

Program EVALPLOT
(Version 2018-1)

by

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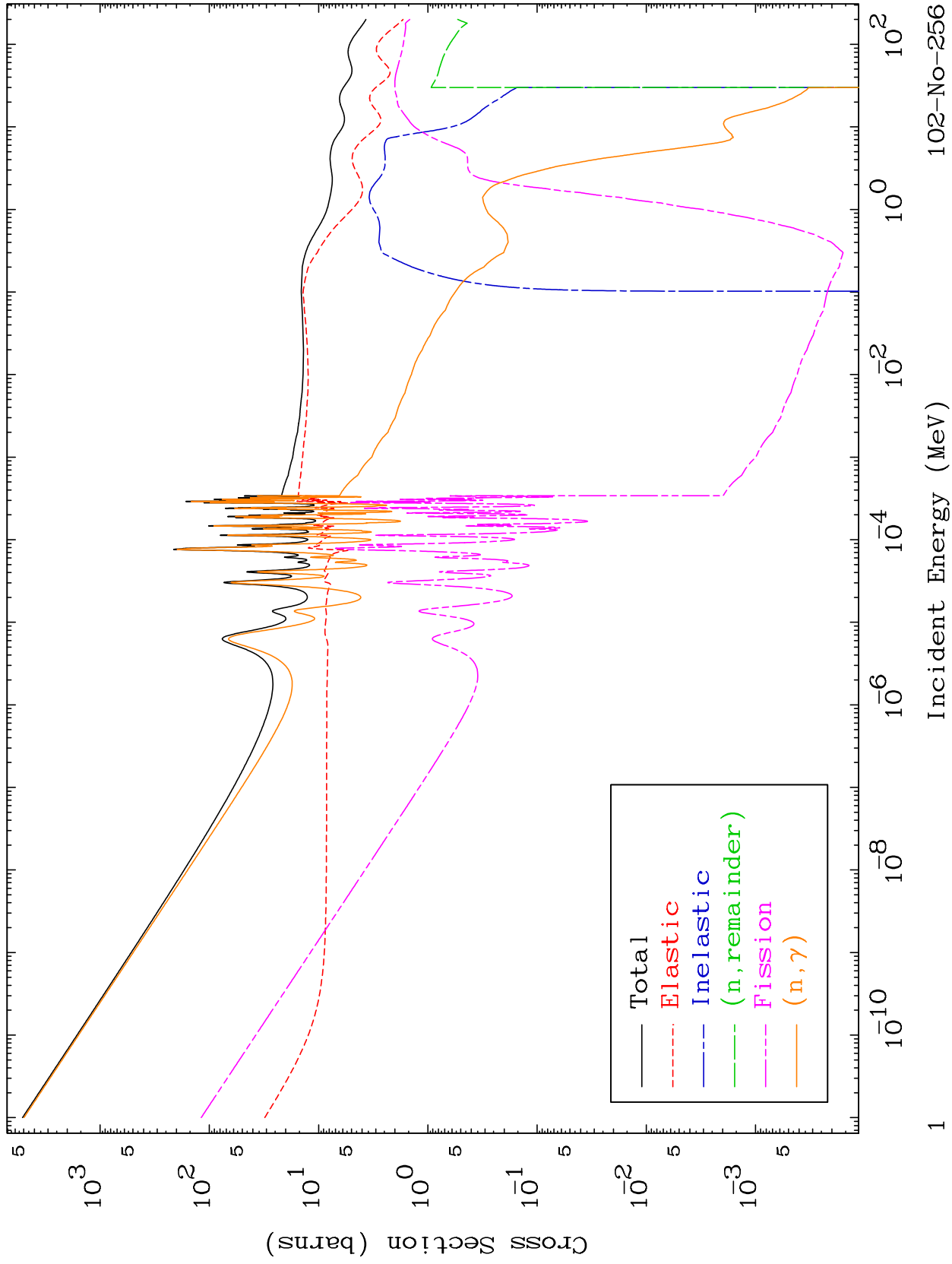
Press Mouse Button to Start

