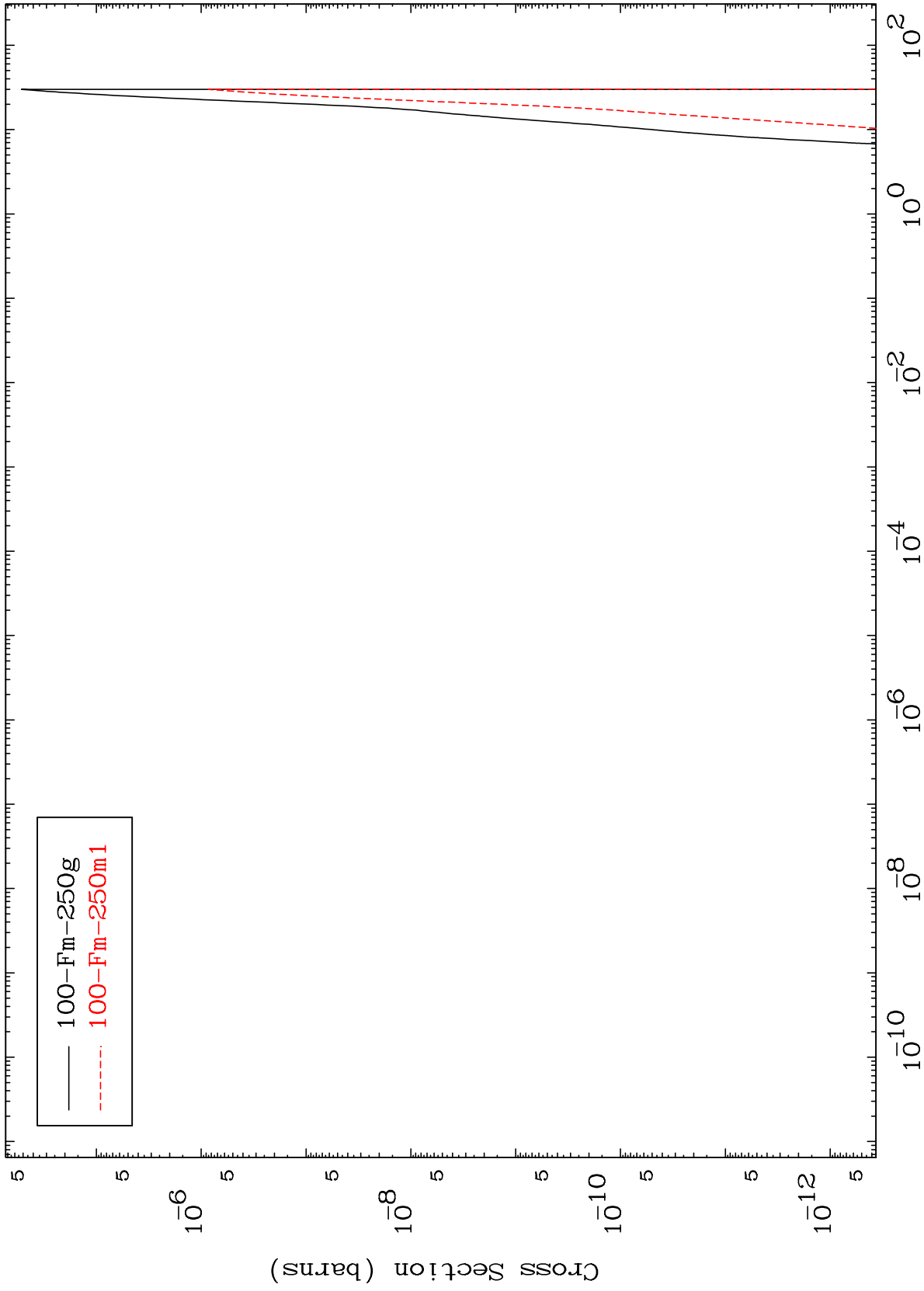


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(n,2p)

102-No-251

Radionuclide Production Cross Section



100-Fm-250g
100-Fm-250m1