

Program EVALPLOT
(Version 2018-1)

by

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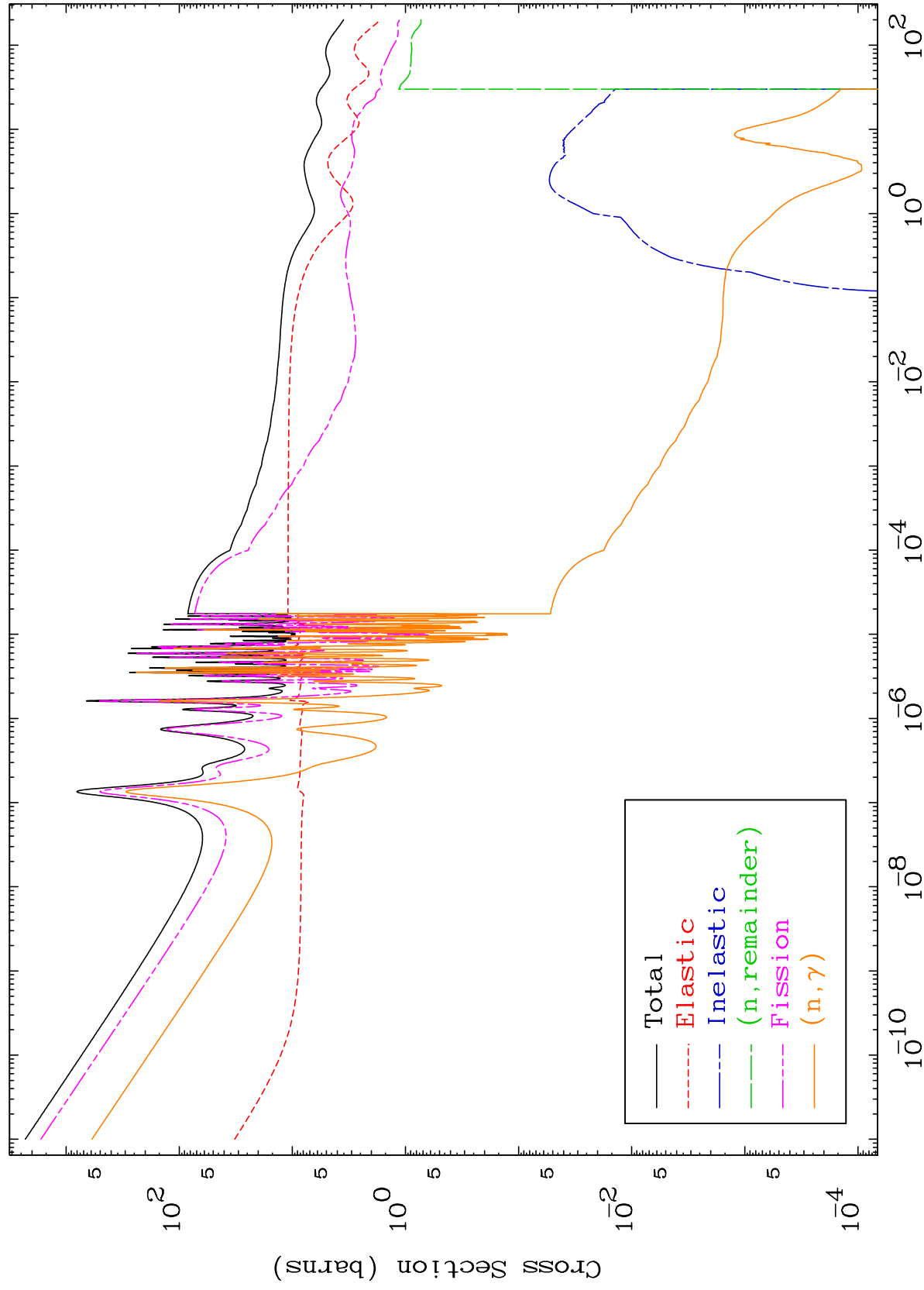
Press Mouse Button to Start

MAT 9328

Major

293 Kelvin Cross Sections

93-Np-231



1

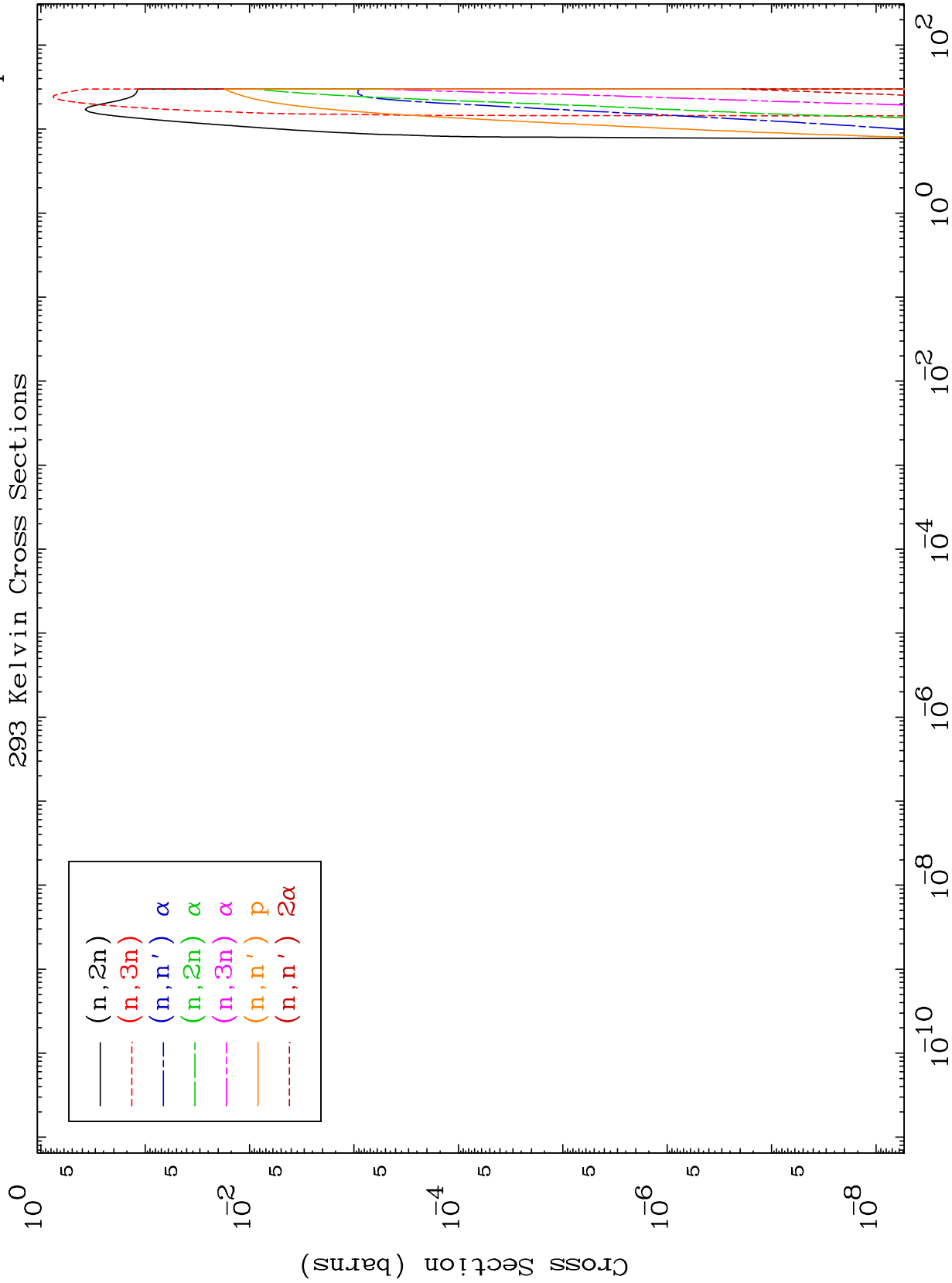
Incident Energy (MeV)