MAT 9974

Major

293 Kelvin Cross Sections

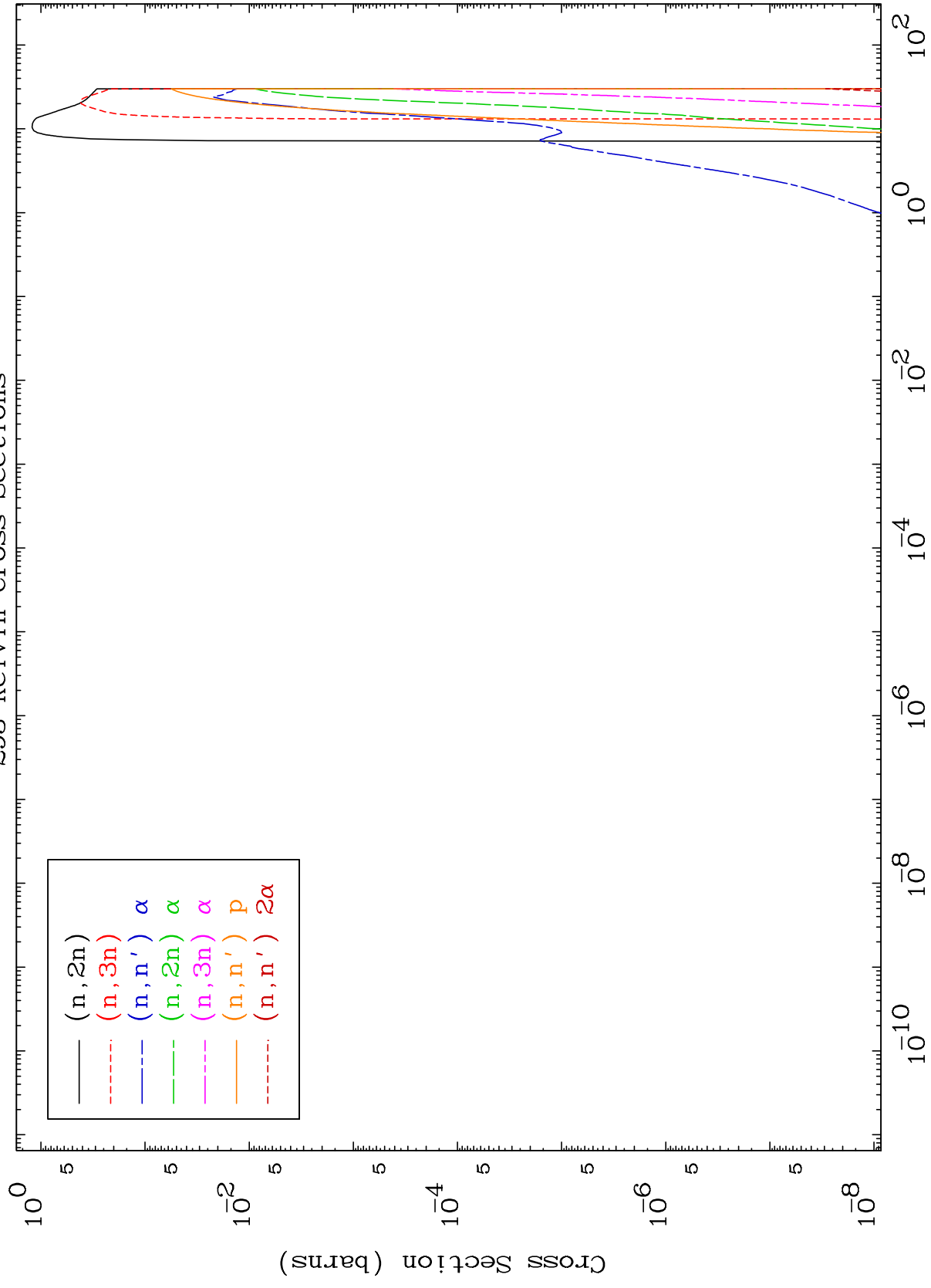
102-No-256

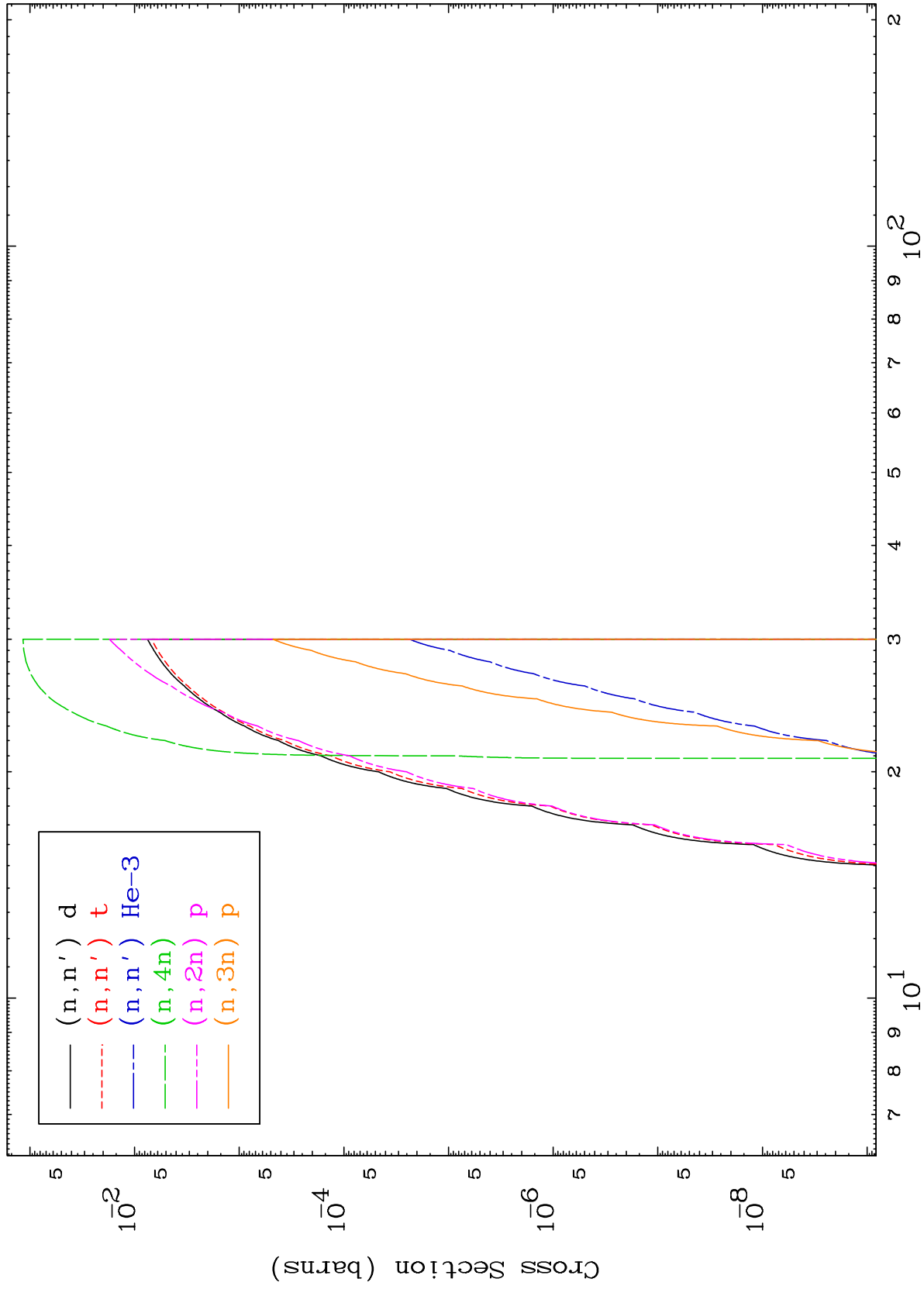


MAT 9974

Neutron Production
293 Kelvin Cross Sections

102-No-256

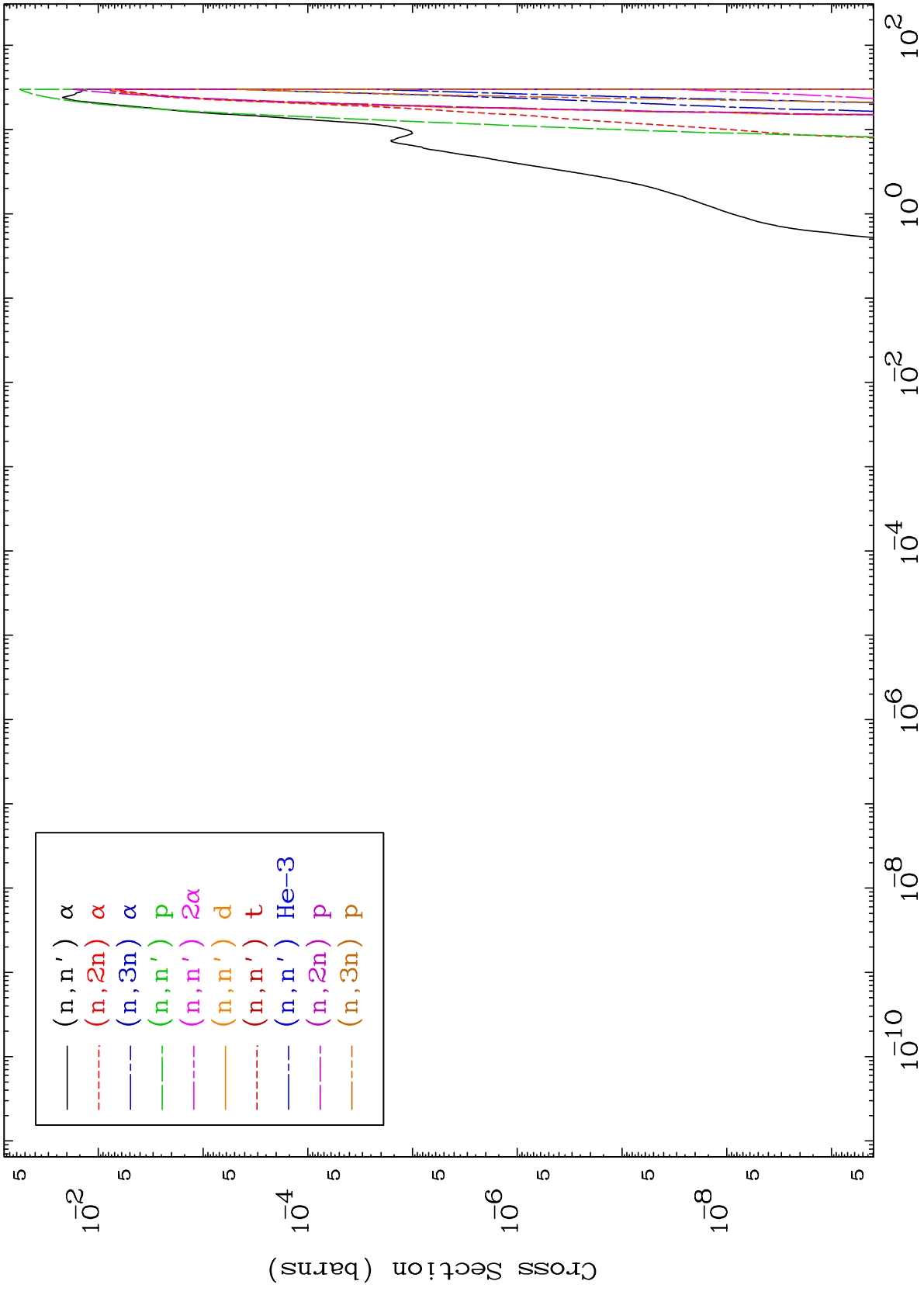




MAT 9974

Charged Particle
293 Kelvin Cross Sections

102-No-256

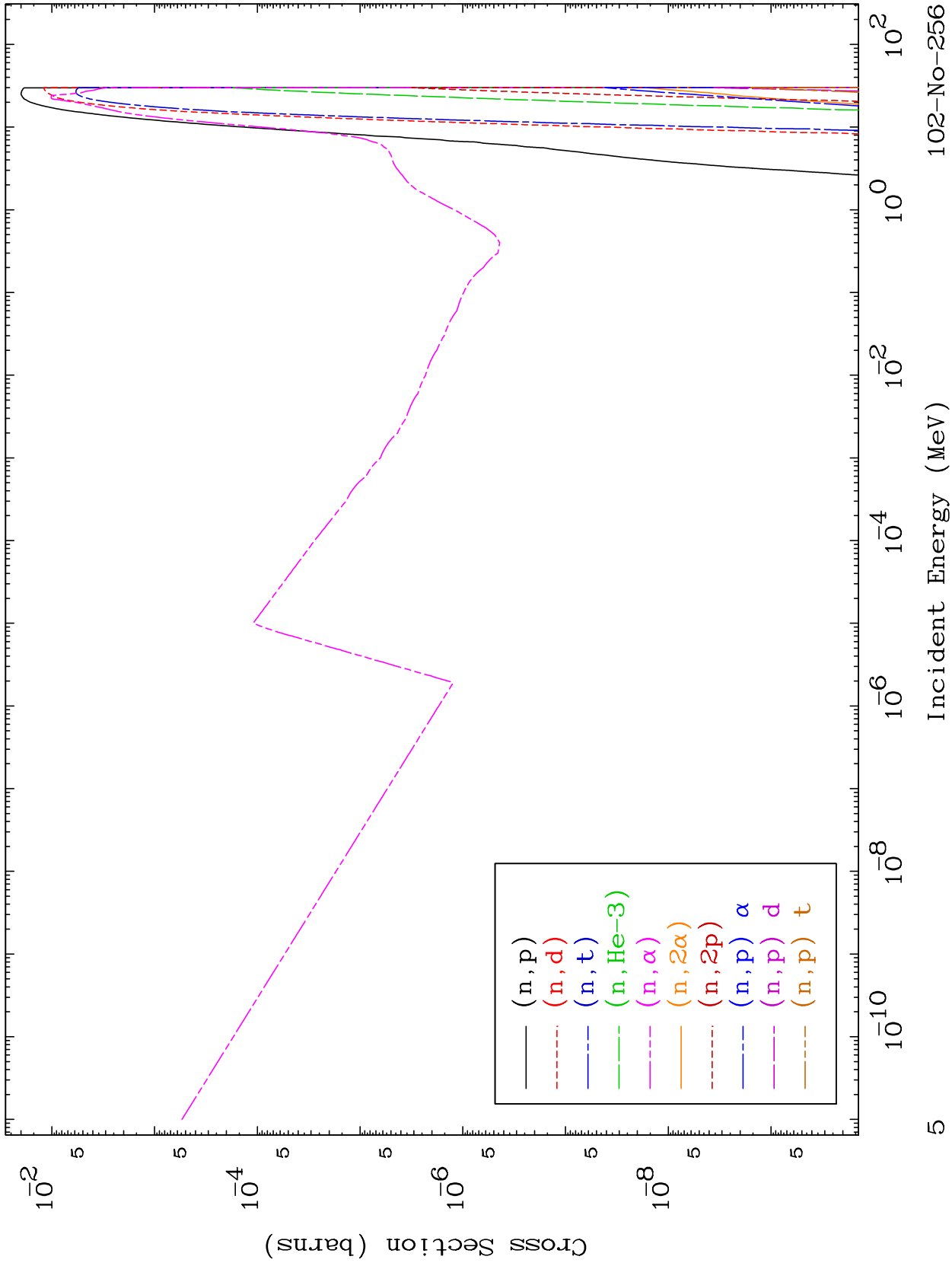


102-No-256

MAT 9974

Charged Particle
293 Kelvin Cross Sections

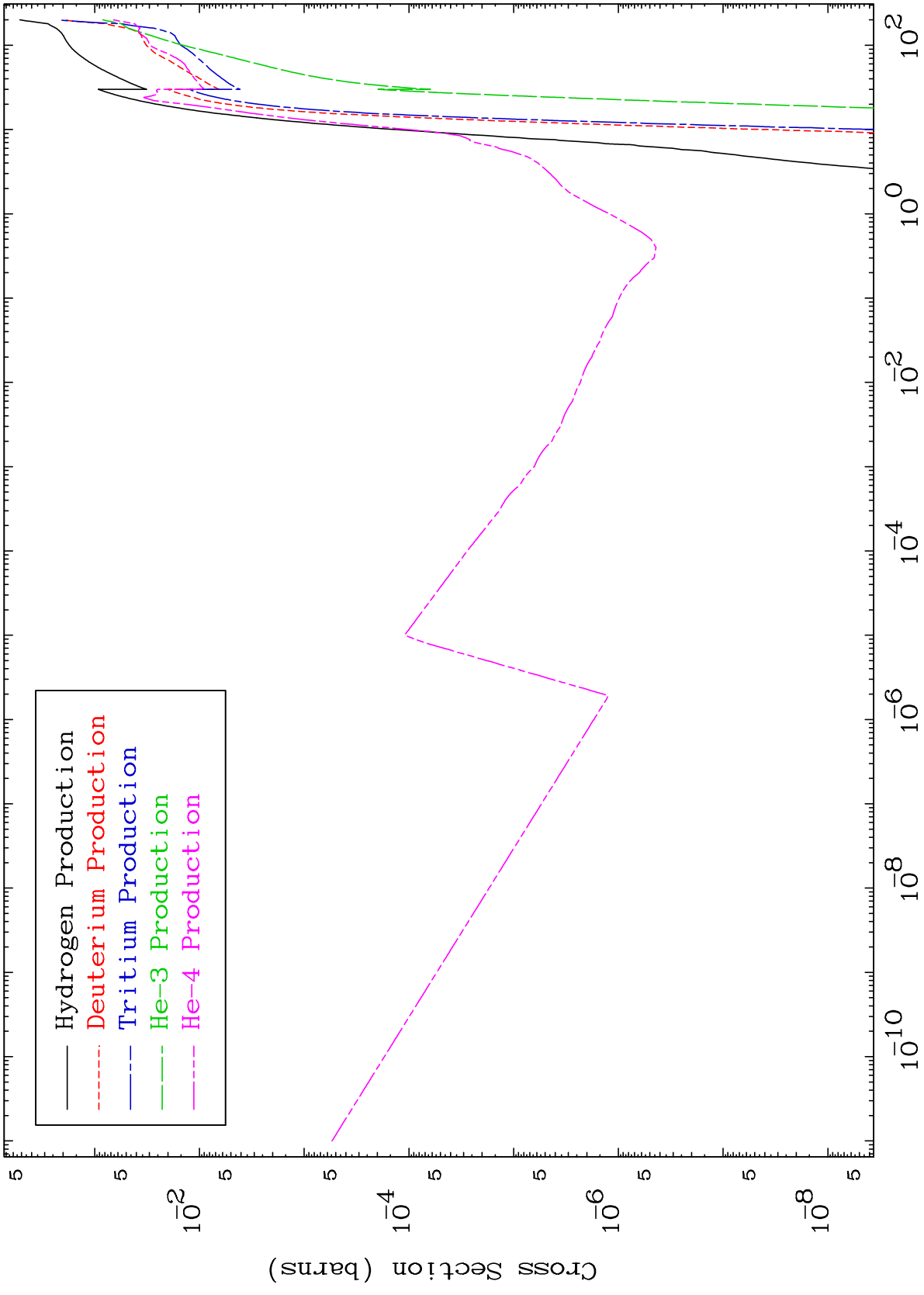
102-No-256



MAT 9974

Particle Production
293 Kelvin Cross Sections

102-No-256



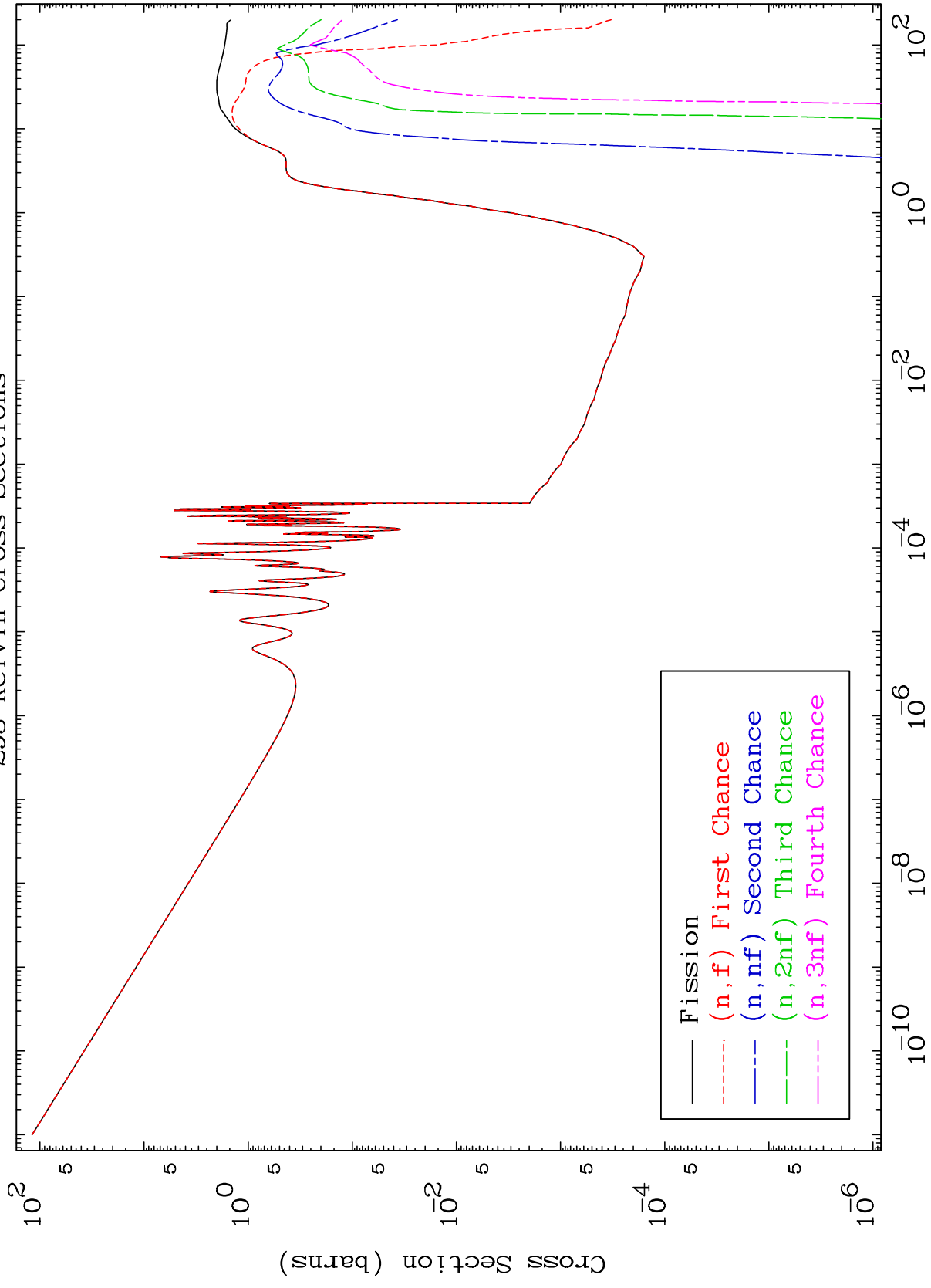
