

Program EVALPLOT
(Version 2018-1)

by

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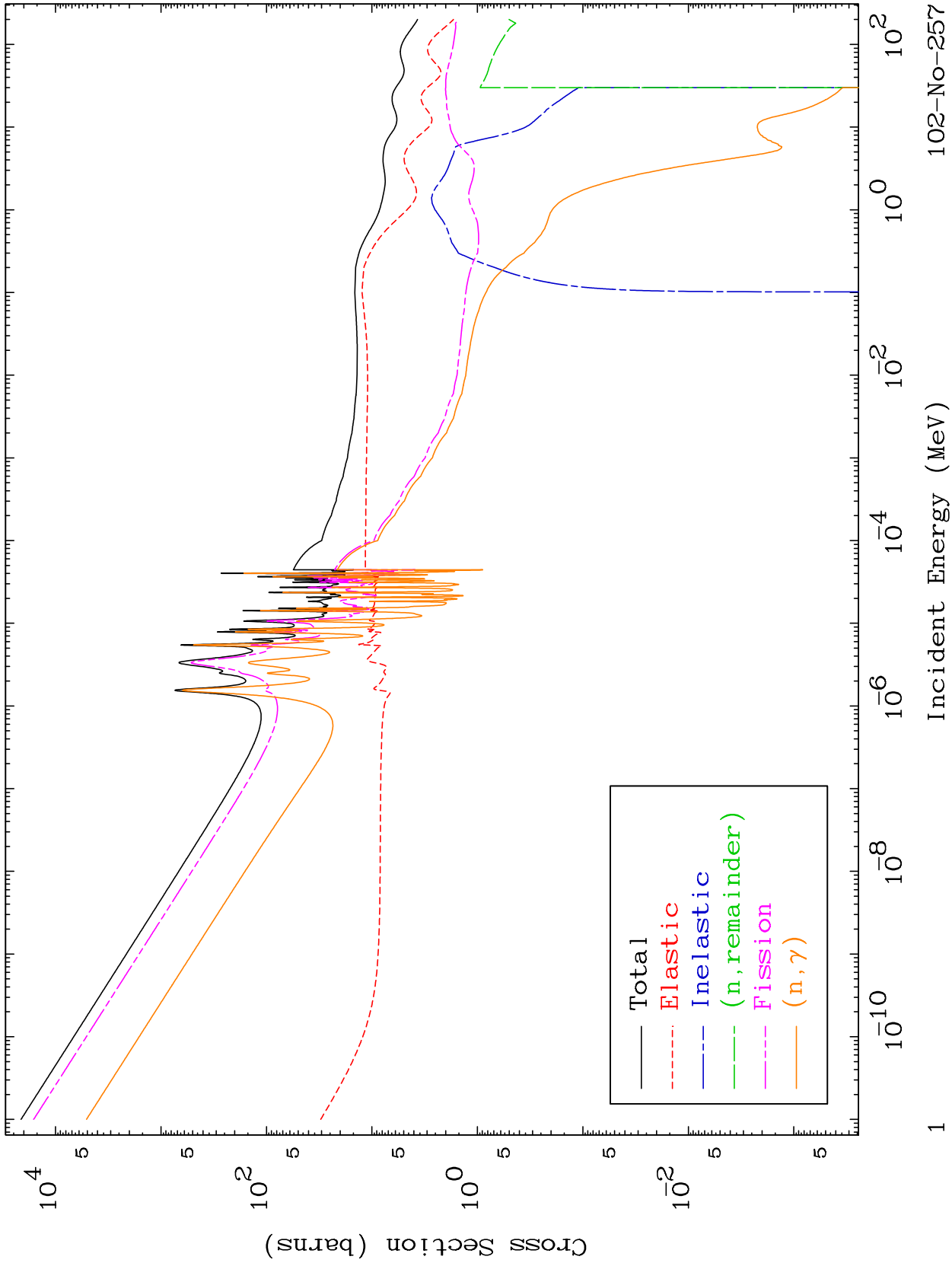
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9975

