

Program EVALPLOT
(Version 2018-1)

by

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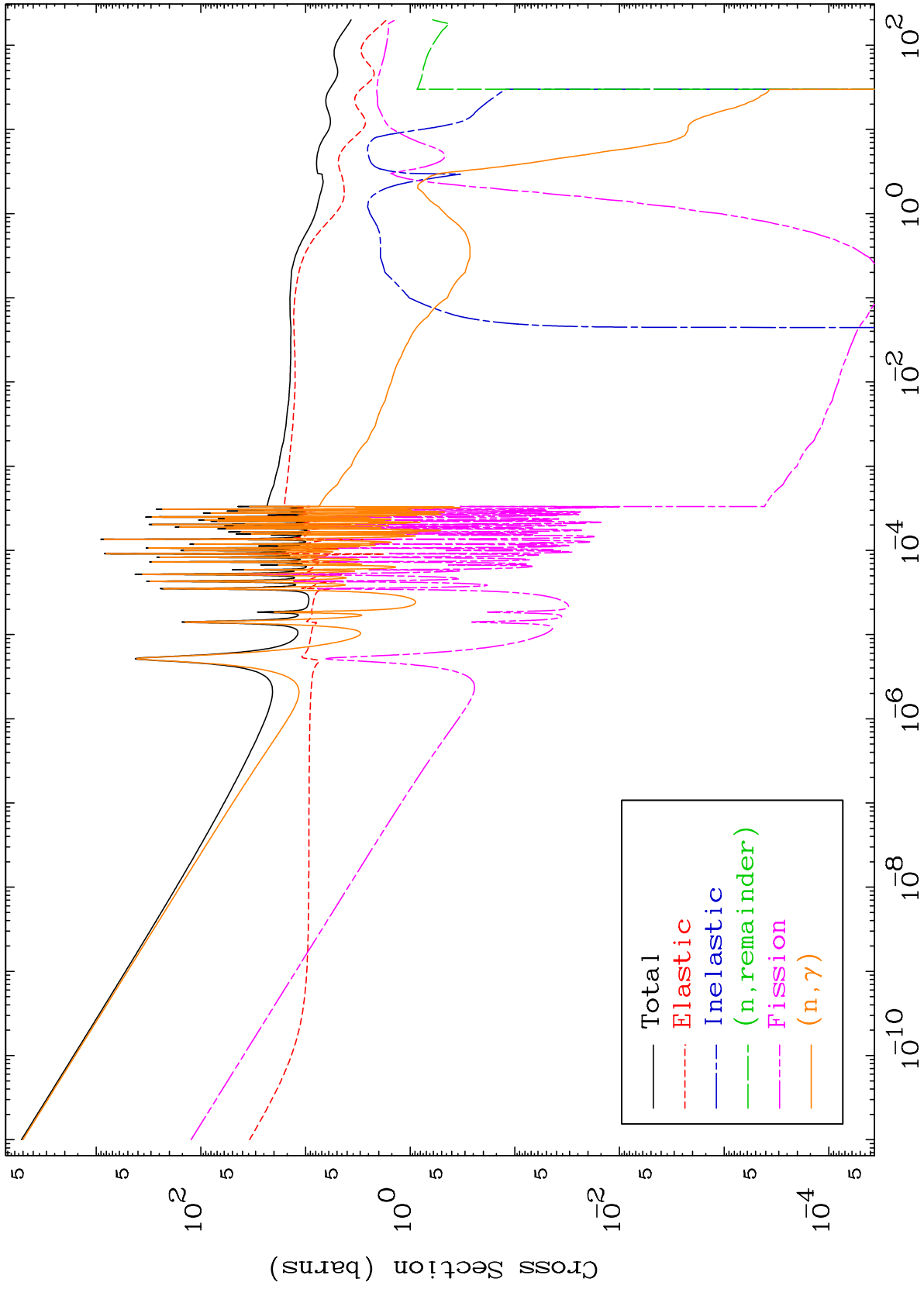
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9972