6

102-No-256

MAT 9974

Fission
293 Kelvin Cross Sections

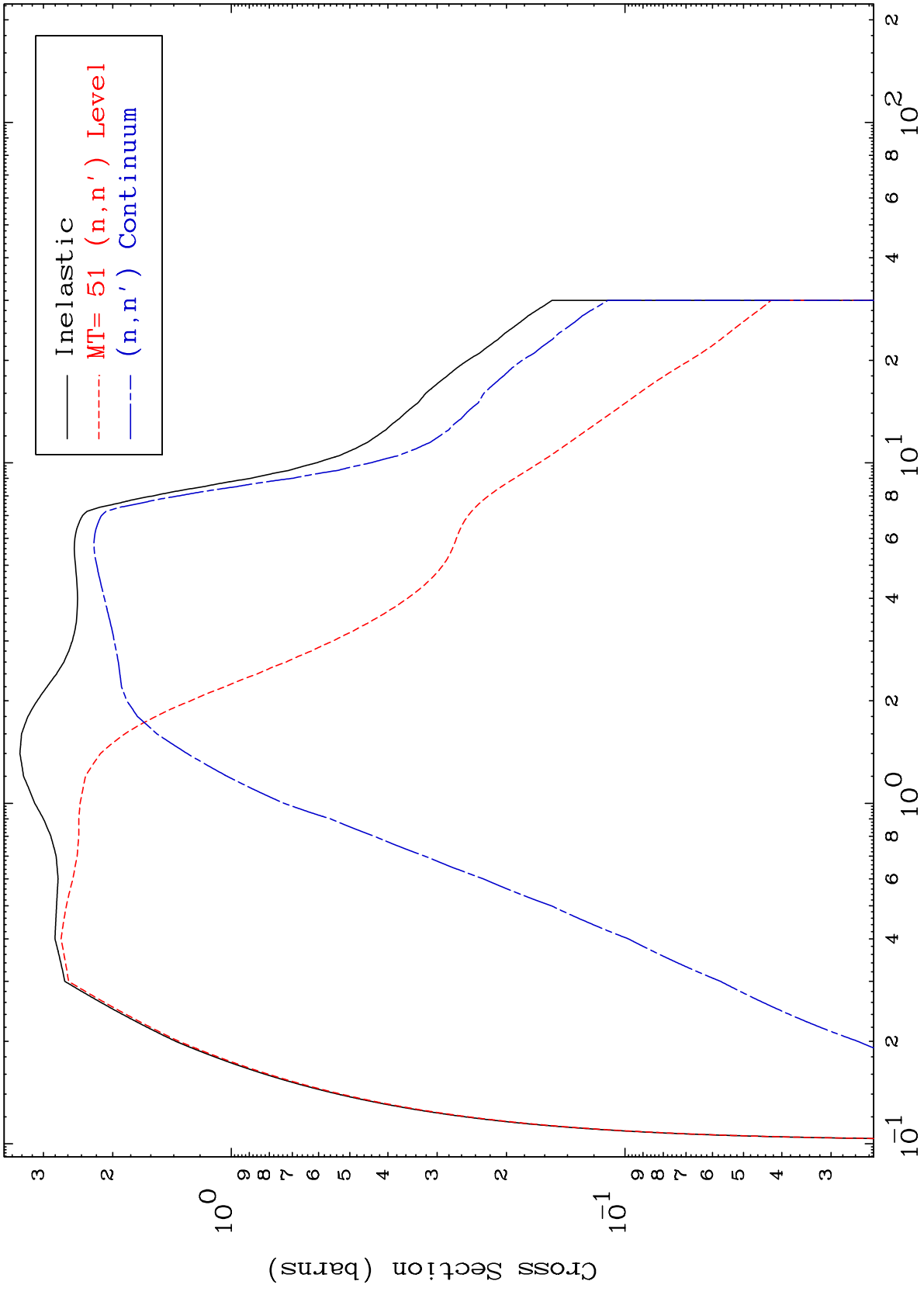
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MAT 9974

(n,n') Level
293 Kelvin Cross Sections

102-No-256



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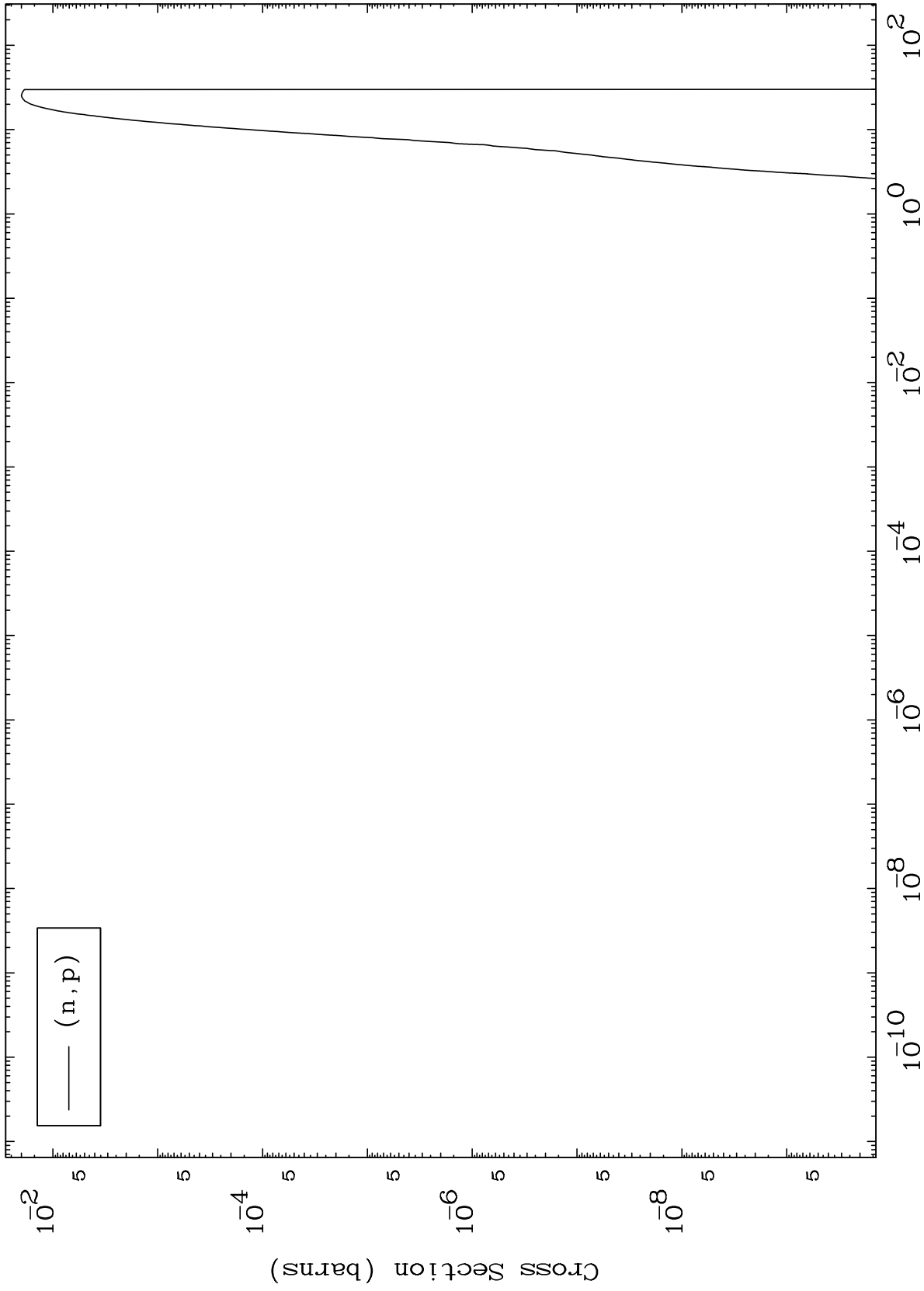
Incident Energy (MeV)

102-No-256

MAT 9974

(n,p) Levels
293 Kelvin Cross Sections

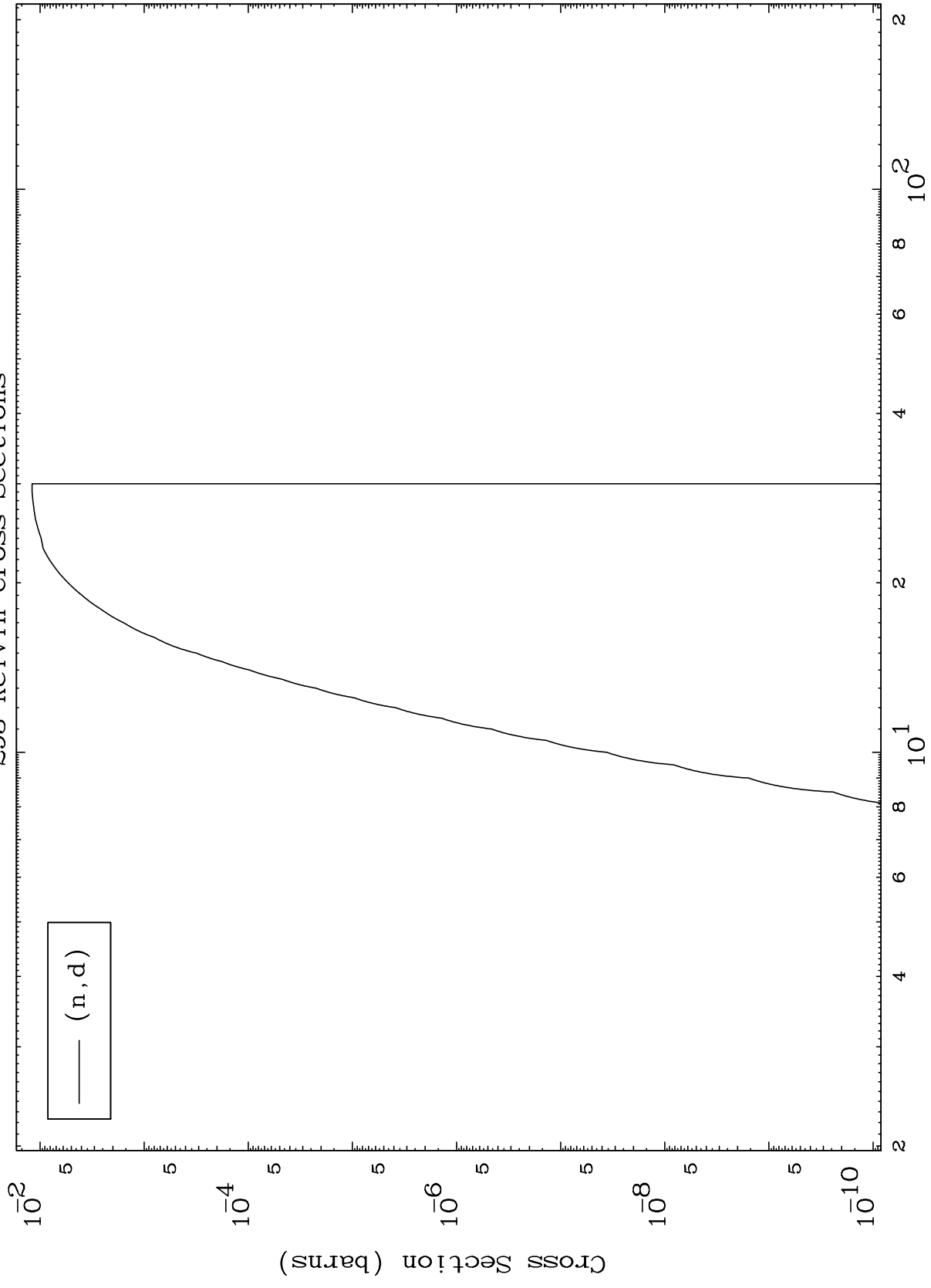
102-No-256



MAT 9974

(n,d) Levels
293 Kelvin Cross Sections

102-No-256



10

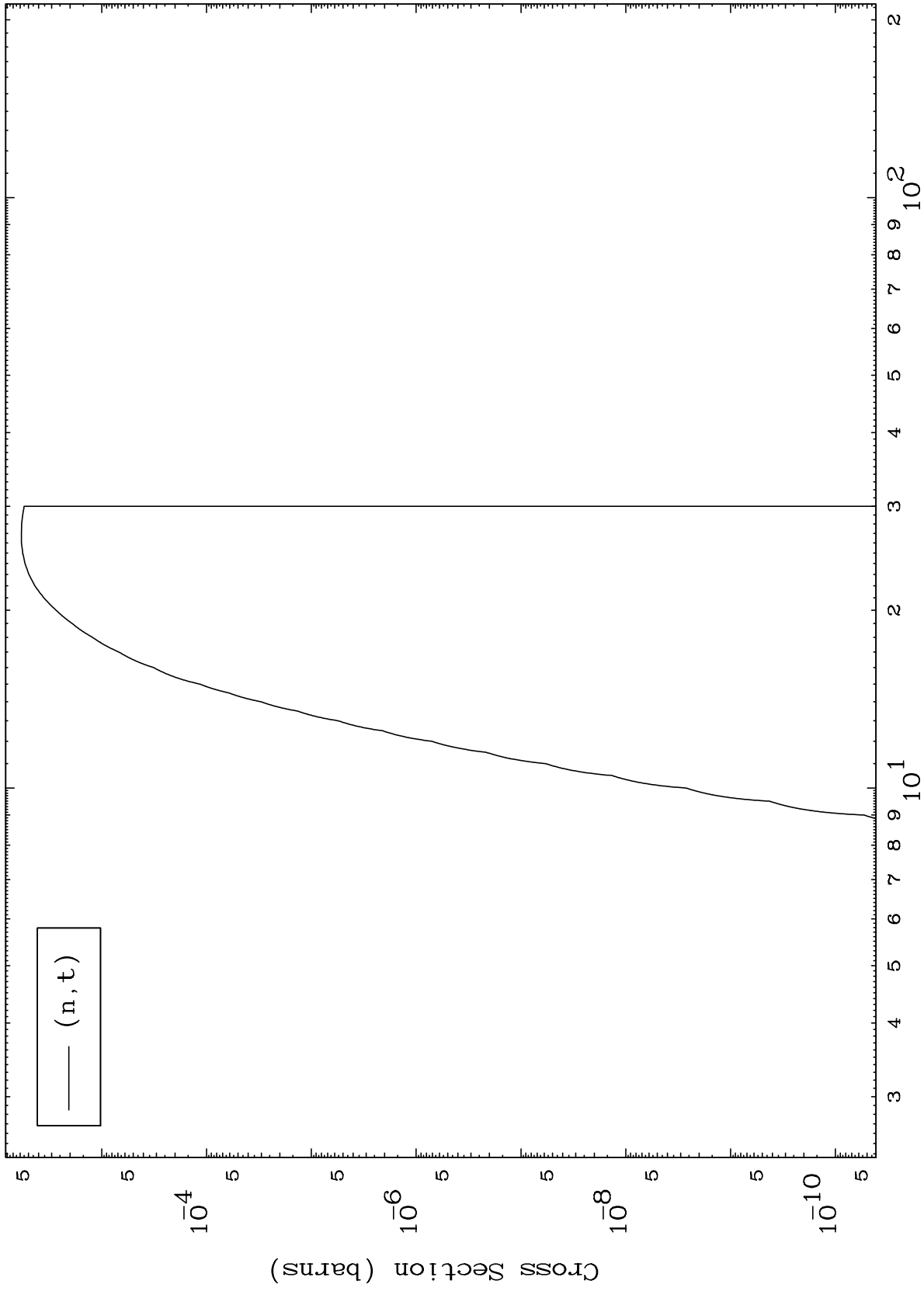
Incident Energy (MeV)