Major
293 Kelvin Cross Sections

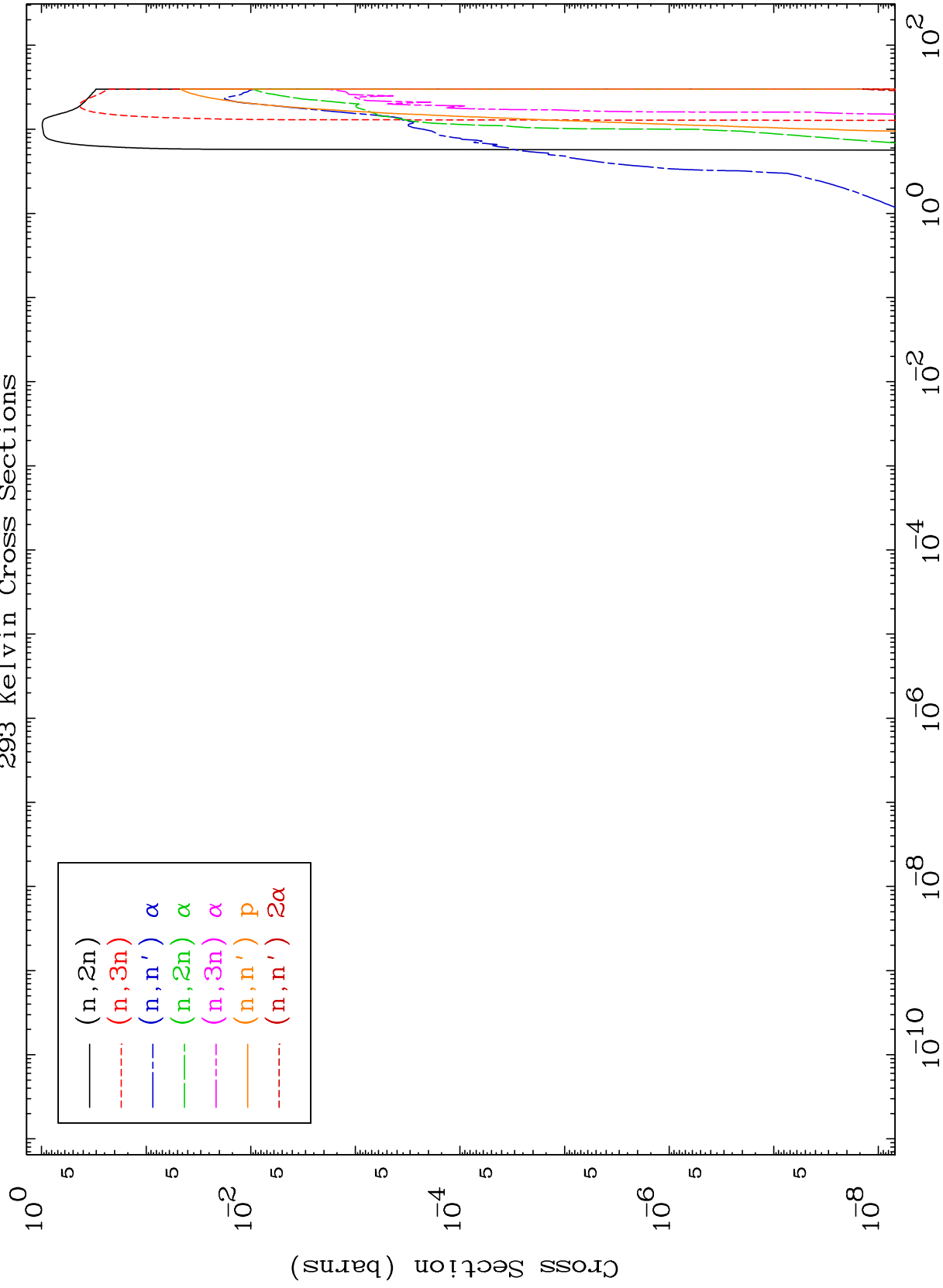
102-No-257



MAT 9975

Neutron Production
293 Kelvin Cross Sections

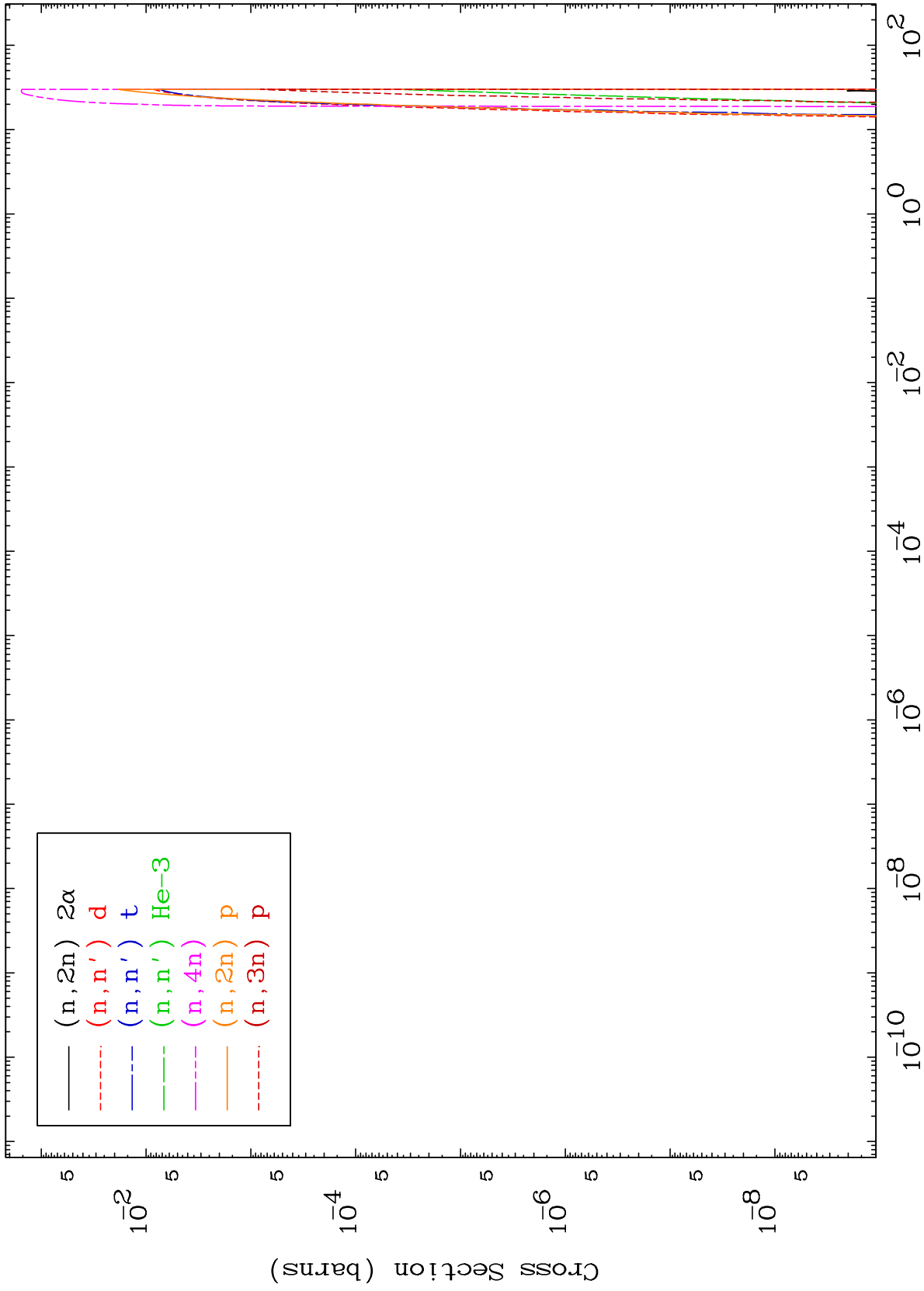
102-No-257



MAT 9975

Neutron Production
293 Kelvin Cross Sections

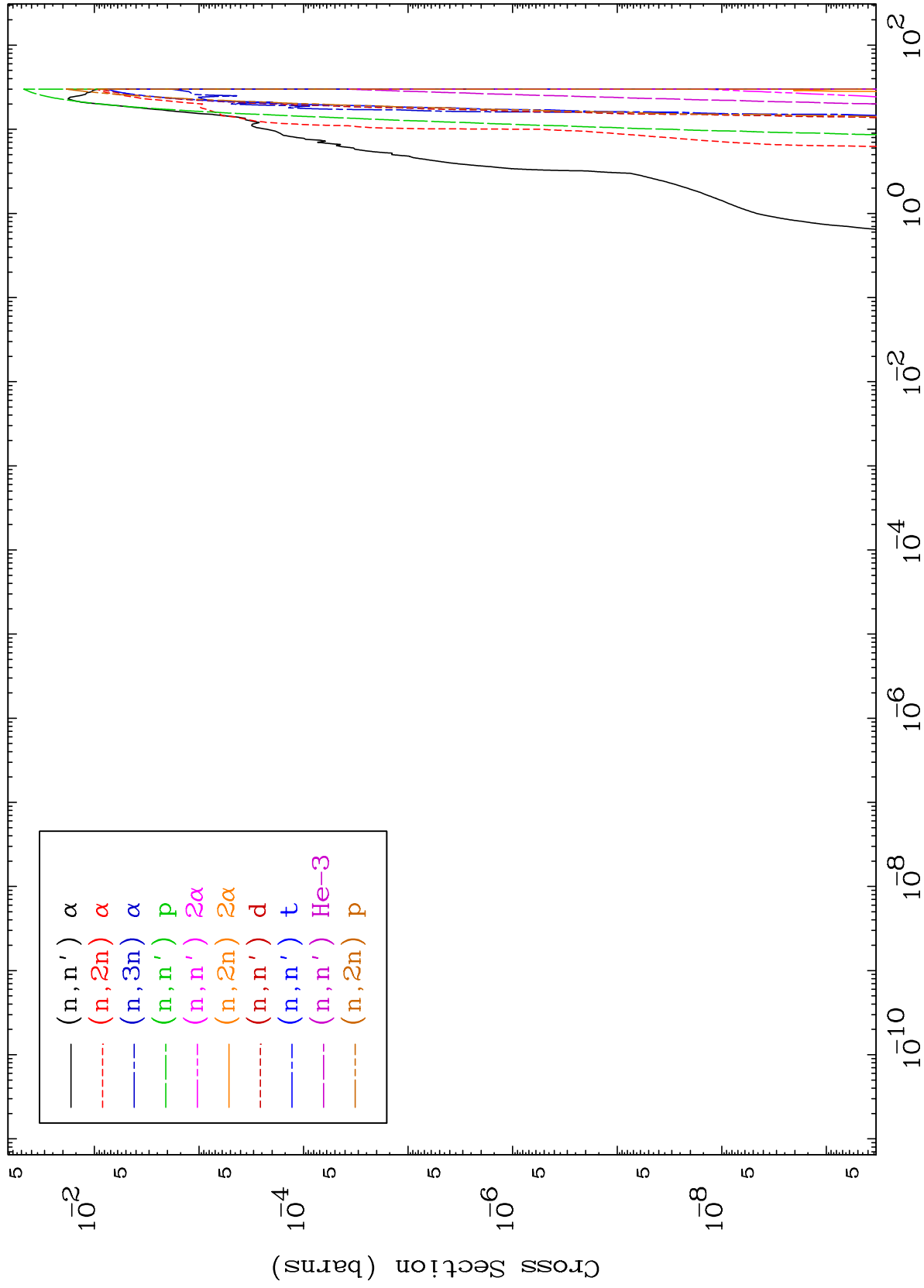
102-No-257



MAT 9975

293 Kelvin Cross Sections
Charged Particle

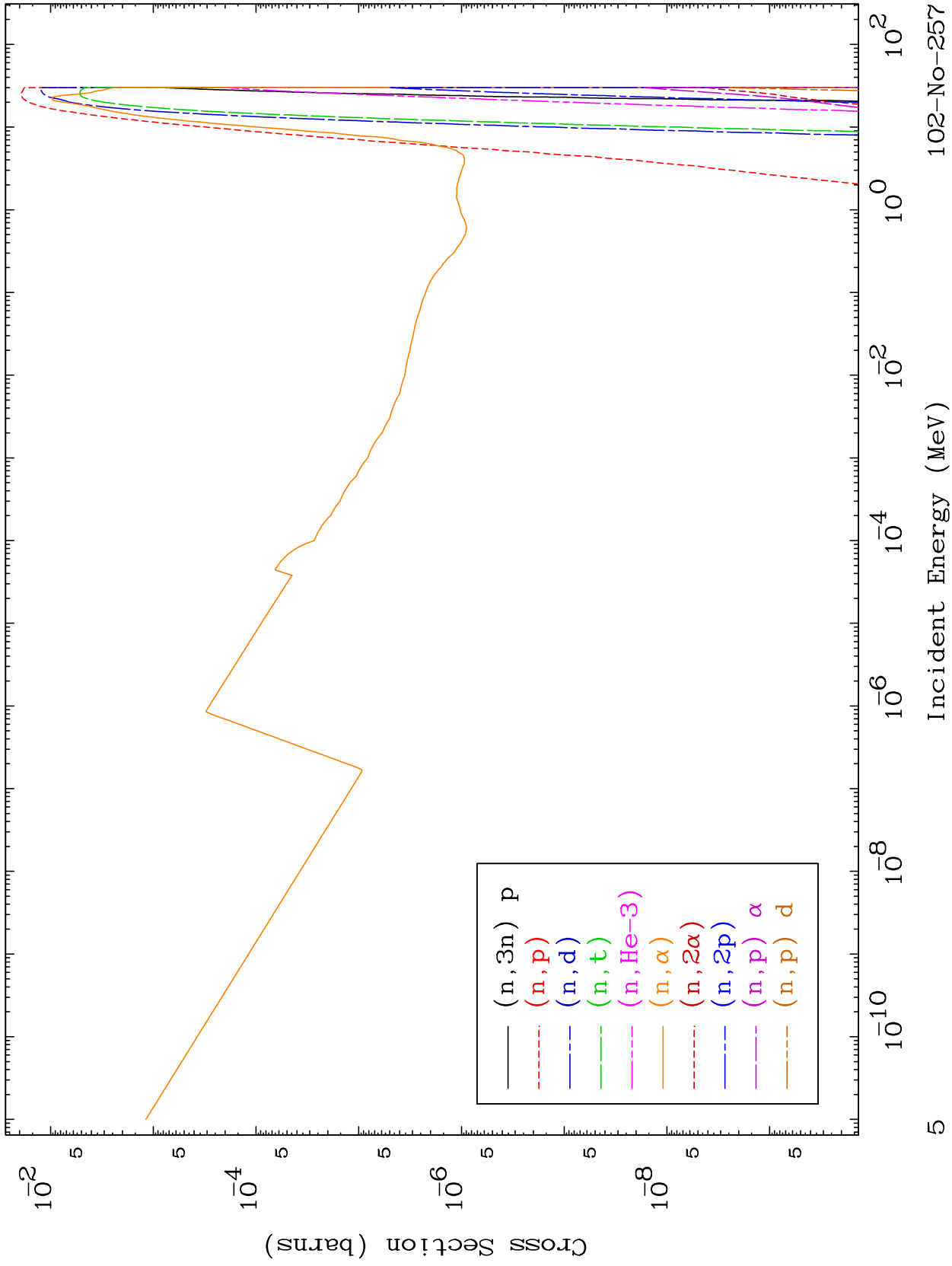
102-No-257



MAT 9975

Charged Particle
293 Kelvin Cross Sections

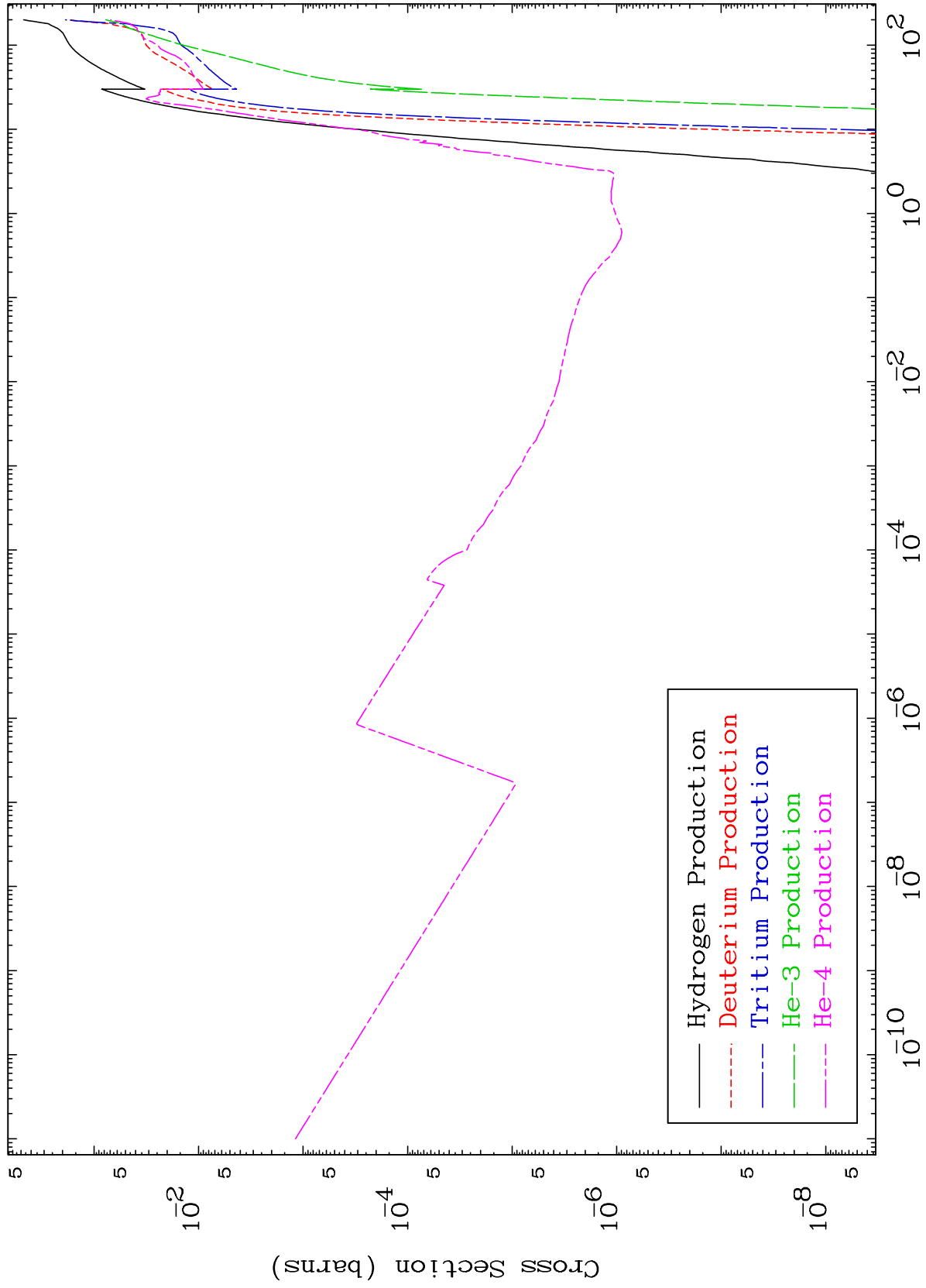
102-No-257



MAT 9975

Particle Production
293 Kelvin Cross Sections

102-No-257