Major
293 Kelvin Cross Sections

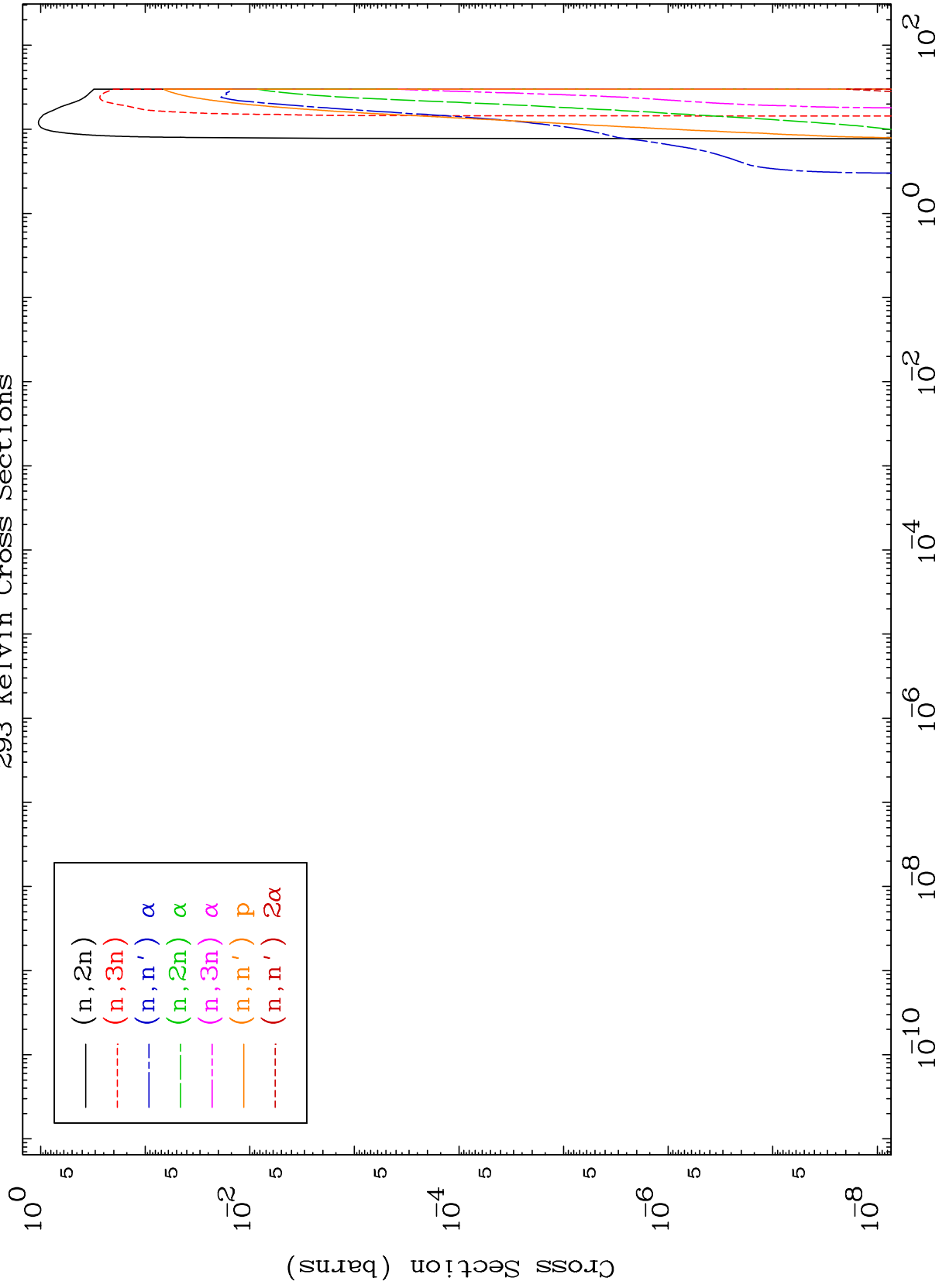
102-No-254



MAT 9972

Neutron Production
293 Kelvin Cross Sections

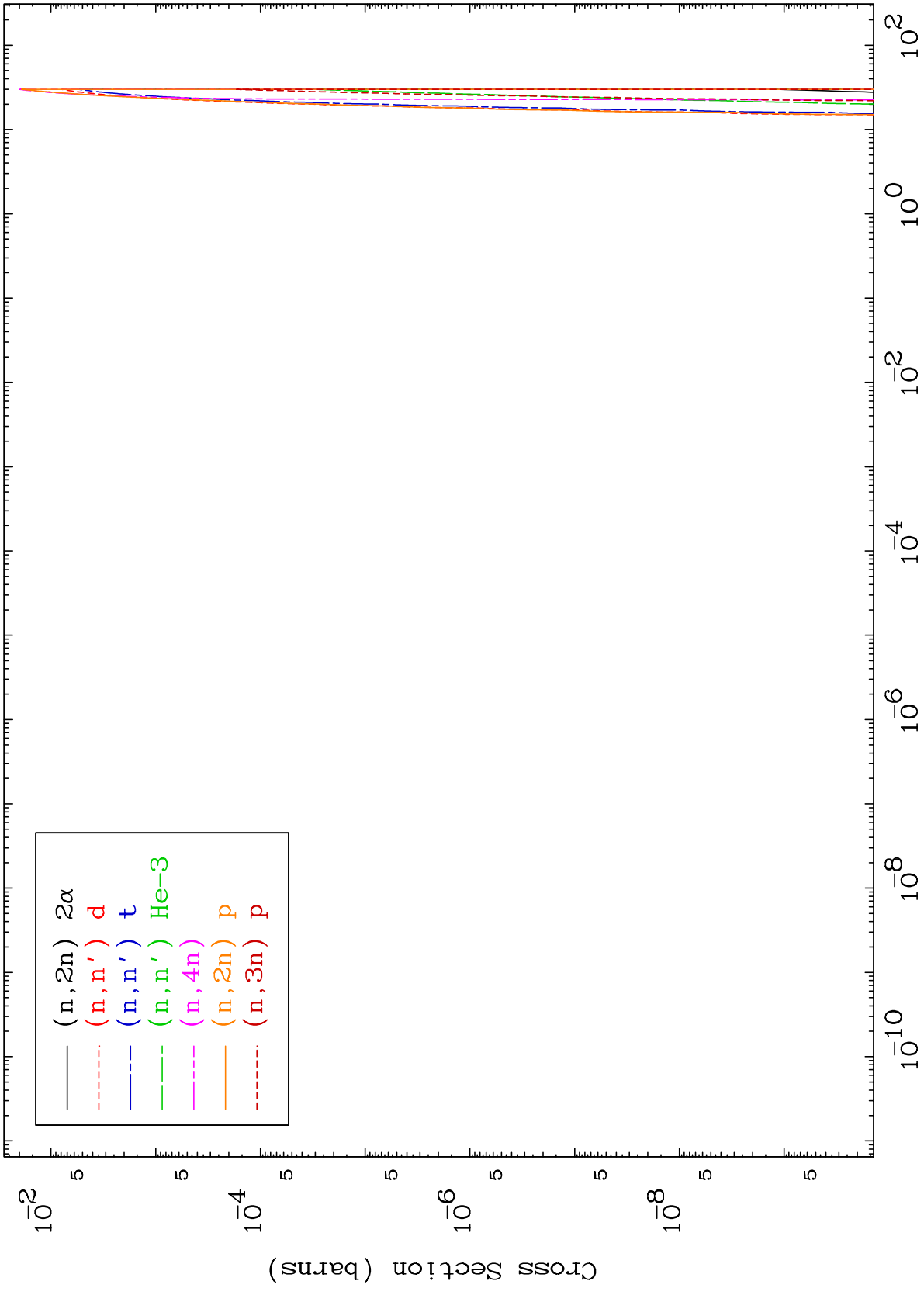
102-No-254



MAT 9972

Neutron Production
293 Kelvin Cross Sections

102-No-254

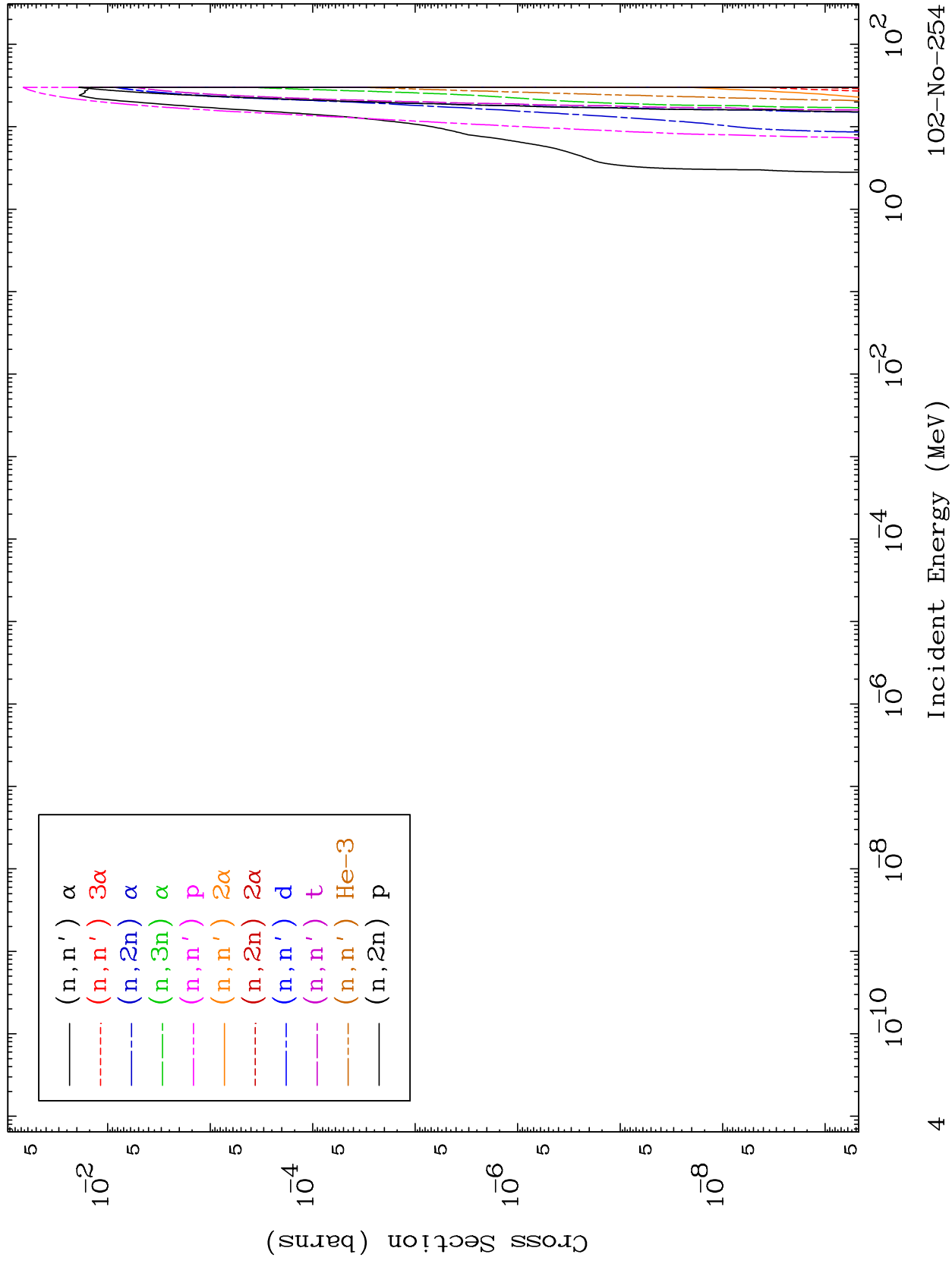


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MAT 9972

293 Kelvin Cross Sections

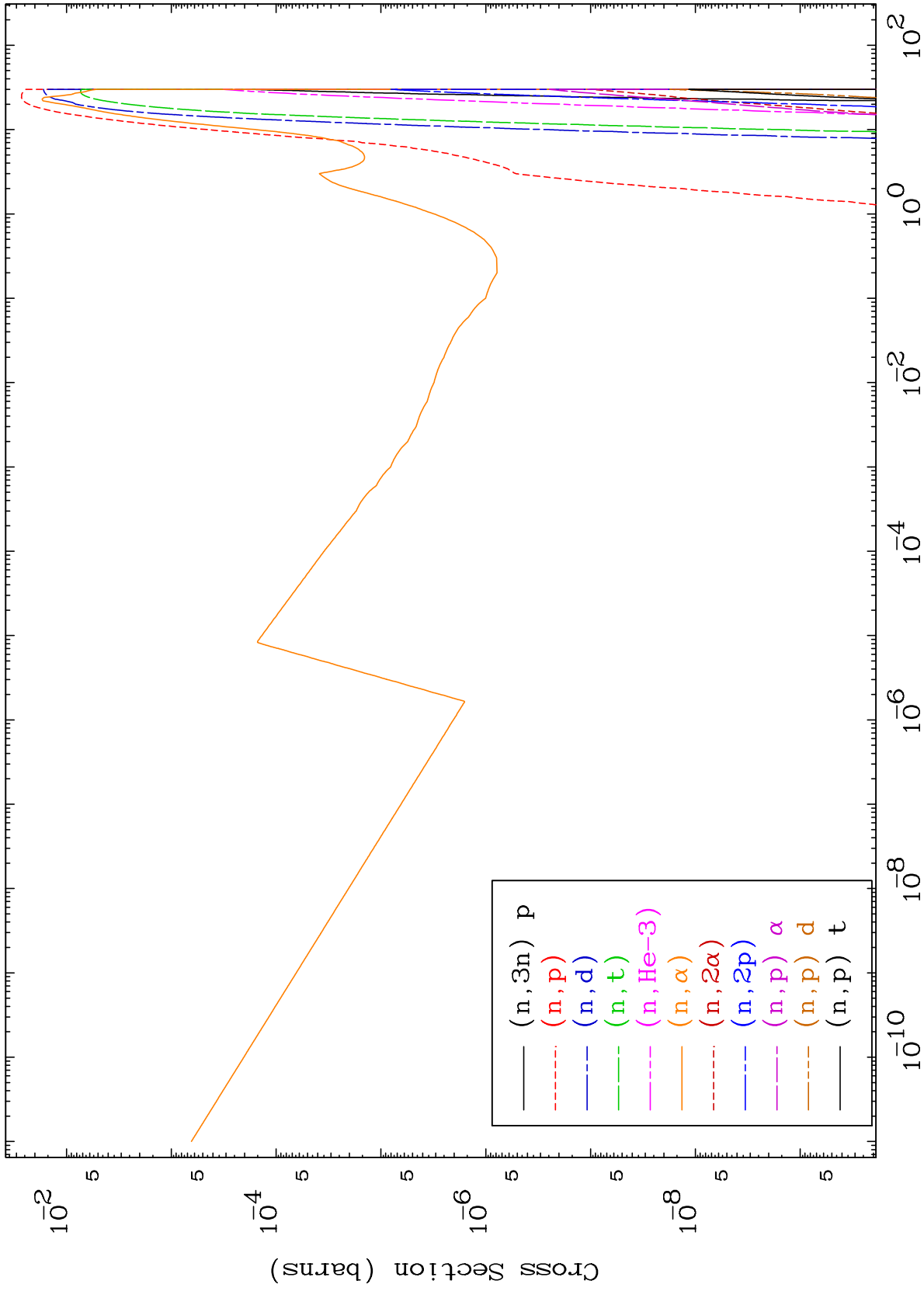
102-No-254



MAT 9972

Charged Particle