102-No-256

MAT 9974

(n,t) Levels
293 Kelvin Cross Sections

102-No-256



11

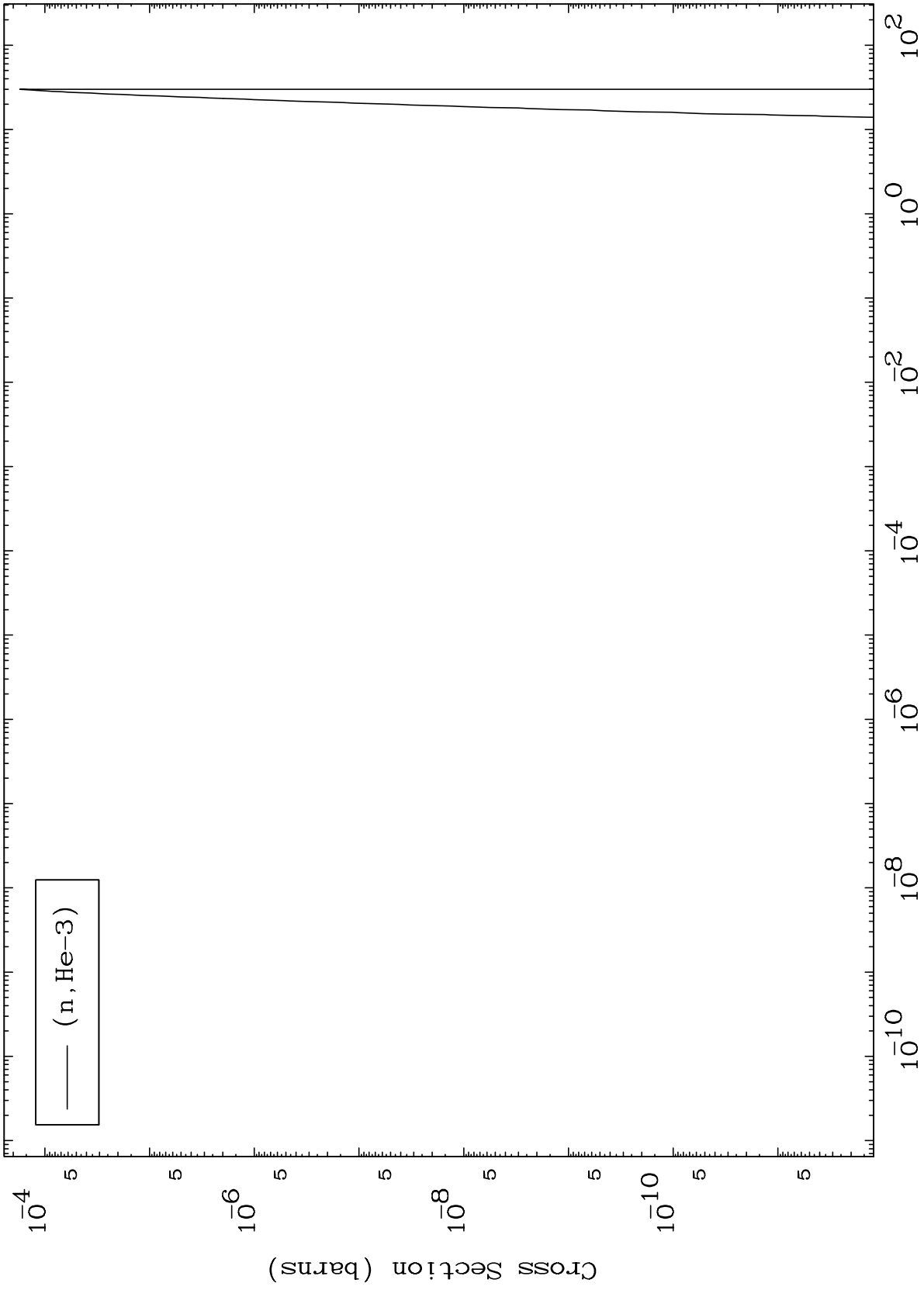
Incident Energy (MeV)

102-No-256

MAT 9974

(n,He3) Levels
293 Kelvin Cross Sections

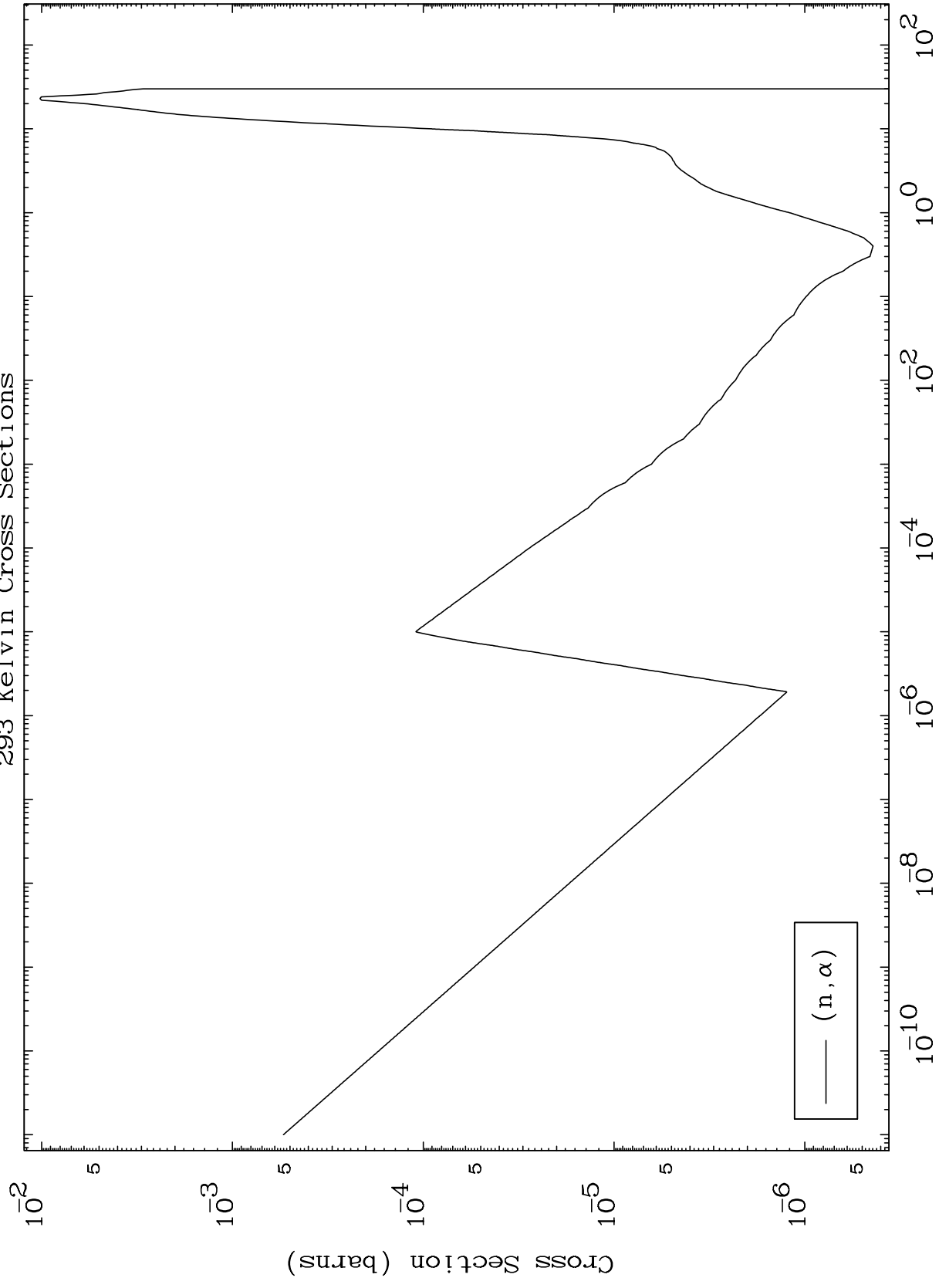
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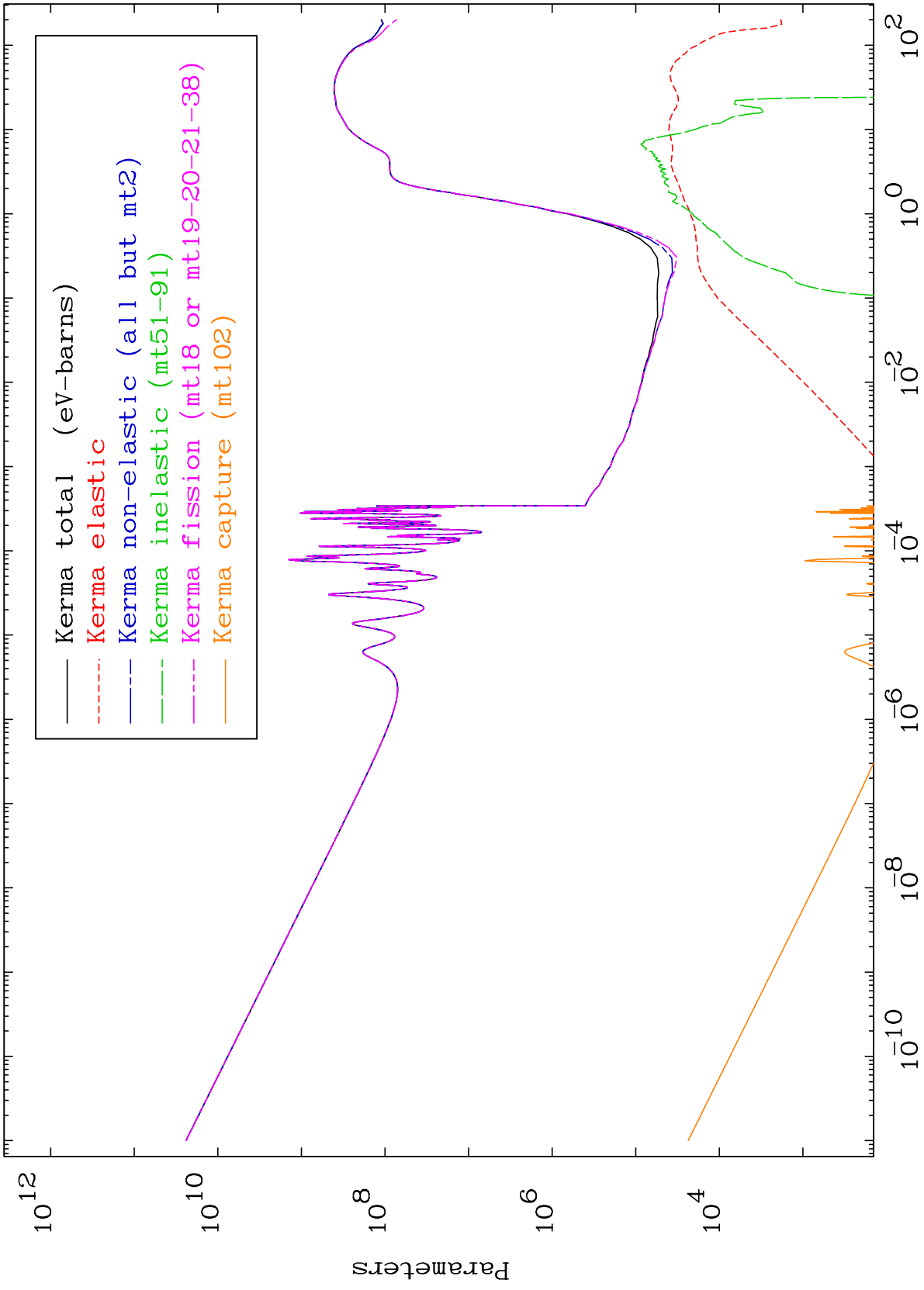