93-Np-231

MAT 9328

Neutron Production
293 Kelvin Cross Sections

93-Np-231



2

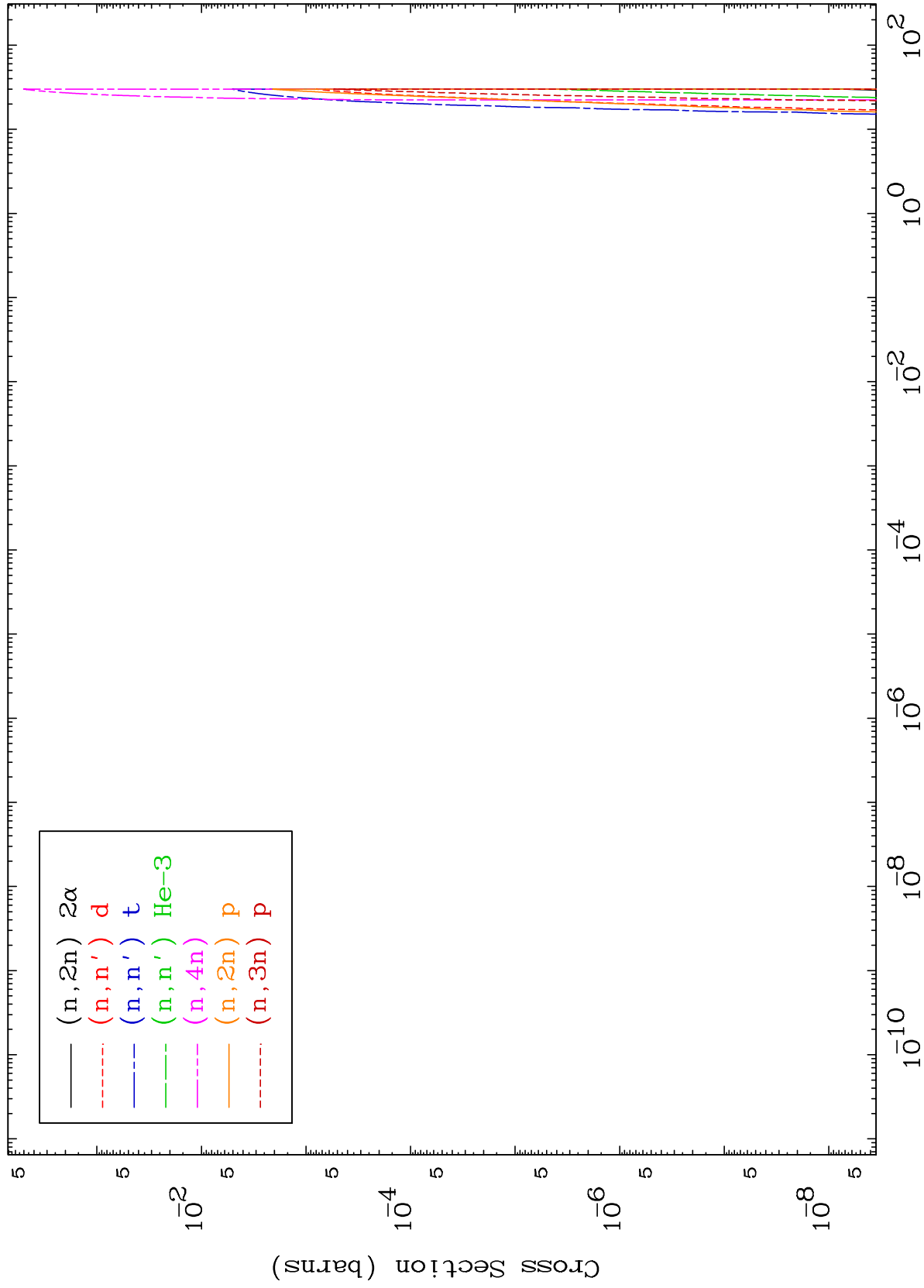
Incident Energy (MeV)