293 Kelvin Cross Sections

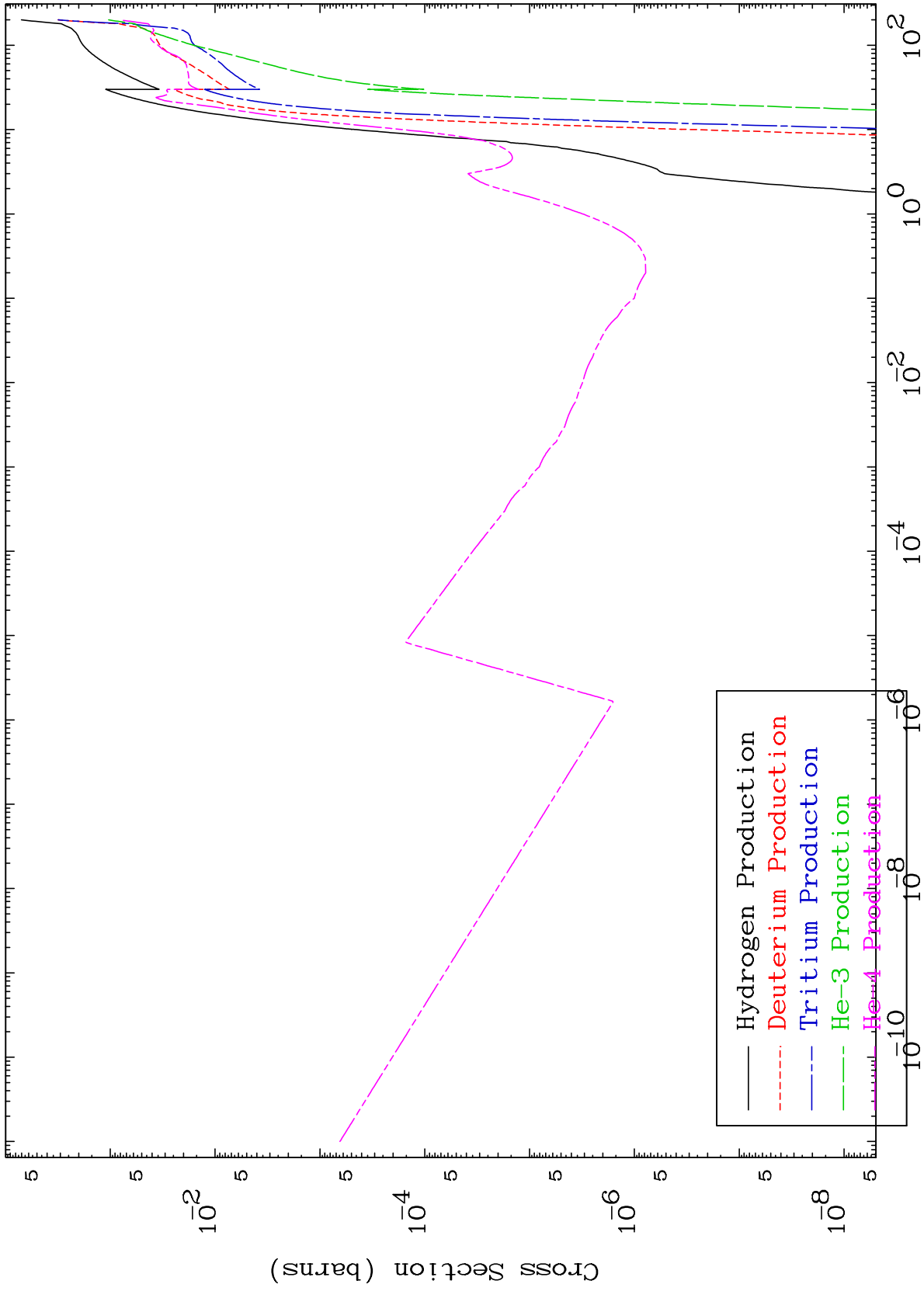
102-No-254



MAT 9972

Particle Production
293 Kelvin Cross Sections

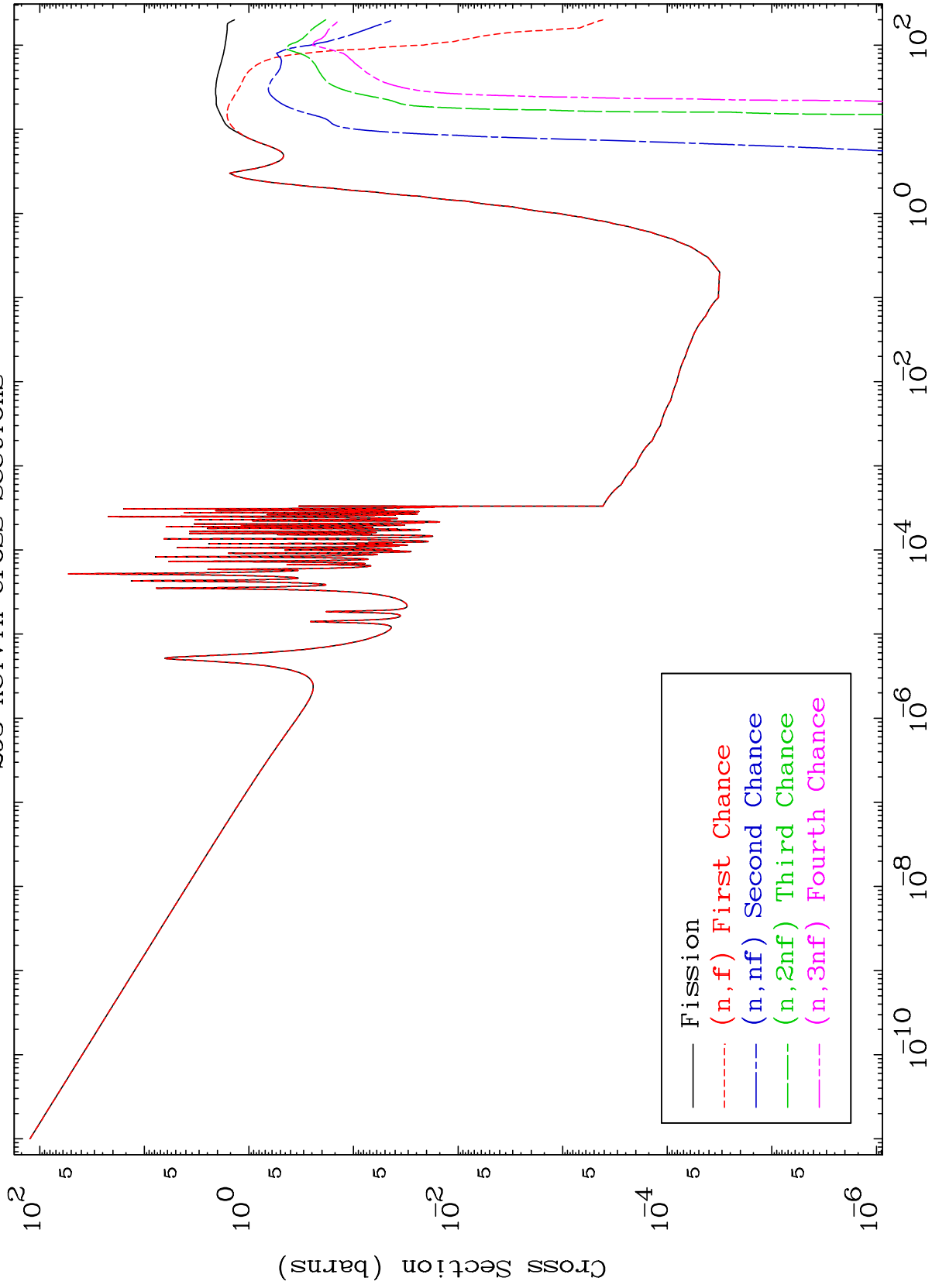
102-No-254



MAT 9972

Fission
293 Kelvin Cross Sections

102-No-254

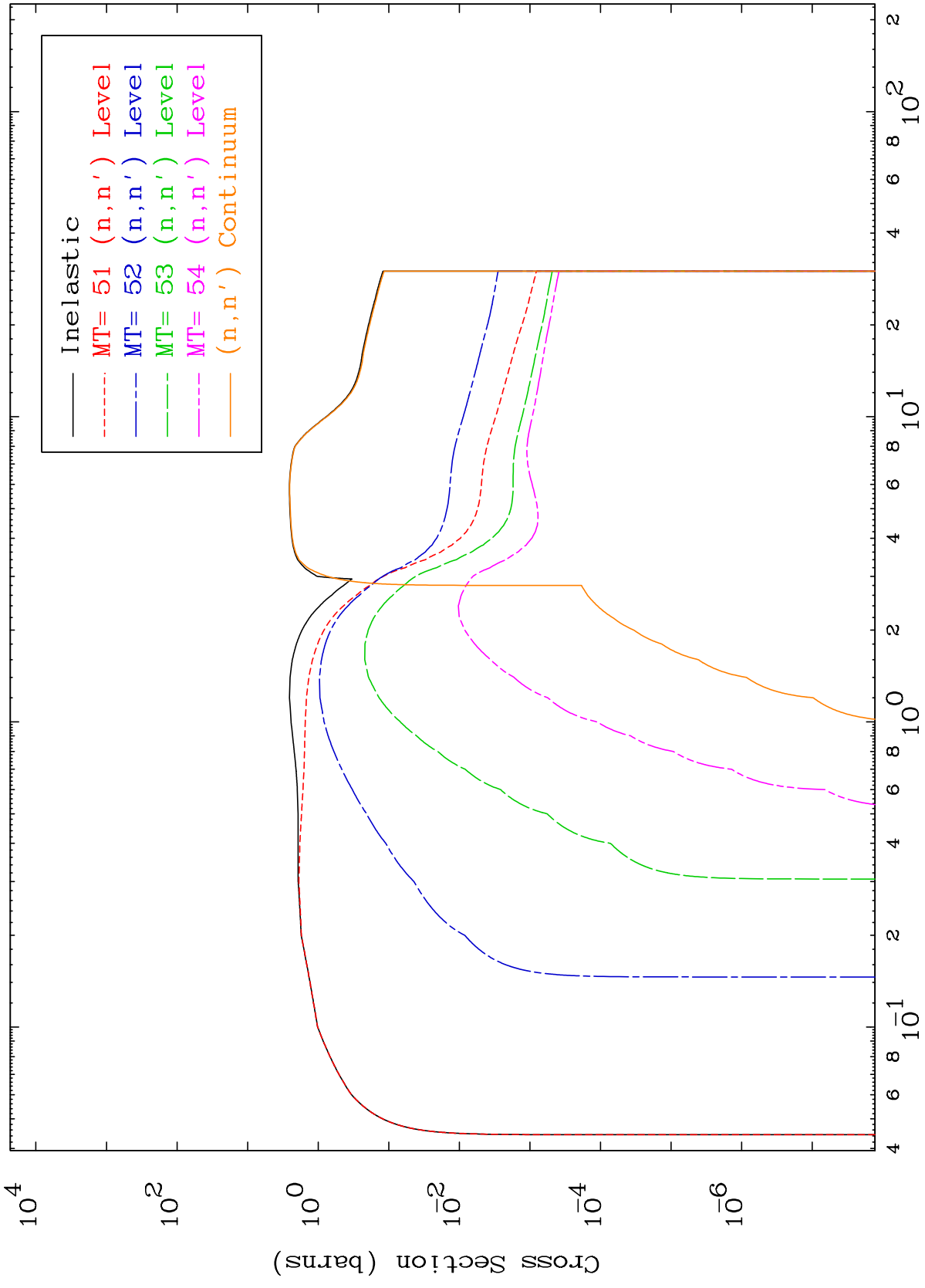


MAT 9972

(n,n') Level

102-No-254

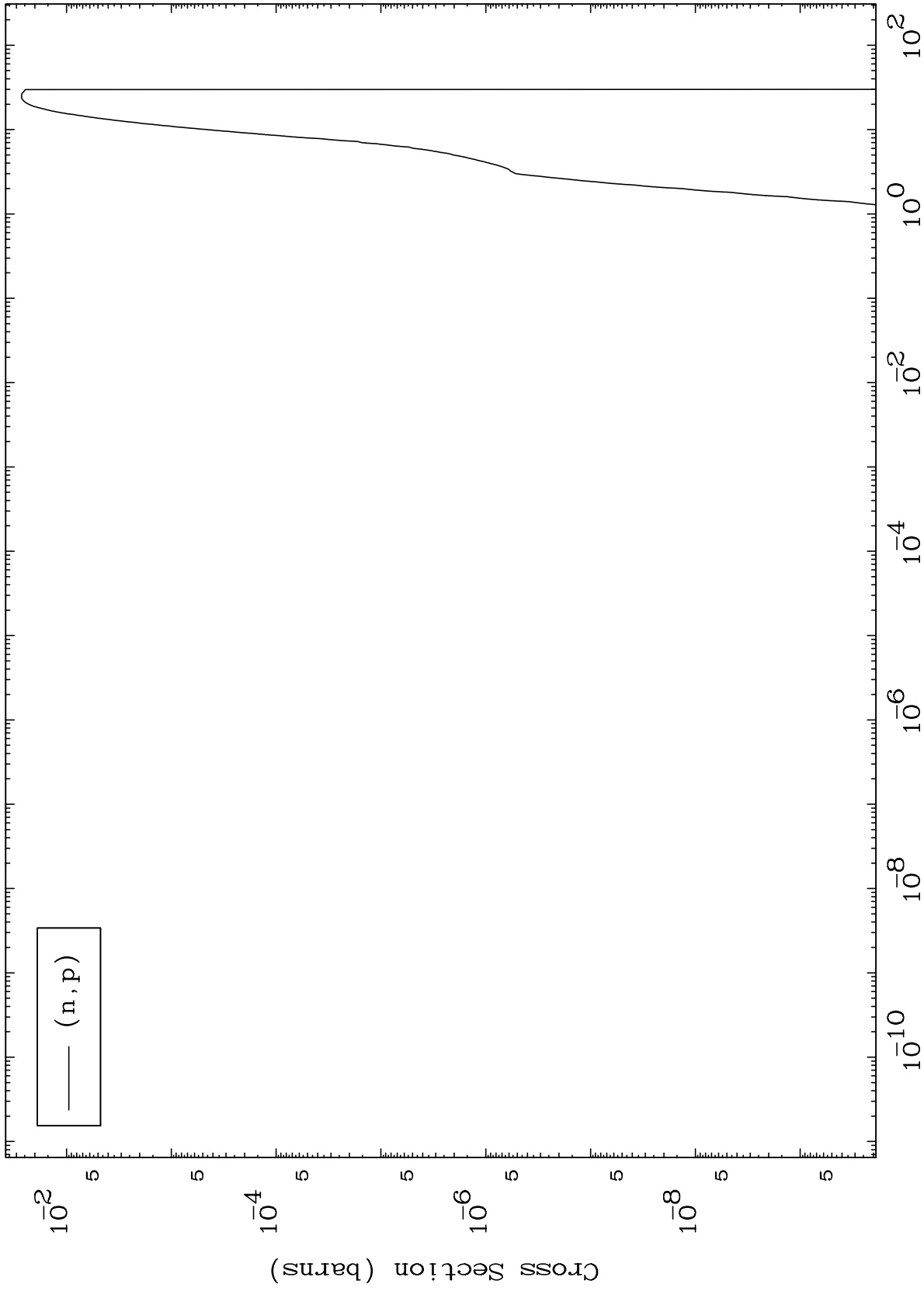
293 Kelvin Cross Sections



MAT 9972

(n,p) Levels
293 Kelvin Cross Sections

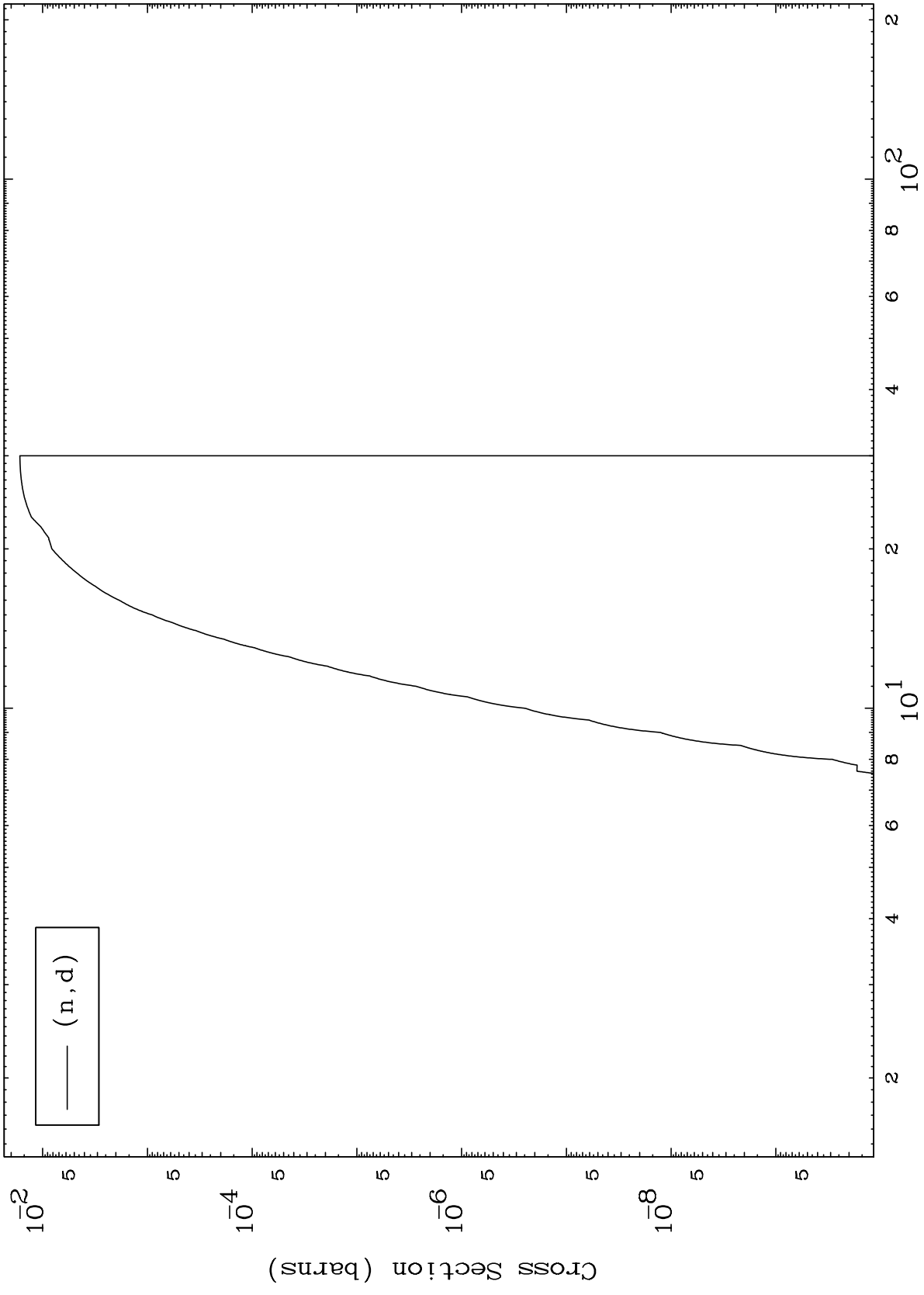
102-No-254