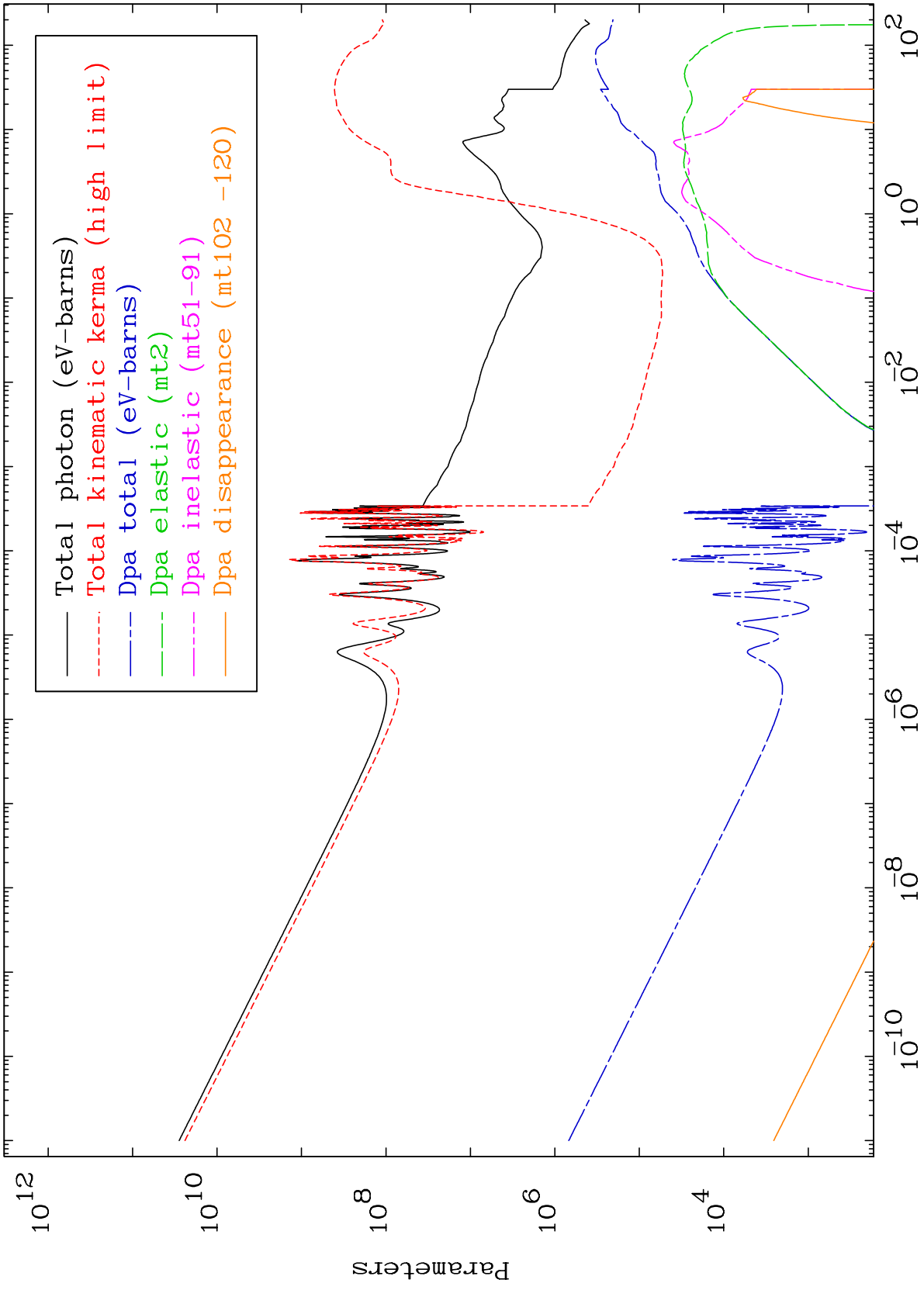
MAT 9974

(n,α) Levels
293 Kelvin Cross Sections

102-No-256



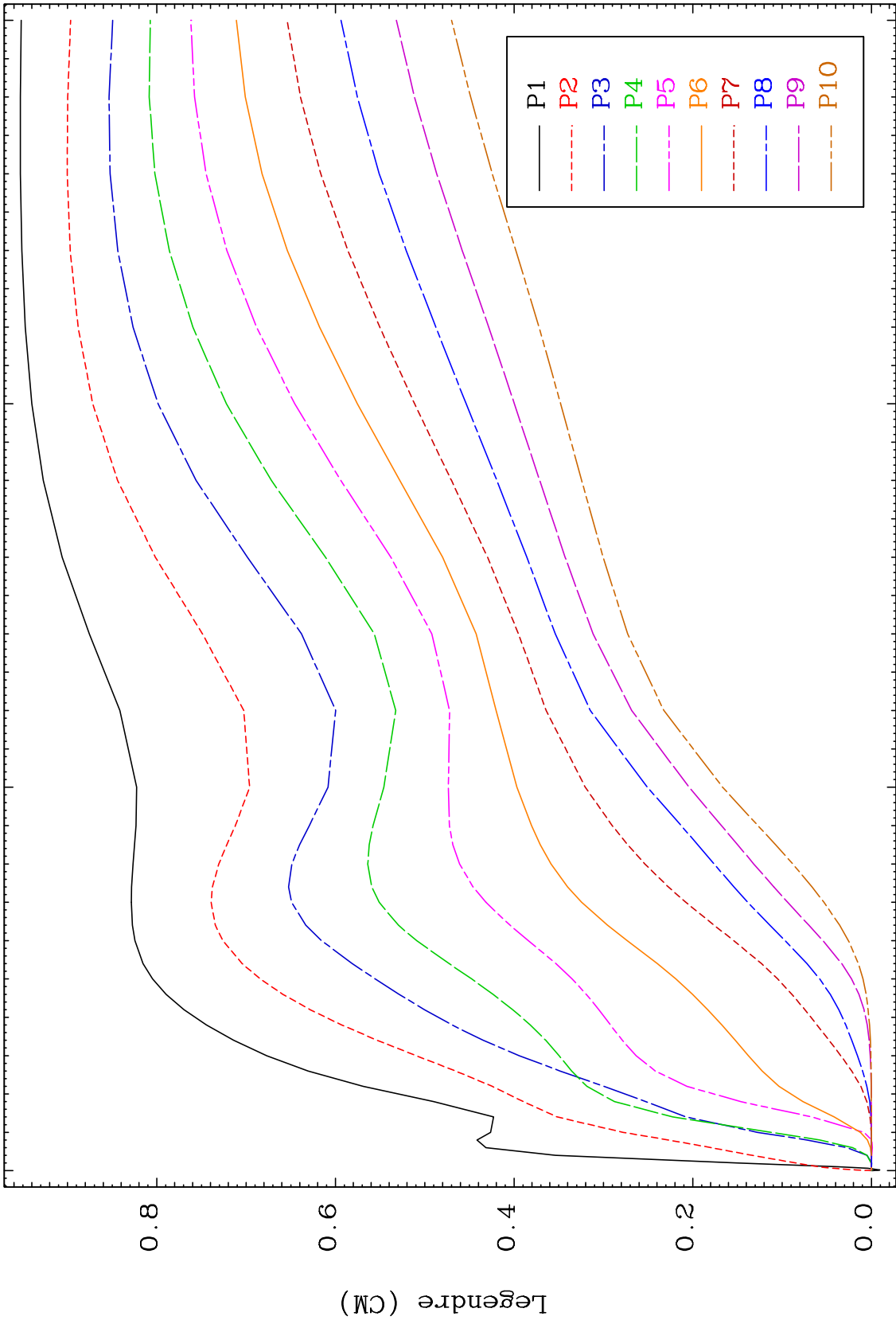




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Elastic Legendre Coefficients

102-No-256



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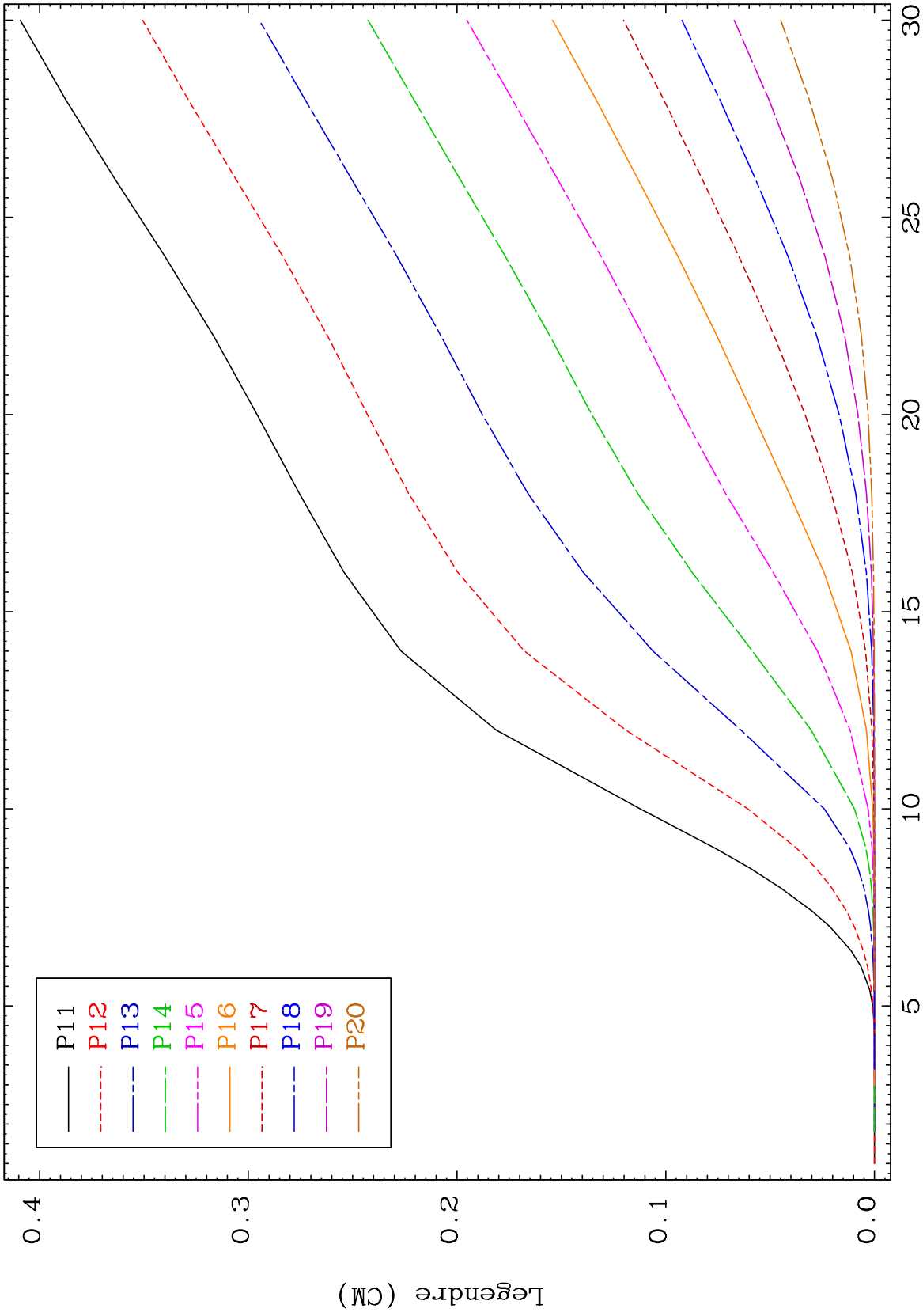
Incident Energy (MeV)

102-No-256

MAT 9974

Elastic Legendre Coefficients

102-No-256



17

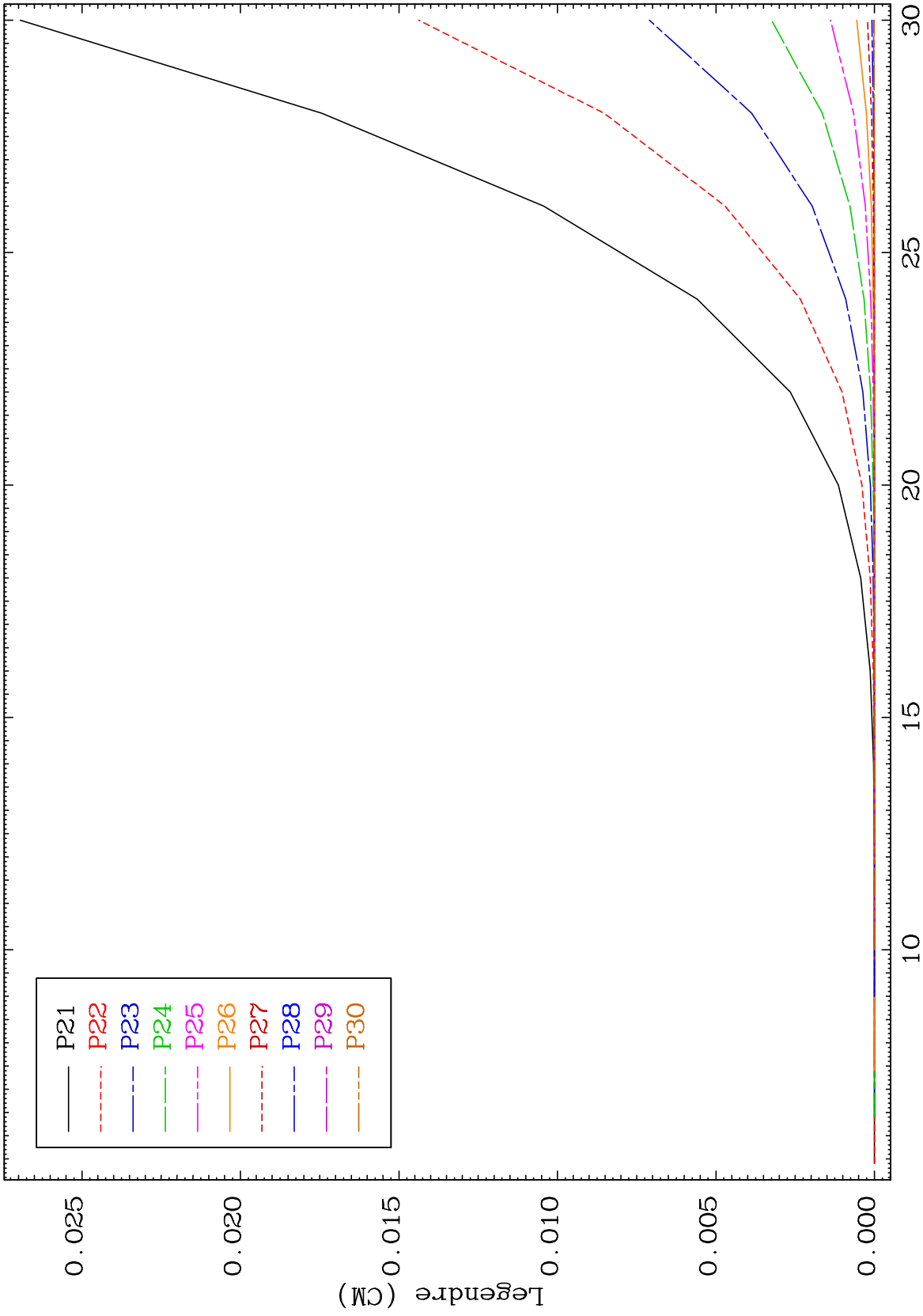
Incident Energy (MeV)

