

Program EVALPLOT
(Version 2018-1)

by

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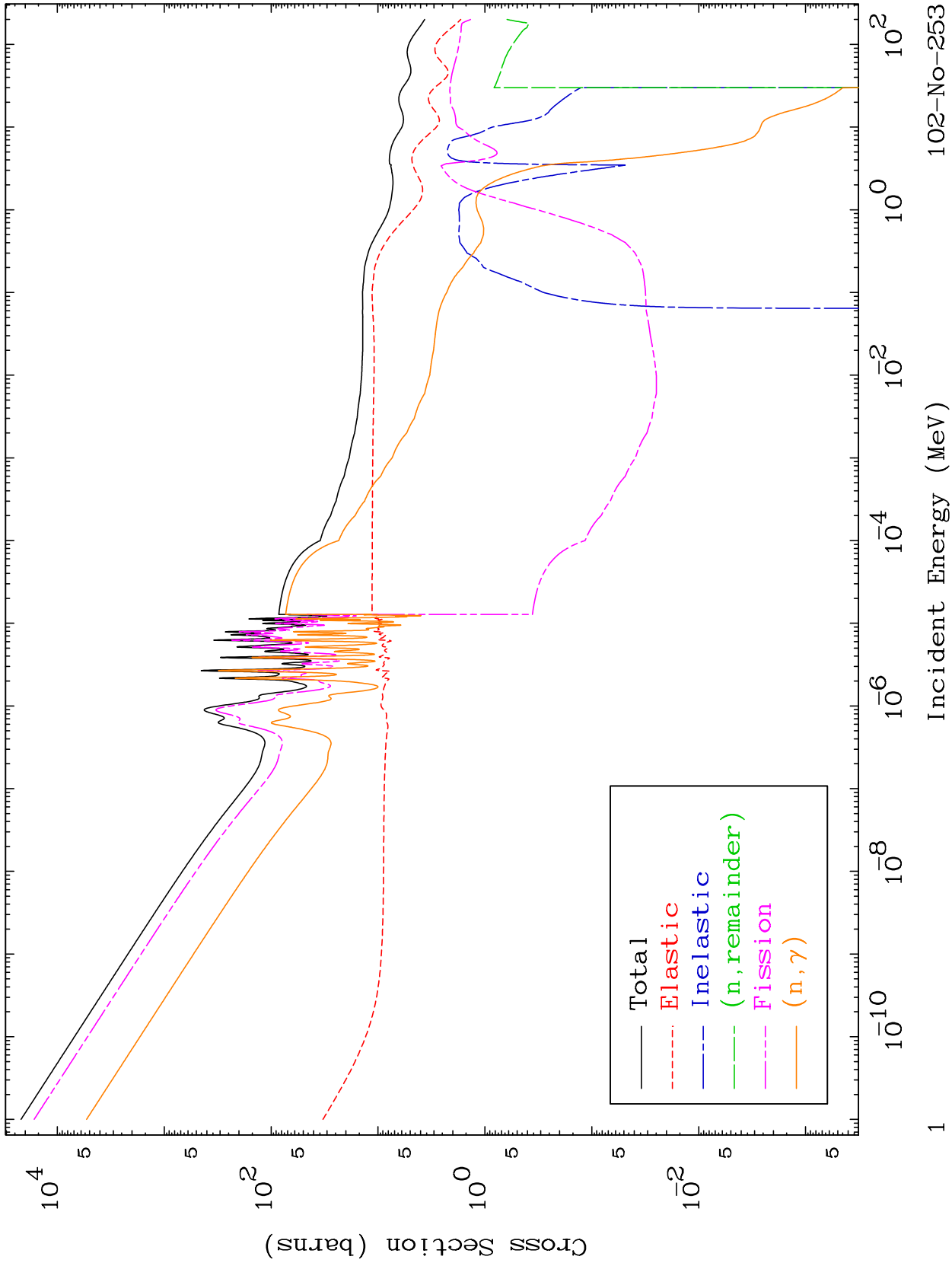
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9971

Major
293 Kelvin Cross Sections

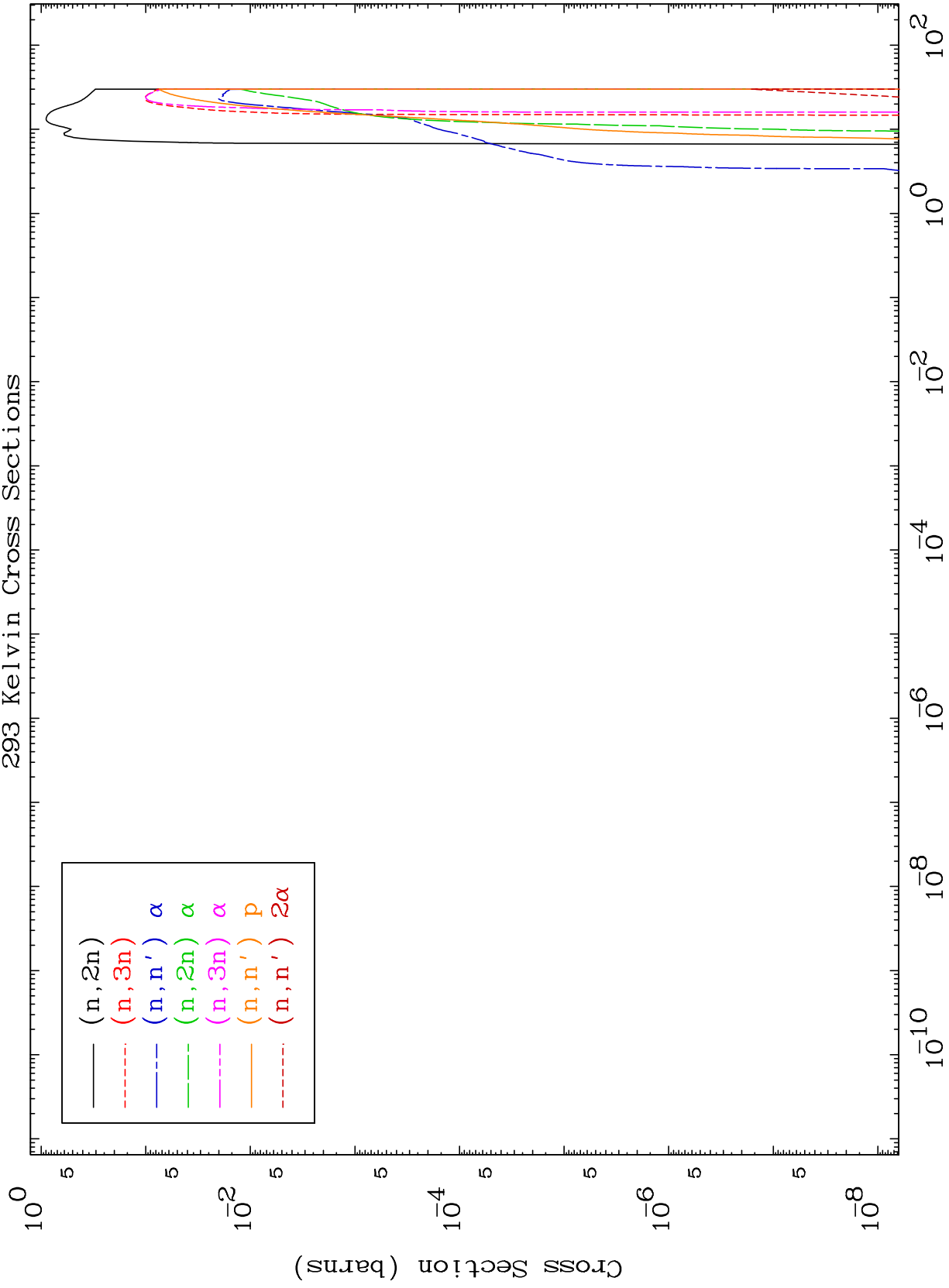
102-No-253



MAT 9971

Neutron Production
293 Kelvin Cross Sections

102-No-253



2

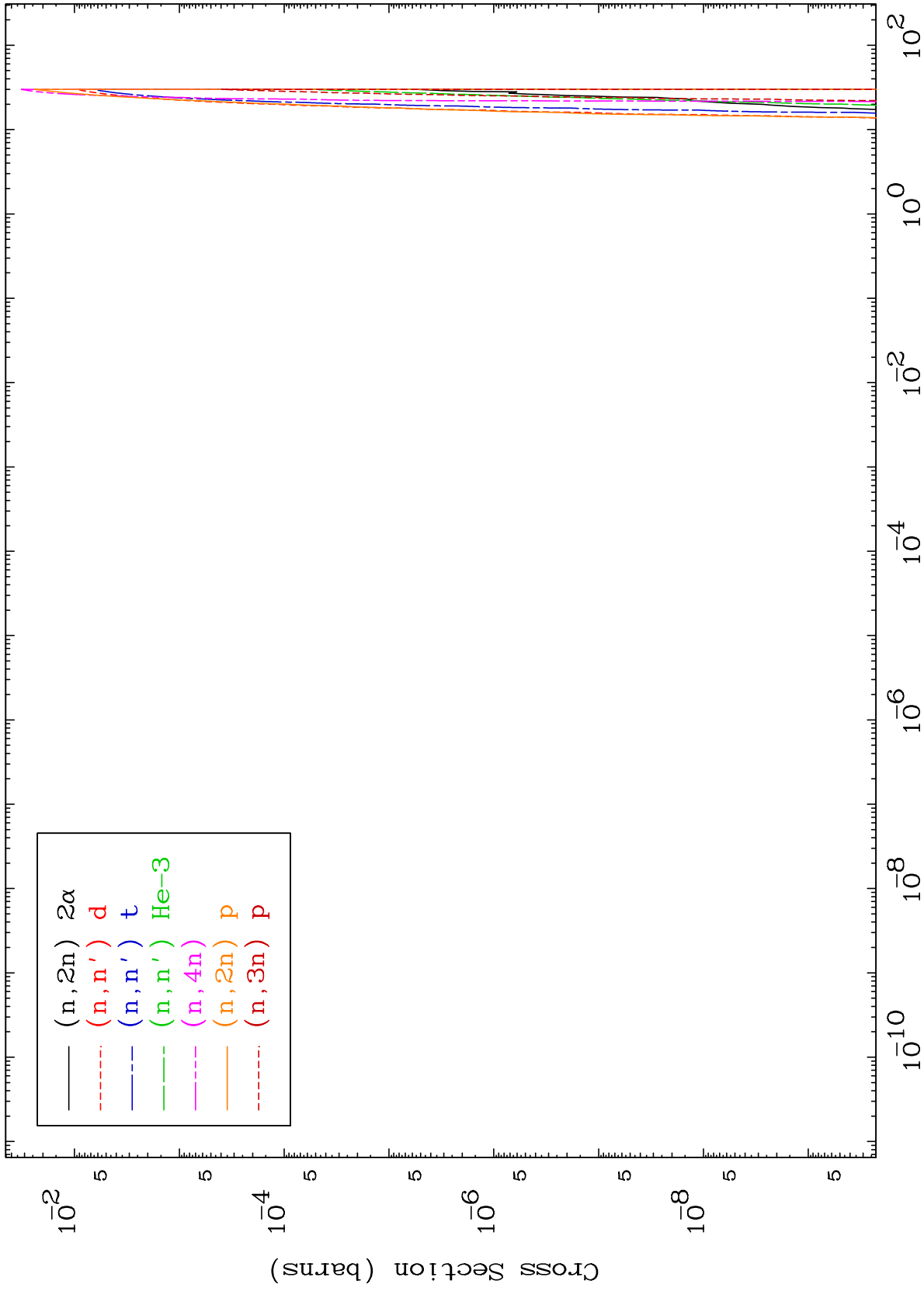
Incident Energy (MeV)