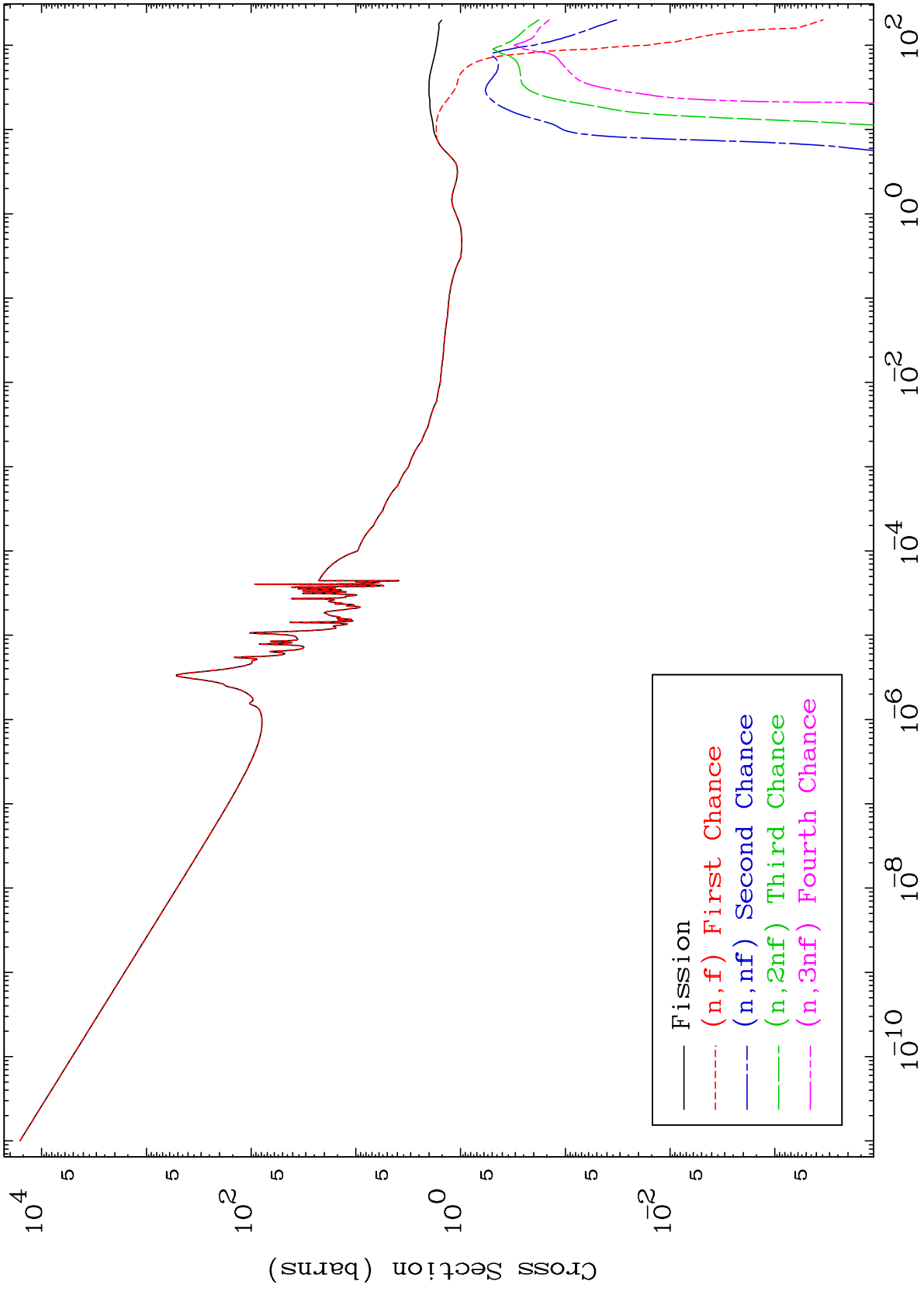


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MAT 9975

Fission
293 Kelvin Cross Sections

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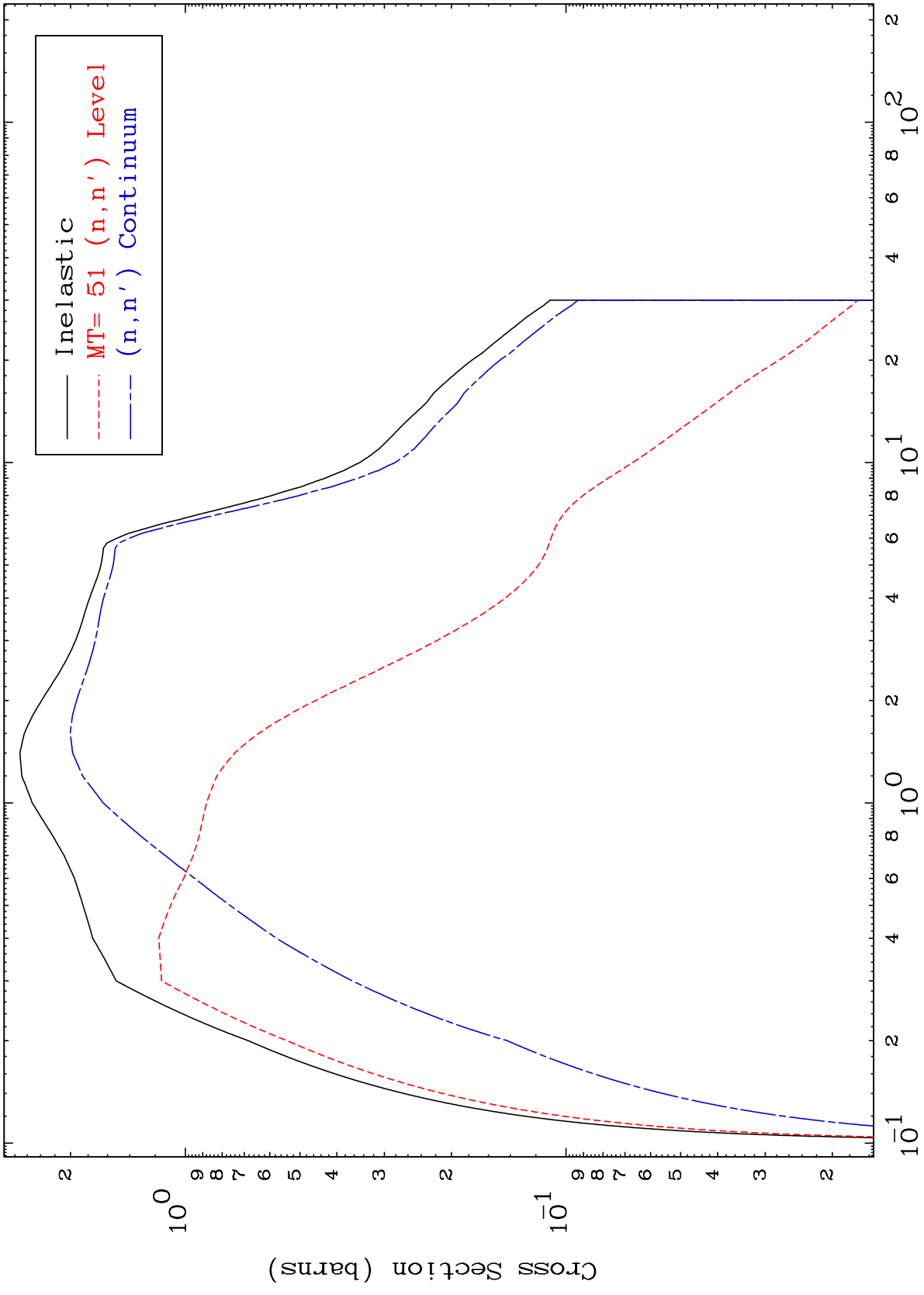
102-No-257

MAT 9975

(n,n') Level

102-No-257

293 Kelvin Cross Sections



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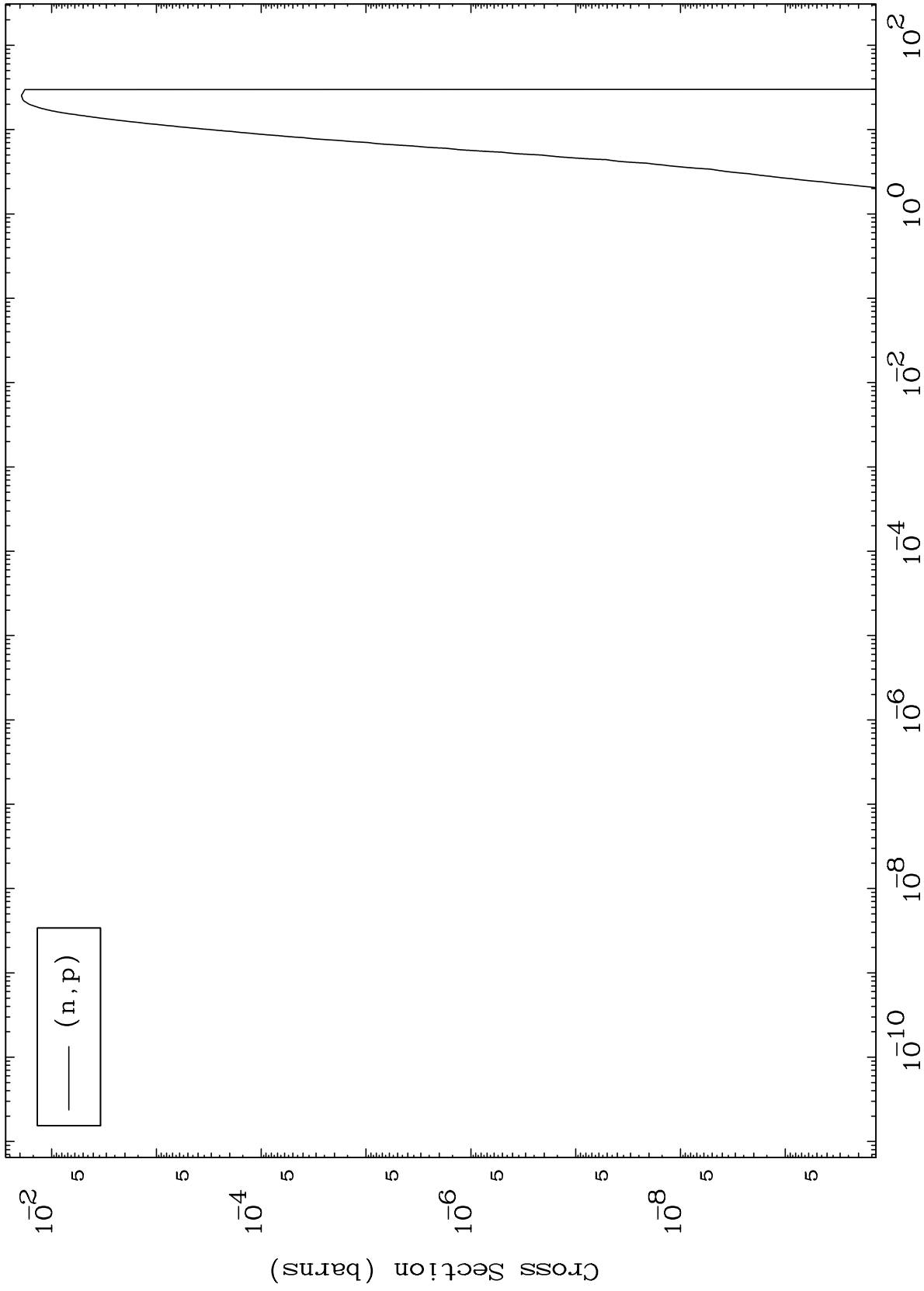
Incident Energy (MeV)

102-No-257

MAT 9975

(n,p) Levels
293 Kelvin Cross Sections

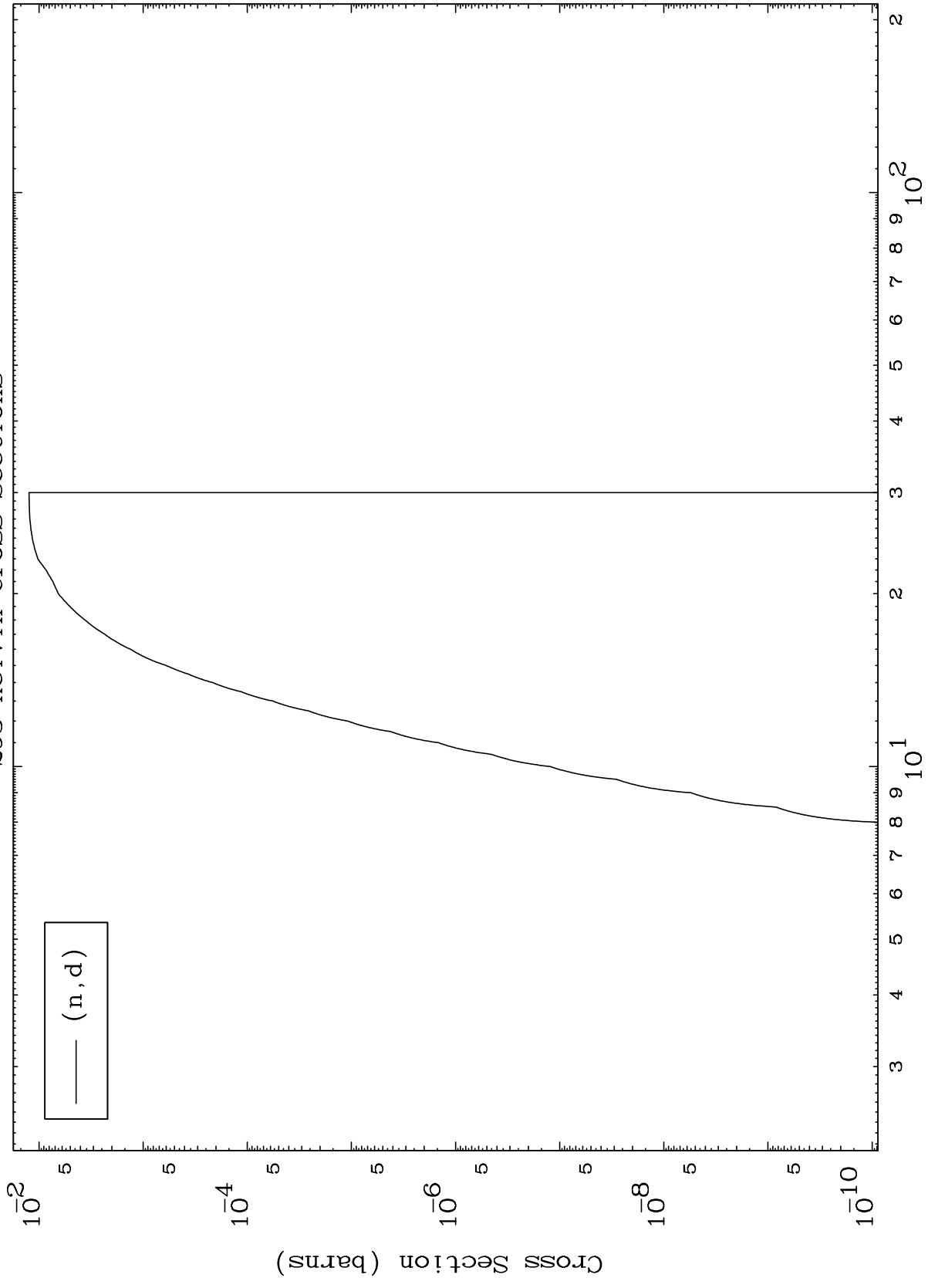
102-No-257



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(n,d) Levels
293 Kelvin Cross Sections