MAT 9972

(n,d) Levels
293 Kelvin Cross Sections

102-No-254



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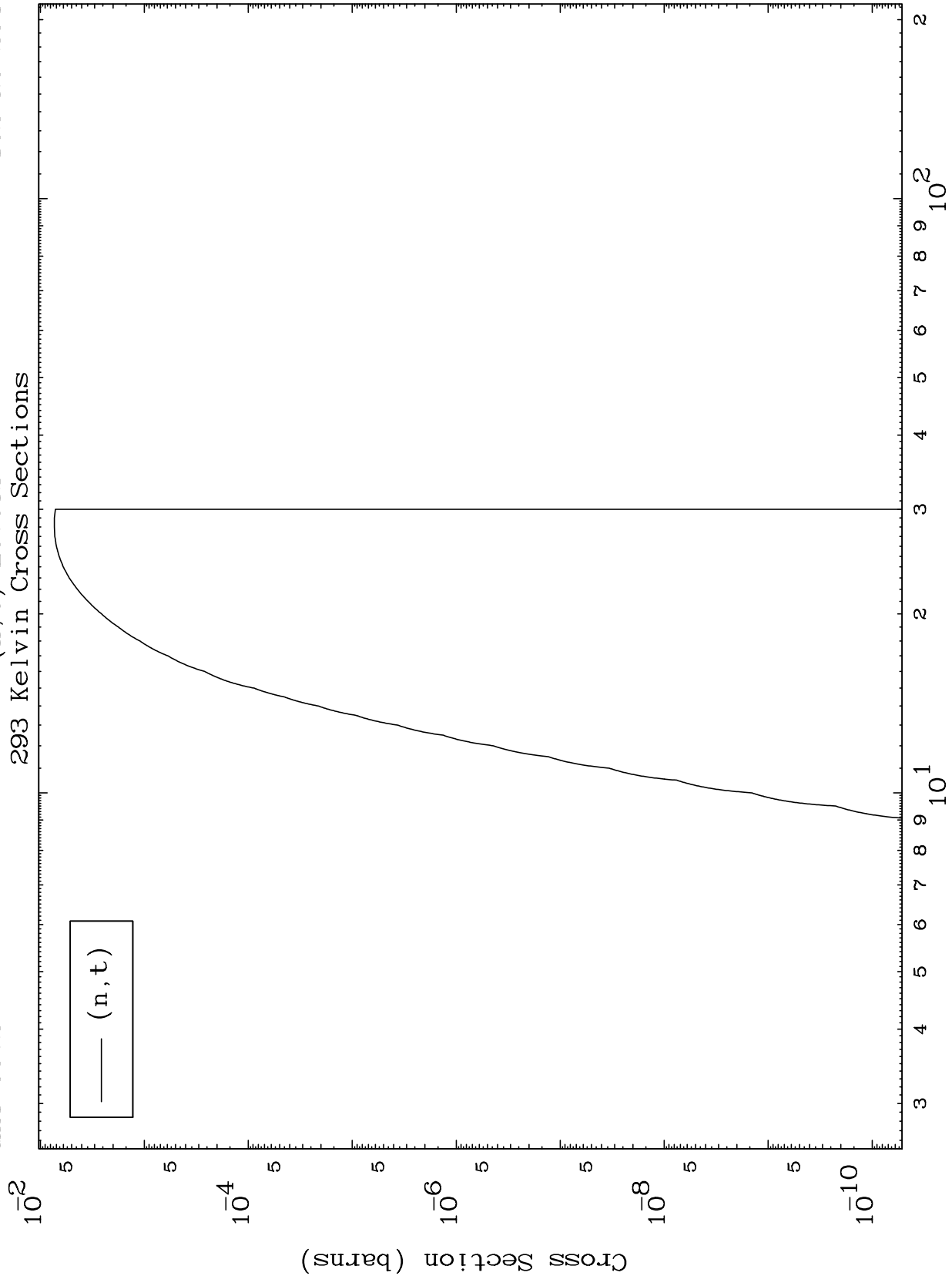
Incident Energy (MeV)

102-No-254

MAT 9972

(n,t) Levels
293 Kelvin Cross Sections

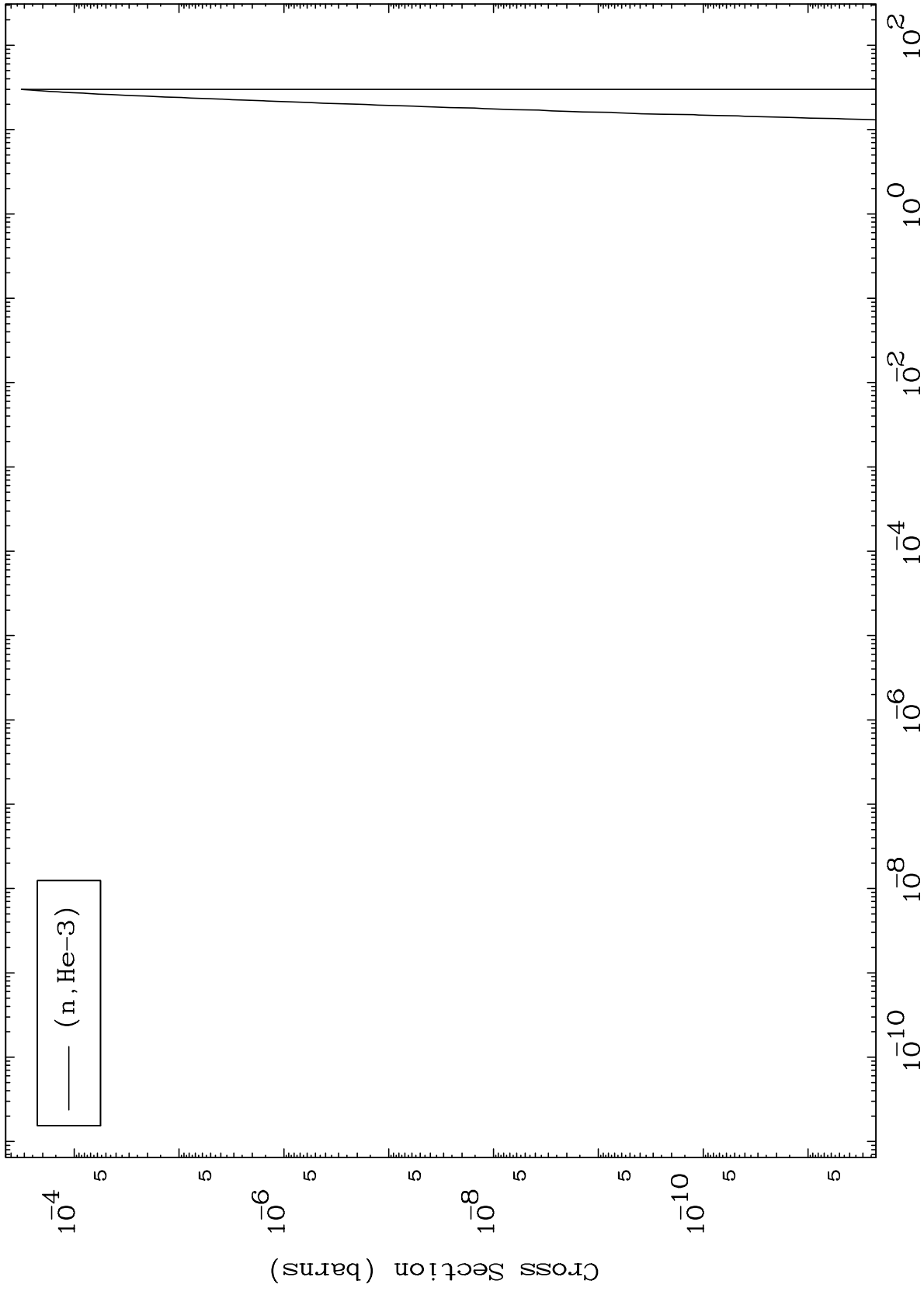
102-No-254



MAT 9972

(n,He3) Levels
293 Kelvin Cross Sections

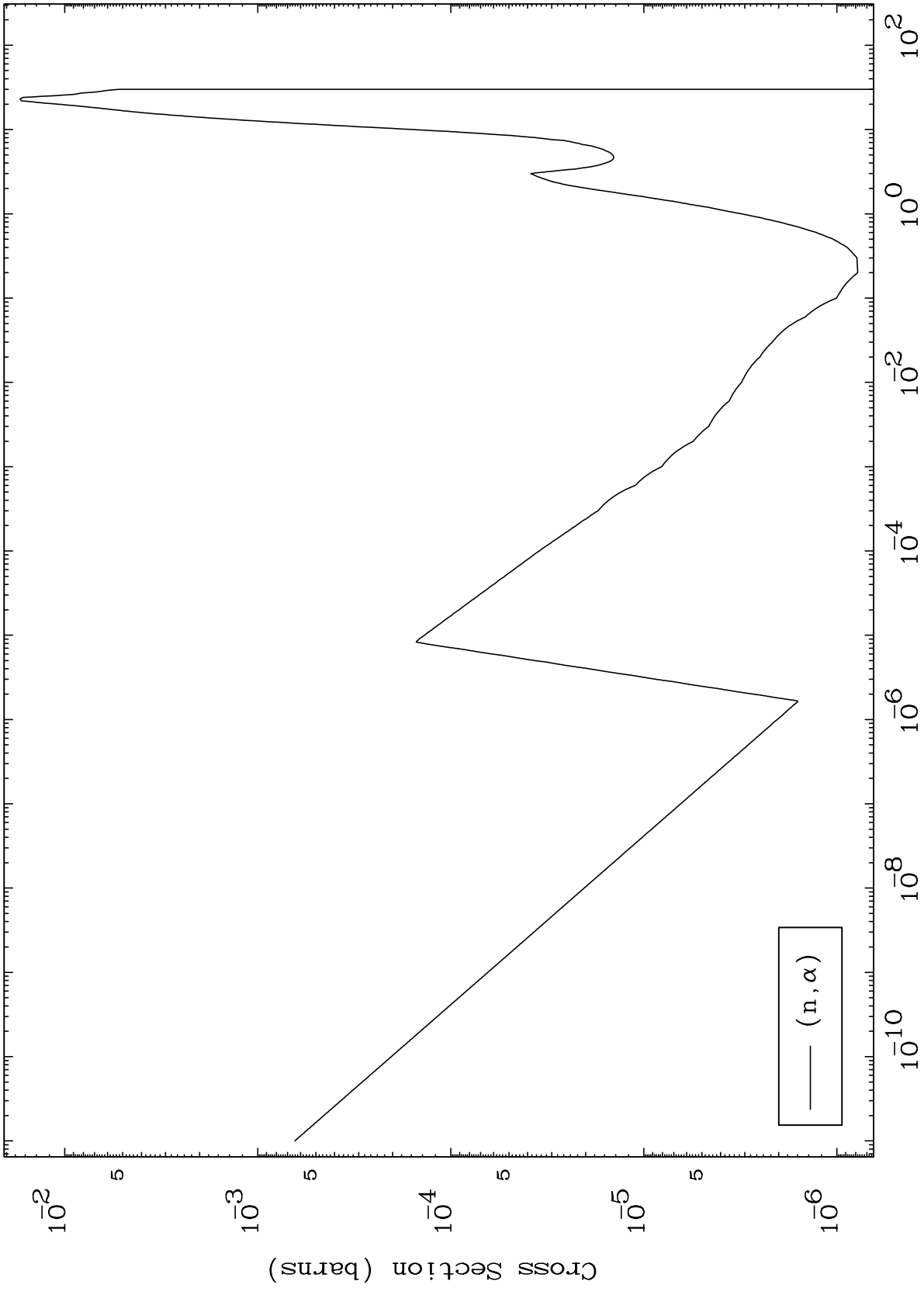
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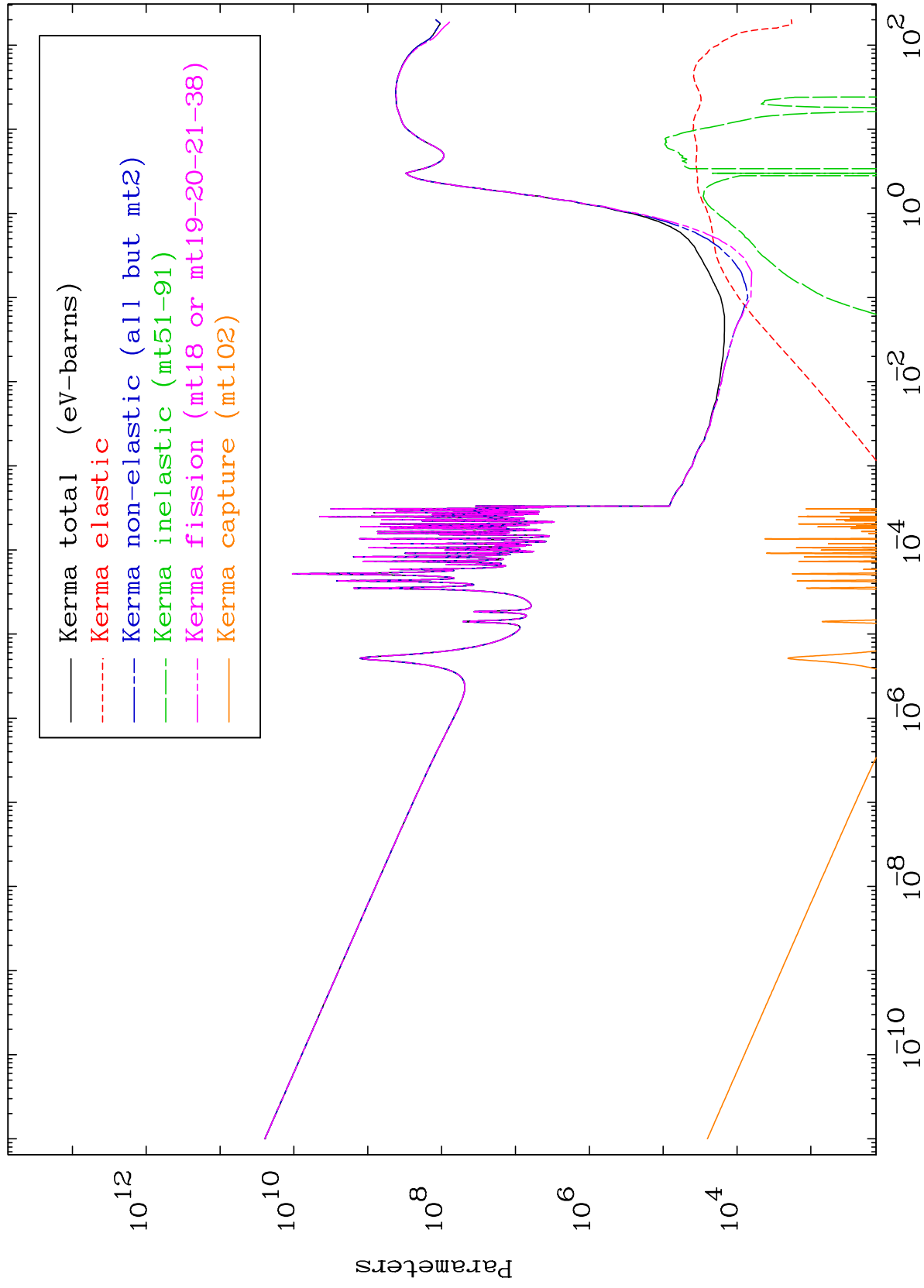