102-No-253

MAT 9971

Neutron Production
293 Kelvin Cross Sections

102-No-253



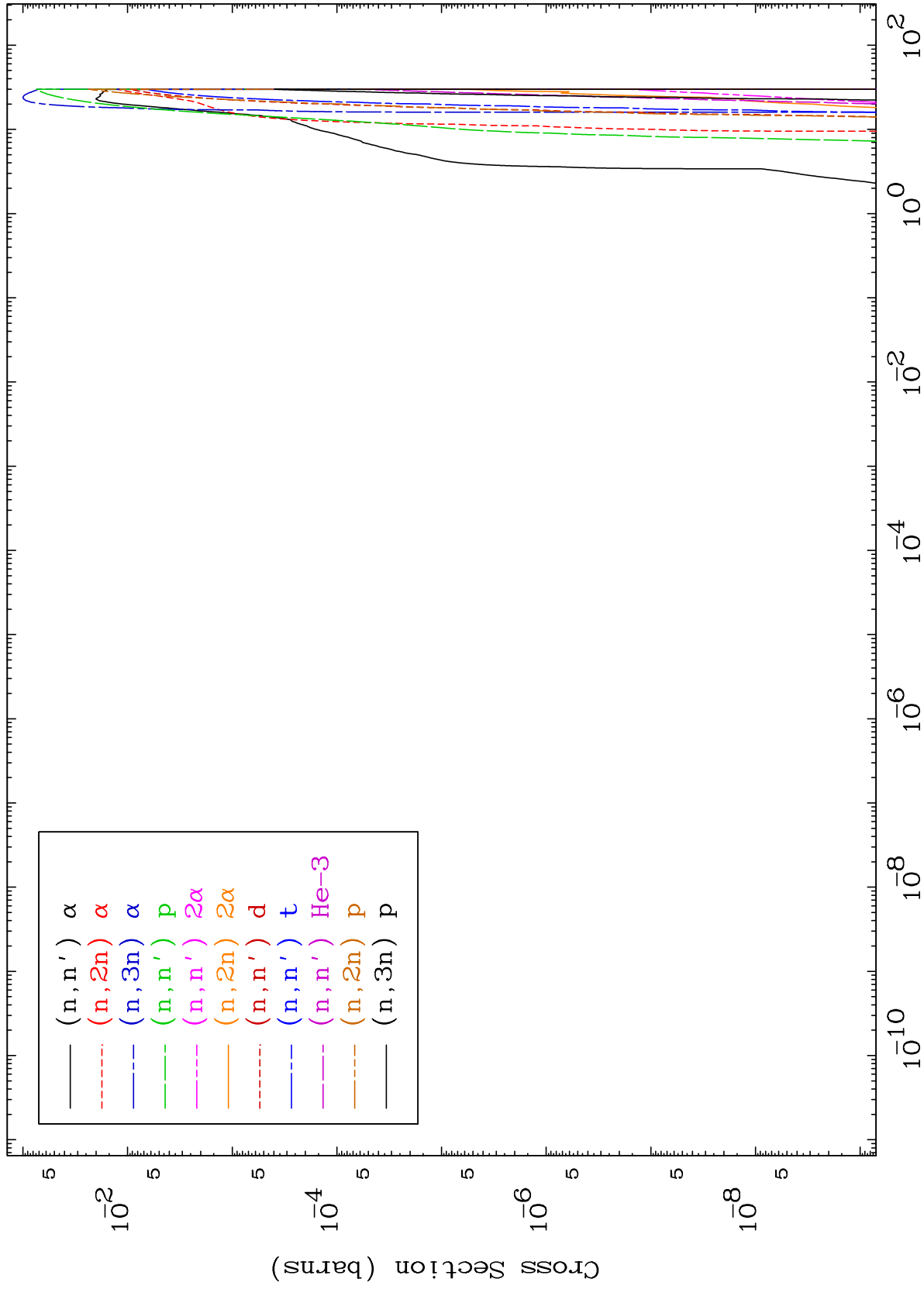
3

102-No-253

MAT 9971

Charged Particle
293 Kelvin Cross Sections

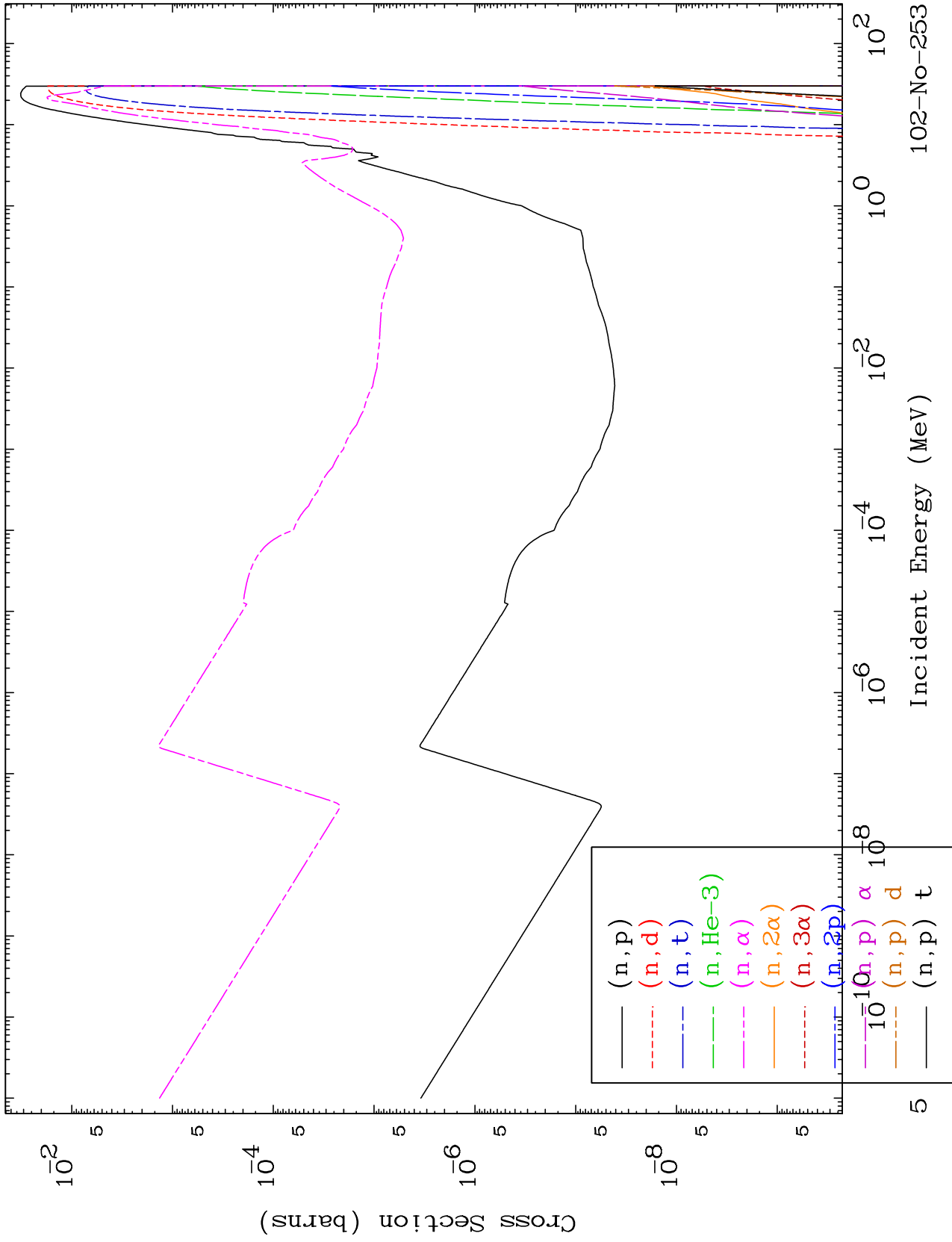
102-No-253



MAT 9971

293 Charged Particle
Kelvin Cross Sections

102-No-253

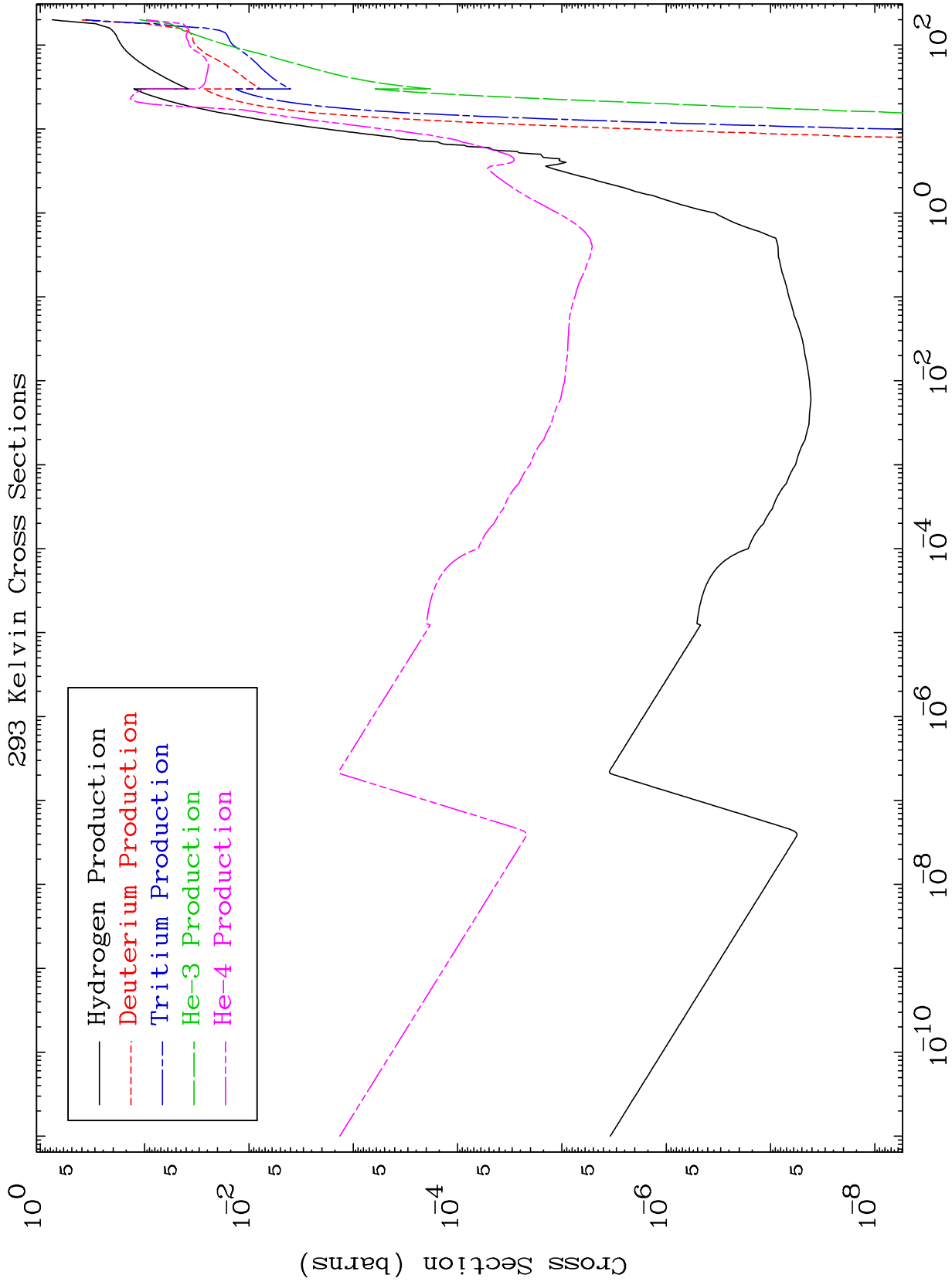


102-No-253

MAT 9971

293 Kelvin Production
Particle Production Cross Sections

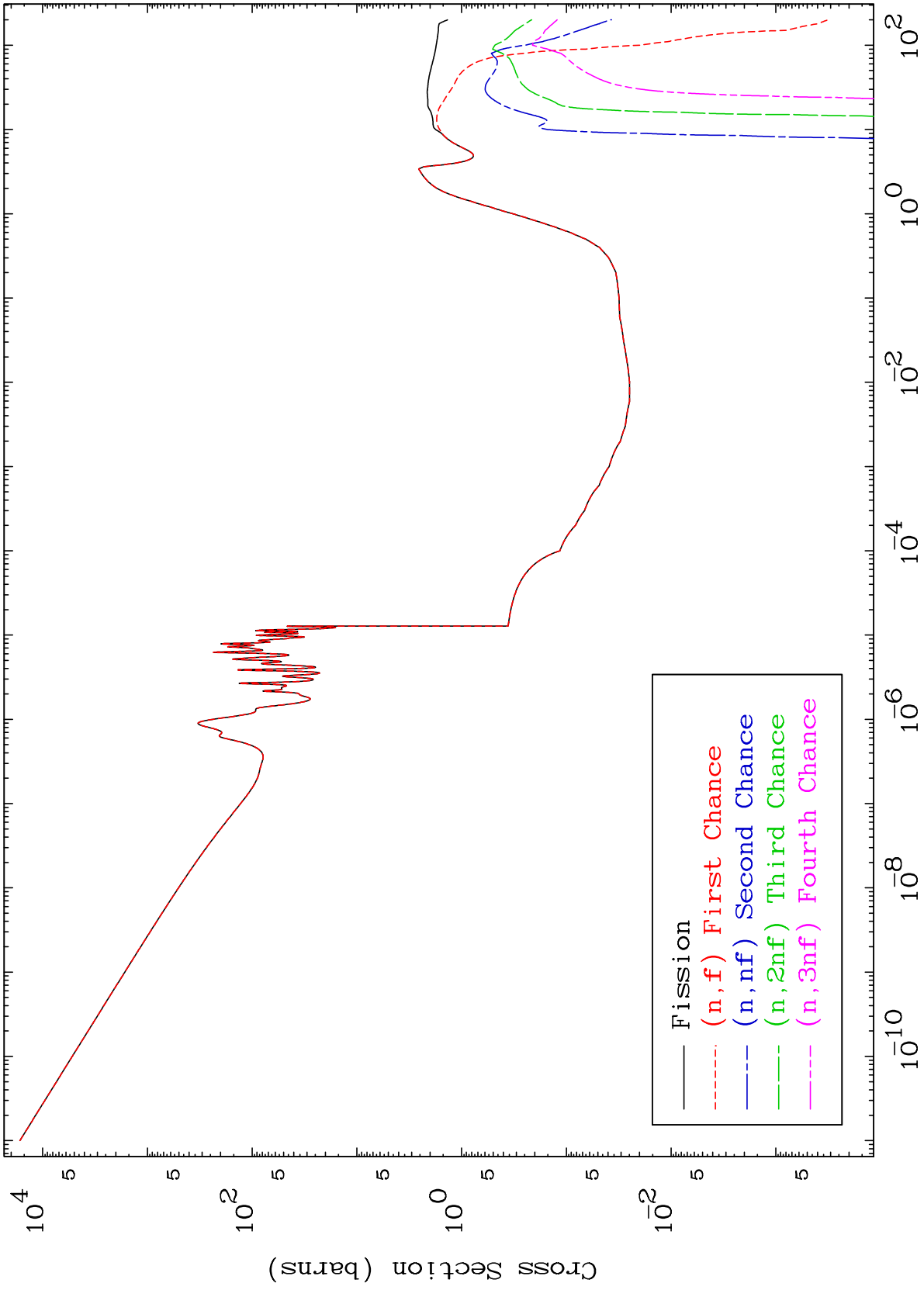
102-No-253



MAT 9971

Fission
293 Kelvin Cross Sections

102-No-253

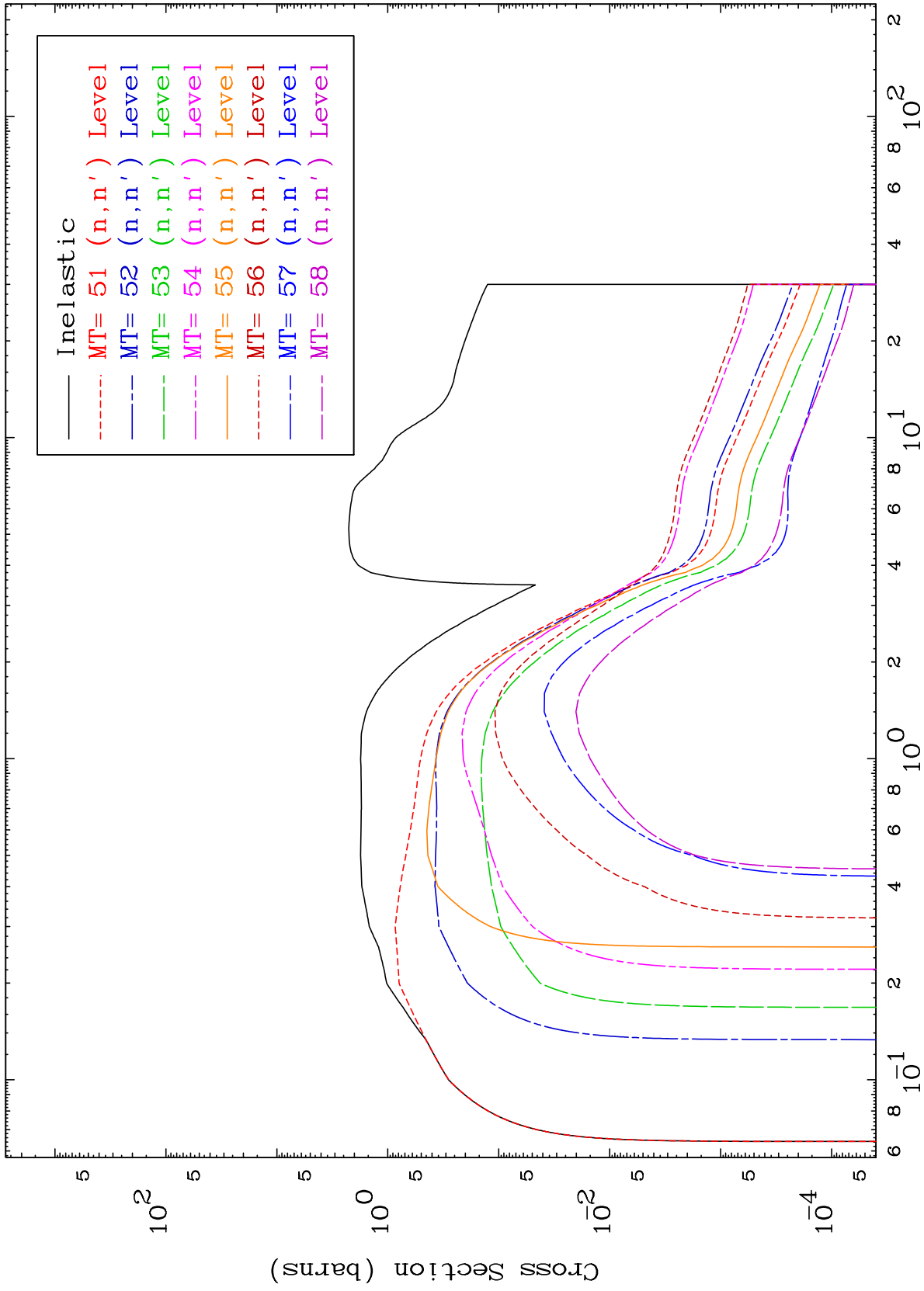


MAT 9971

(n,n') Level

102-No-253

293 Kelvin Cross Sections



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Incident Energy (MeV)