93-Np-231

MAT 9328

Neutron Production
293 Kelvin Cross Sections

93-Np-231



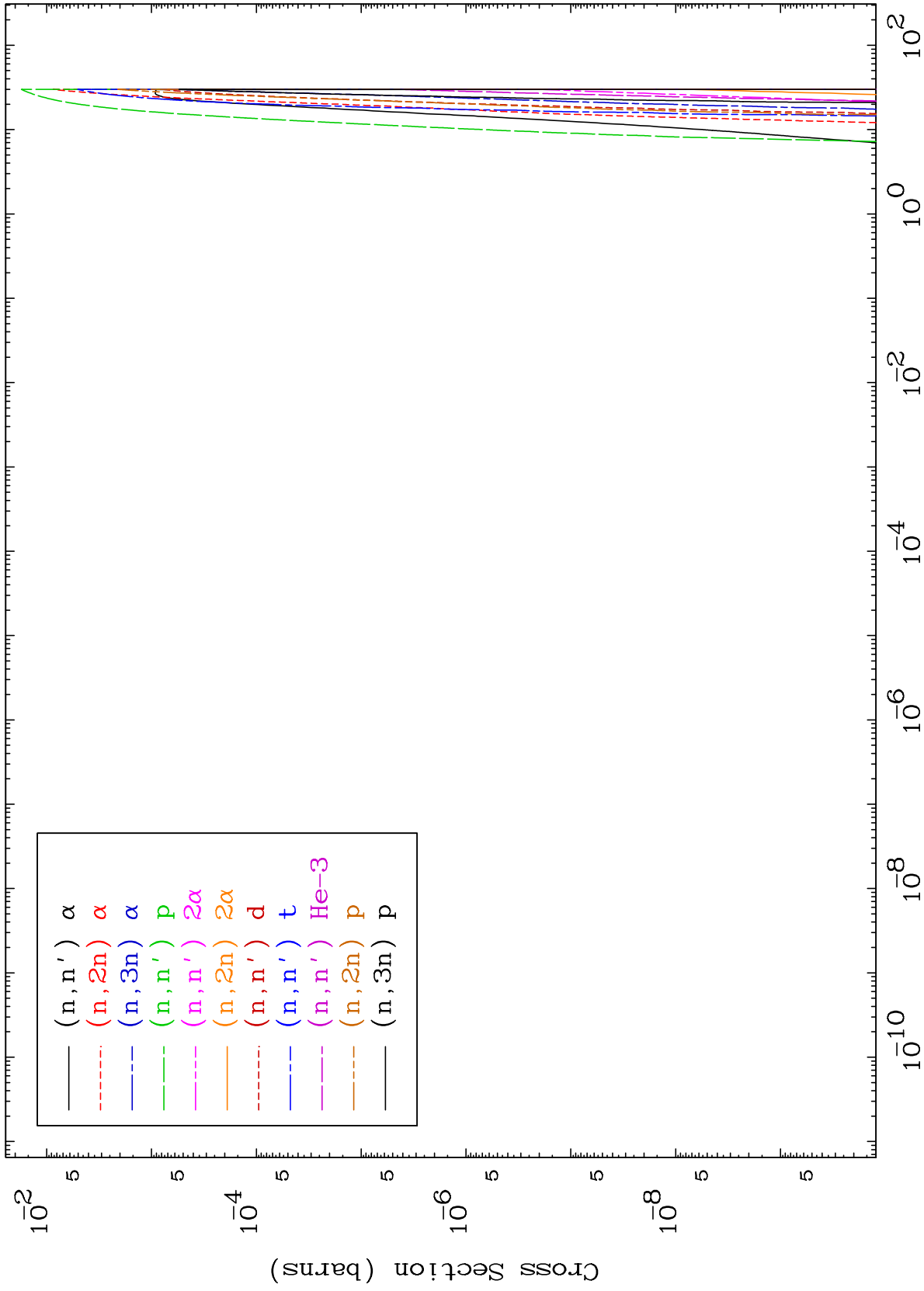
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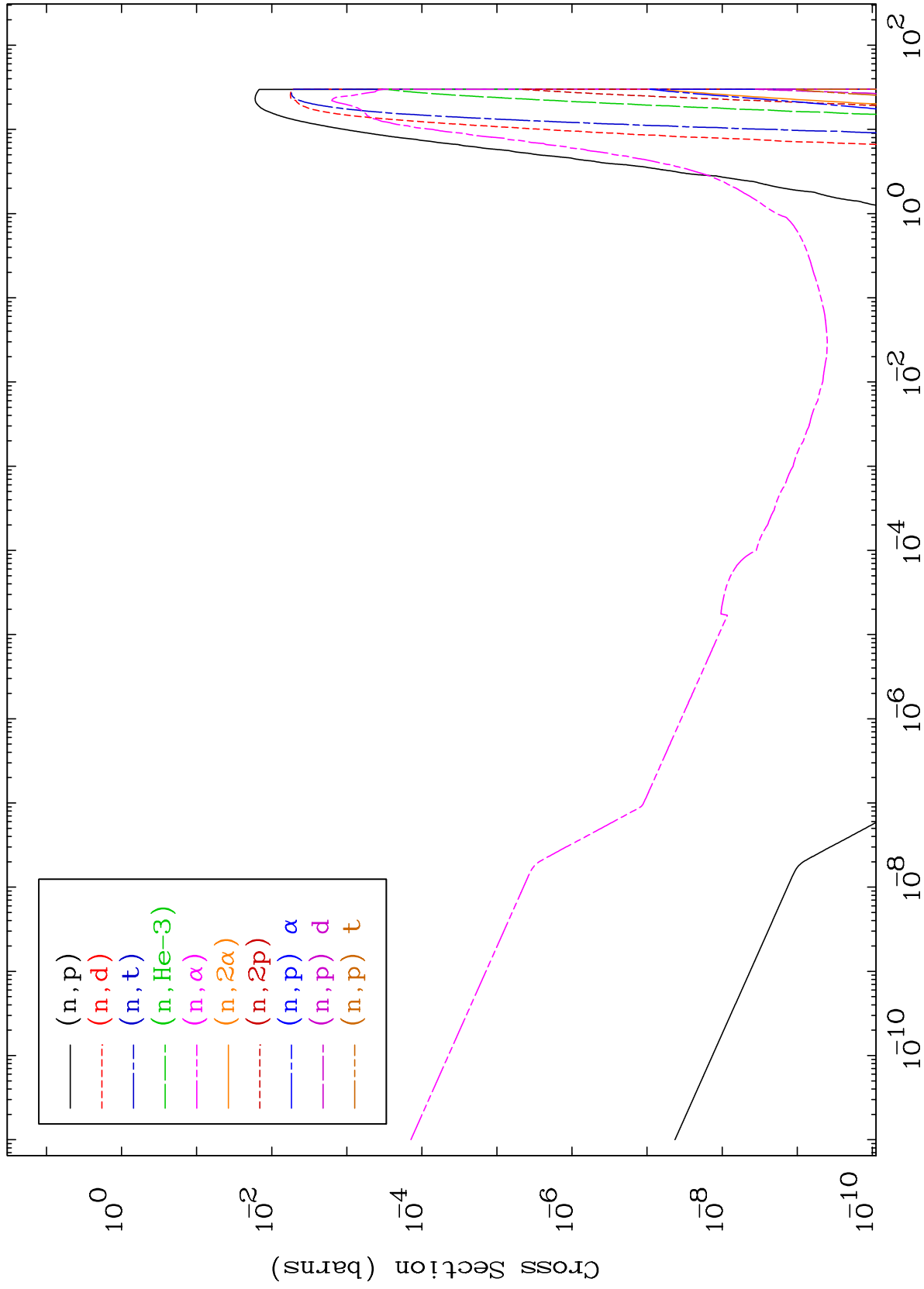
93-Np-231