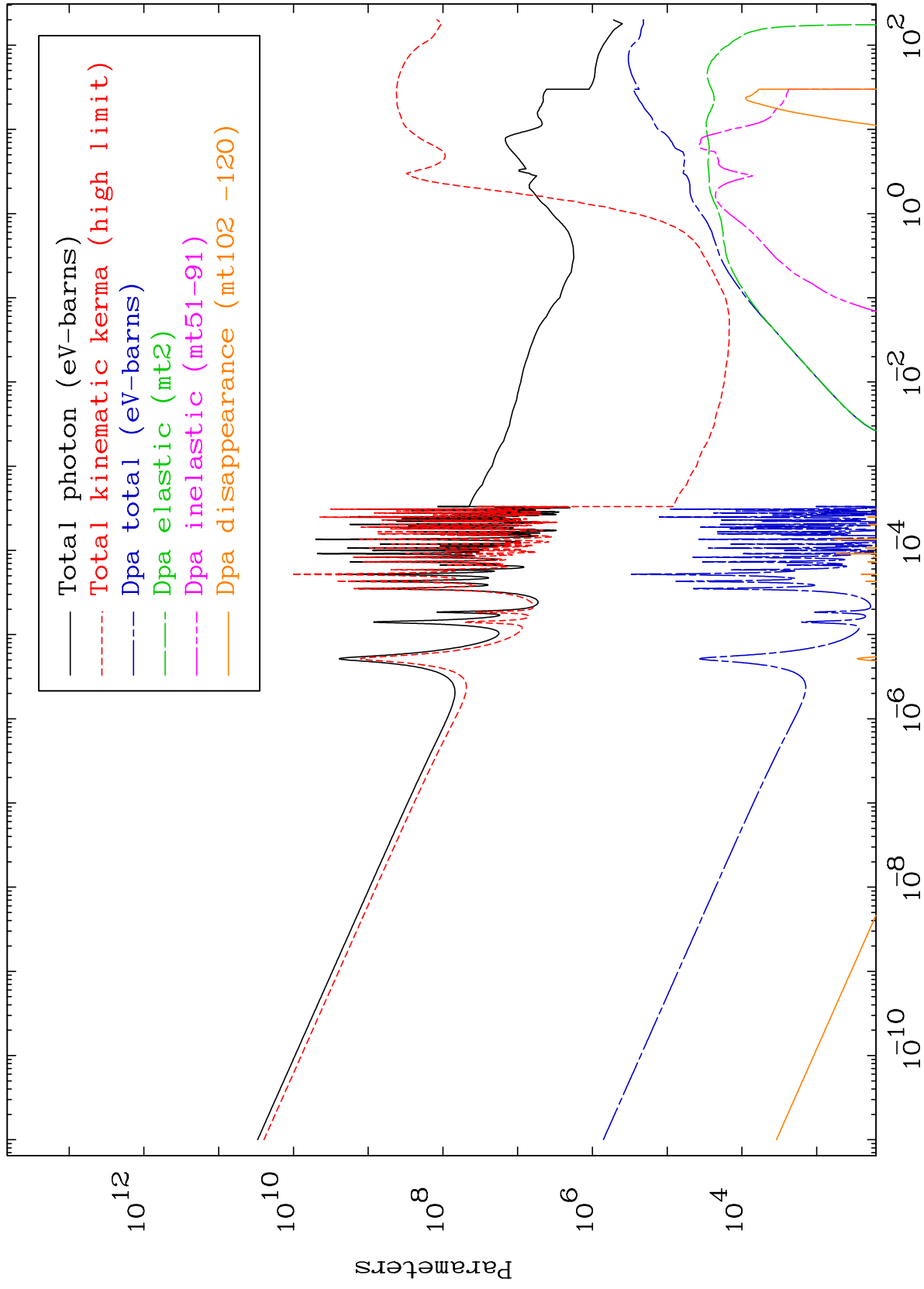
MAT 9972

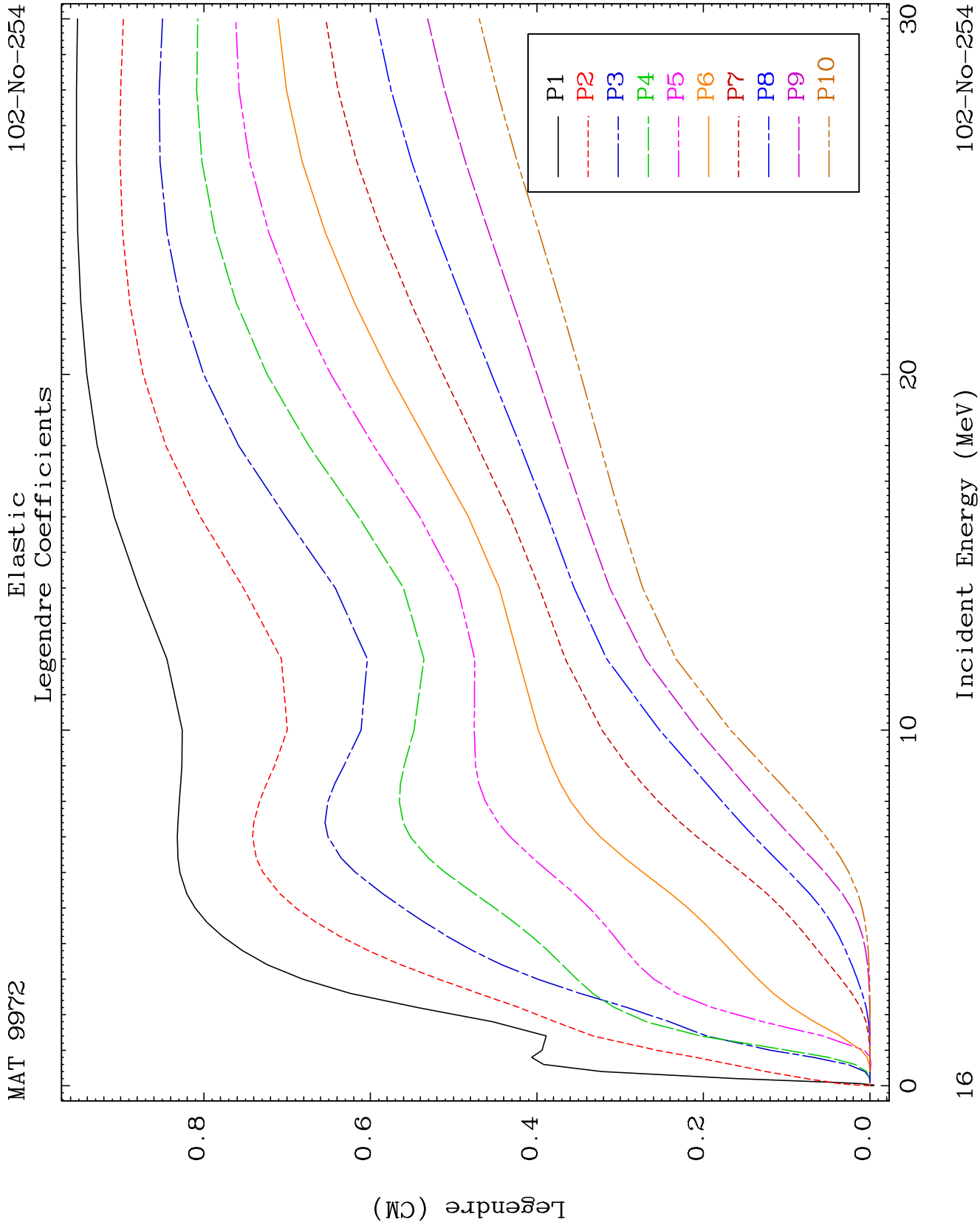
(n,α) Levels
293 Kelvin Cross Sections

102-No-254





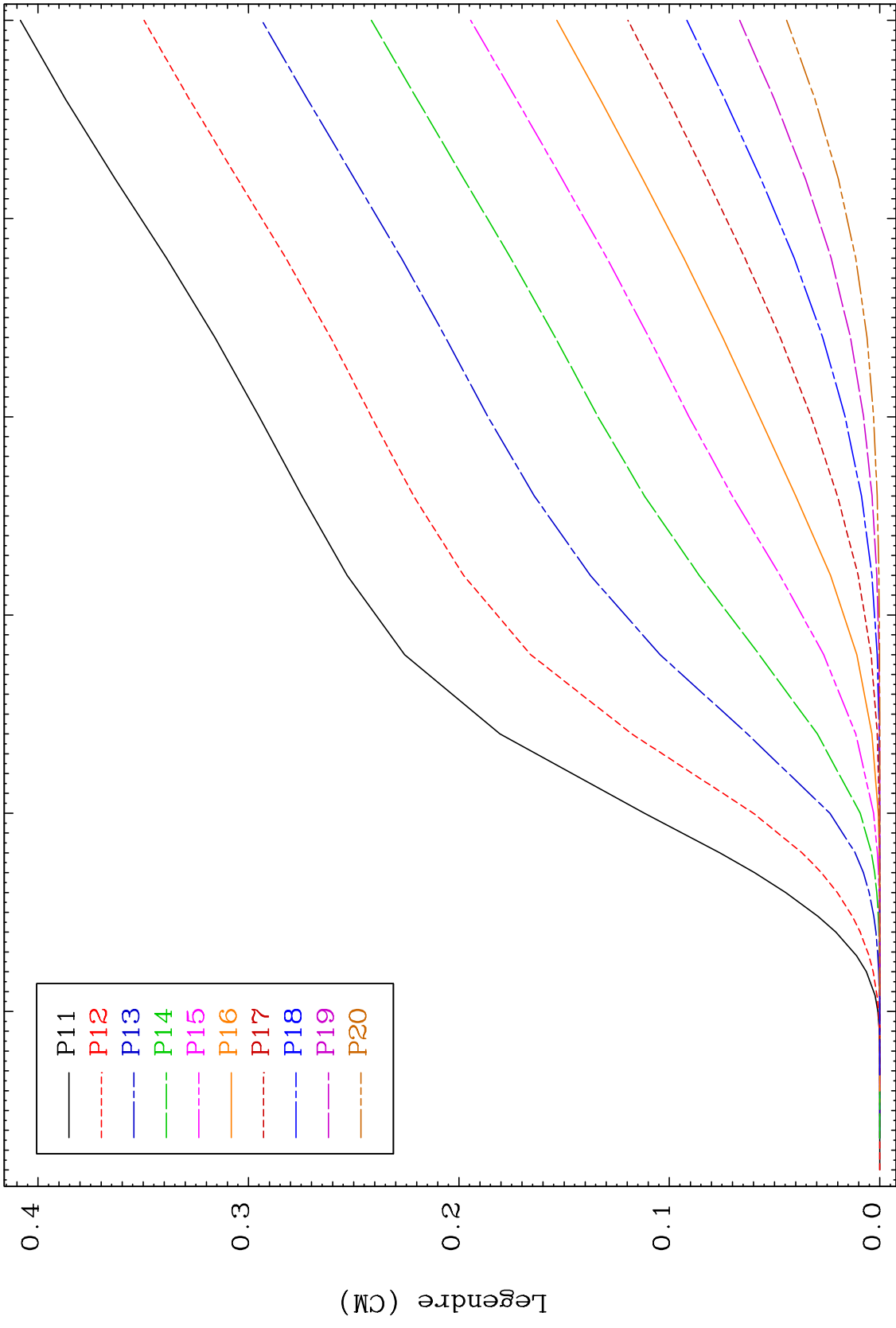




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Elastic Legendre Coefficients