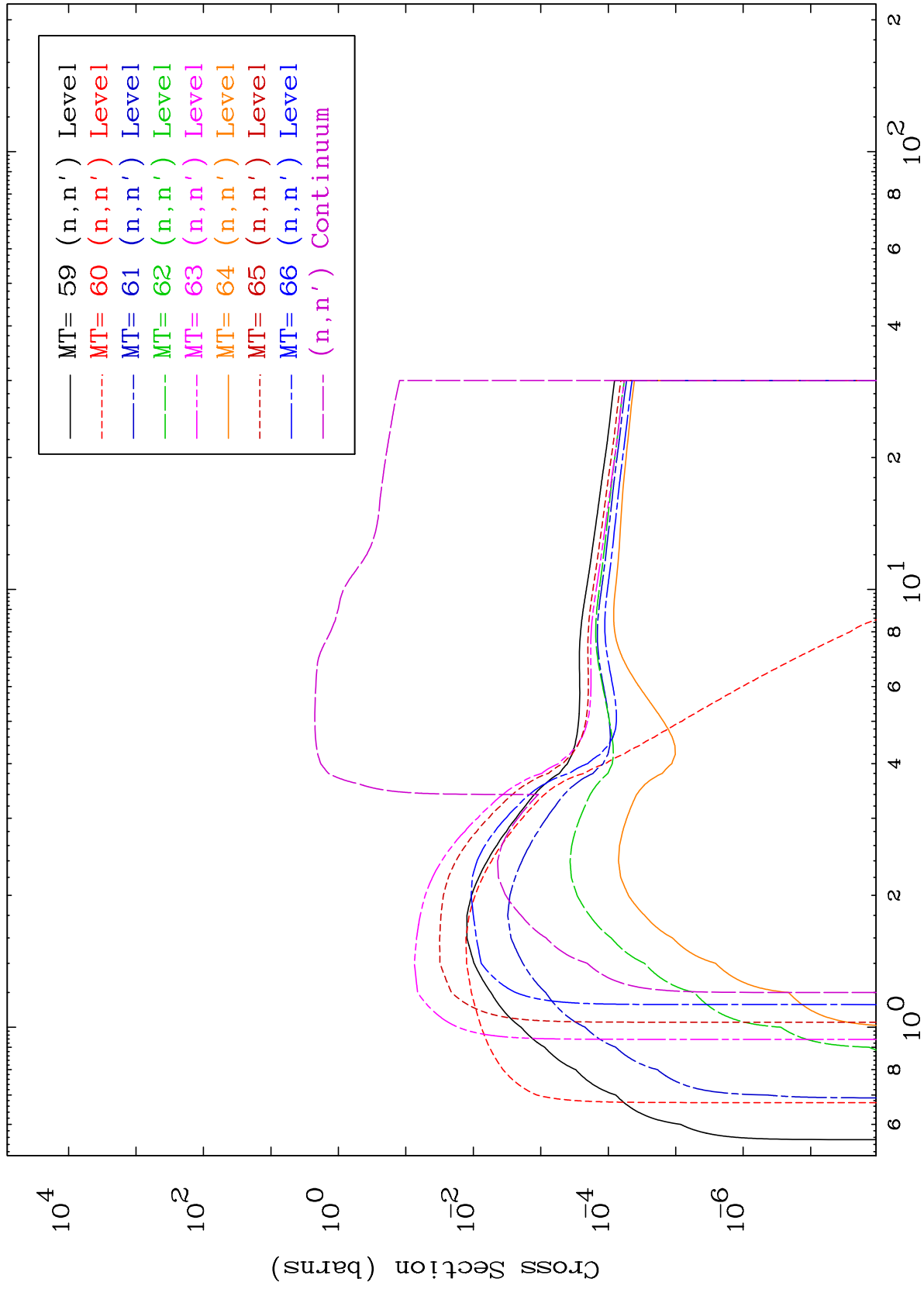
102-No-253

MAT 9971

(n,n') Level

102-No-253

293 Kelvin Cross Sections



9

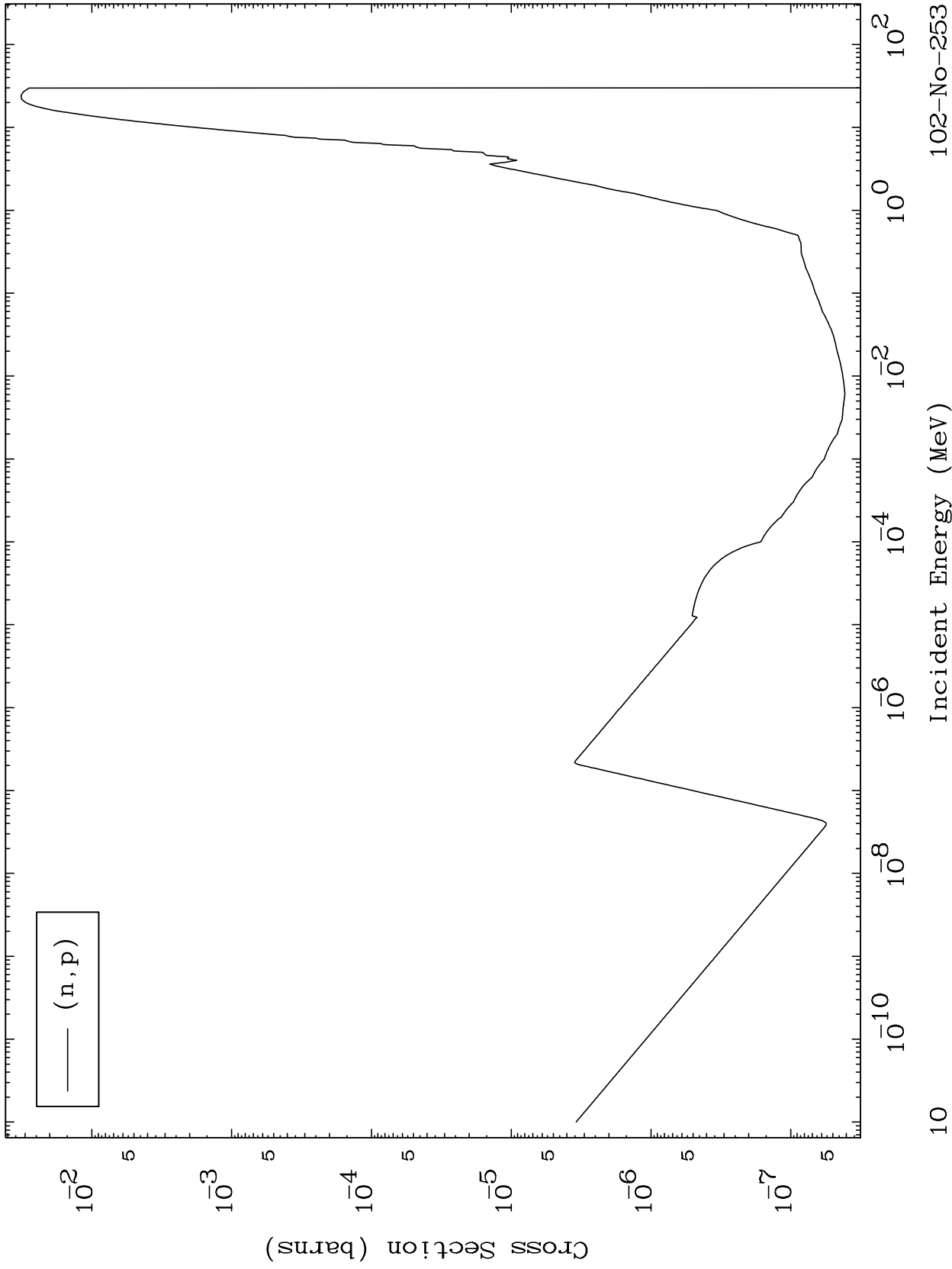
Incident Energy (MeV)