102-No-256

MAT 9974

Elastic Legendre Coefficients

102-No-256



18

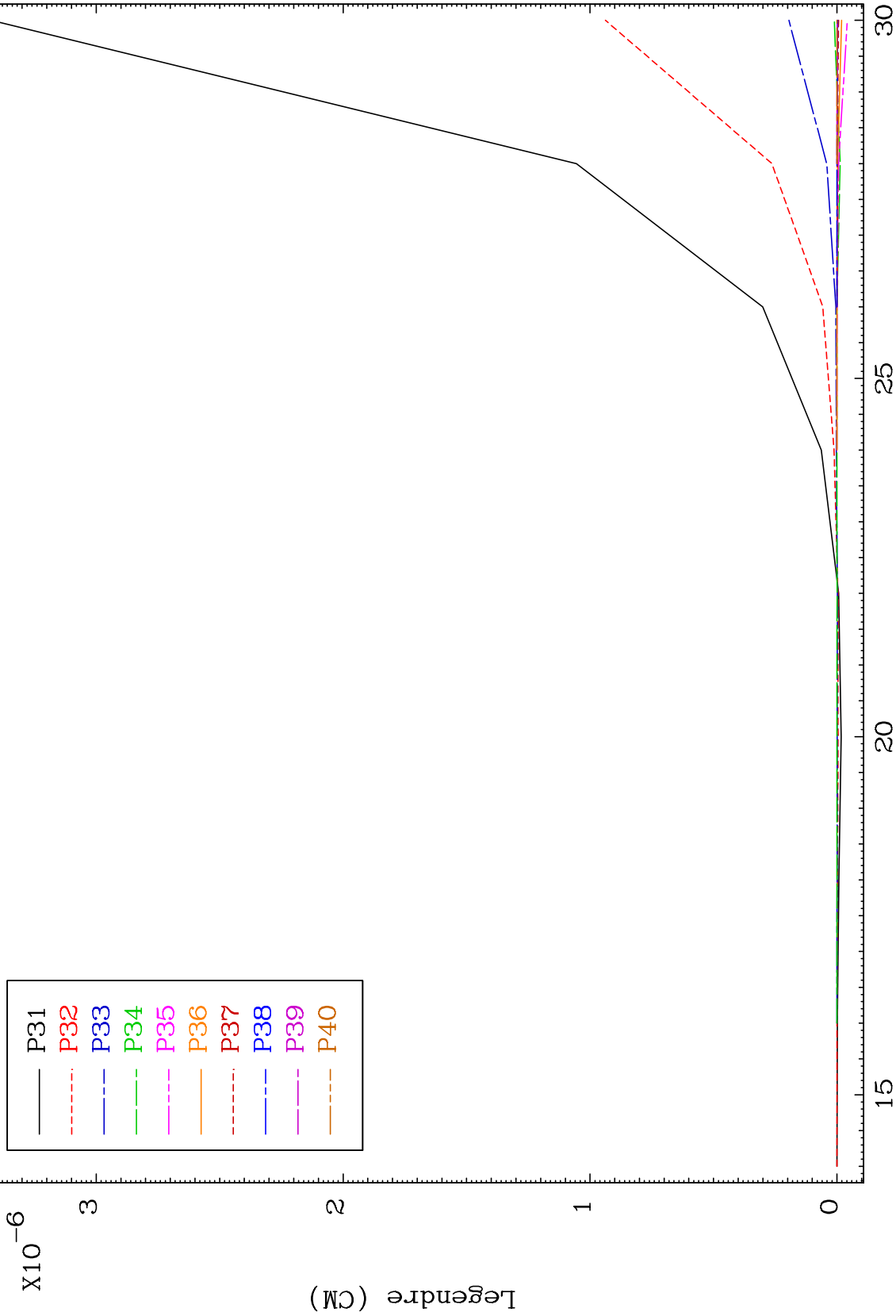
Incident Energy (MeV)

102-No-256

MAT 9974

Elastic Legendre Coefficients

102-No-256



19

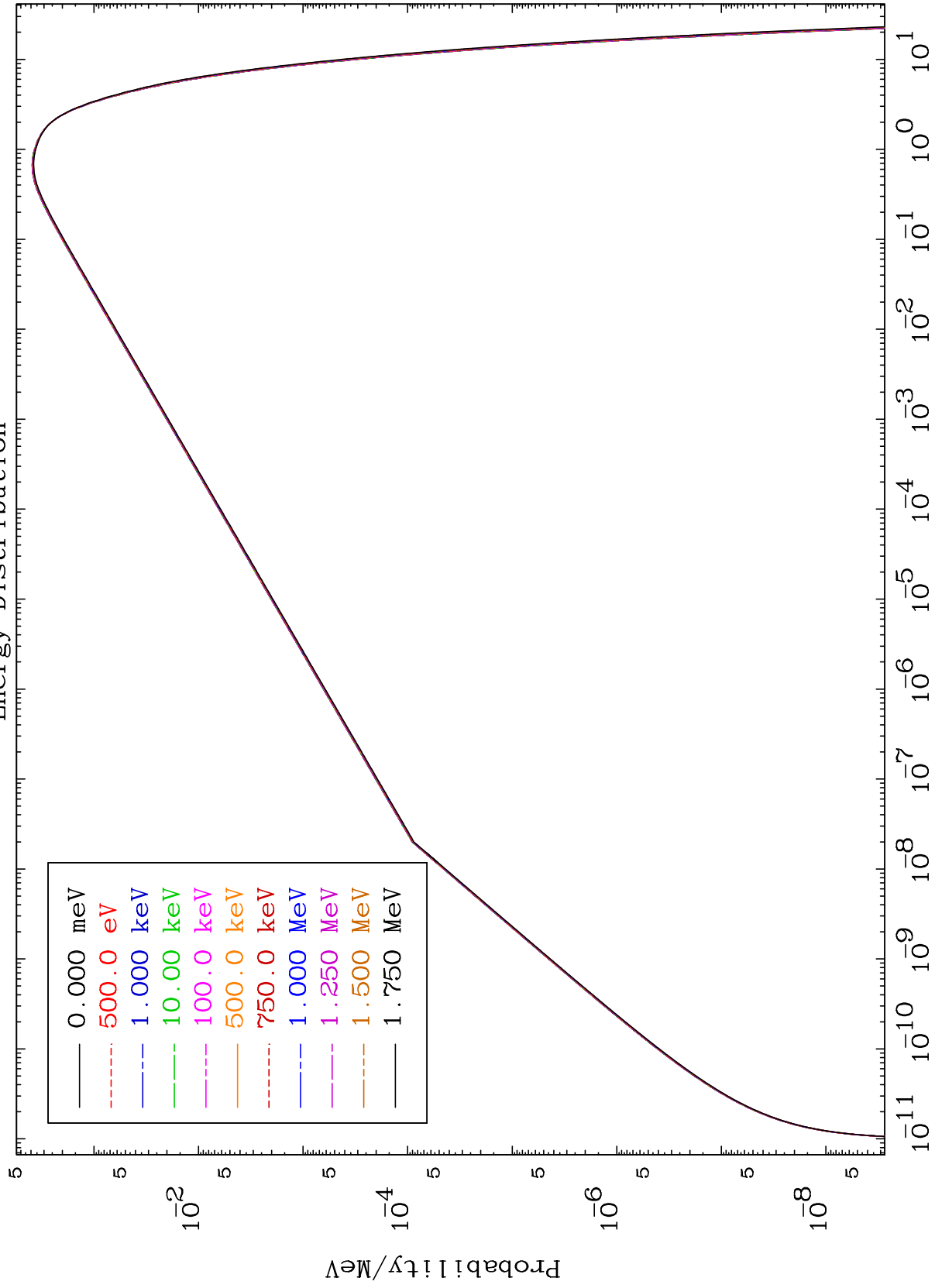
Incident Energy (MeV)