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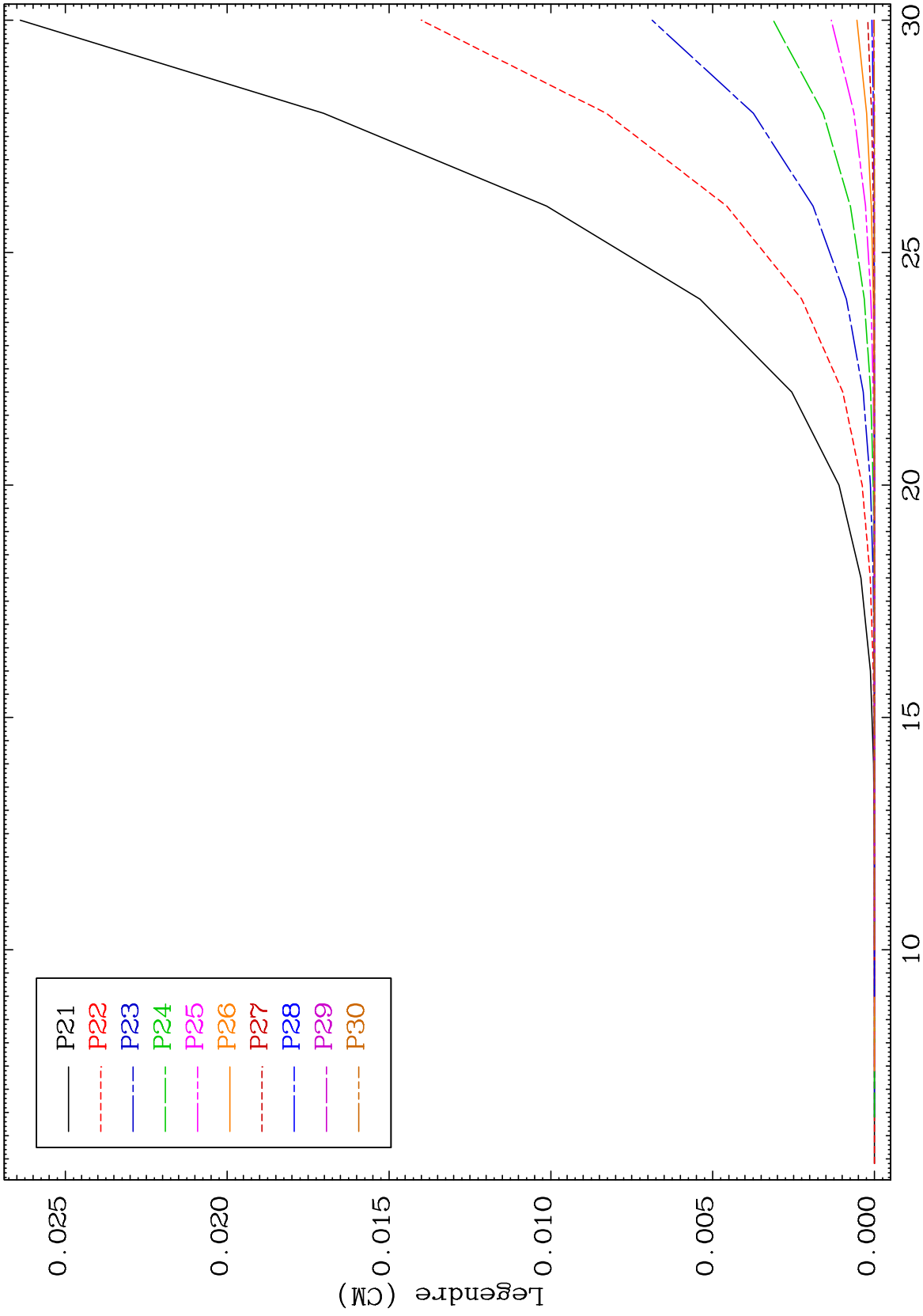
Incident Energy (MeV)

102-No-254

MAT 9972

Elastic Legendre Coefficients

102-No-254



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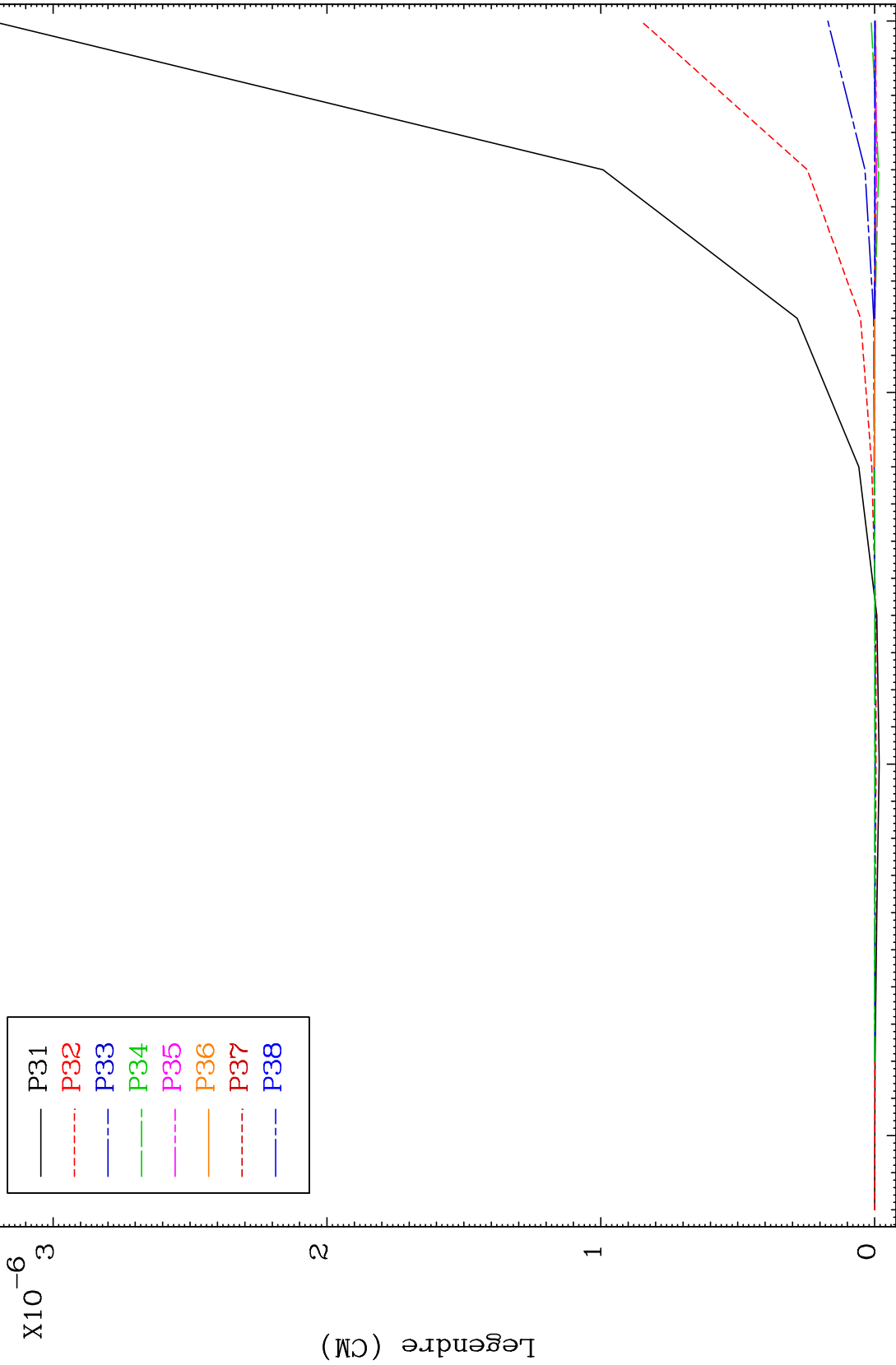
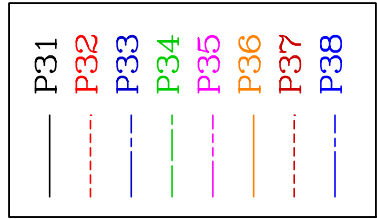
Incident Energy (MeV)

102-No-254

MAT 9972

Elastic Legendre Coefficients

102-No-254



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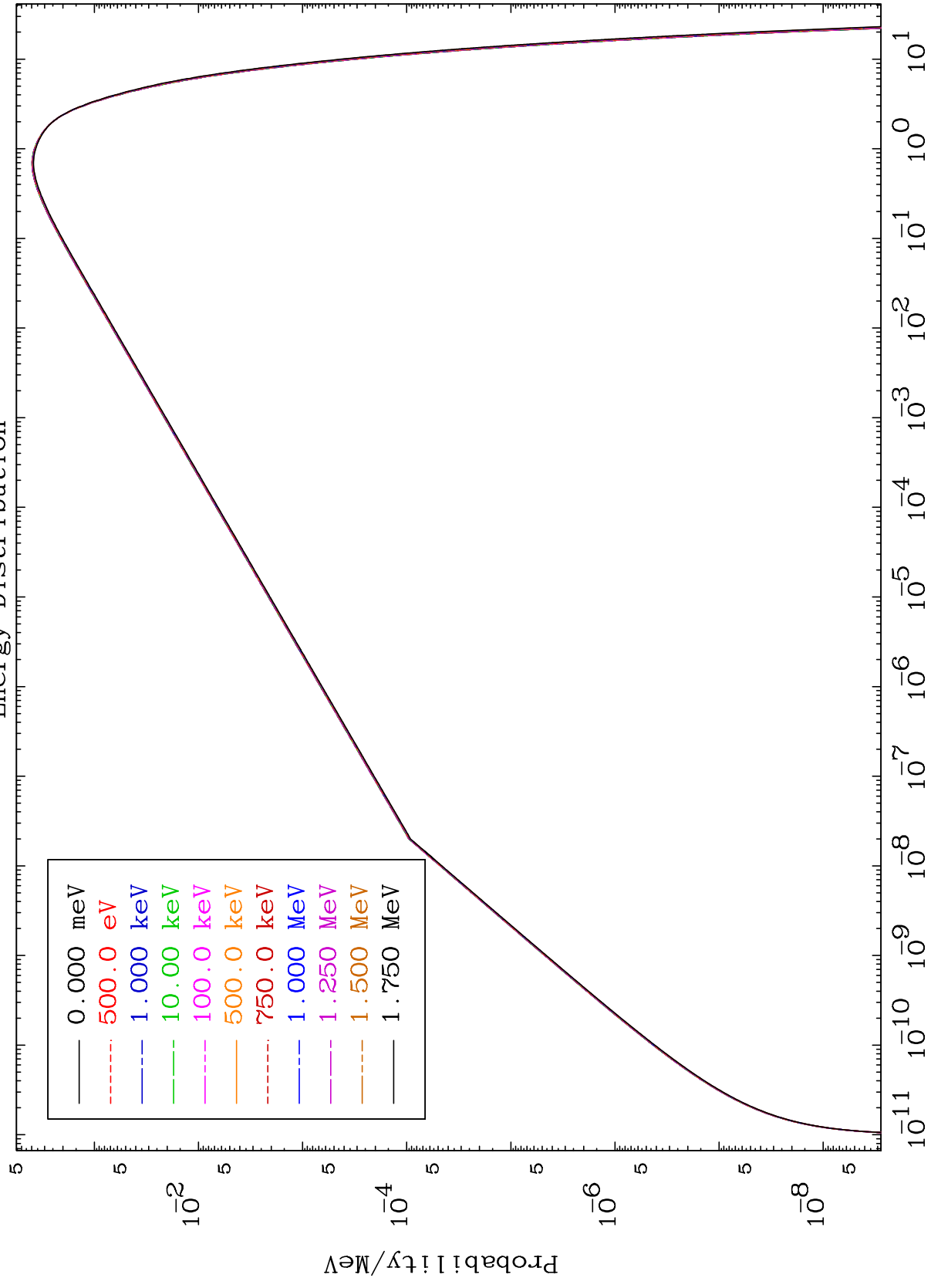
Incident Energy (MeV)