102-No-253

MAT 9971

(n,p) Levels
293 Kelvin Cross Sections

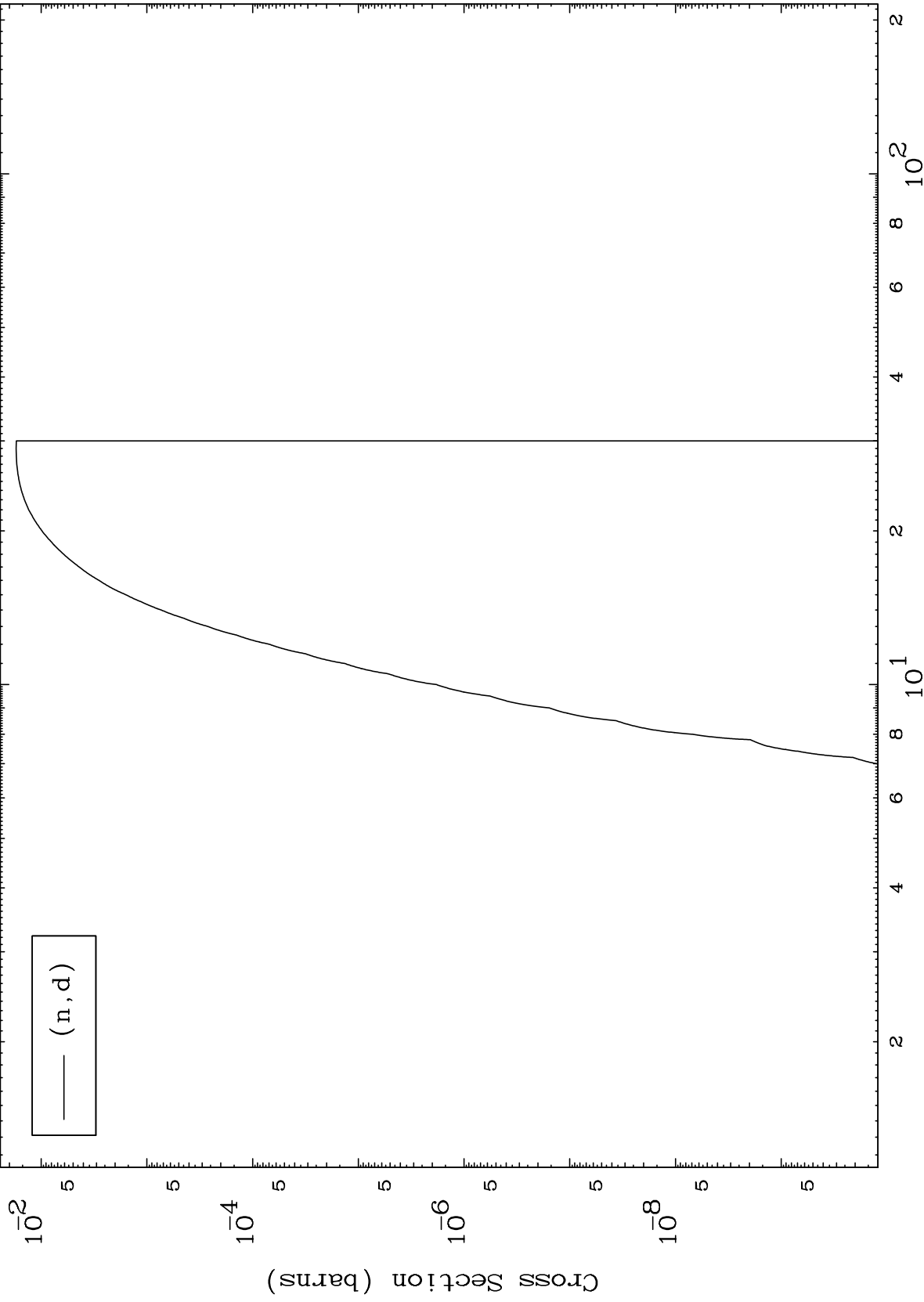
102-No-253



MAT 9971

(n,d) Levels
293 Kelvin Cross Sections

102-No-253



11

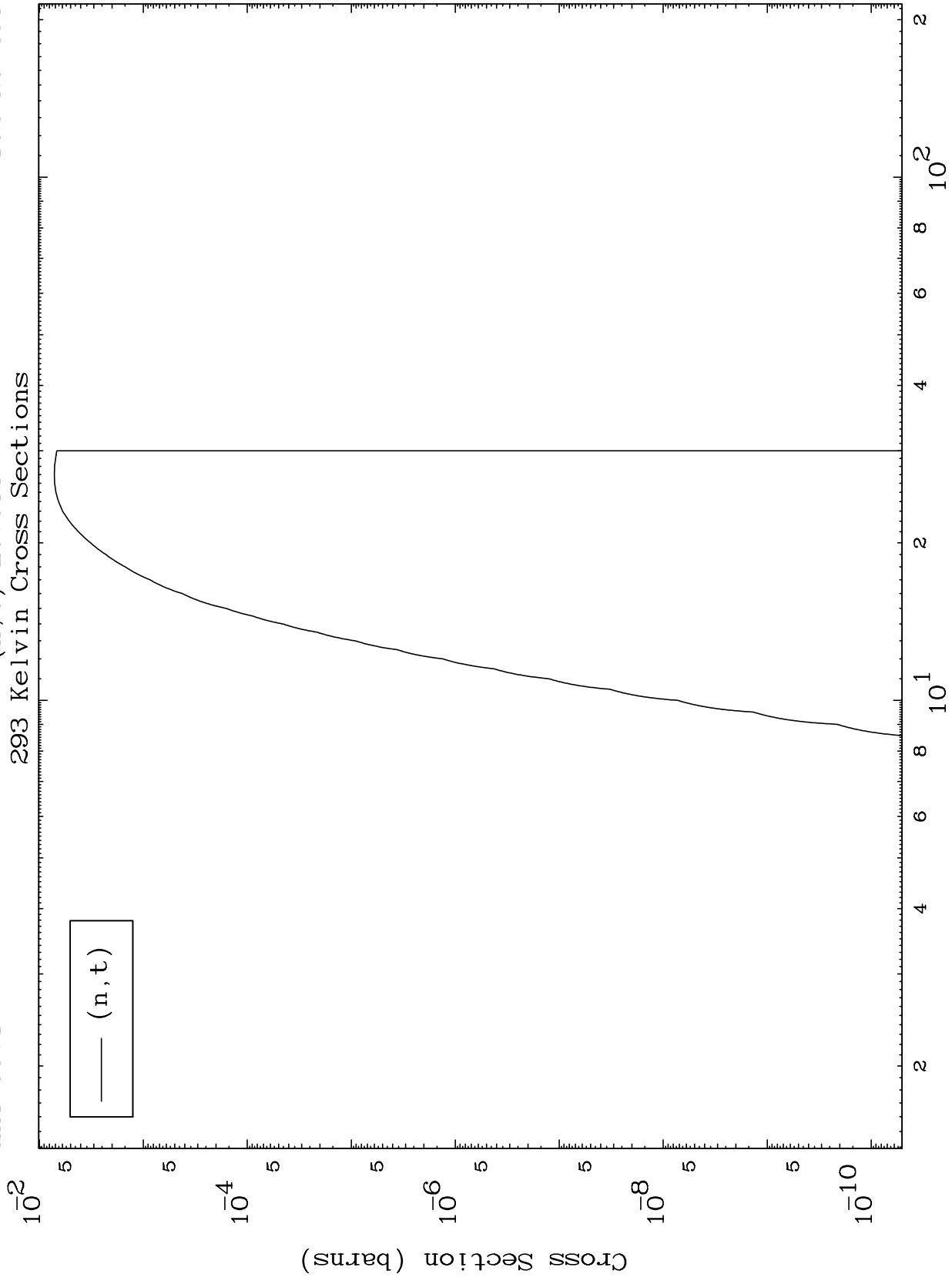
Incident Energy (MeV)

102-No-253

MAT 9971

(n,t) Levels
293 Kelvin Cross Sections

102-No-253



12

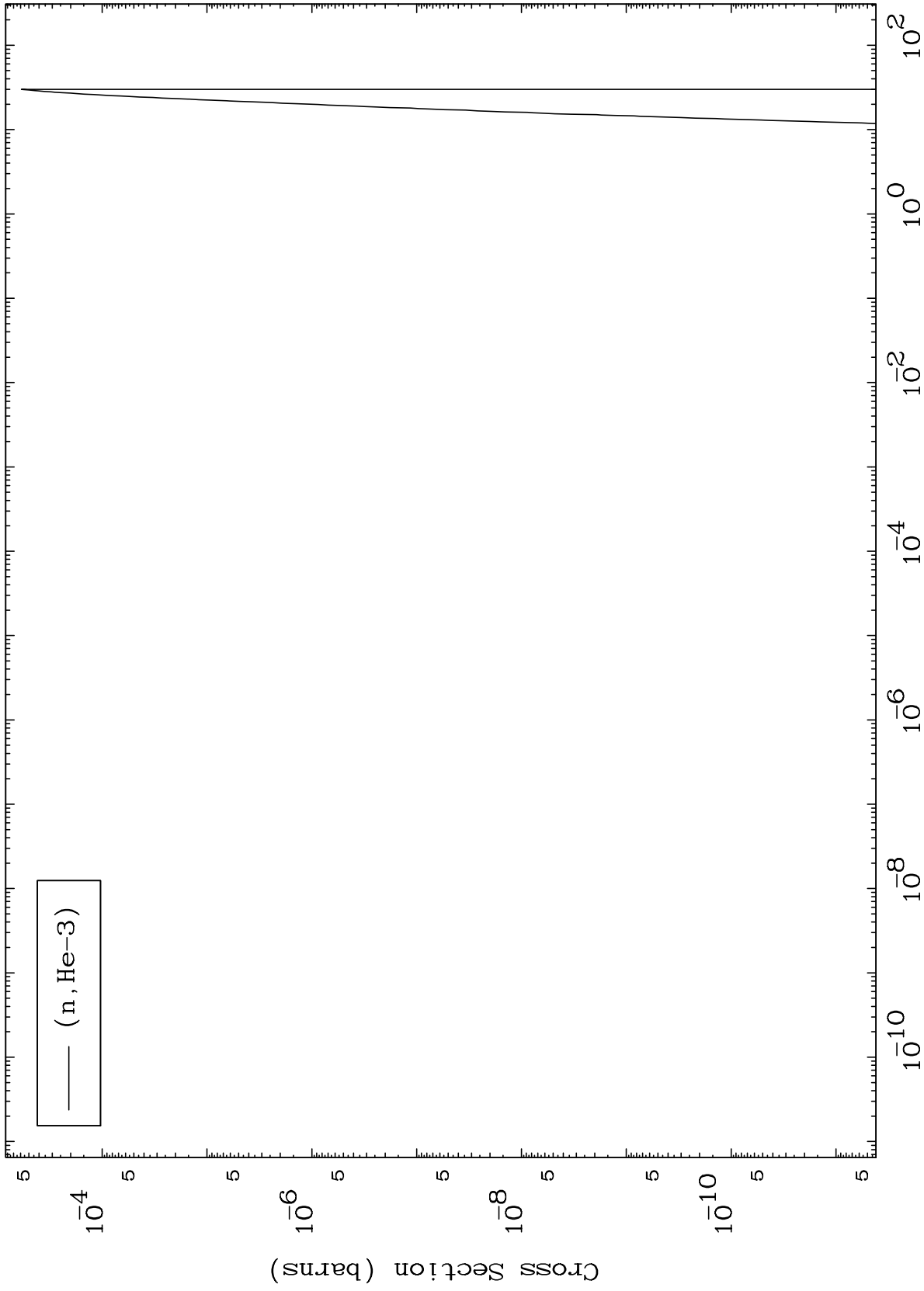
Incident Energy (MeV)

102-No-253

MAT 9971

(n,He3) Levels
293 Kelvin Cross Sections

102-No-253



13

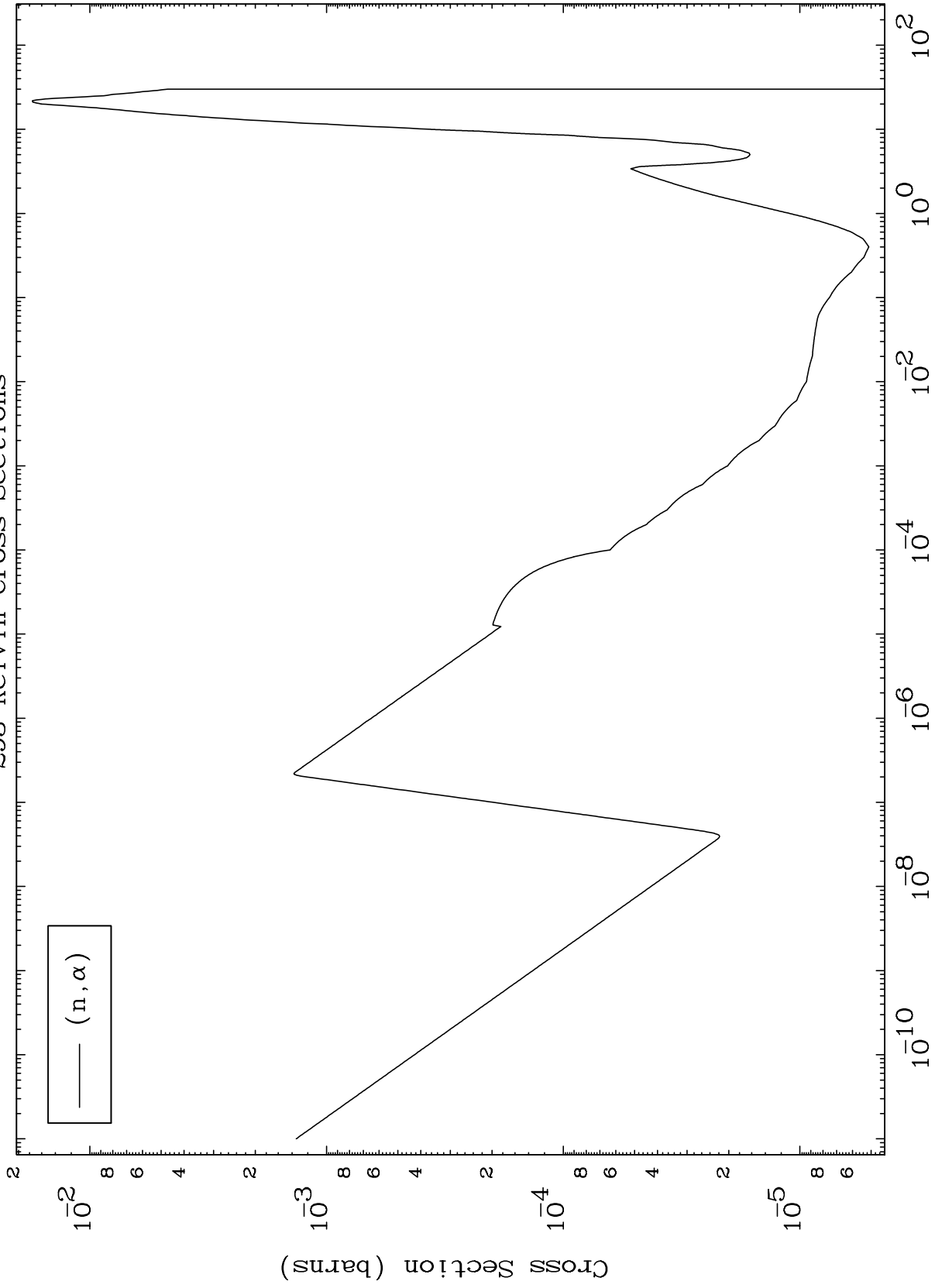
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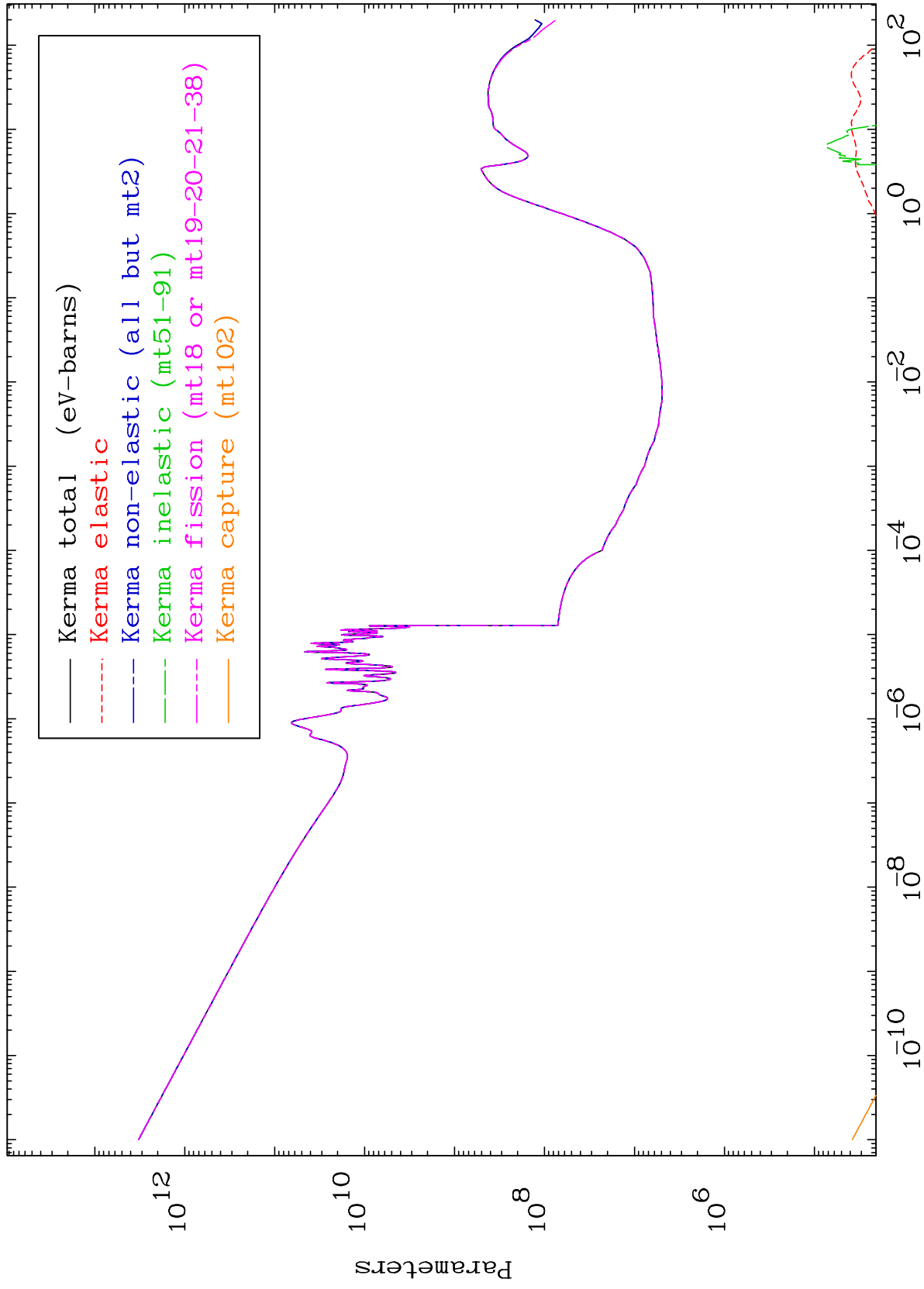
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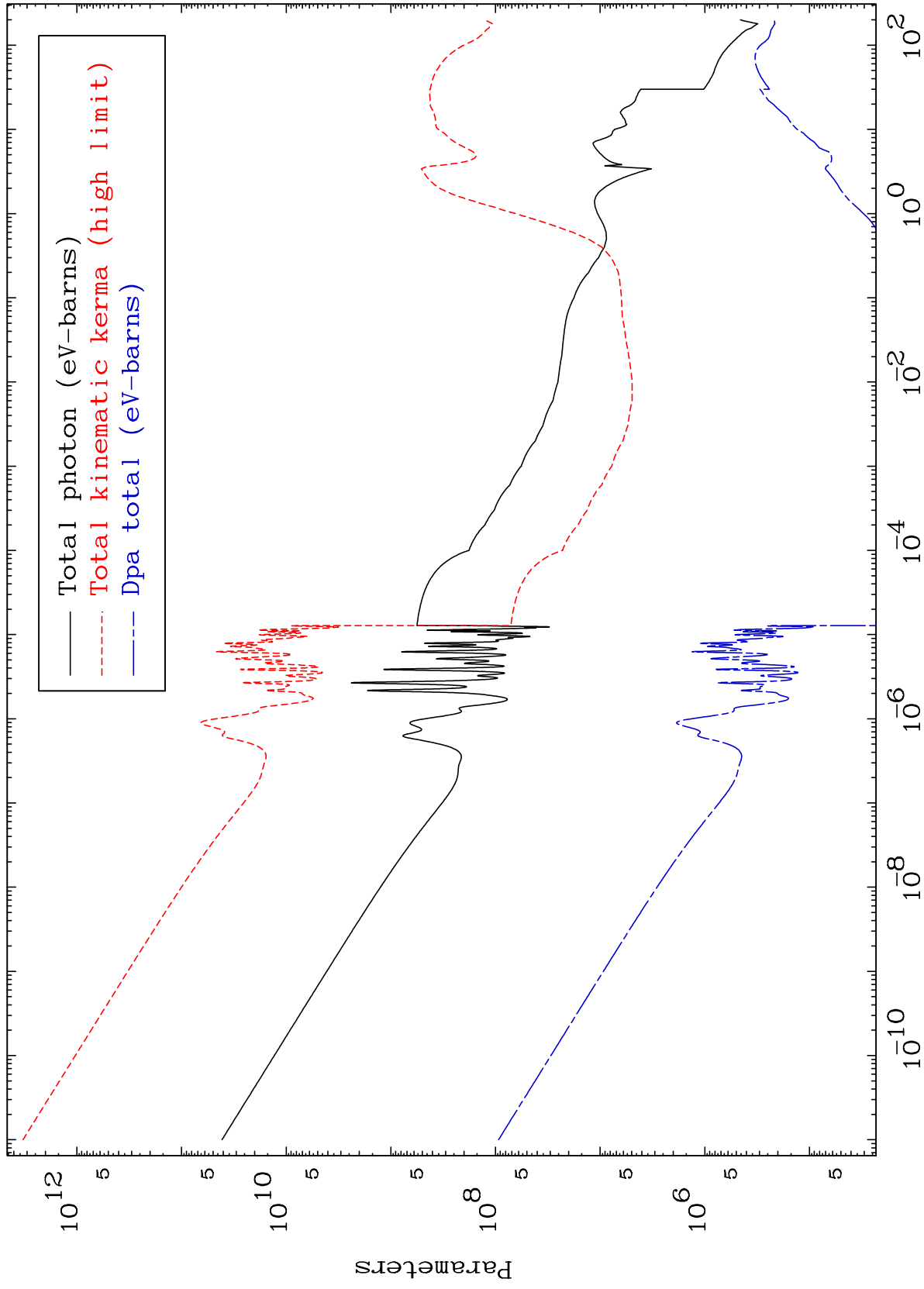
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(n,α) Levels
293 Kelvin Cross Sections