102-No-256

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Fission Energy Distribution

102-No-256

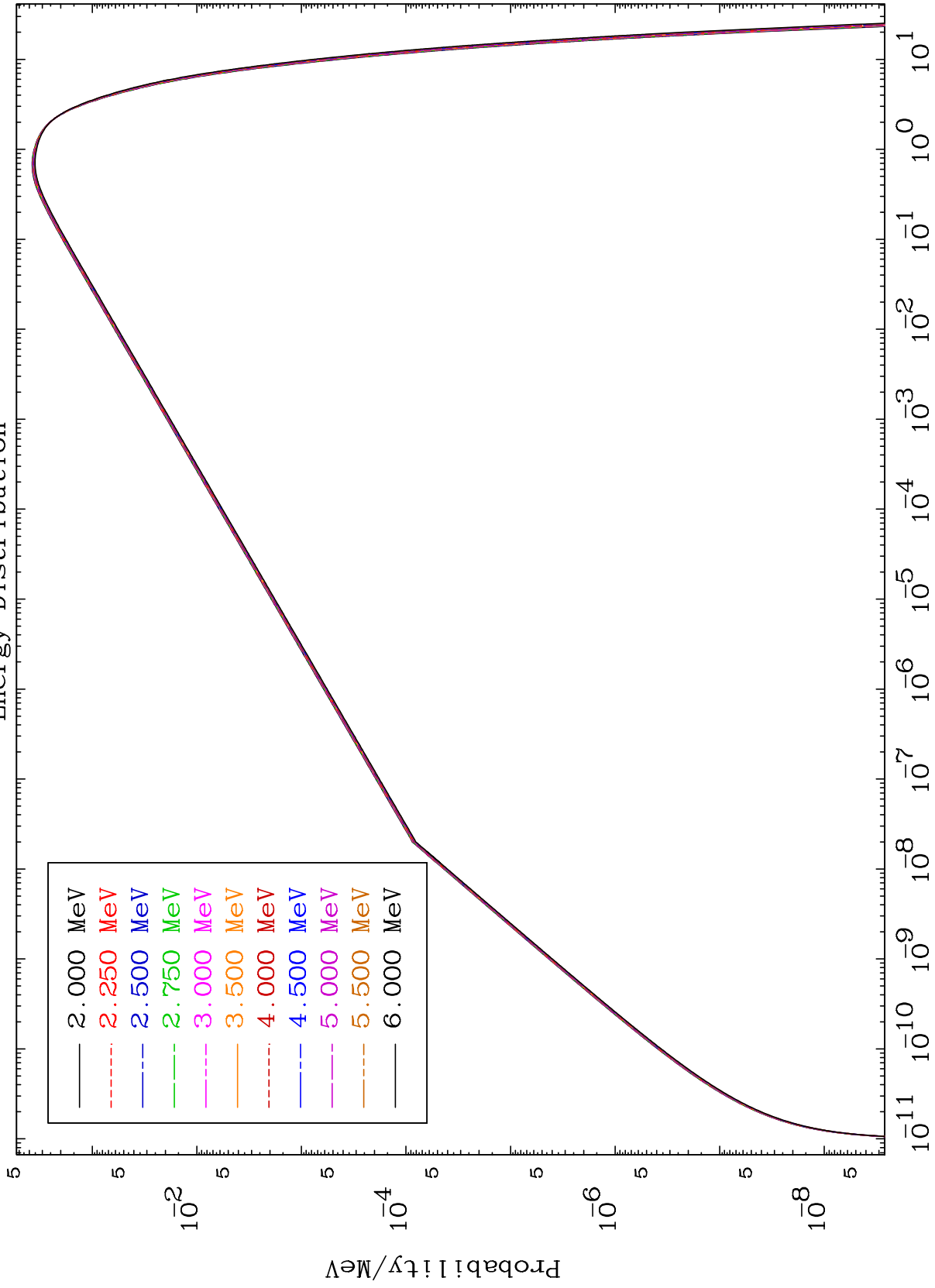


102-No-256

MAT 9974

Fission Energy Distribution

102-No-256

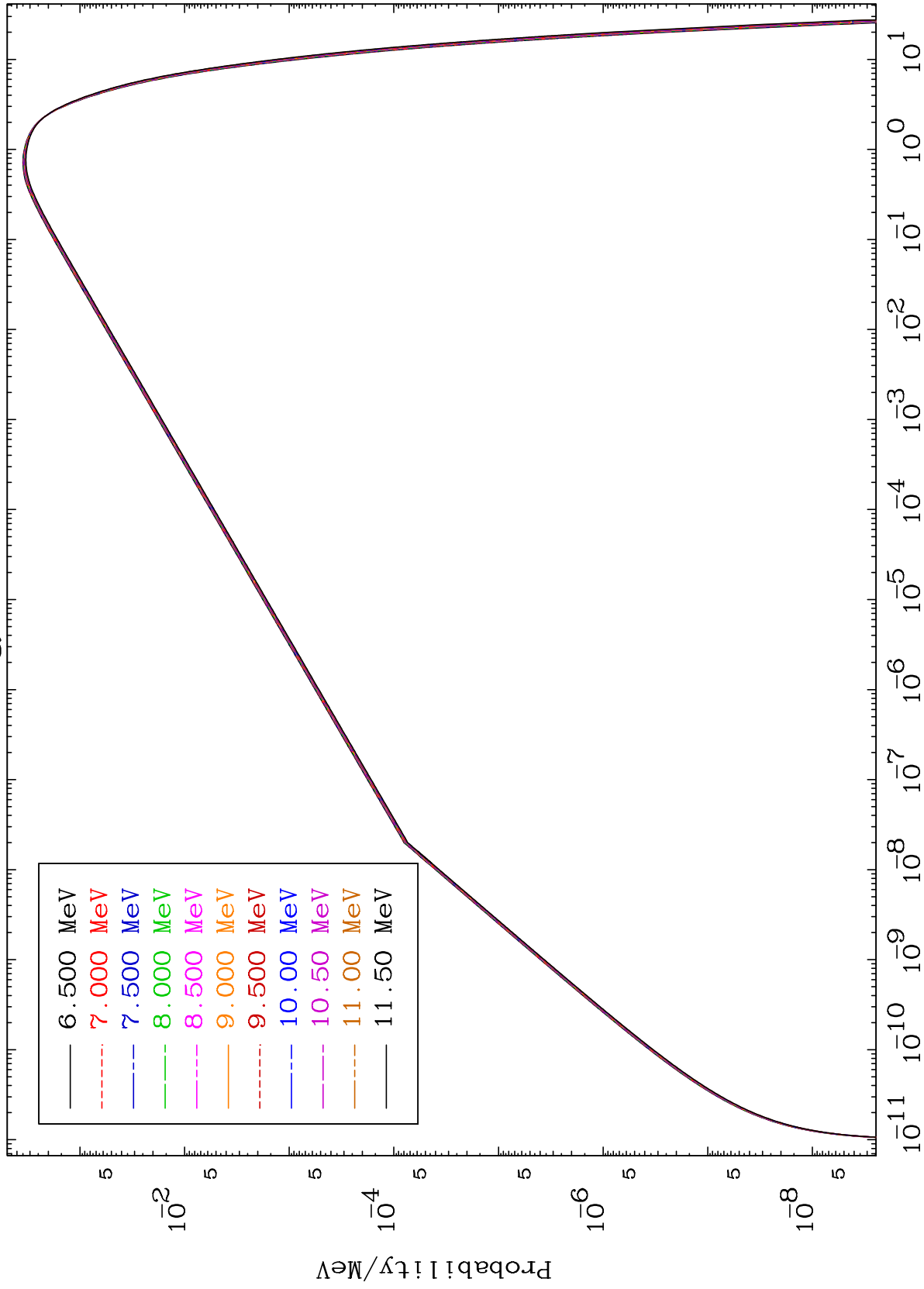


102-No-256

MAT 9974

Fission Energy Distribution

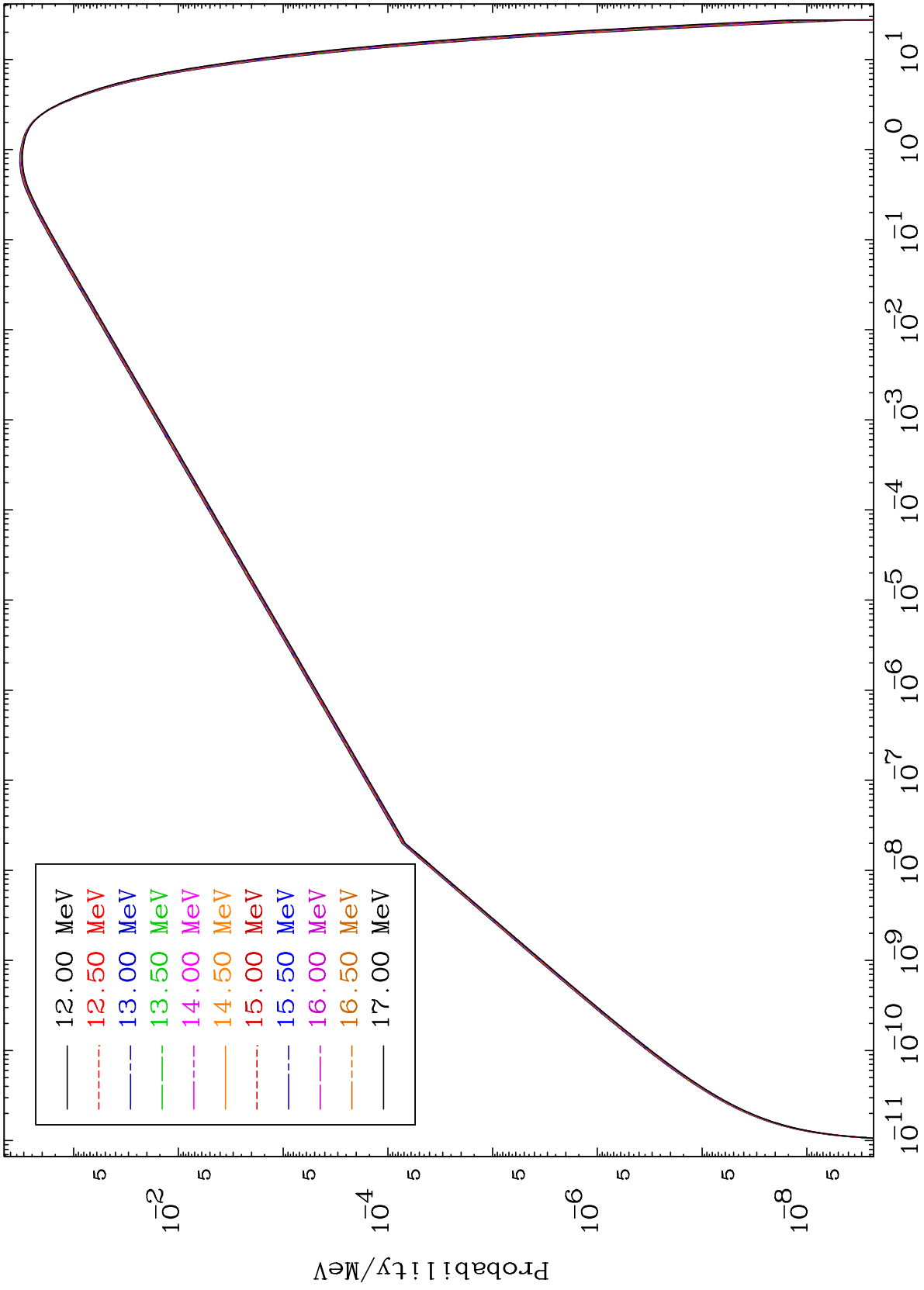
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Fission Energy Distribution

102-No-256

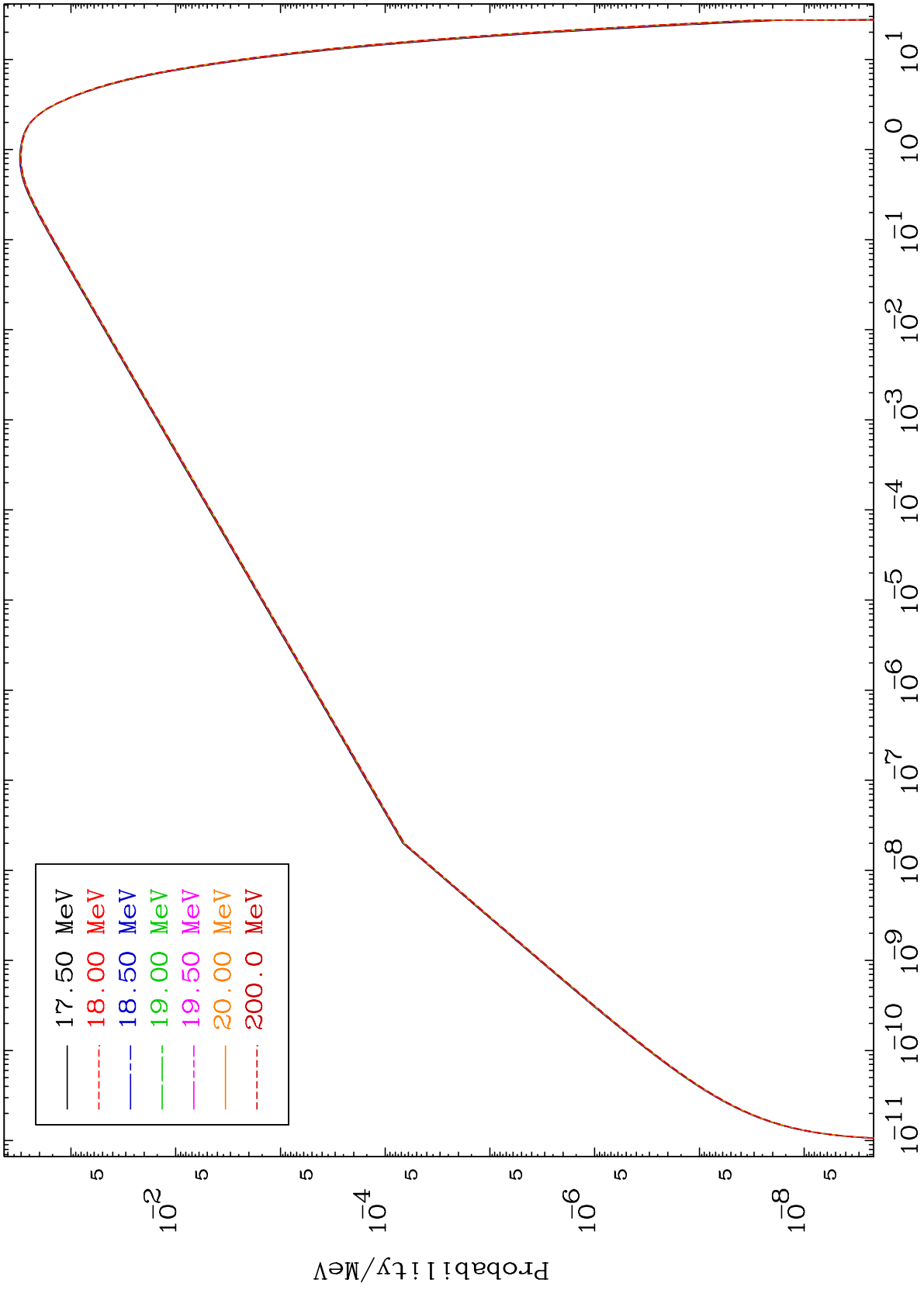


102-No-256

MAT 9974

Fission Energy Distribution

102-No-256



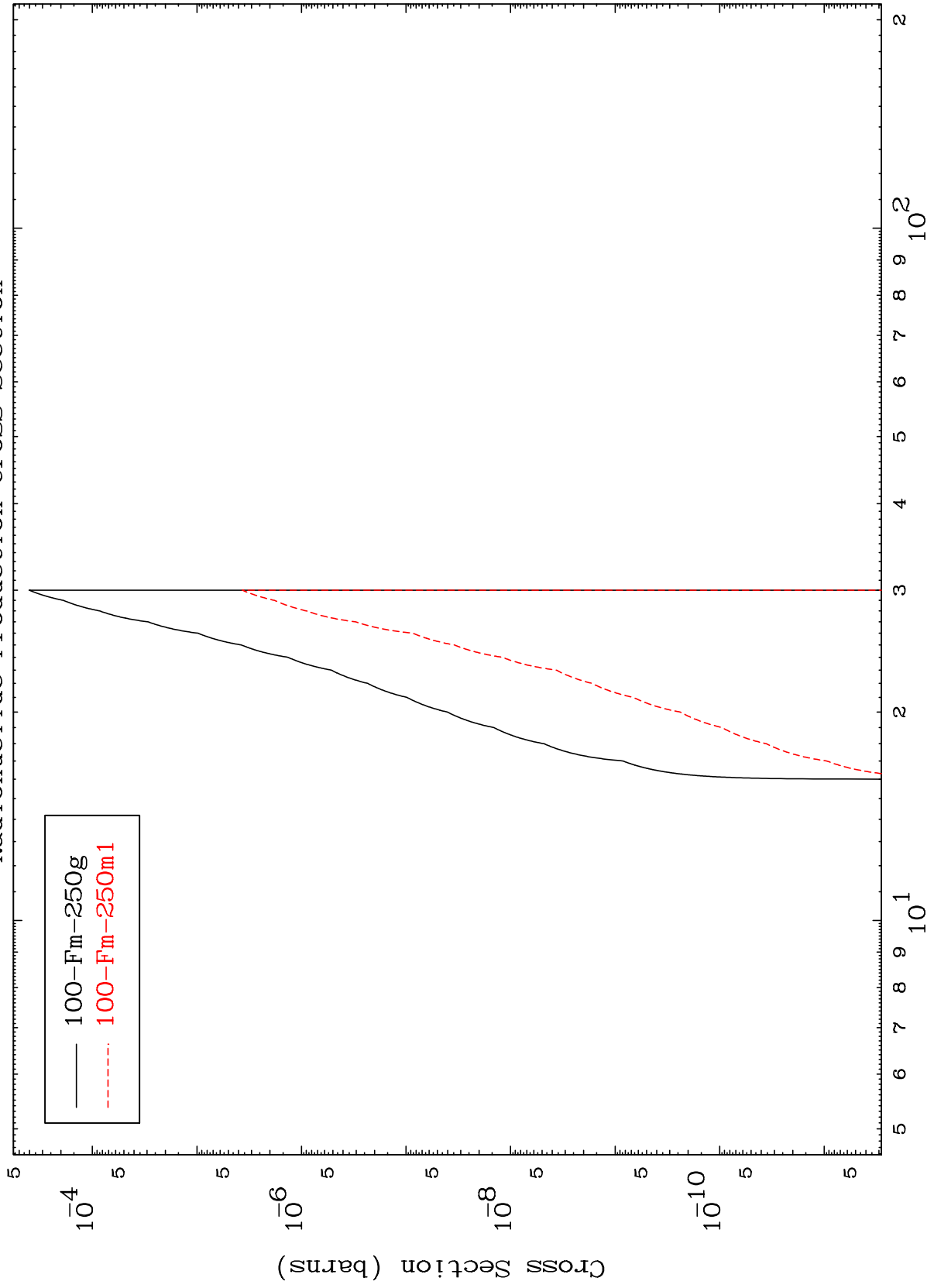
102-No-256

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(n,3n) α

102-No-256

Radionuclide Production Cross Section



25

Incident Energy (MeV)