MAT 9328

Charged Particle
293 Kelvin Cross Sections

93-Np-231

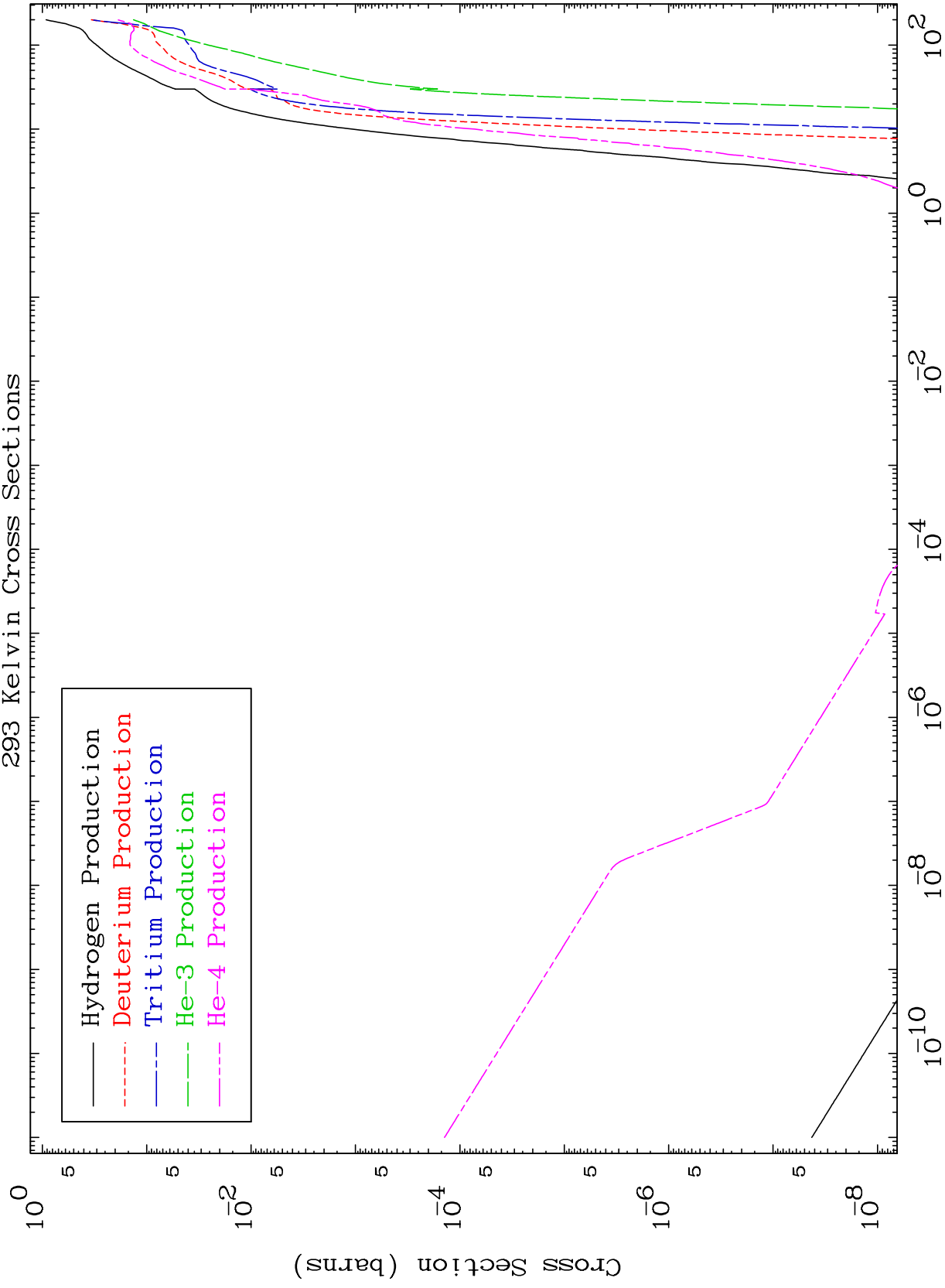




MAT 9328

Particle Production
293 Kelvin Cross Sections

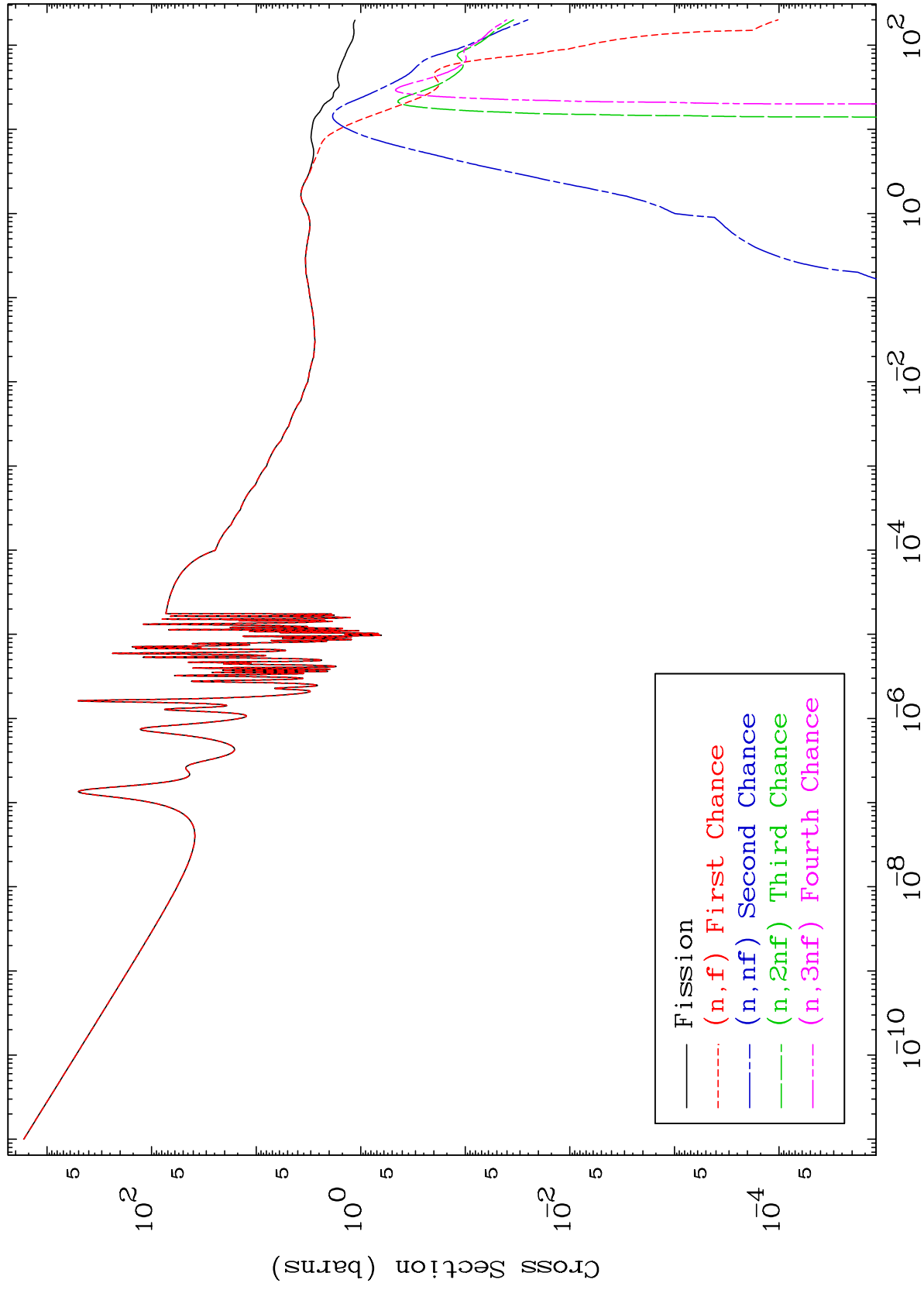
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MAT 9328