102-No-253



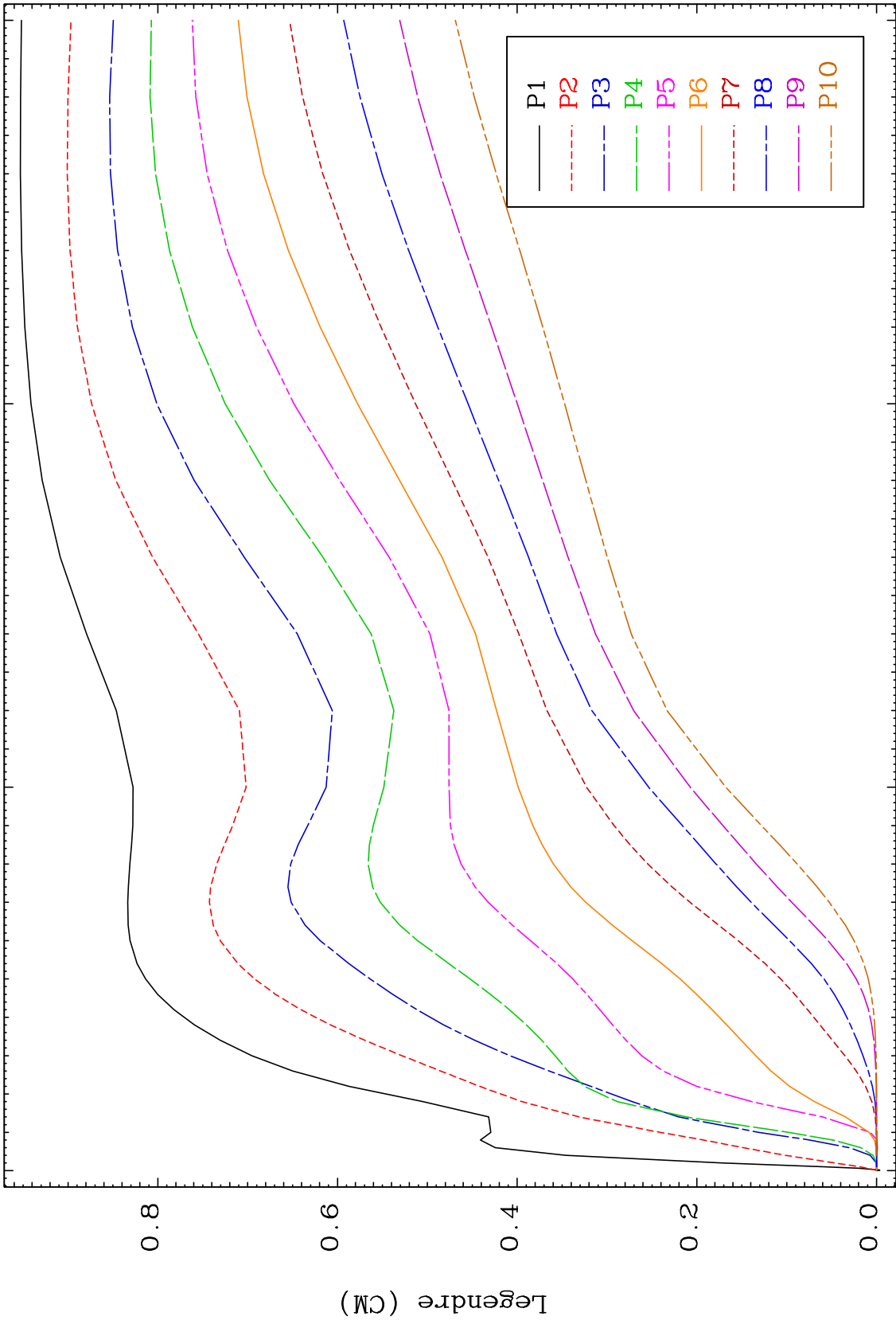




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Elastic Legendre Coefficients

102-No-253



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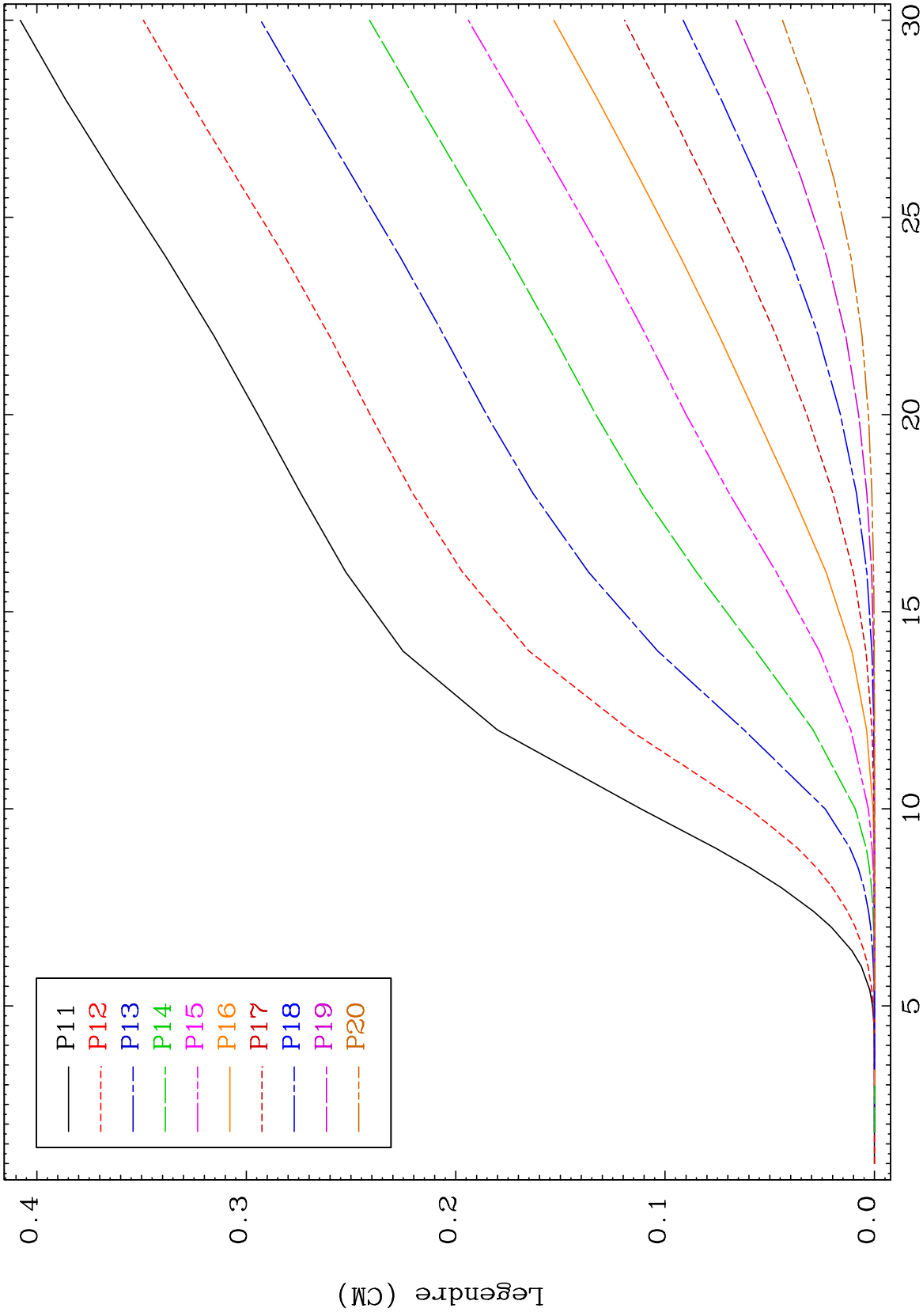
Incident Energy (MeV)

102-No-253

MAT 9971

Elastic Legendre Coefficients

102-No-253



18

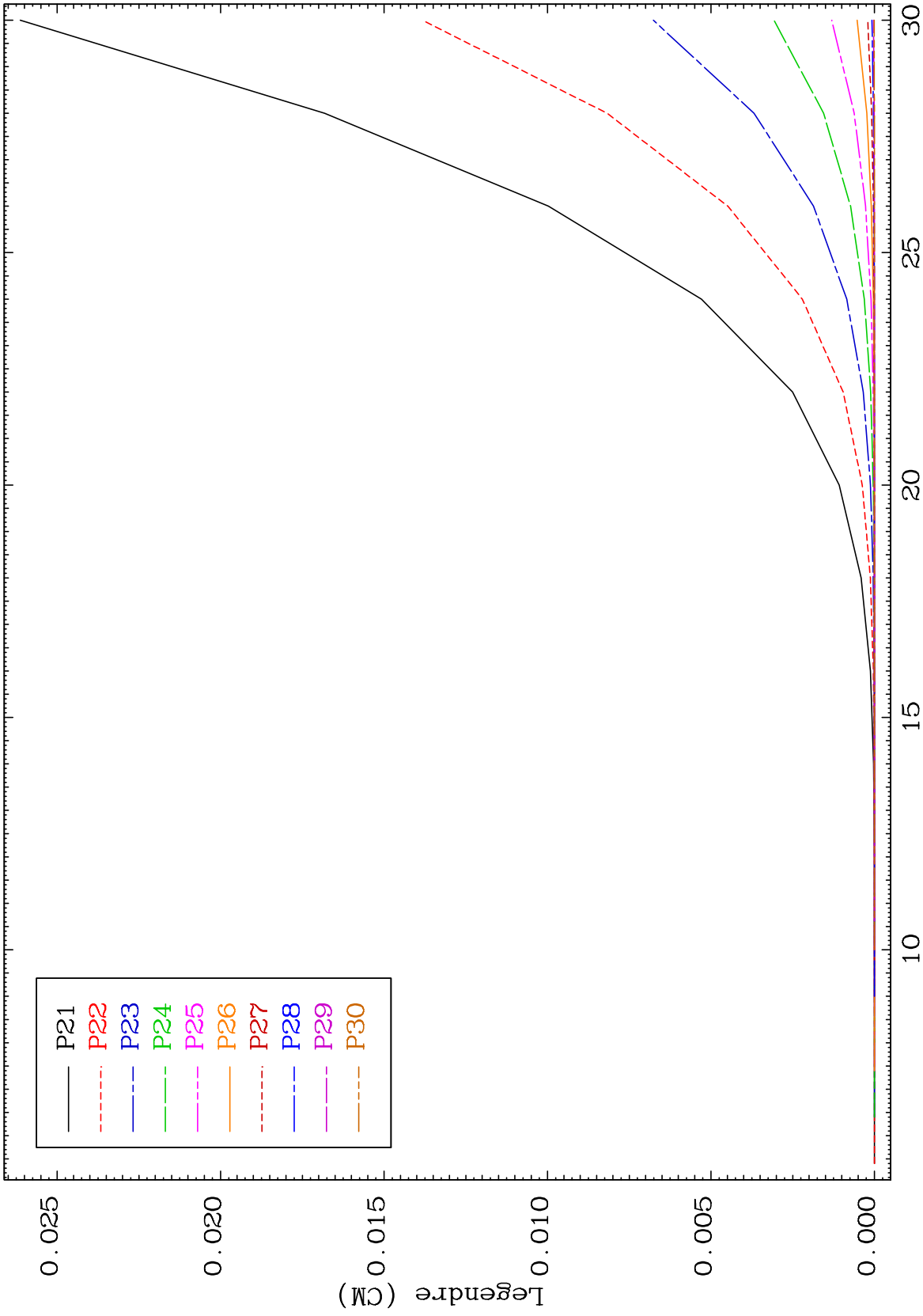
Incident Energy (MeV)