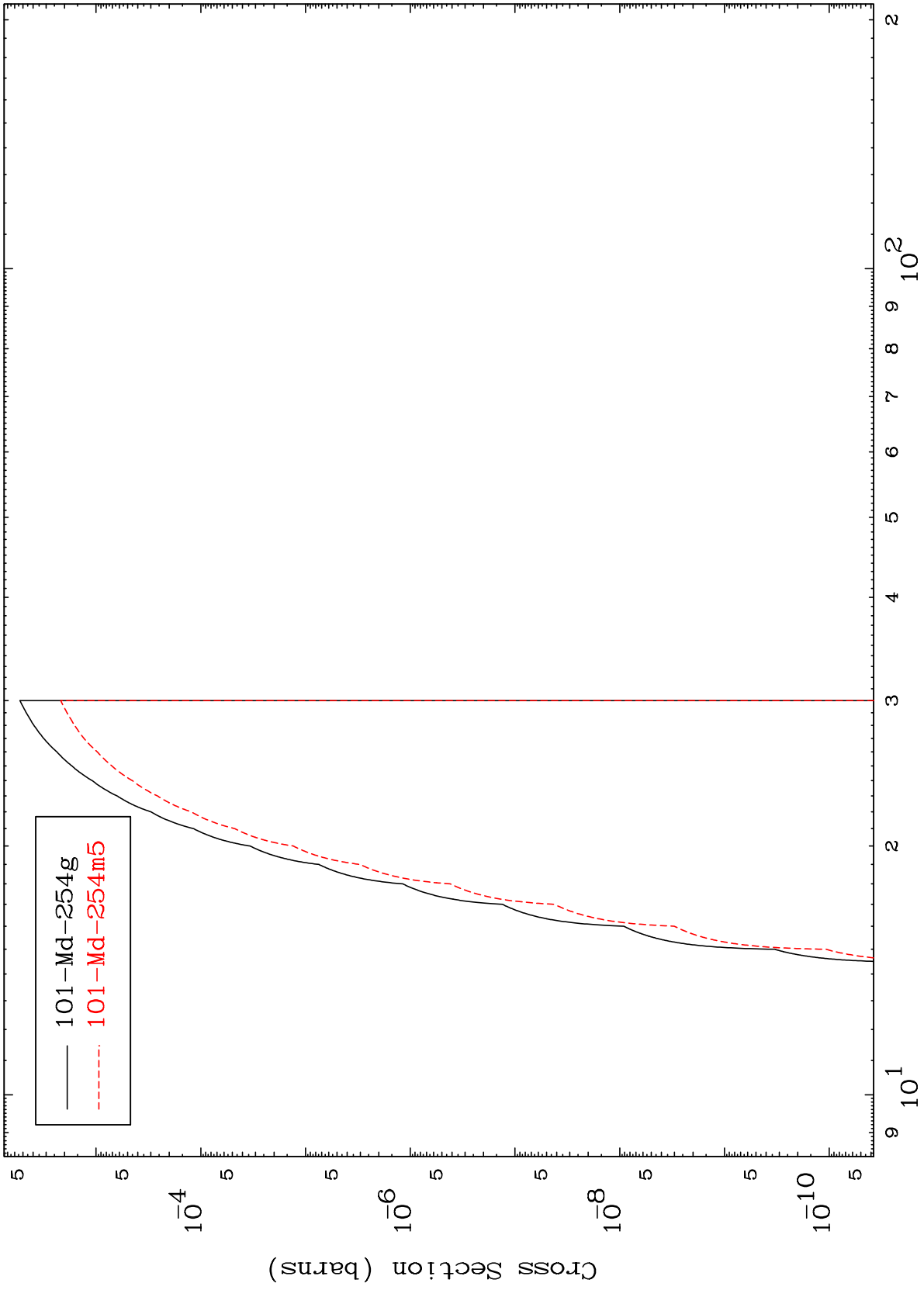
102-No-256

MAT 9974

(n,n') d

102-No-256

Radionuclide Production Cross Section



26

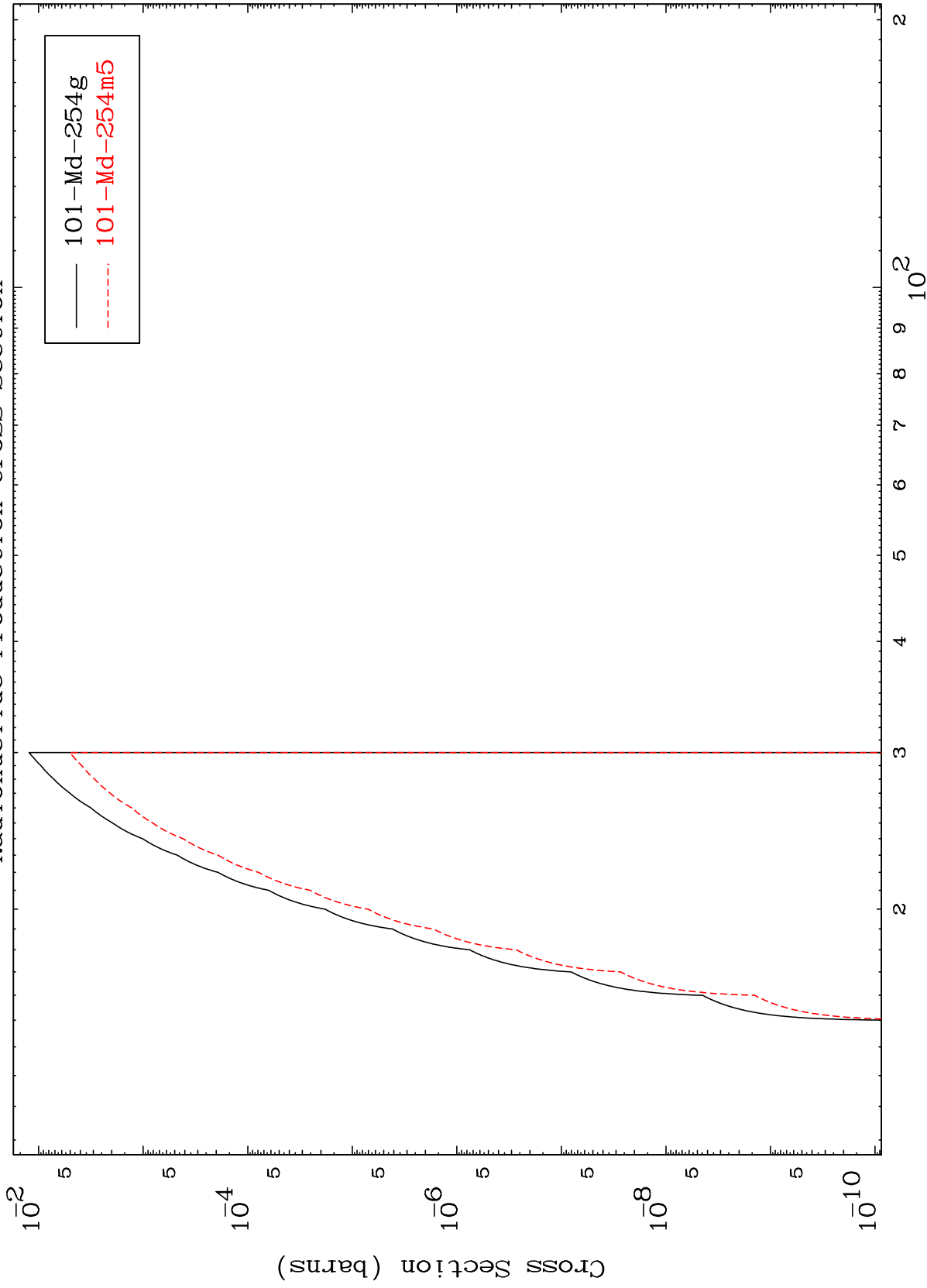
102-No-256

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(n,2n) p

102-No-256

Radionuclide Production Cross Section



27

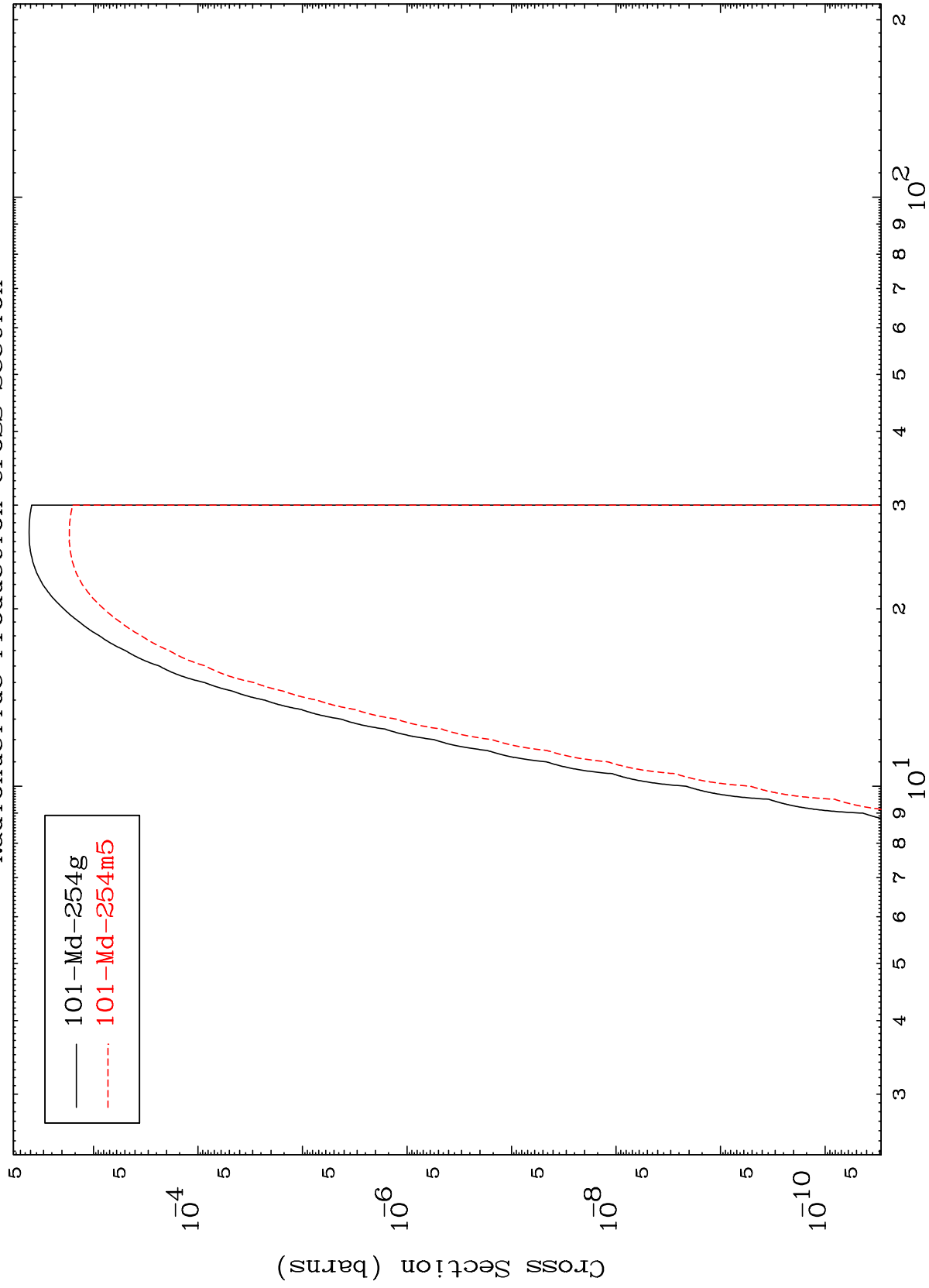
Incident Energy (MeV)

102-No-256

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102-No-256

(n, t)
Radionuclide Production Cross Section



28

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