102-No-254

MAT 9972

Fission Energy Distribution

102-No-254

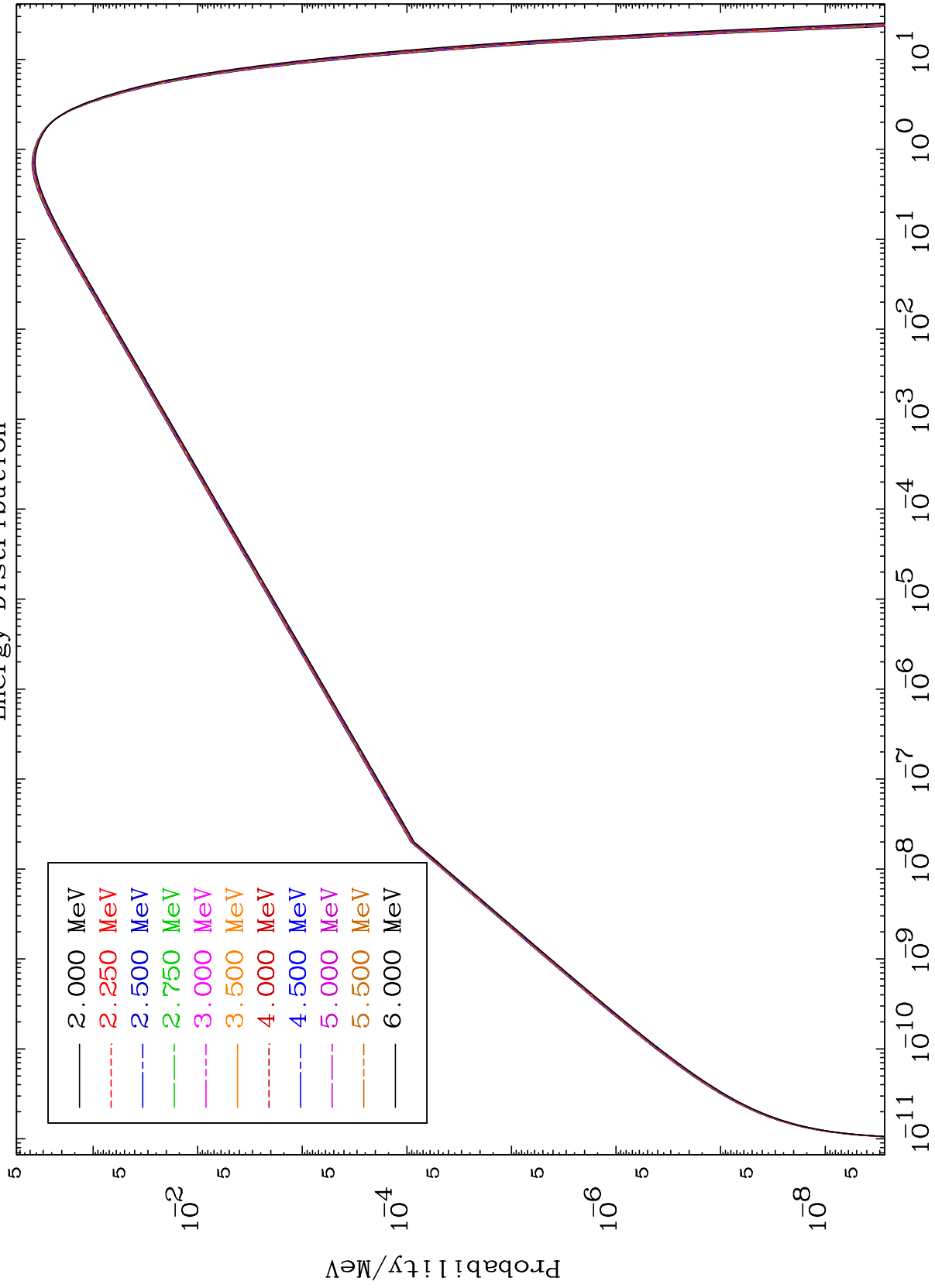


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Fission
Energy Distribution

102-No-254

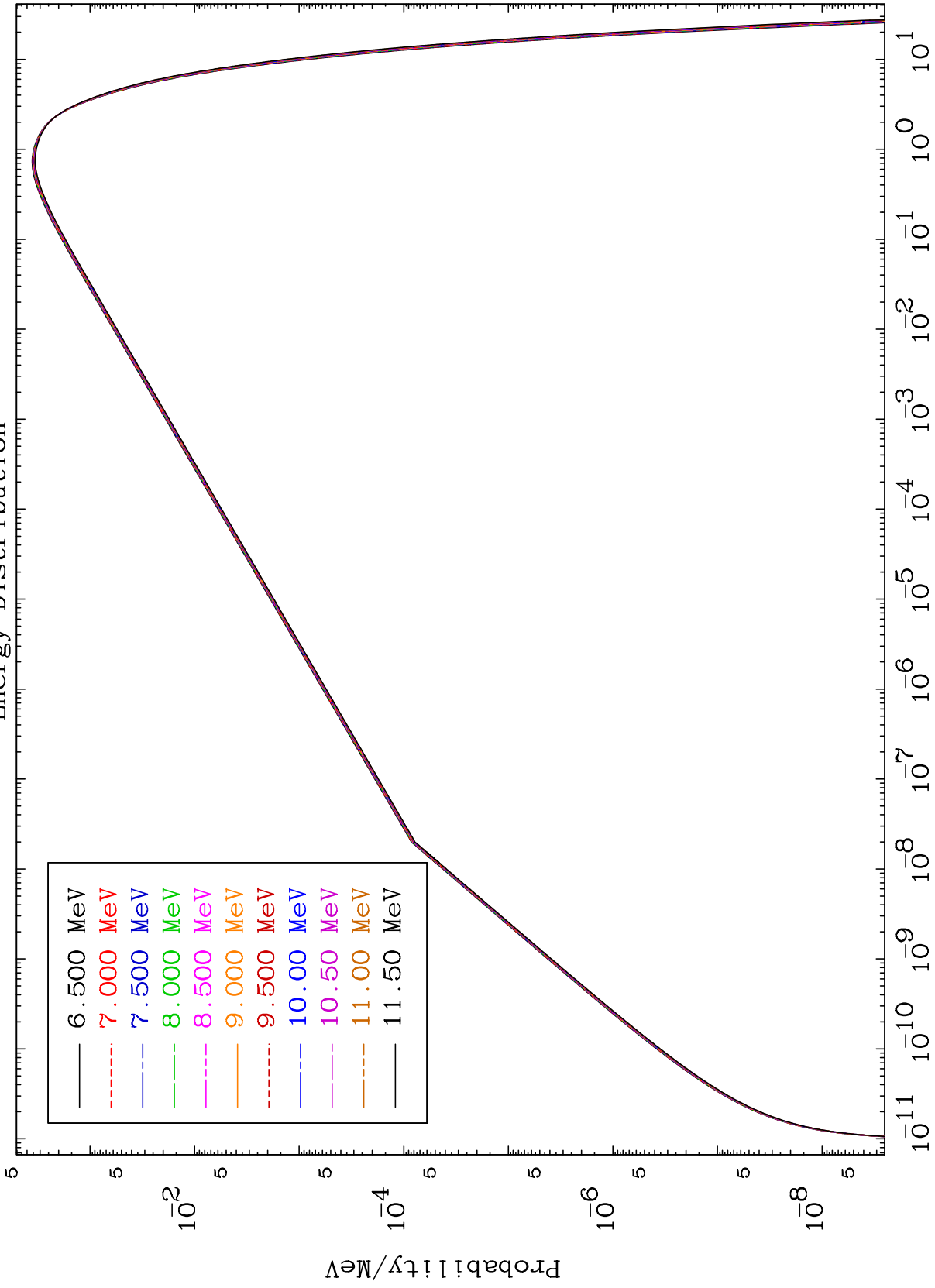


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MAT 9972

Fission Energy Distribution

102-No-254

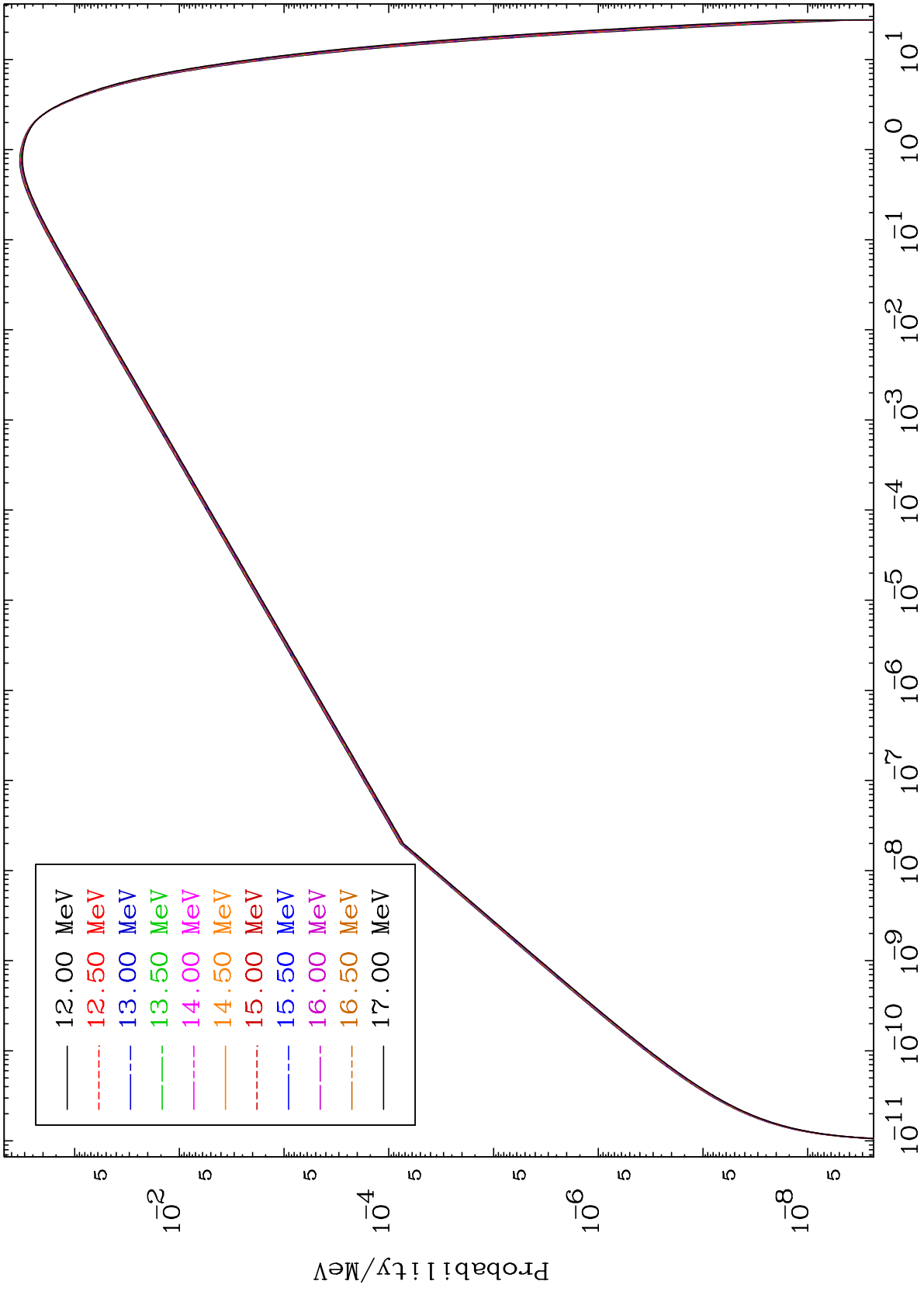


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MAT 9972

Fission Energy Distribution

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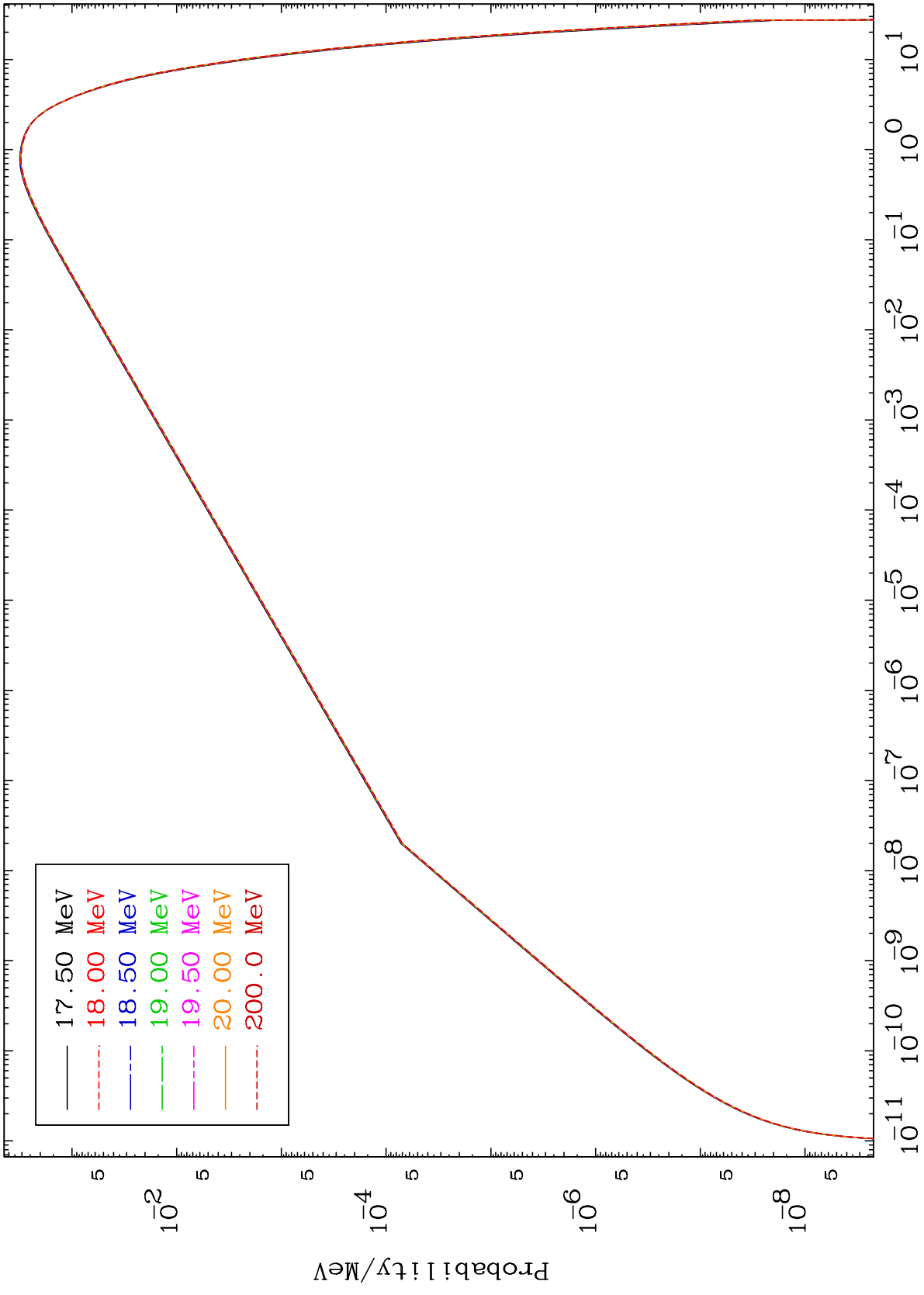