102-No-253

MAT 9971

Elastic Legendre Coefficients

102-No-253



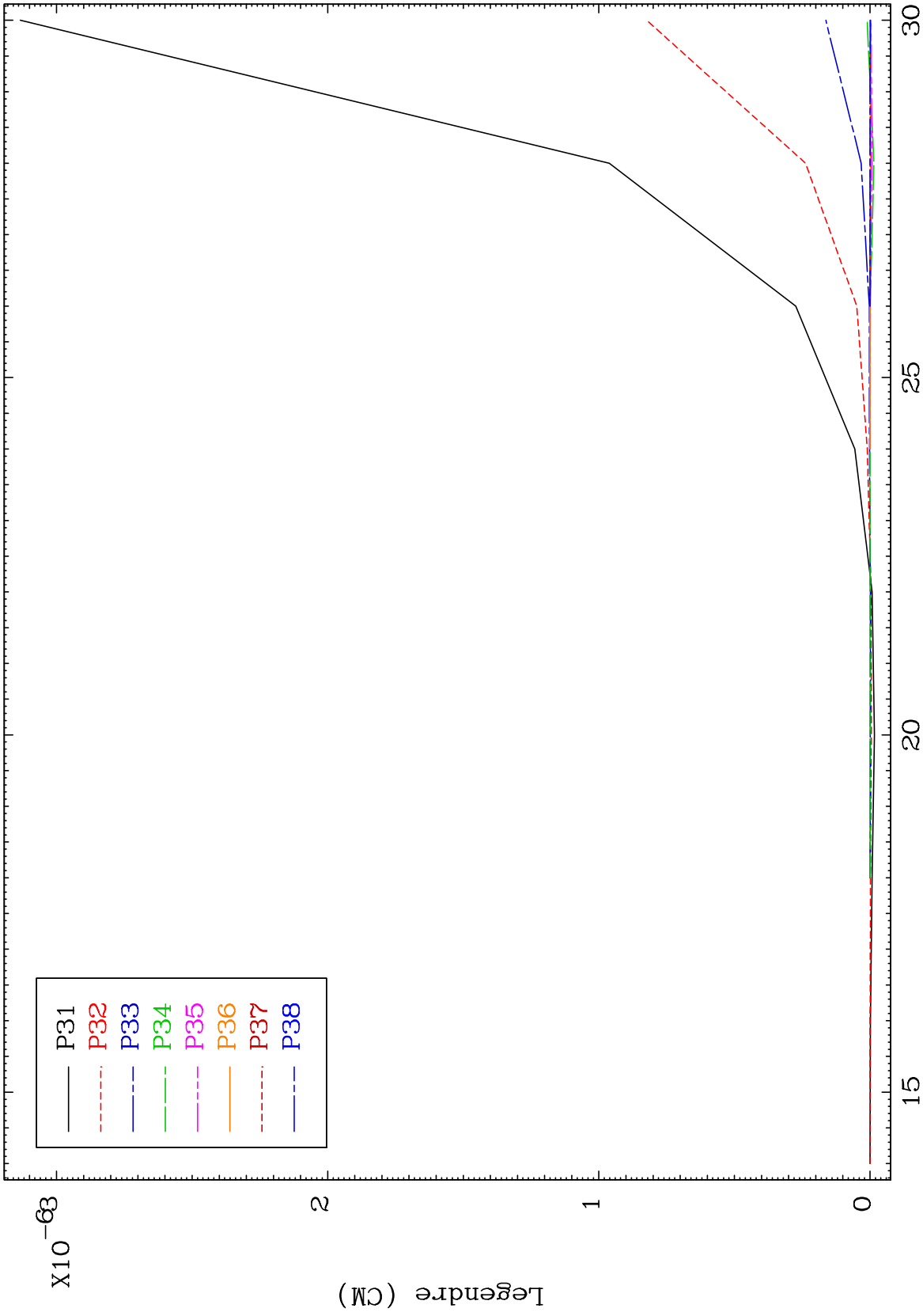
19

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MAT 9971

Elastic Legendre Coefficients

102-No-253



20

Incident Energy (MeV)

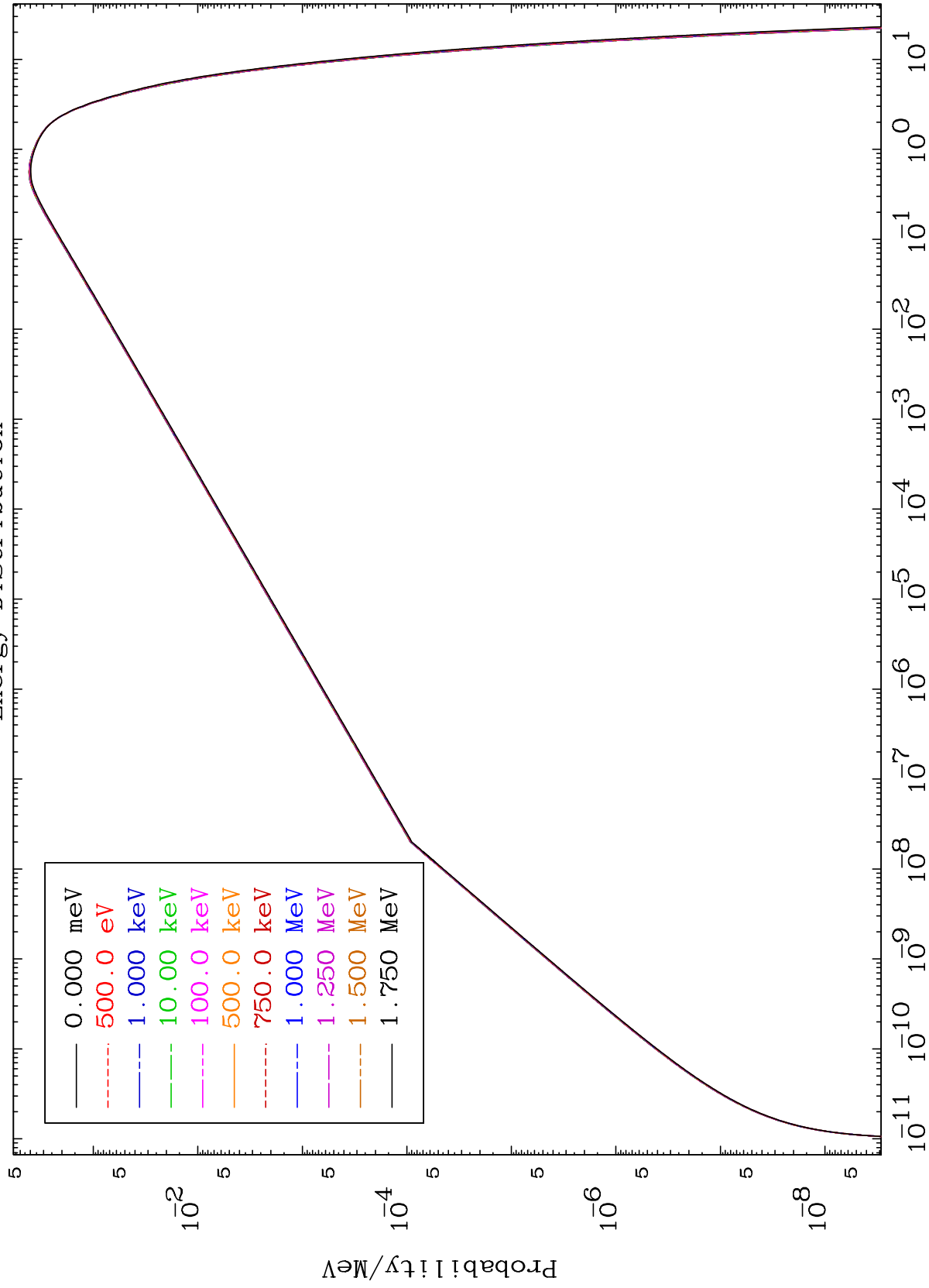
102-No-253

30

MAT 9971

Fission Energy Distribution

102-No-253

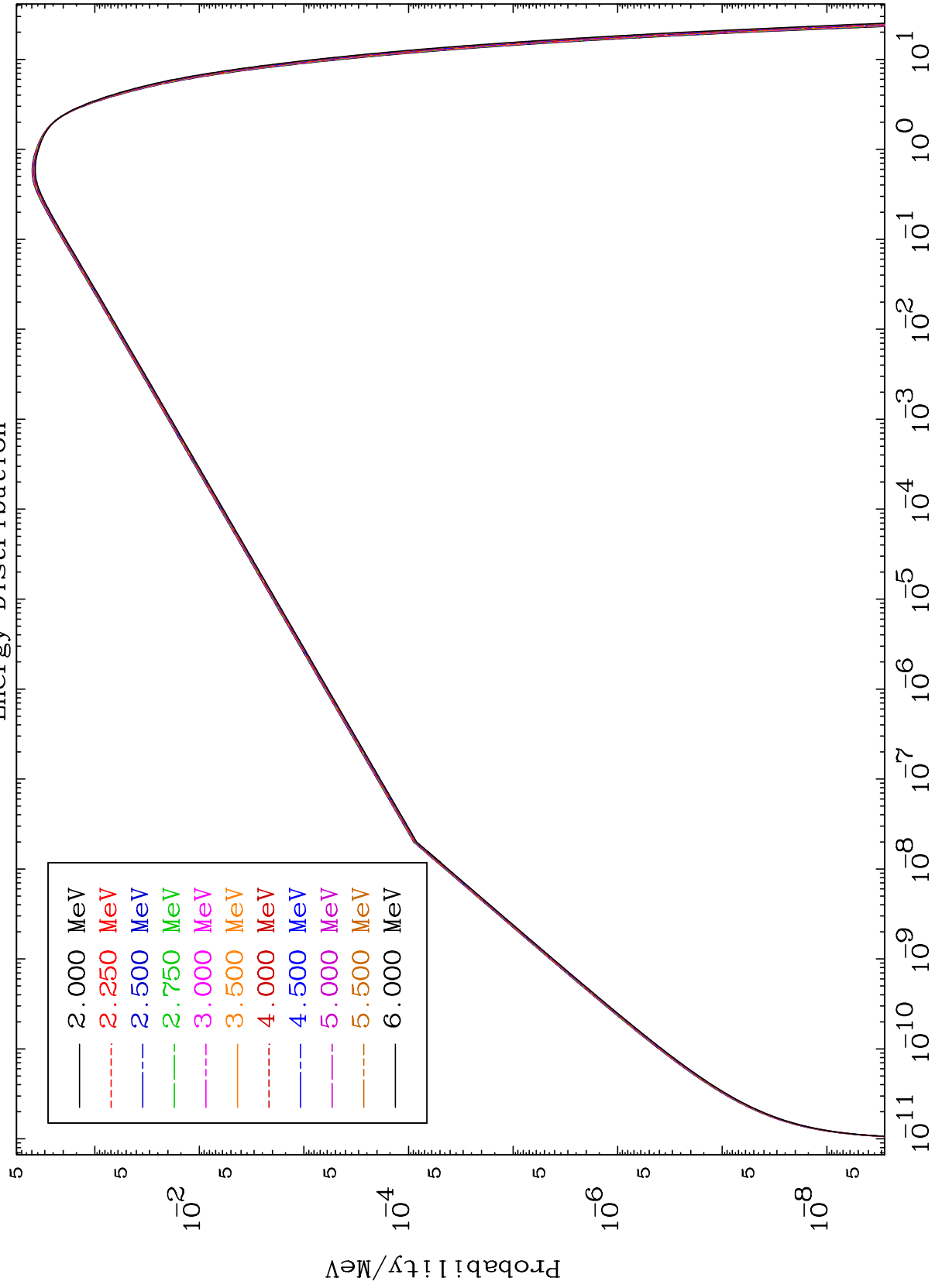


102-No-253

MAT 9971

Fission Energy Distribution

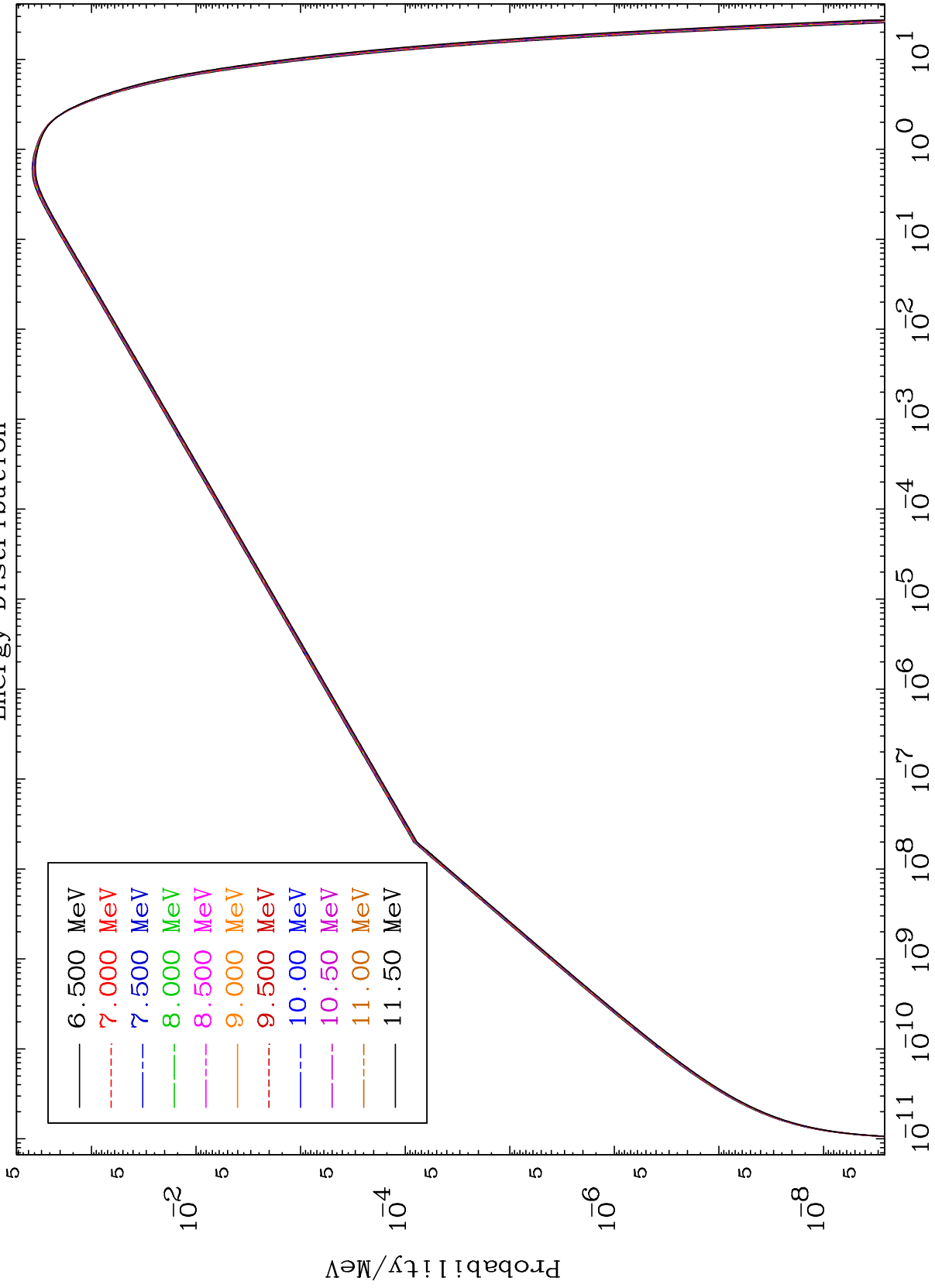
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MAT 9971