102-No-257



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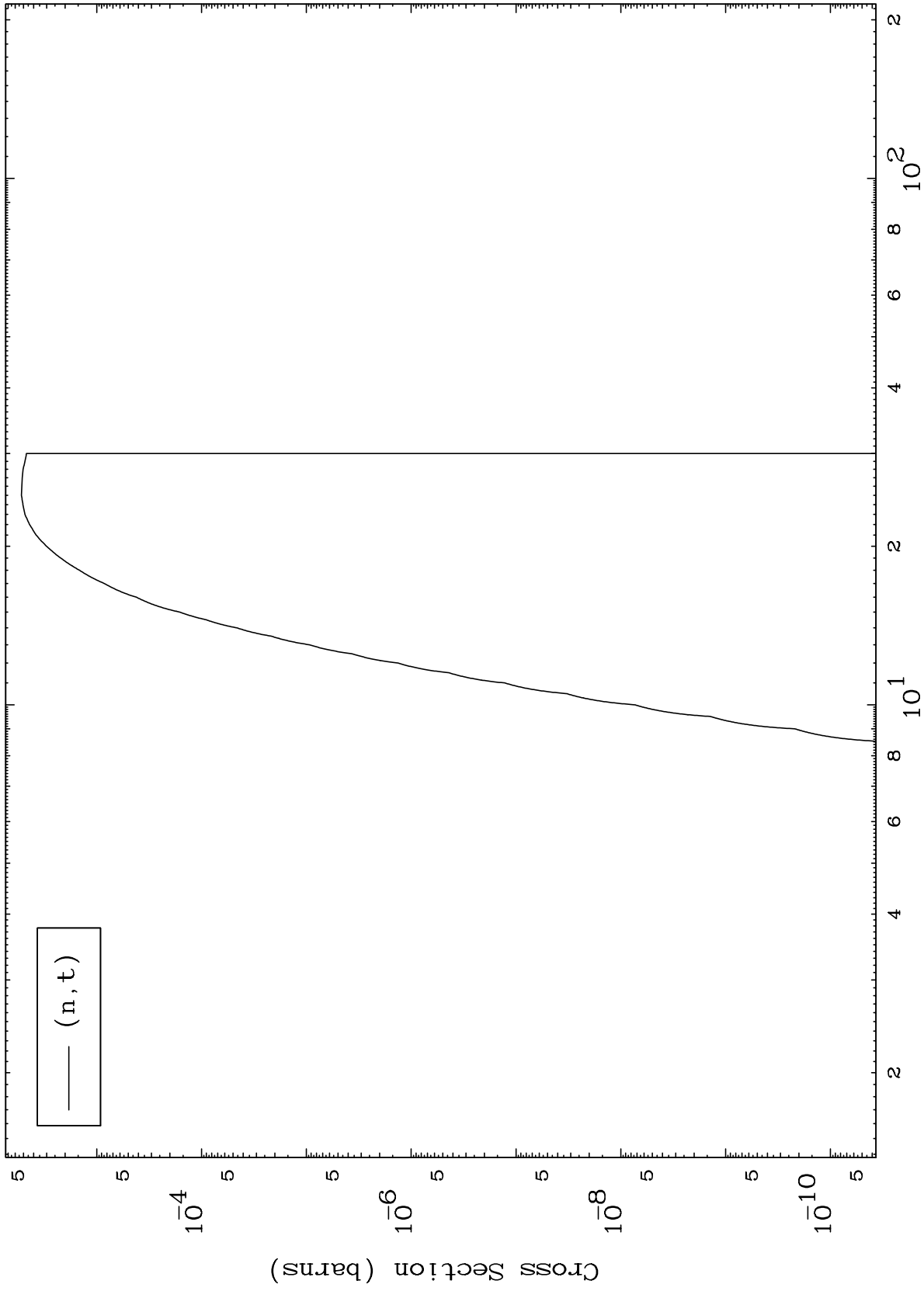
Incident Energy (MeV)

102-No-257

MAT 9975

(n,t) Levels
293 Kelvin Cross Sections

102-No-257



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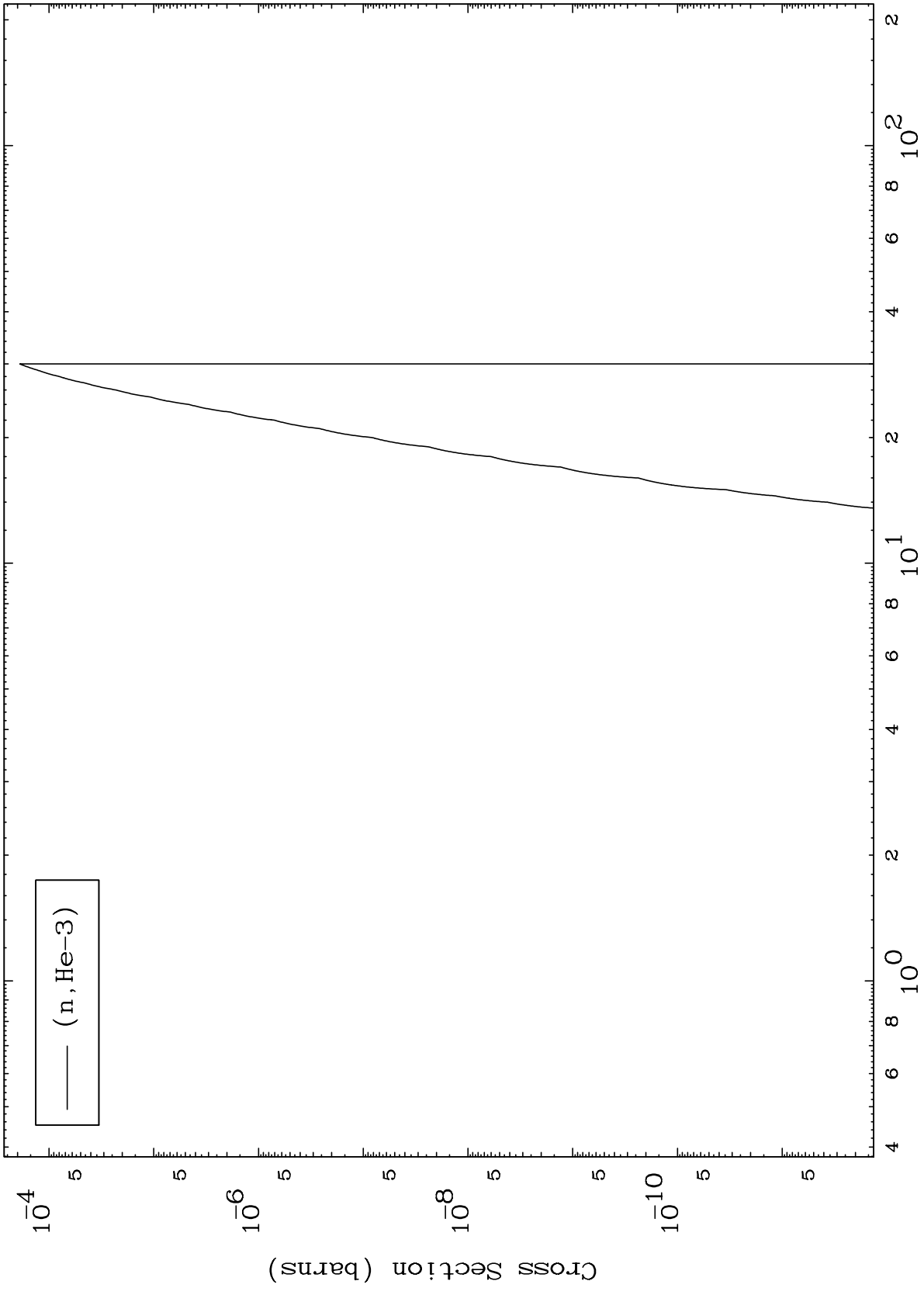
Incident Energy (MeV)

102-No-257

MAT 9975

(n,He3) Levels
293 Kelvin Cross Sections

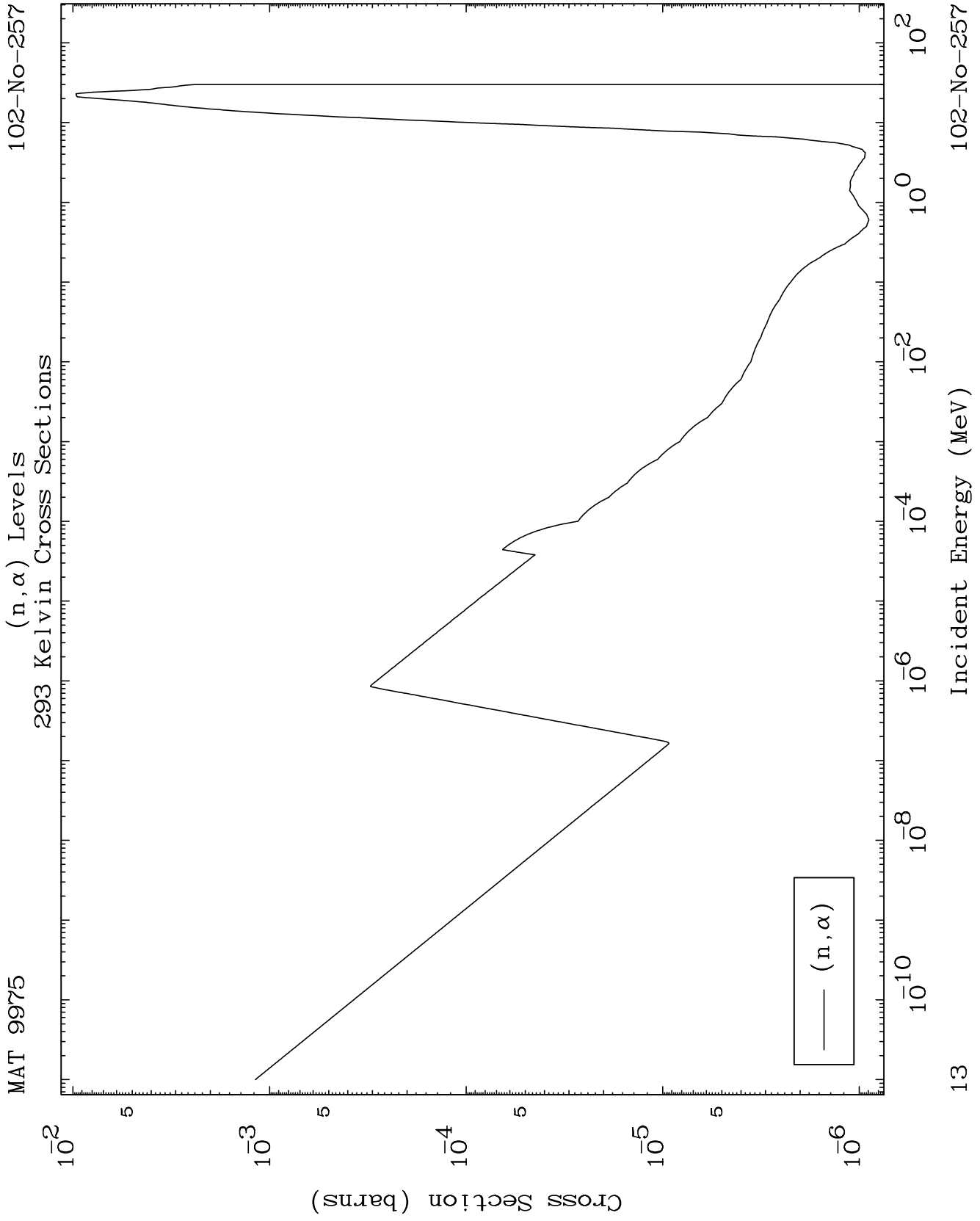
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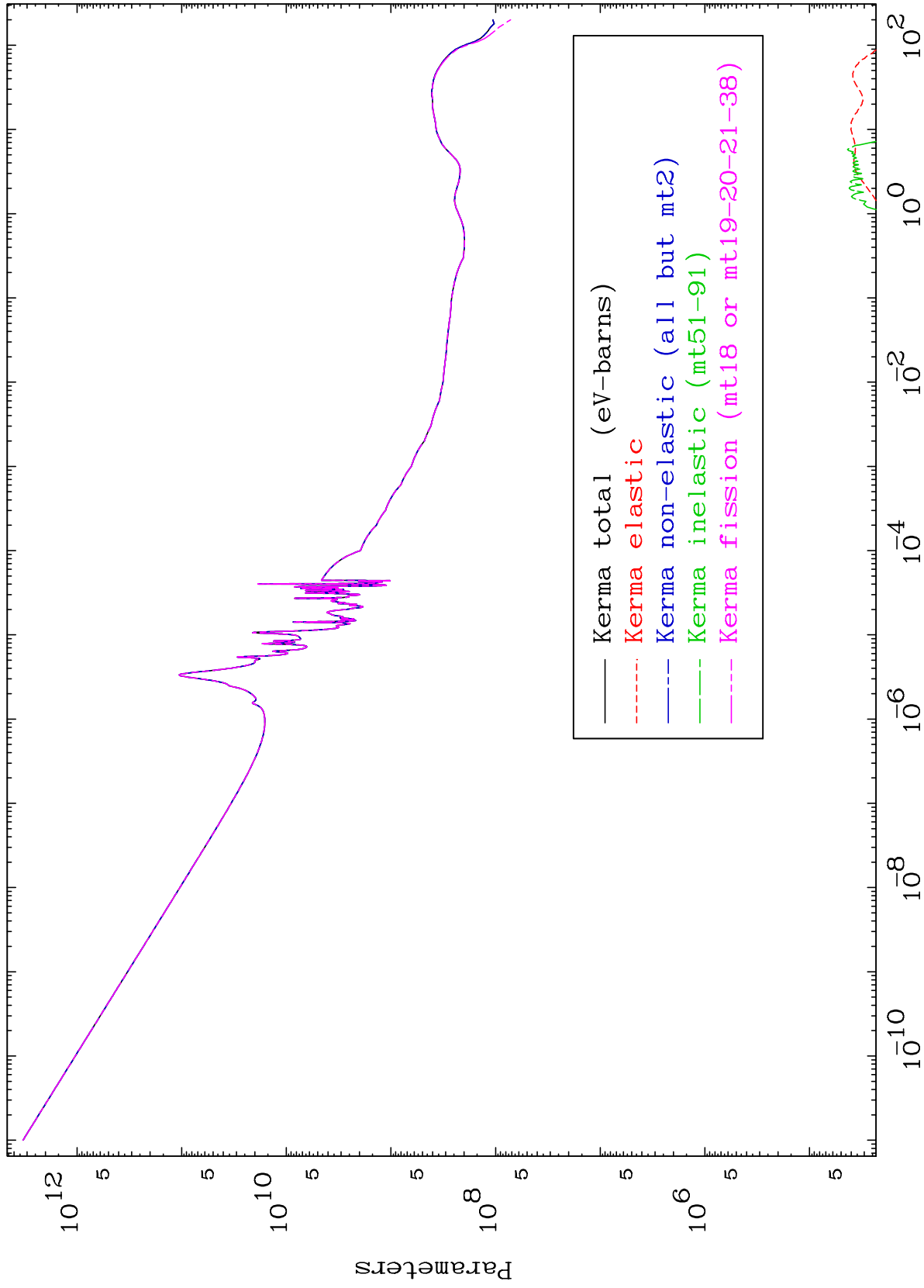