Fission
293 Kelvin Cross Sections

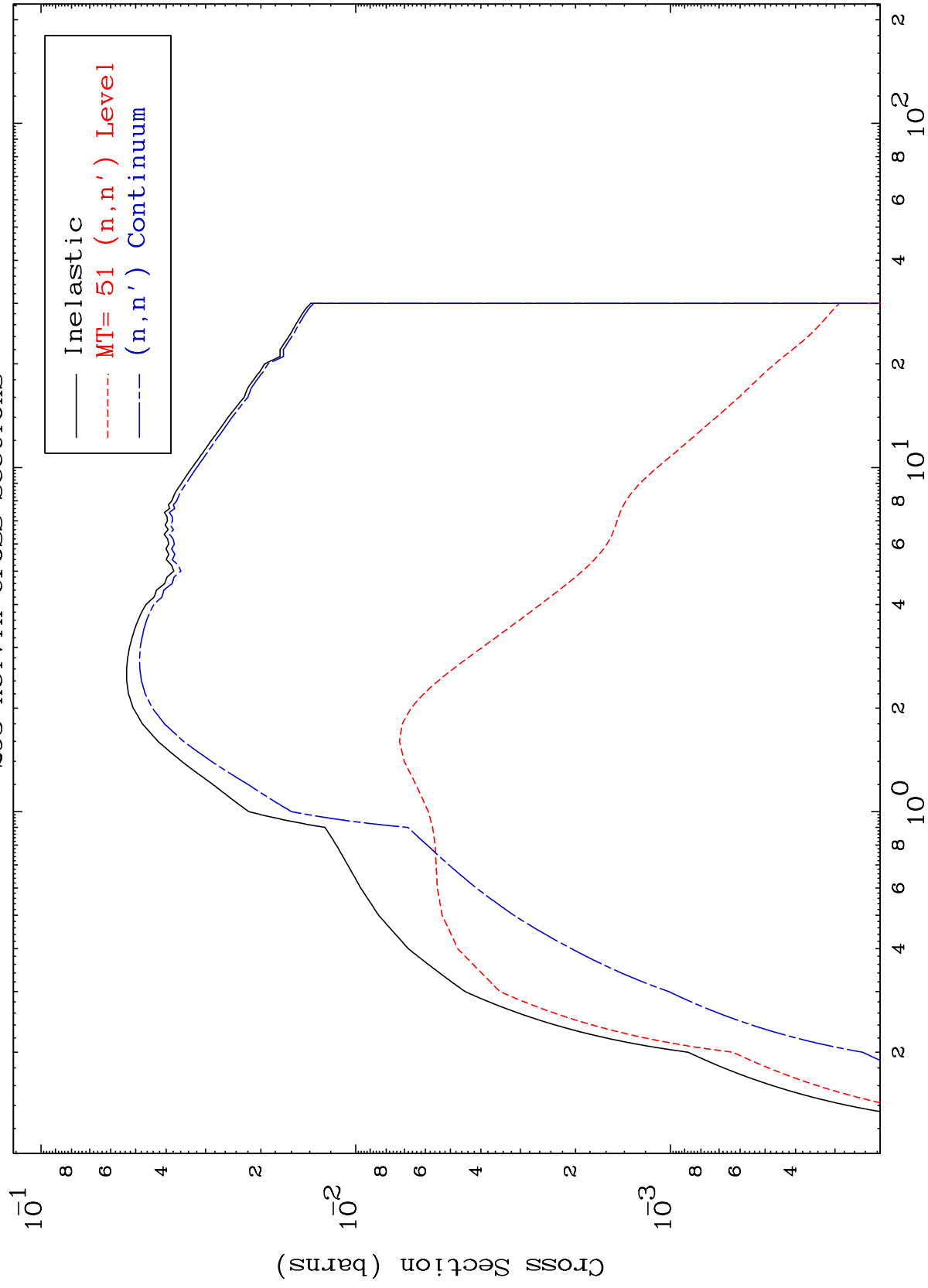
93-Np-231



MAT 9328

(n,n') Level
293 Kelvin Cross Sections

93-Np-231



8

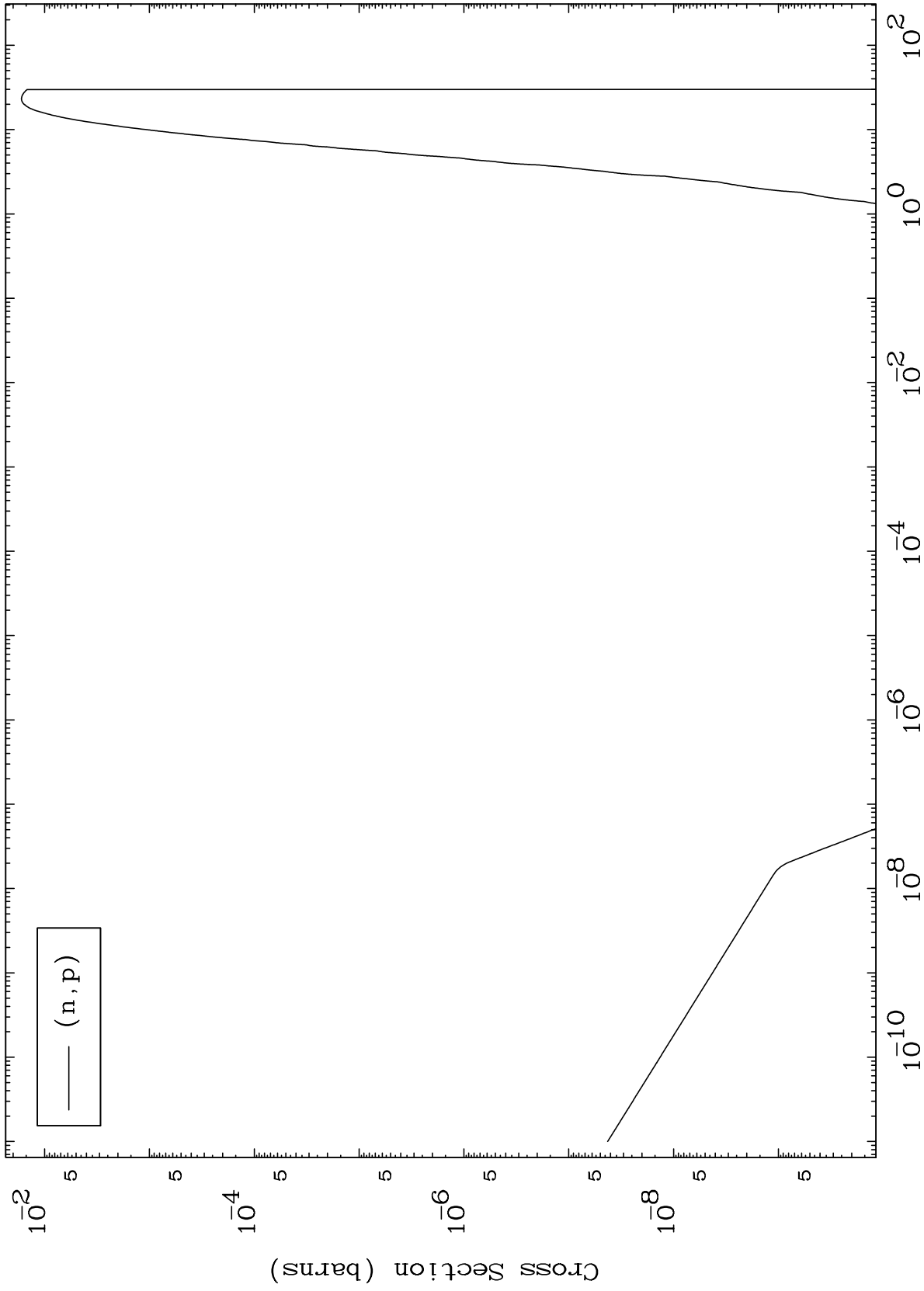
Incident Energy (MeV)