Fission Energy Distribution

102-No-253

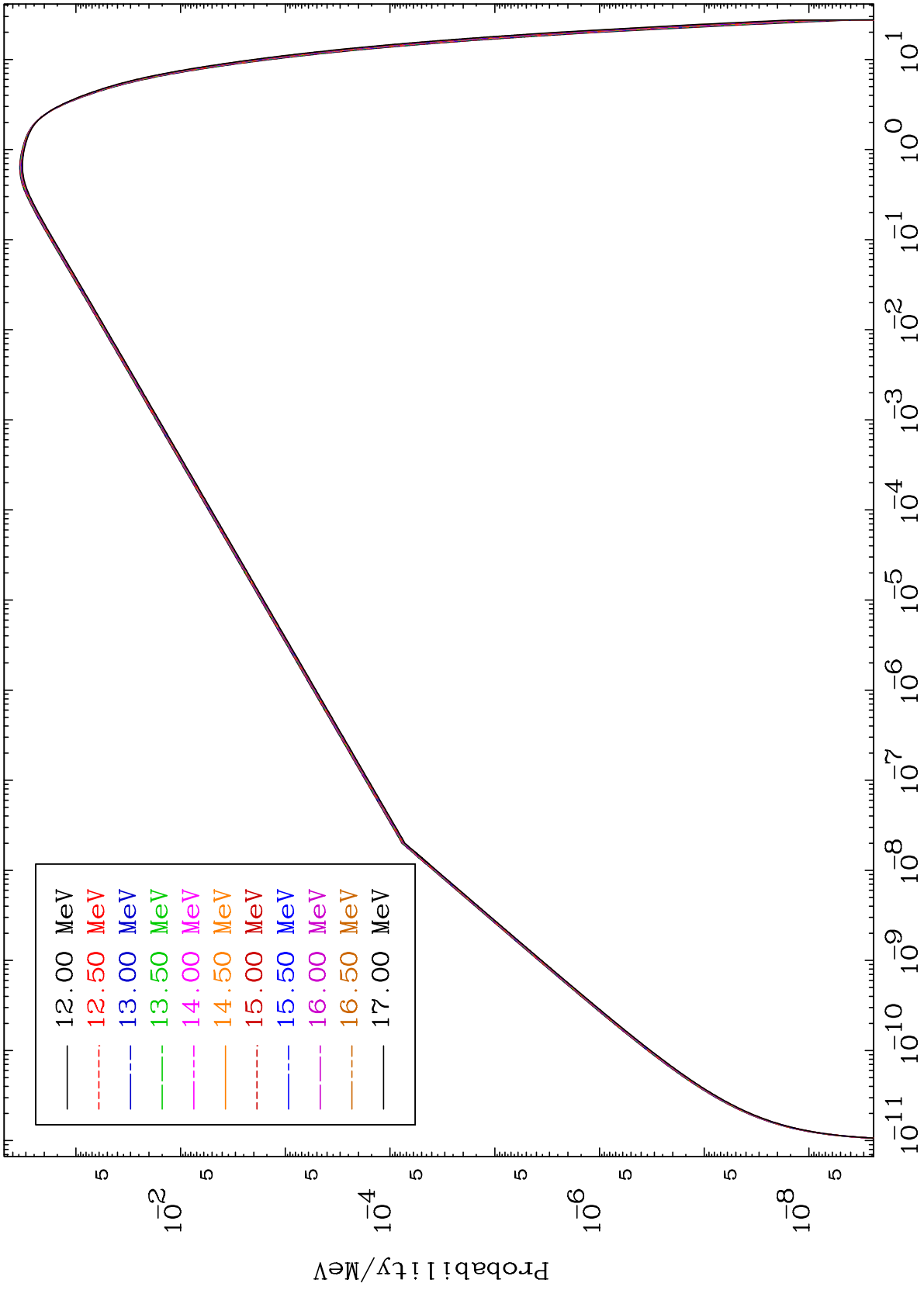


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MAT 9971

Fission Energy Distribution

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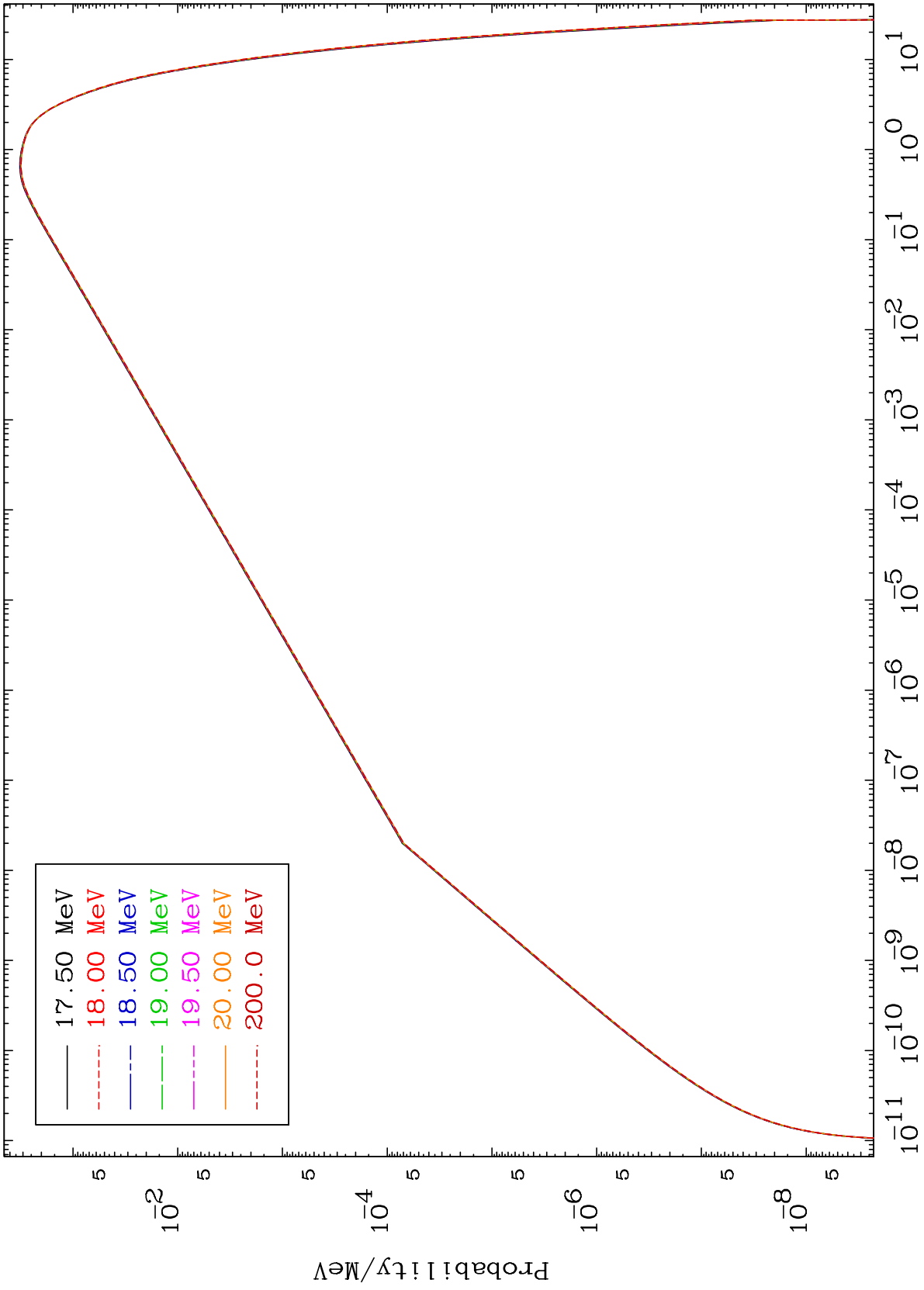


102-No-253

MAT 9971

Fission Energy Distribution

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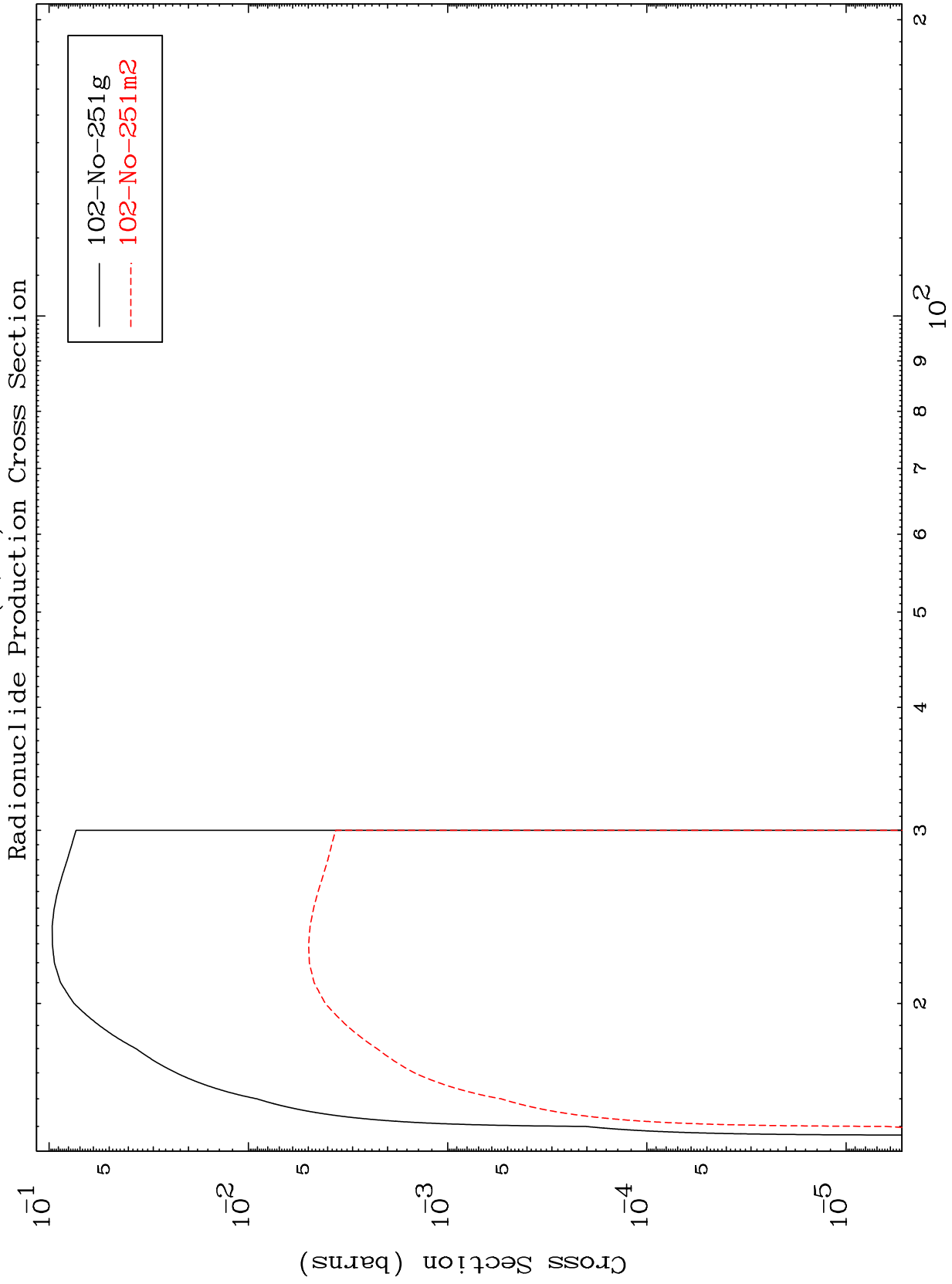


102-No-253

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(n,3n)