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Incident Energy (MeV)

102-No-257

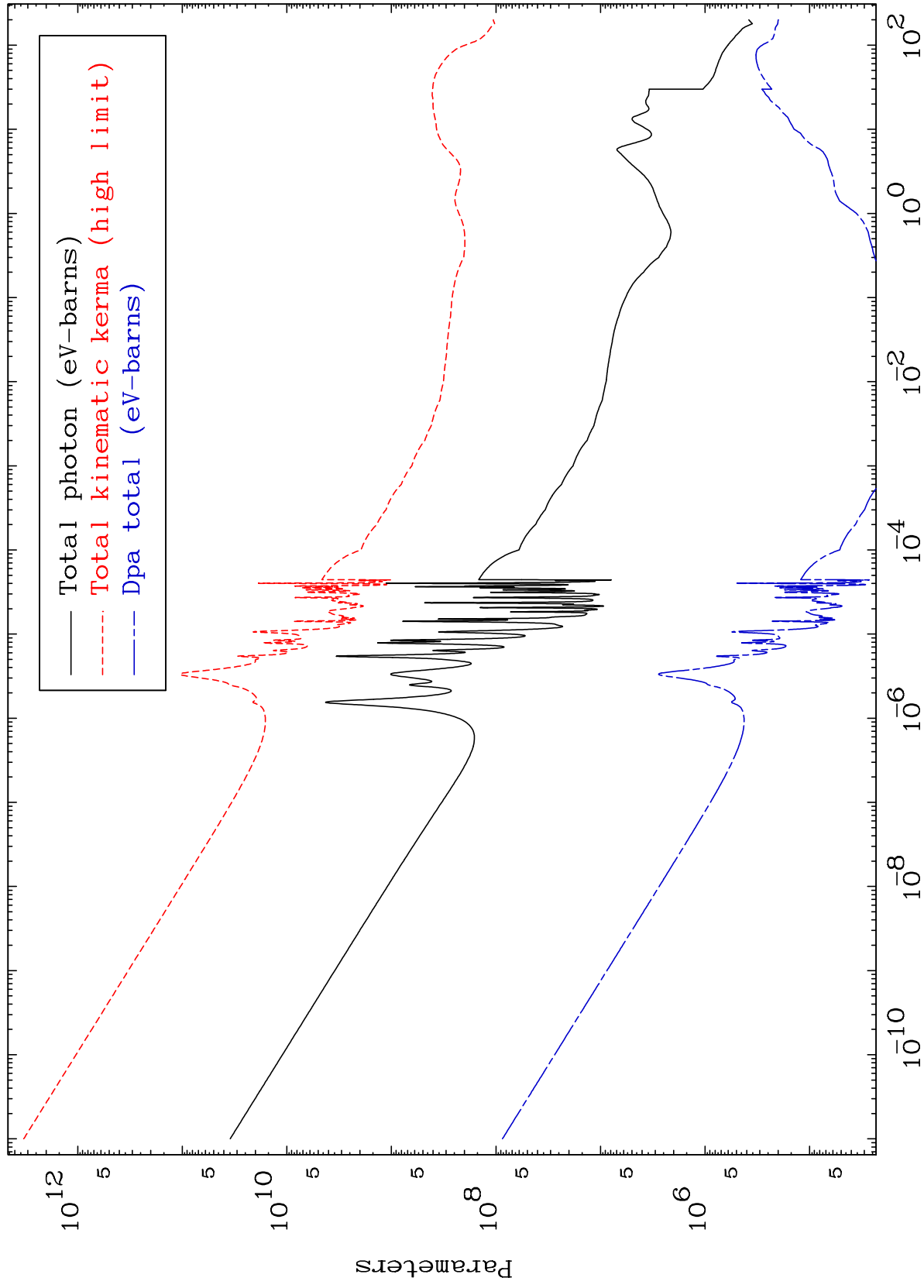




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Energy Release
Parameters

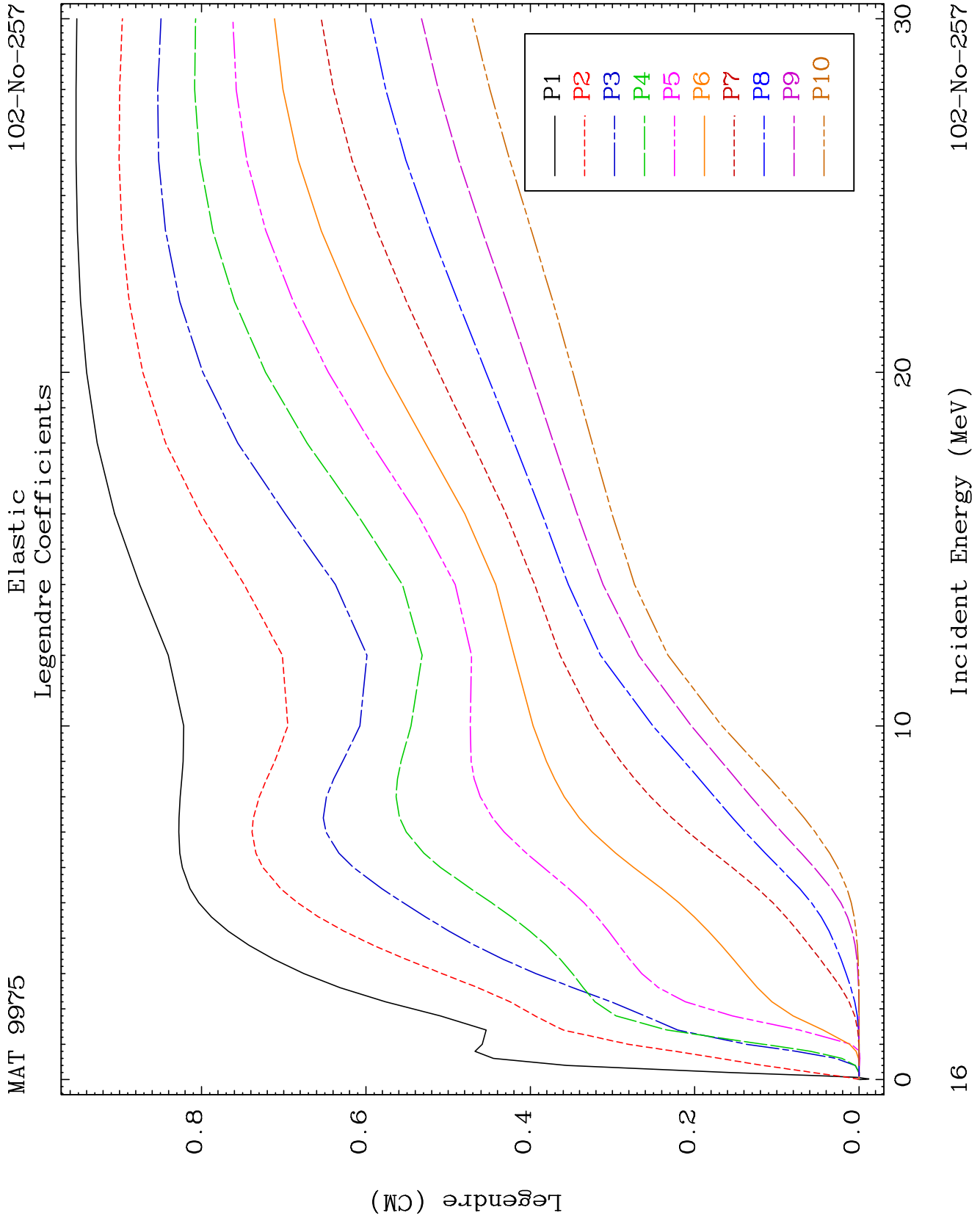
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Incident Energy (MeV)

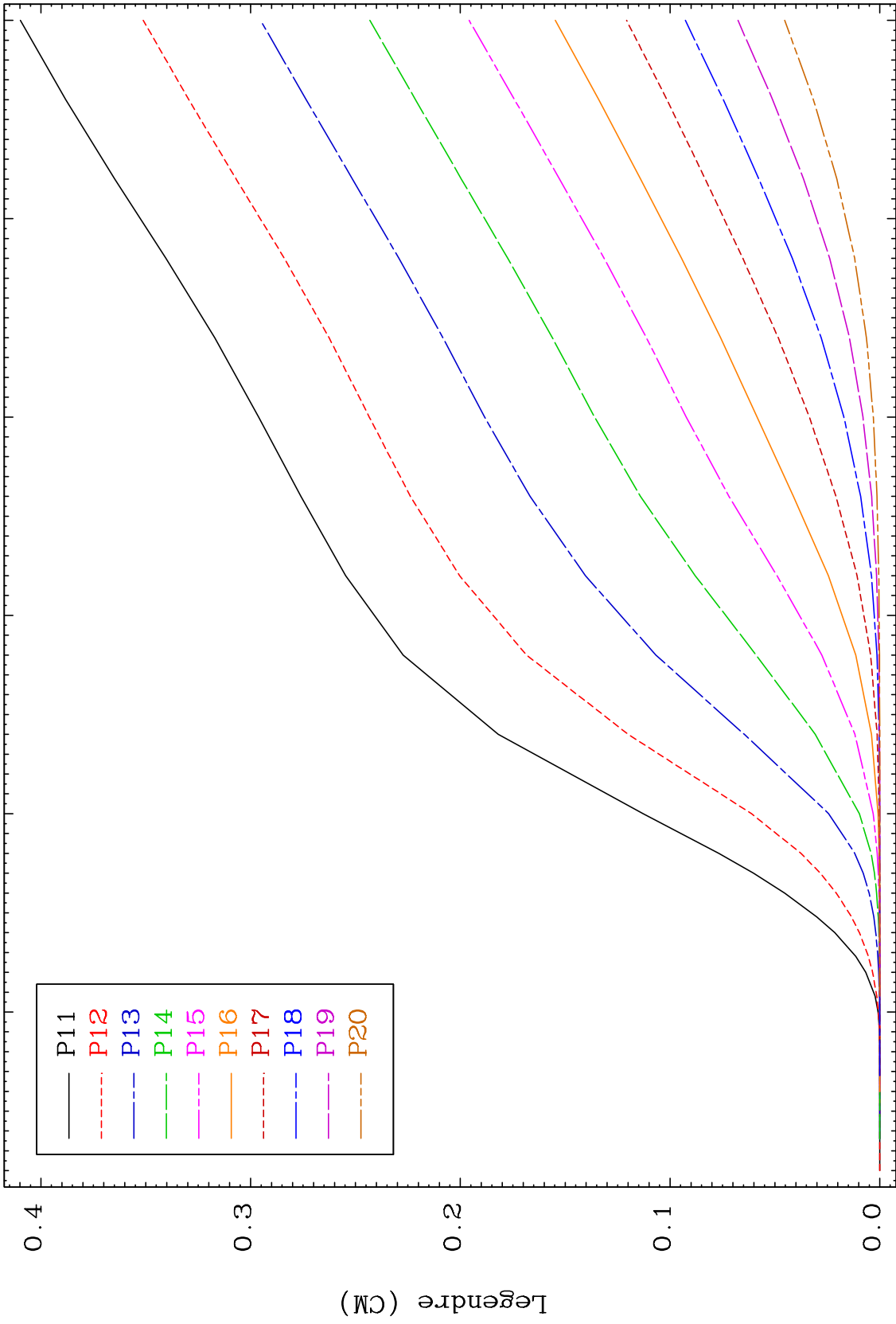
102-No-257



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Elastic Legendre Coefficients