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MAT 9972

Fission Energy Distribution

102-No-254



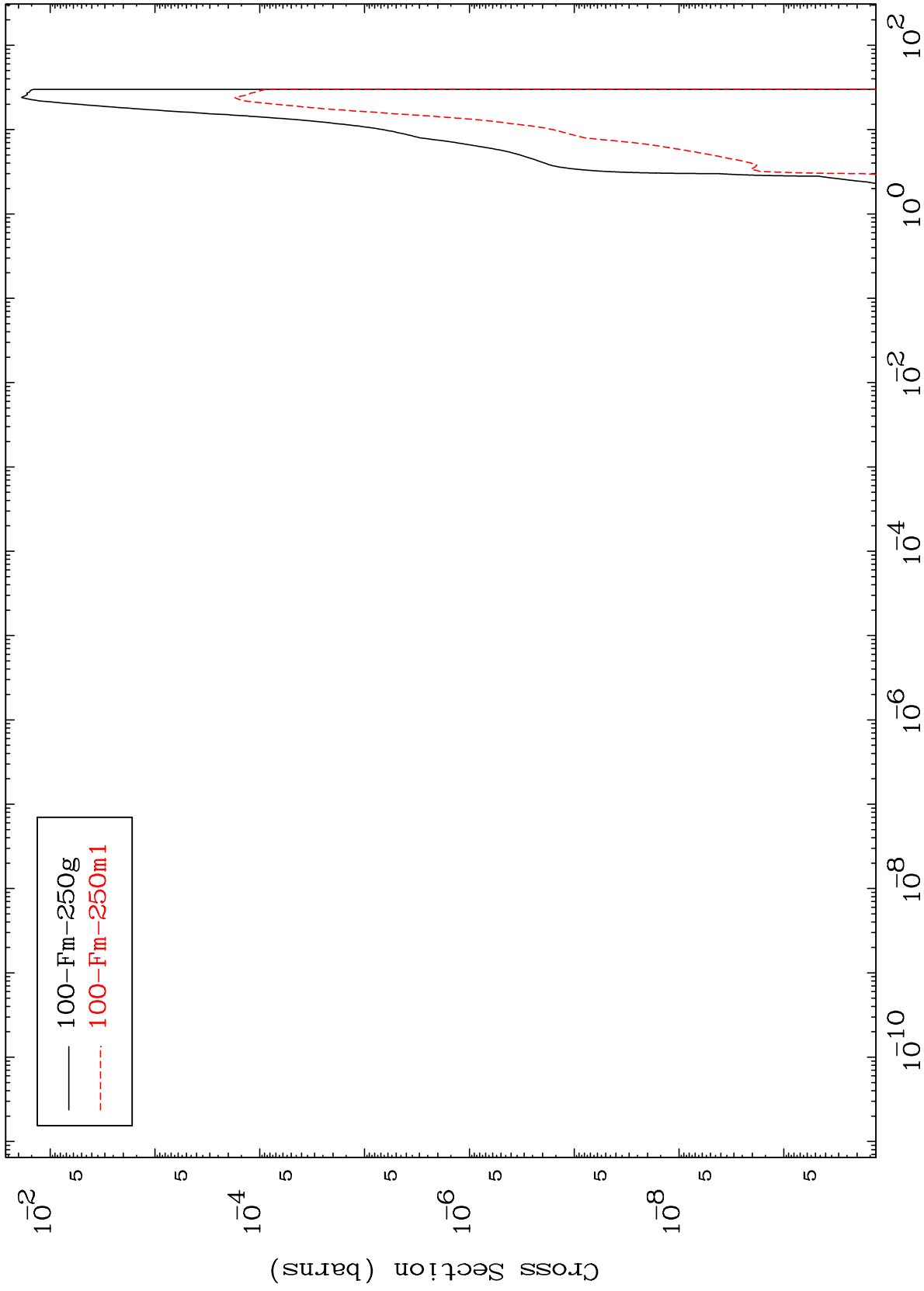
102-No-254

MAT 9972

(n,n') α

102-No-254

Radionuclide Production Cross Section



25

Incident Energy (MeV)

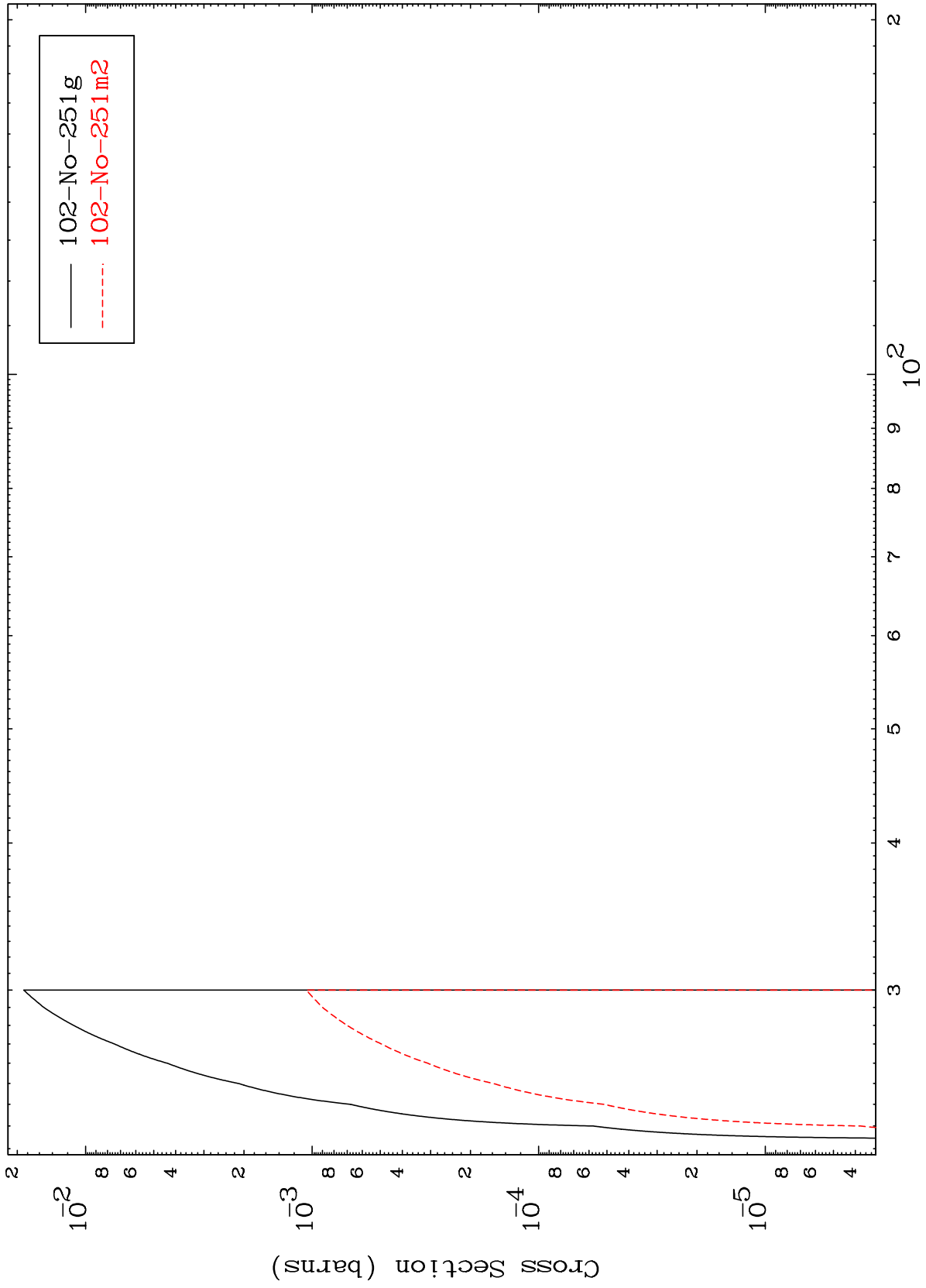
102-No-254

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(n,4n)

102-No-254

Radionuclide Production Cross Section



26

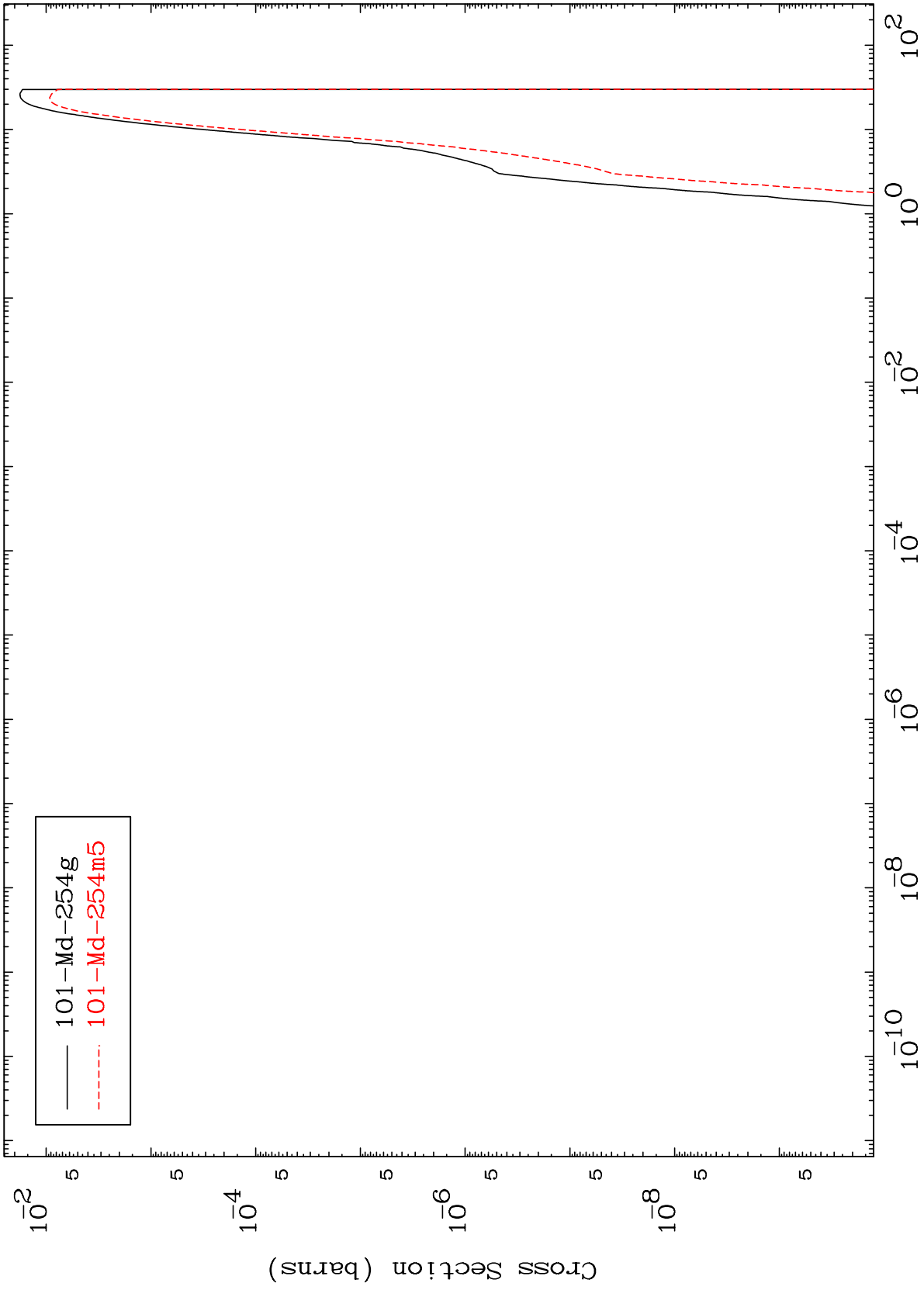
Incident Energy (MeV)

102-No-254

MAT 9972

(n,p)
Radionuclide Production Cross Section

102-No-254



MAT 9972

(n,p) α

102-No-254

Radionuclide Production Cross Section