93-Np-231

MAT 9328

(n,p) Levels
293 Kelvin Cross Sections

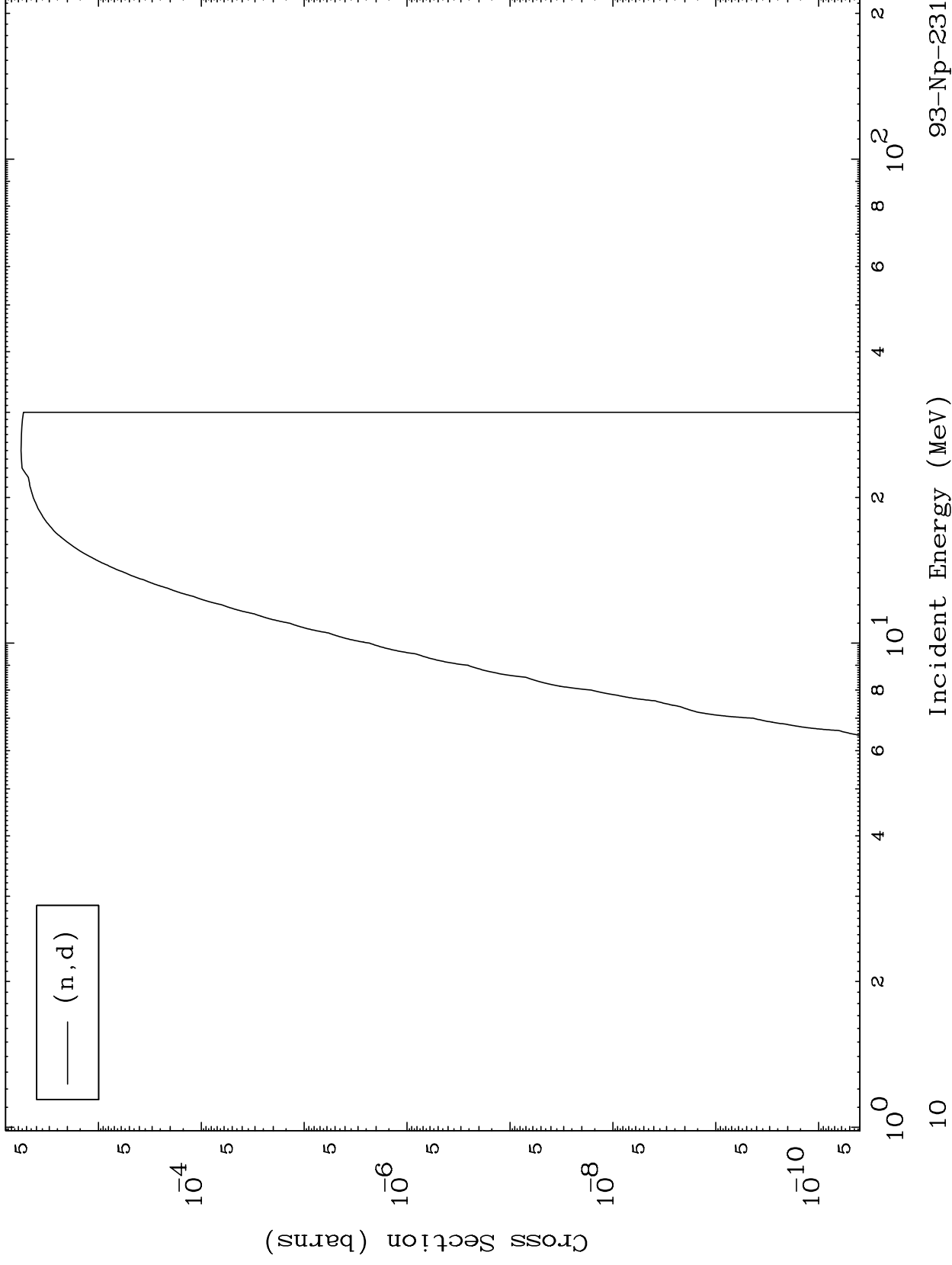
93-Np-231



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(n,d) Levels
293 Kelvin Cross Sections