102-No-257



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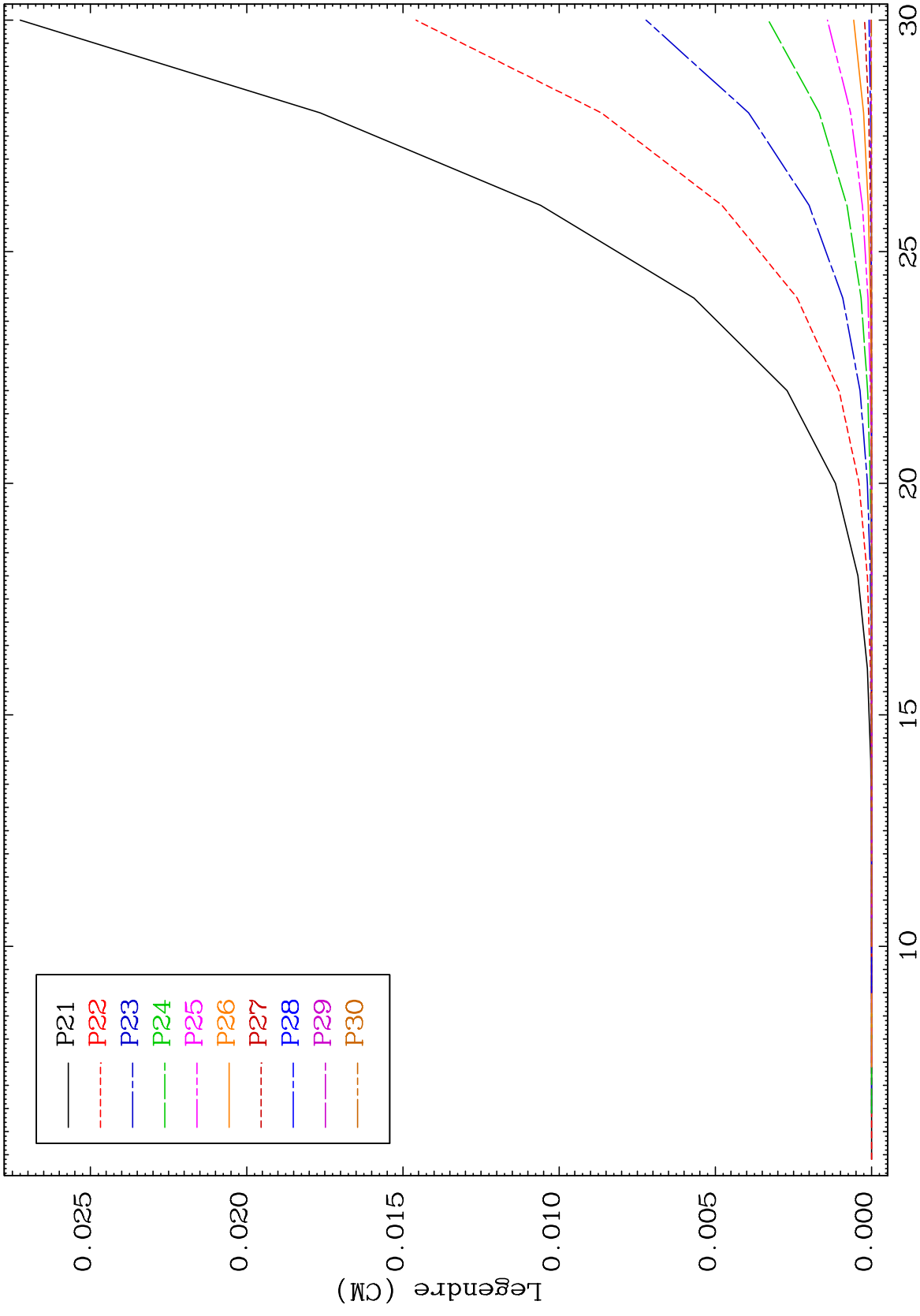
Incident Energy (MeV)

102-No-257

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Elastic
Legendre Coefficients

102-No-257



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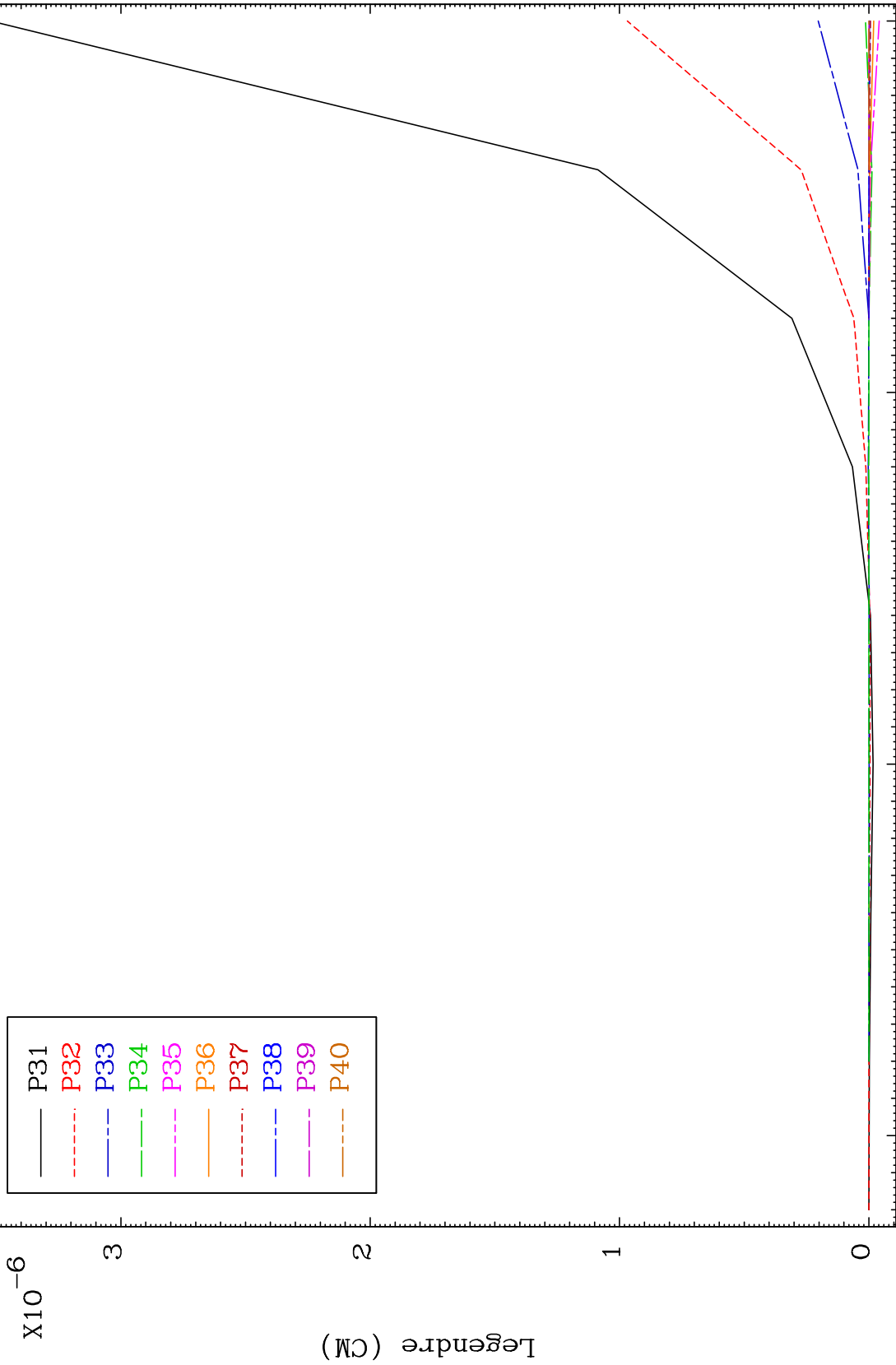
Incident Energy (MeV)

102-No-257

MAT 9975

Elastic Legendre Coefficients

102-No-257



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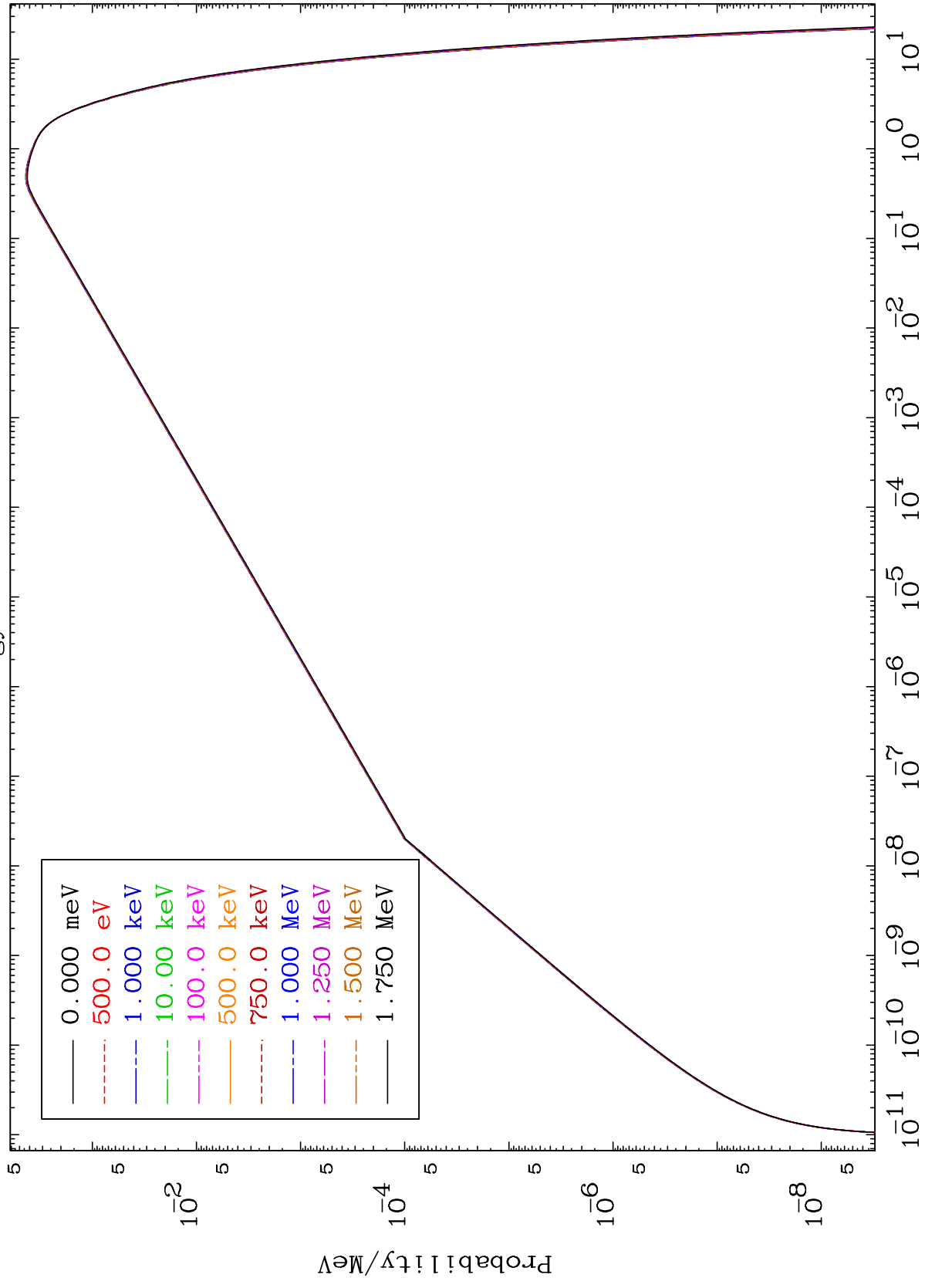
Incident Energy (MeV)