102-No-253



26

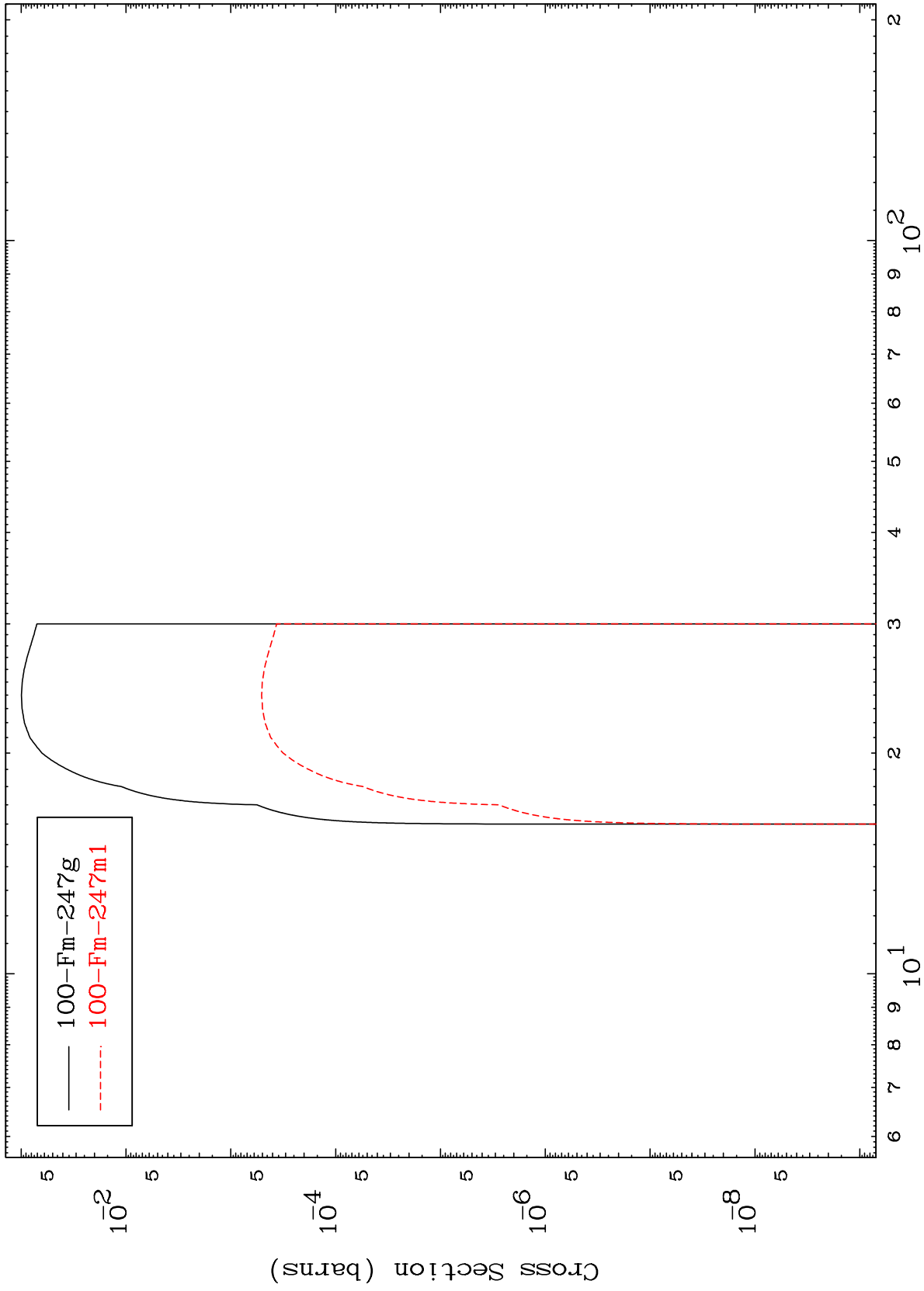
102-No-253

MAT 9971

(n,3n) α

102-No-253

Radionuclide Production Cross Section



27

Incident Energy (MeV)

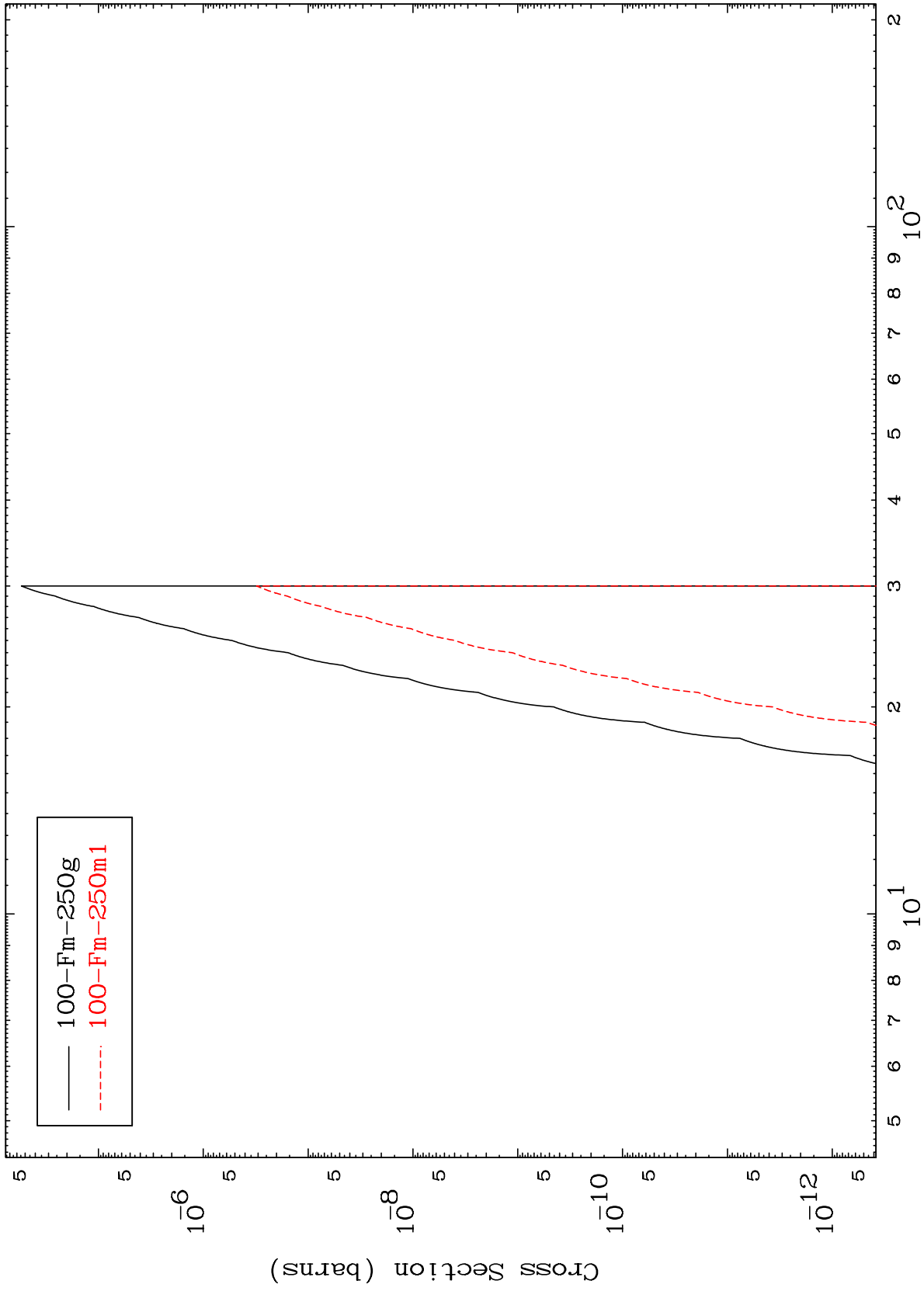
102-No-253

MAT 9971

(n, n') He-3

102-No-253

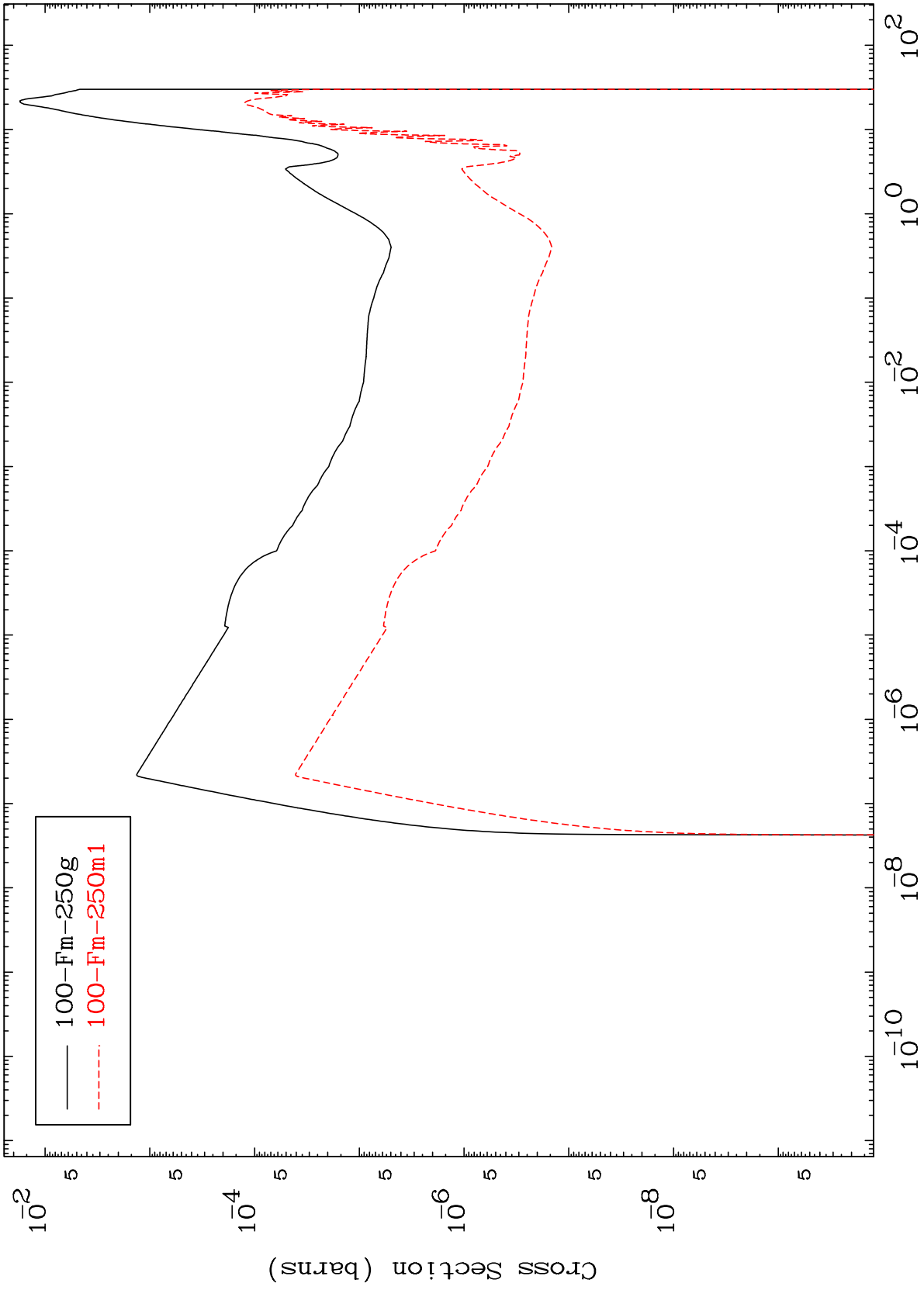
Radionuclide Production Cross Section



MAT 9971

102-No-253

Radionuclide Production Cross Section



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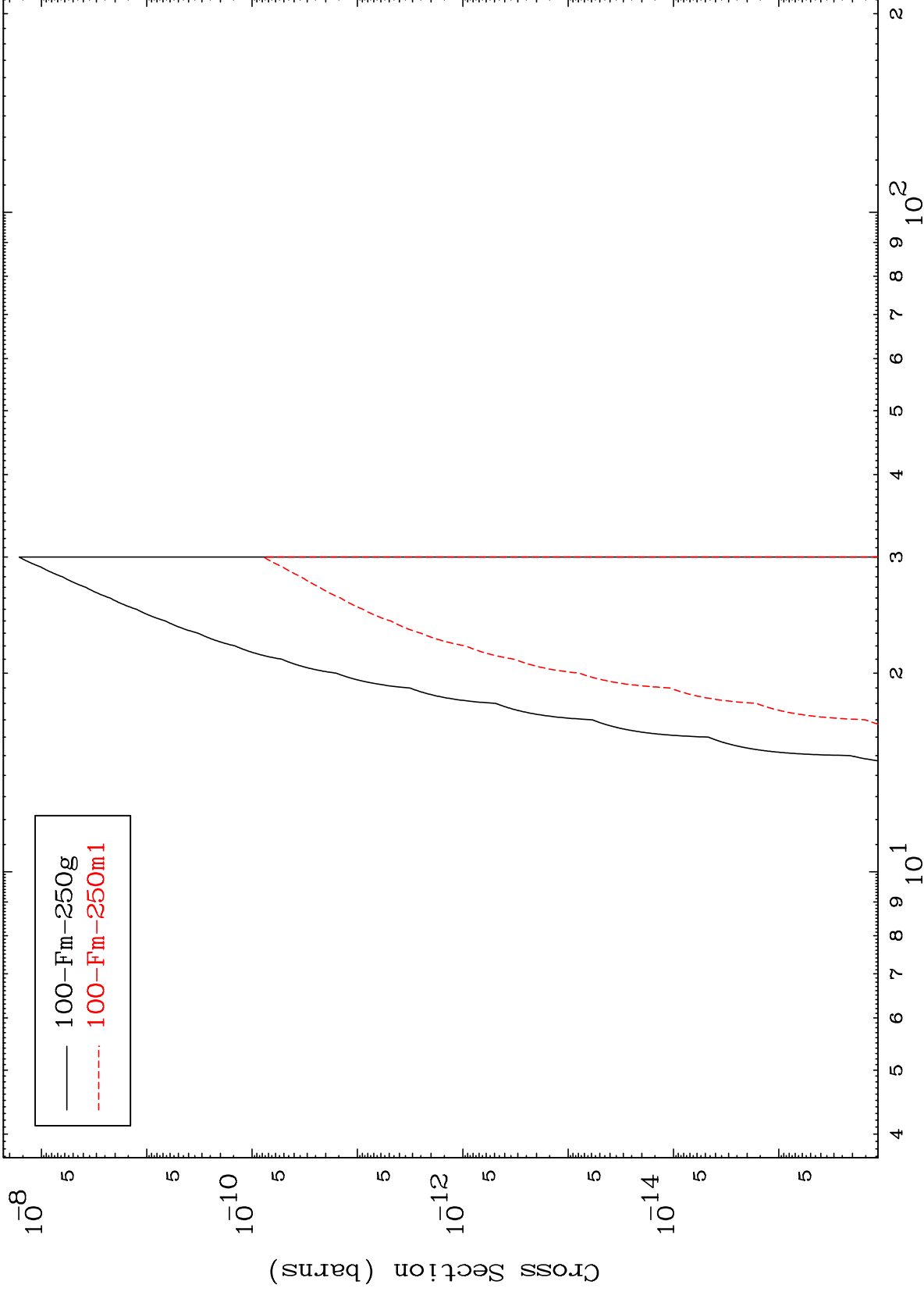
102-No-253

MAT 9971

(n,p) t

102-No-253

Radionuclide Production Cross Section



30

Incident Energy (MeV)

102-No-253