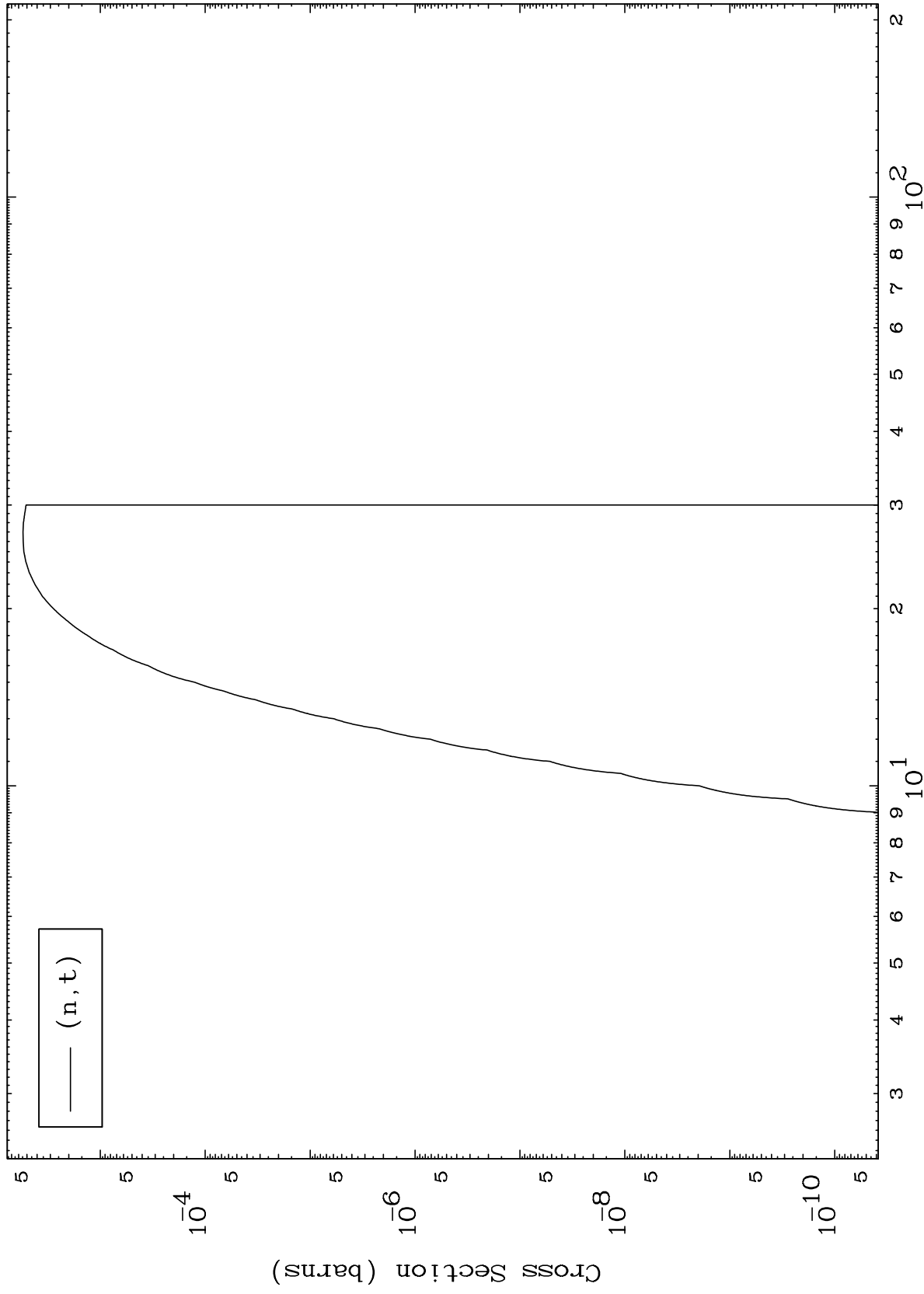
93-Np-231



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93-Np-231

(n,t) Levels
293 Kelvin Cross Sections



11

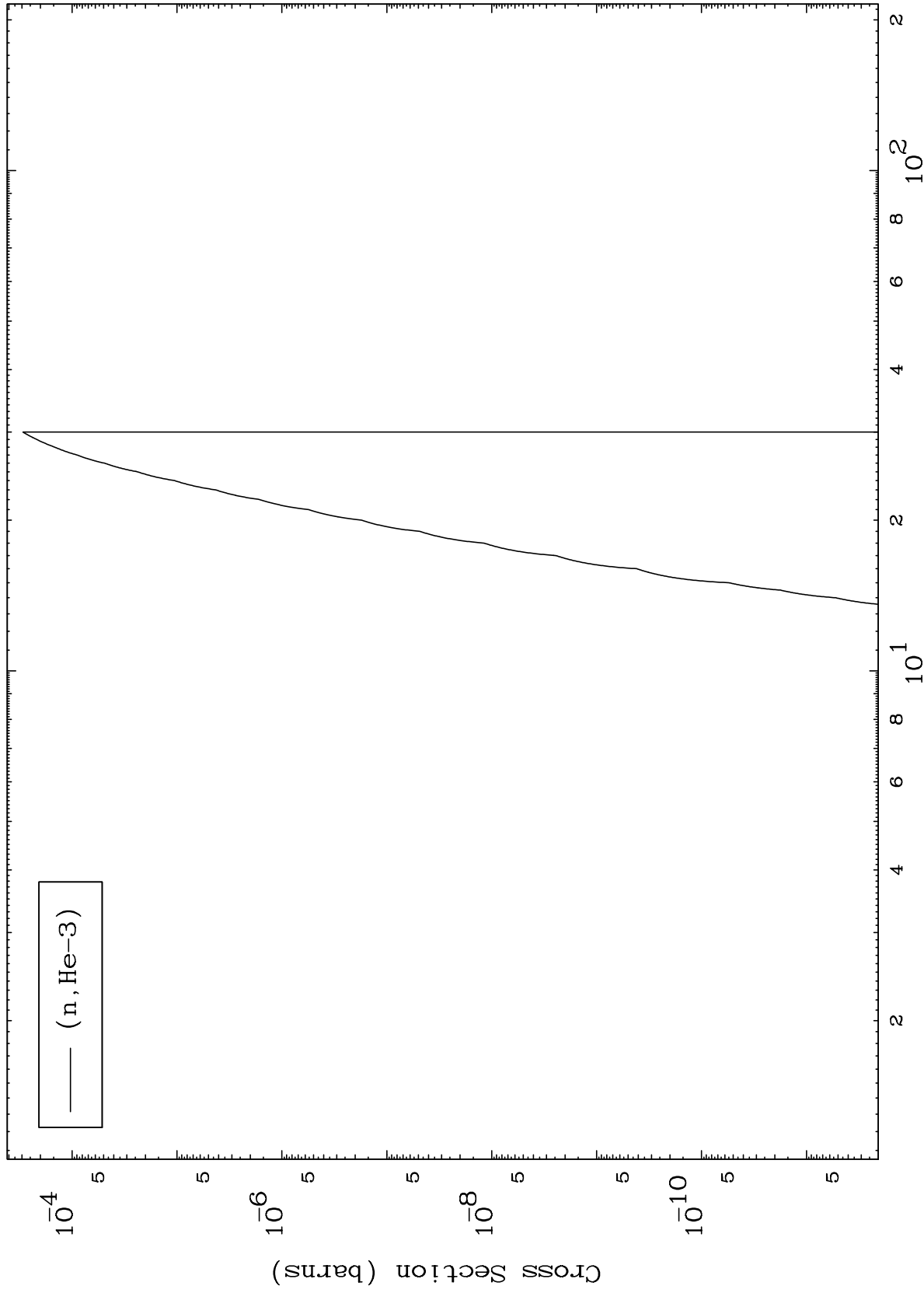
Incident Energy (MeV)