102-No-257

MAT 9975

Fission Energy Distribution

102-No-257

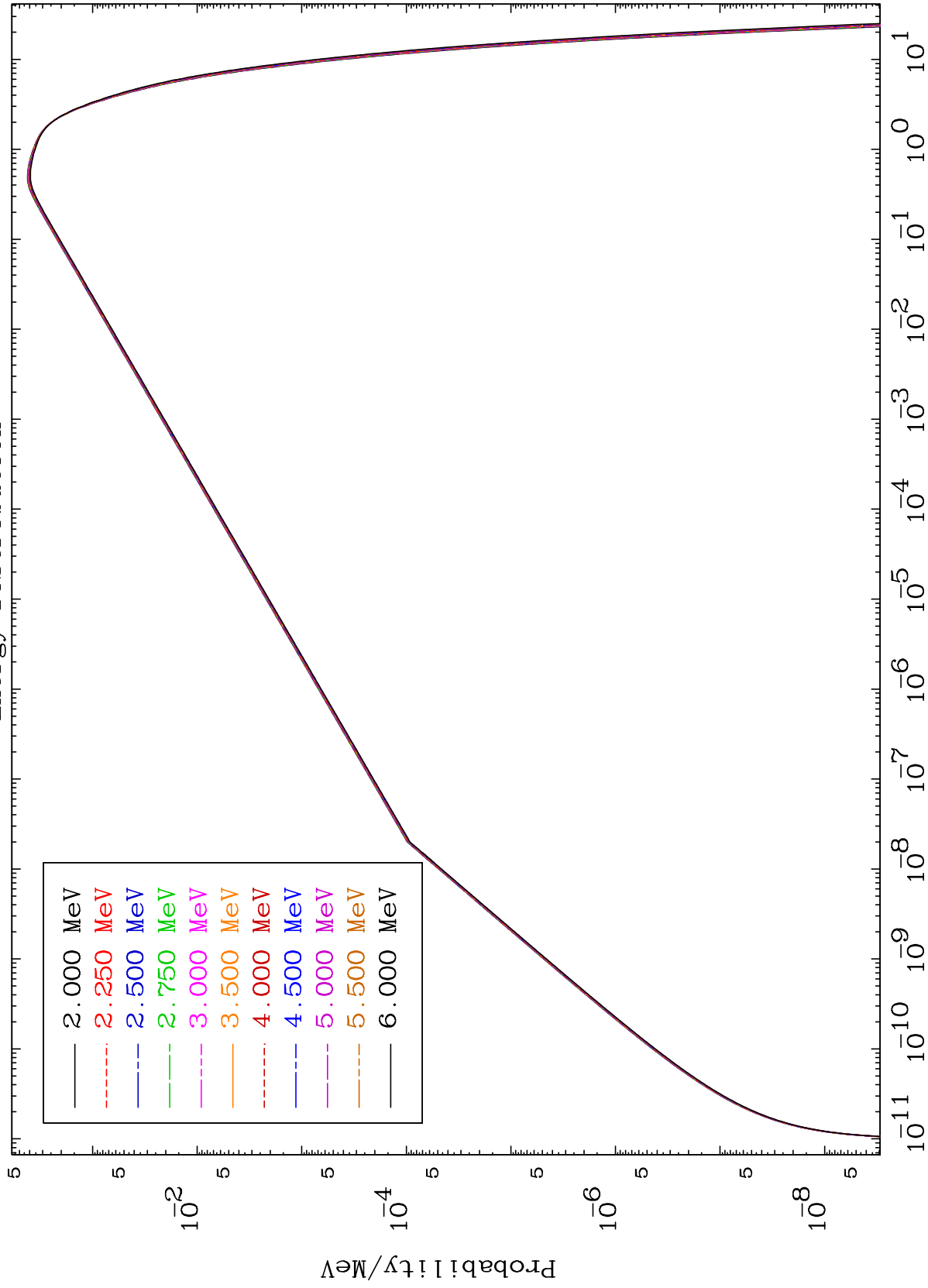


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MAT 9975

Fission Energy Distribution

102-No-257

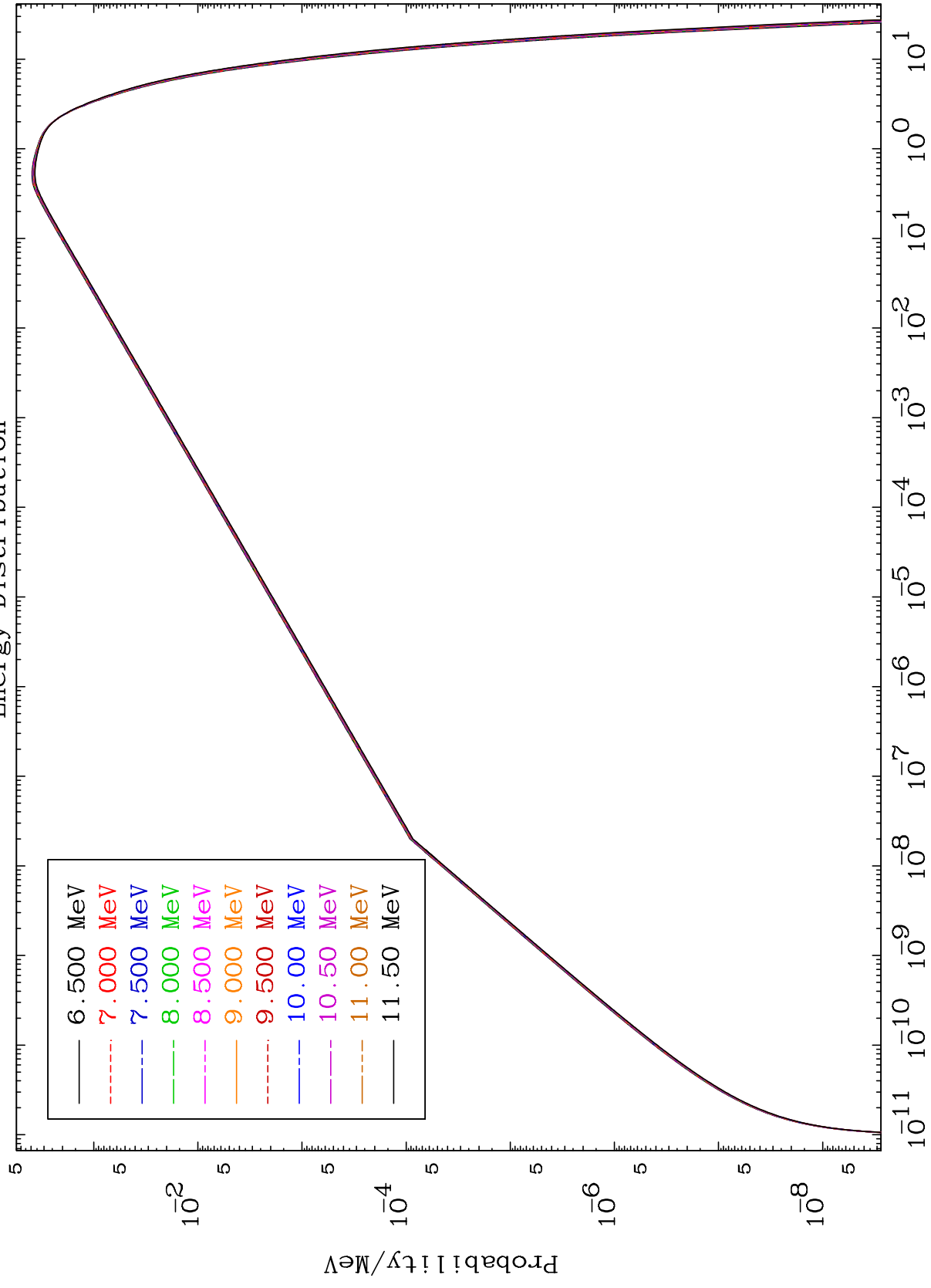


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MAT 9975

Fission Energy Distribution

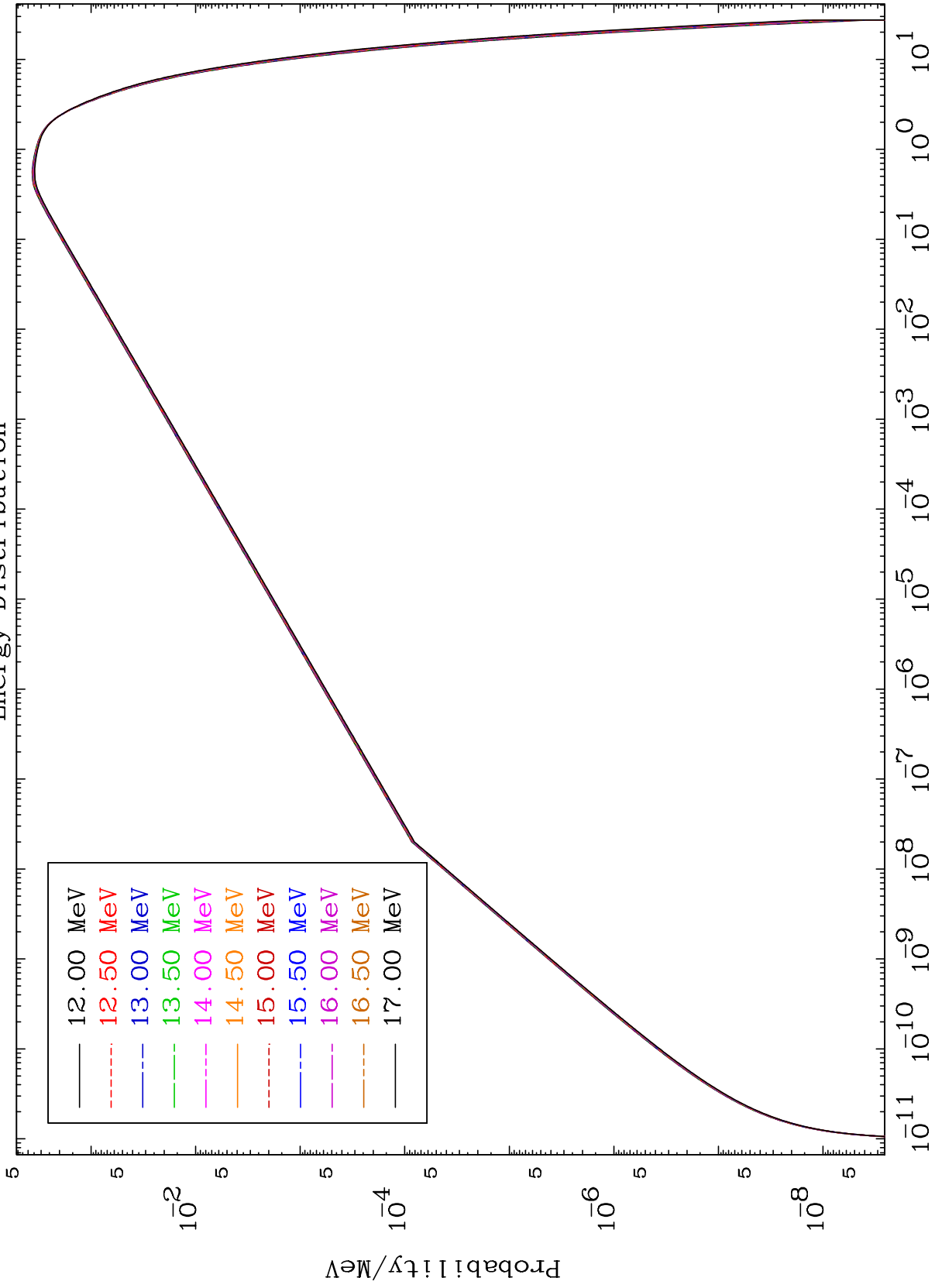
102-No-257



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Fission Energy Distribution

102-No-257

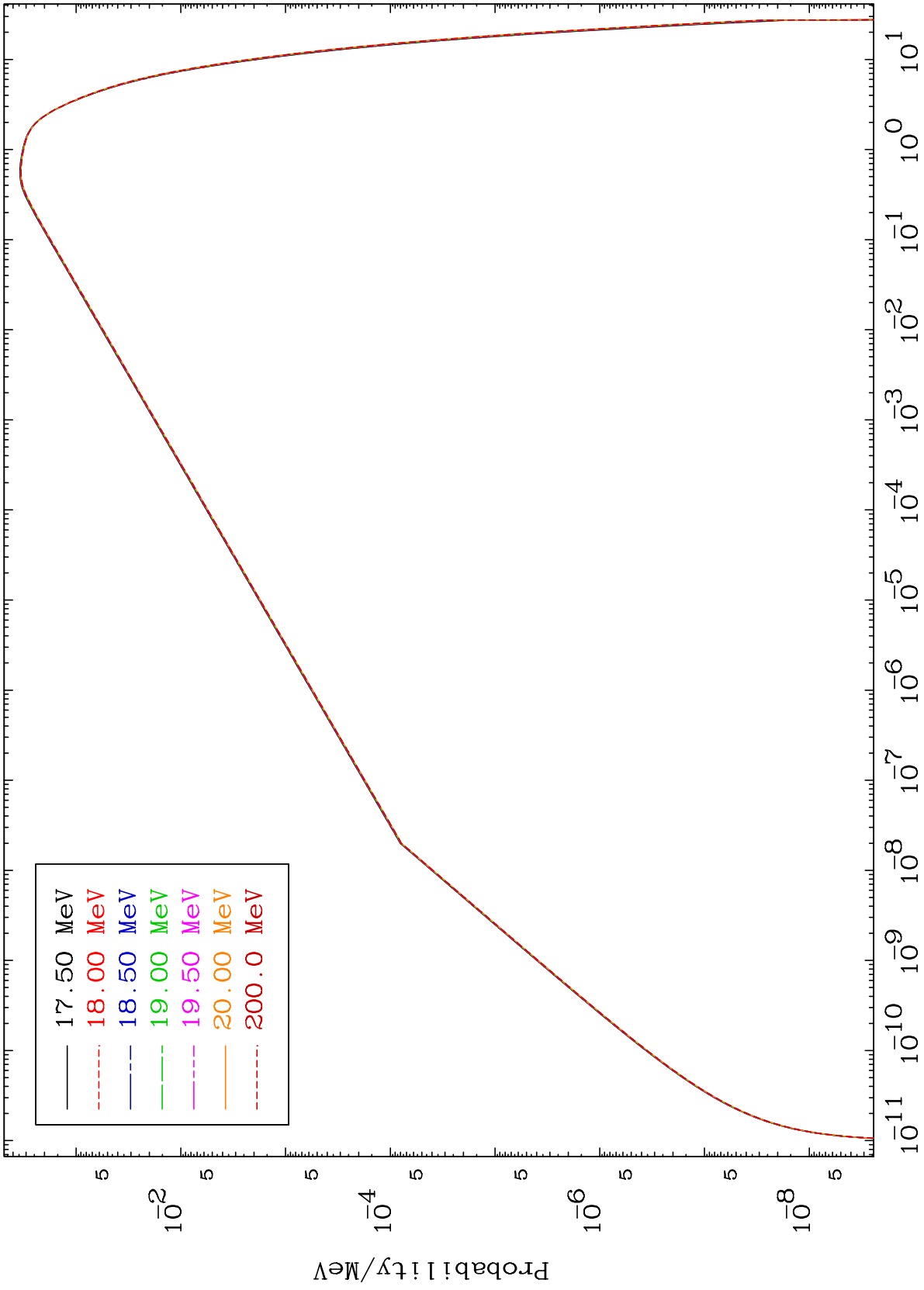


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Fission Energy Distribution

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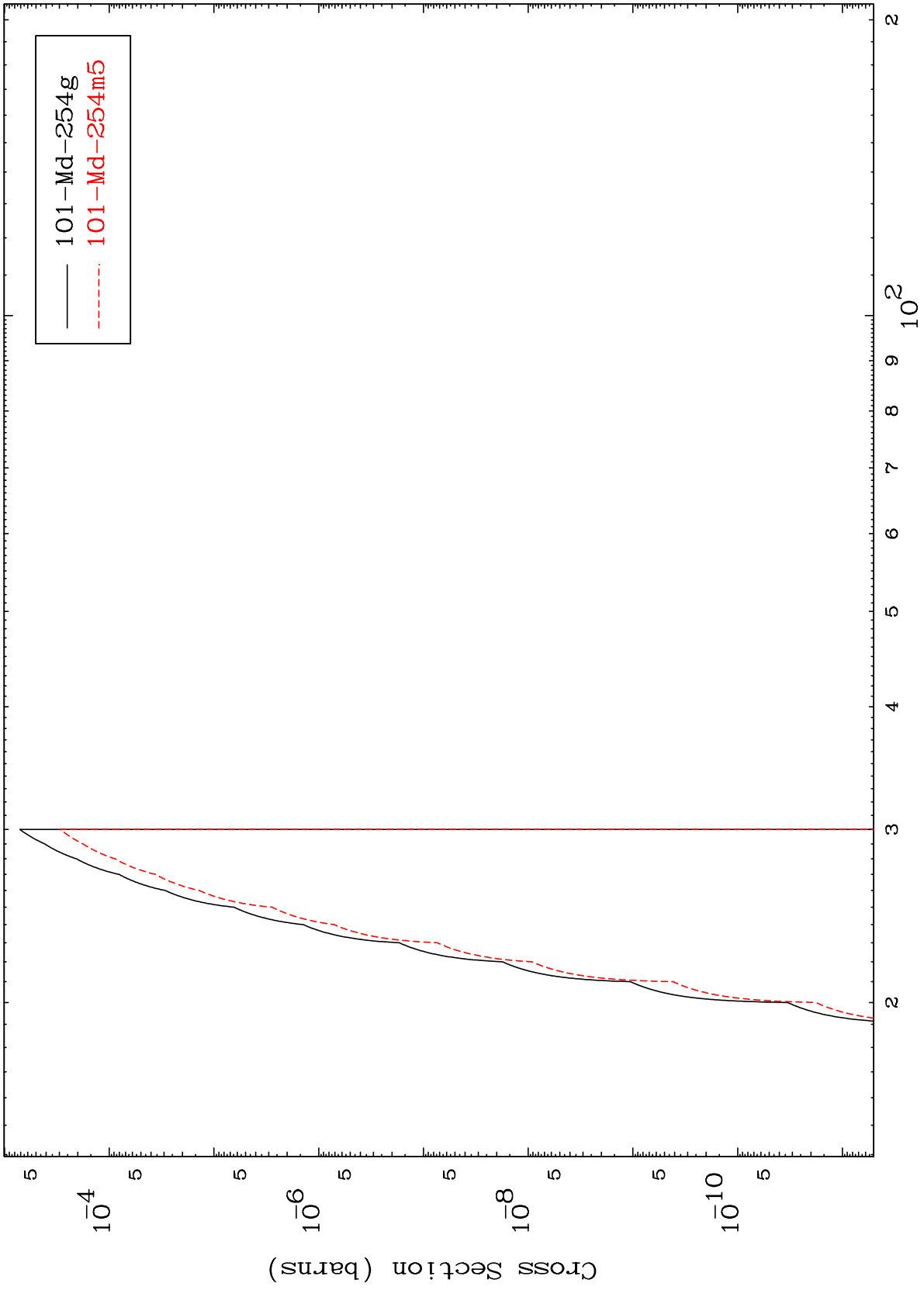


MAT 9975

(n,2n) d

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

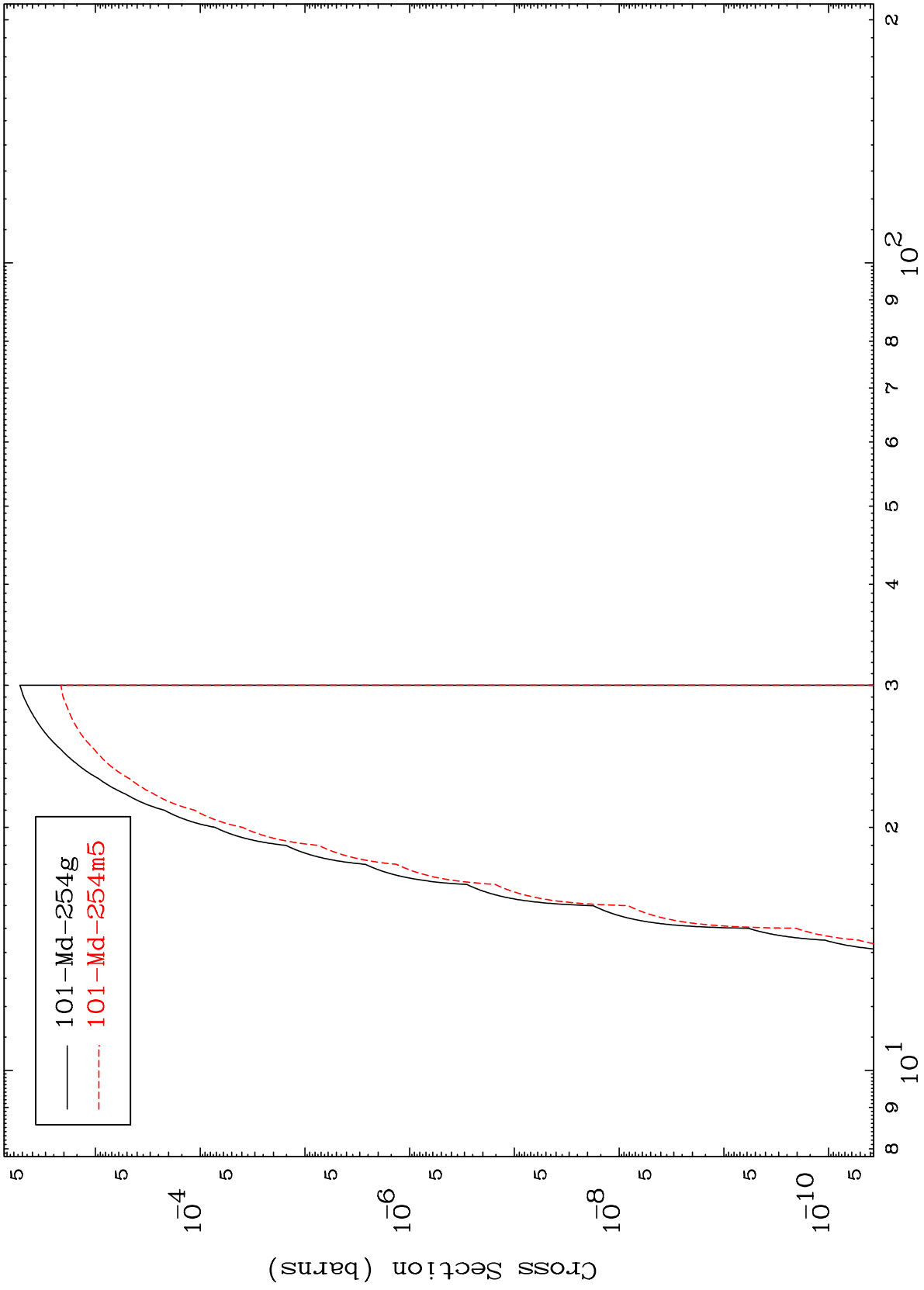
102-No-257

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(n,n') t

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Radionuclide Production Cross Section



26

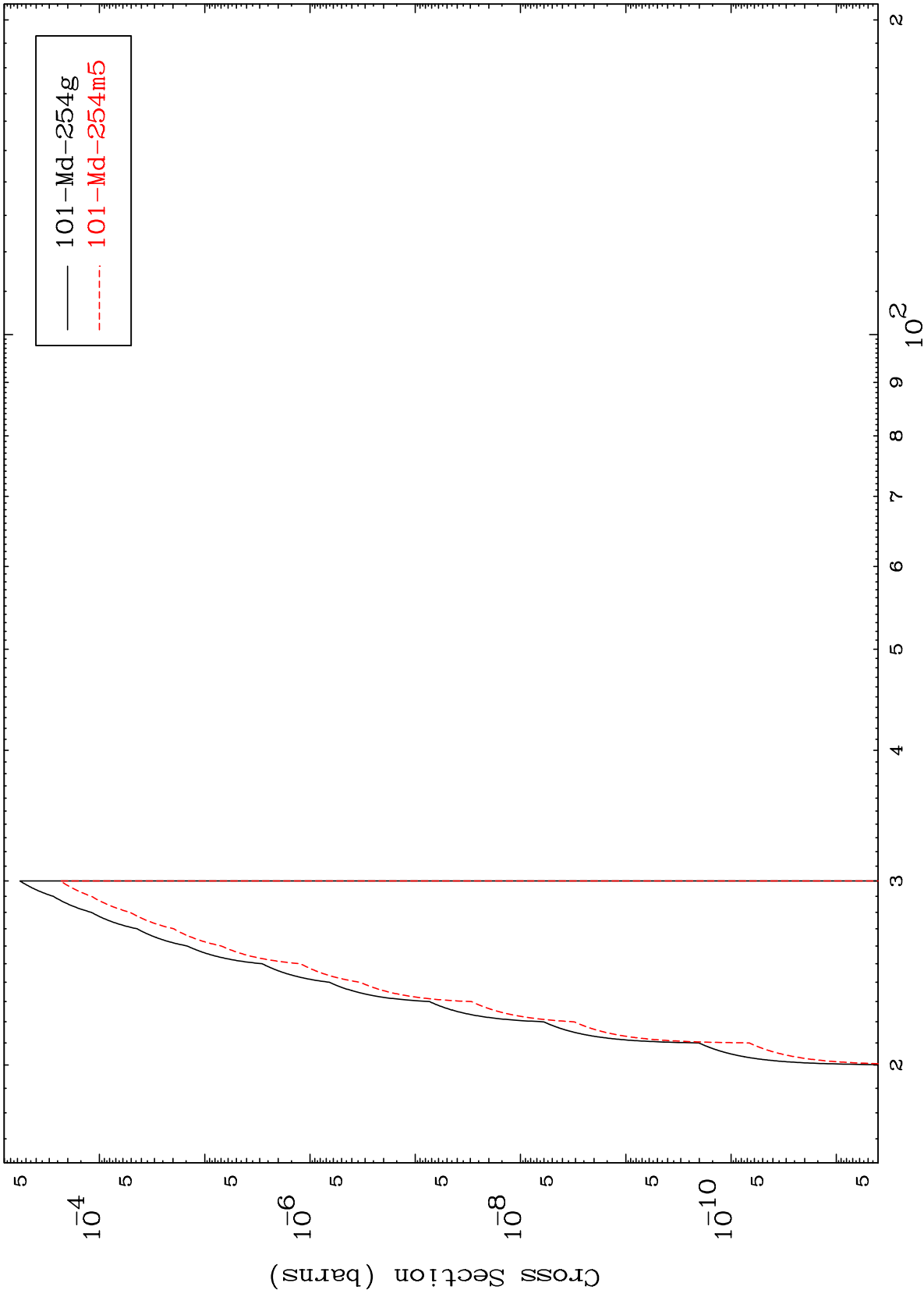
102-No-257

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(n,3n) p

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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