93-Np-231

MAT 9328

(n,He3) Levels
293 Kelvin Cross Sections

93-Np-231



12

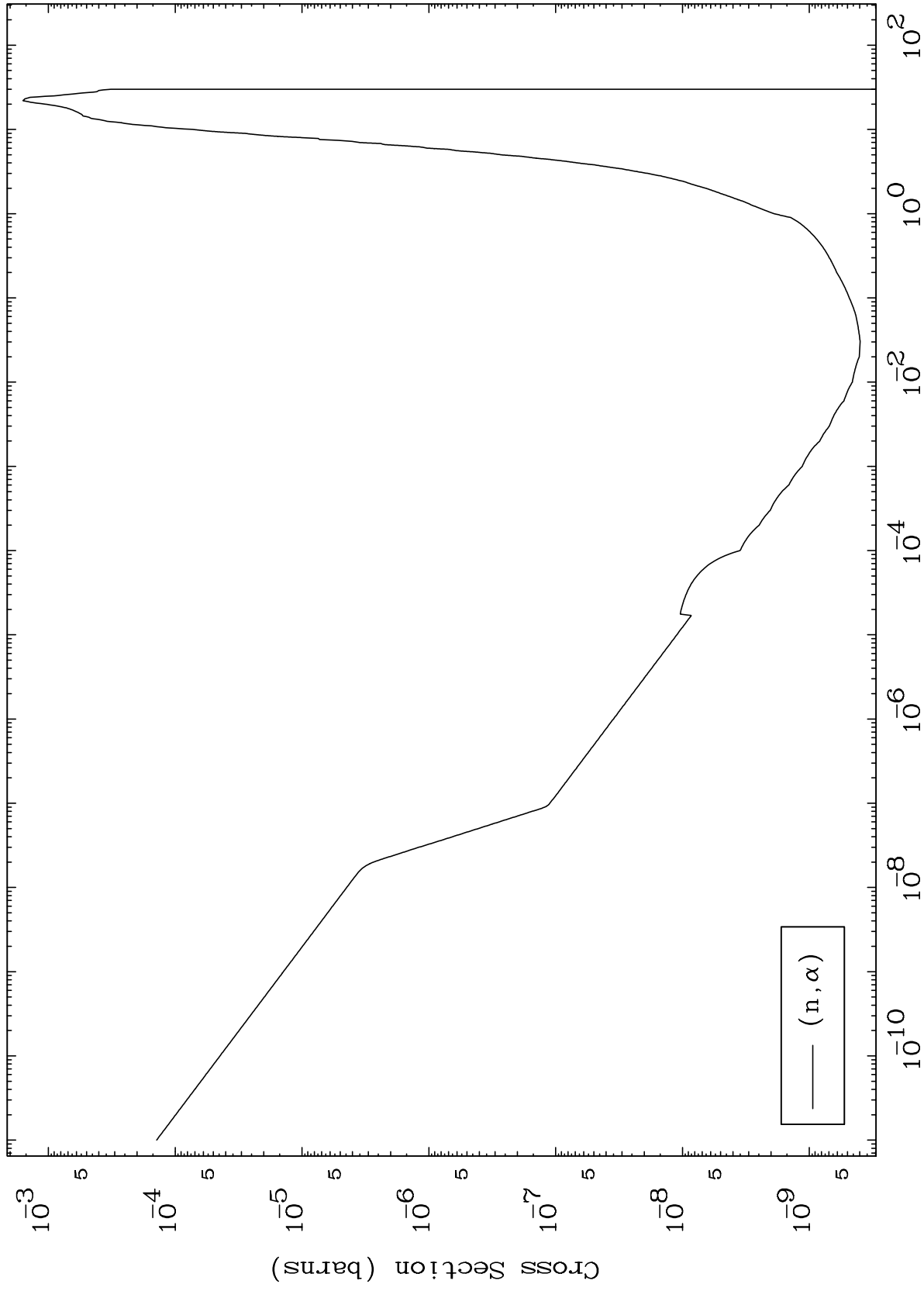
Incident Energy (MeV)

93-Np-231

MAT 9328

(n, α) Levels
293 Kelvin Cross Sections

93-Np-231



13

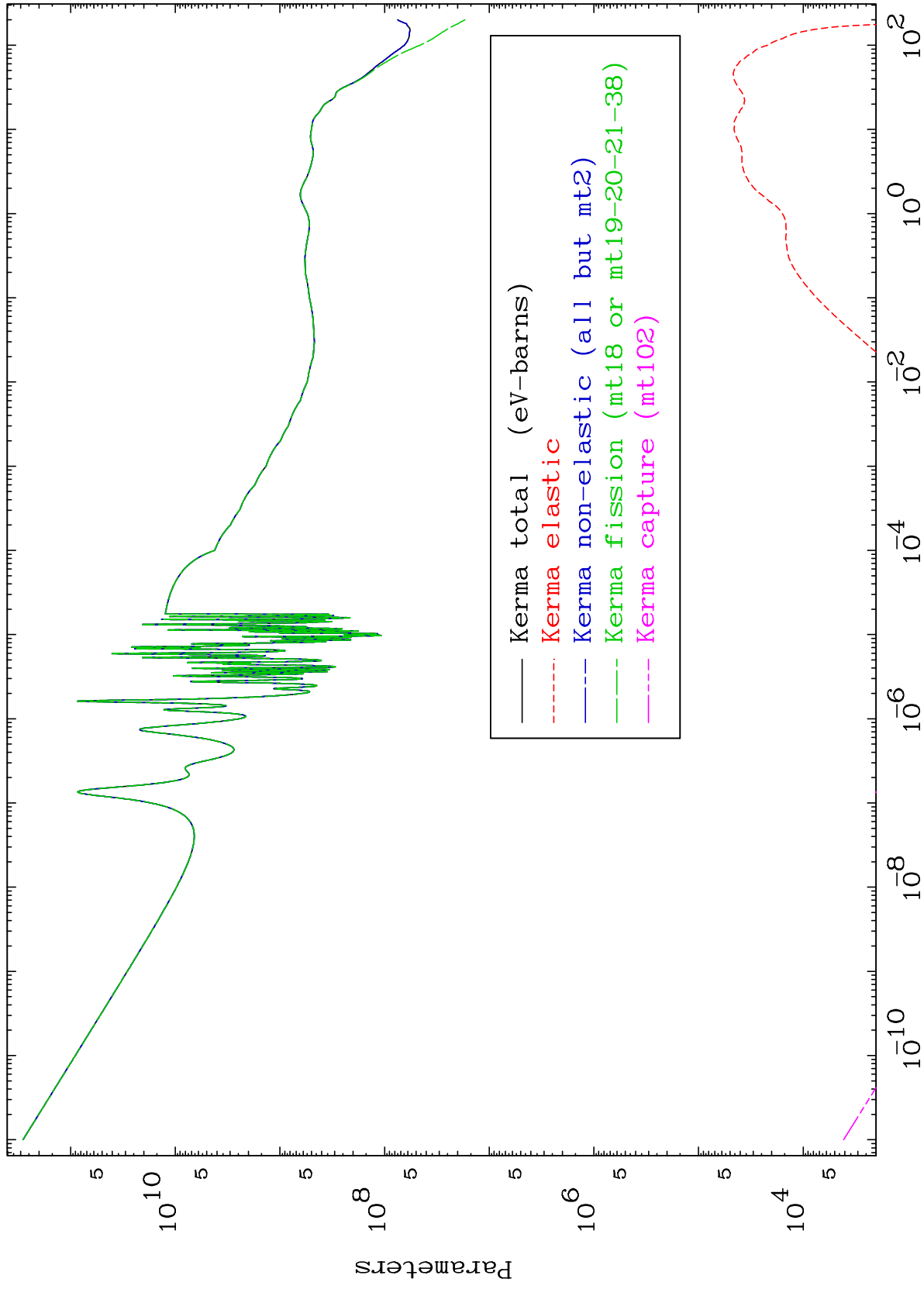
Incident Energy (MeV)

93-Np-231

MAT 9328

Energy Release
Parameters

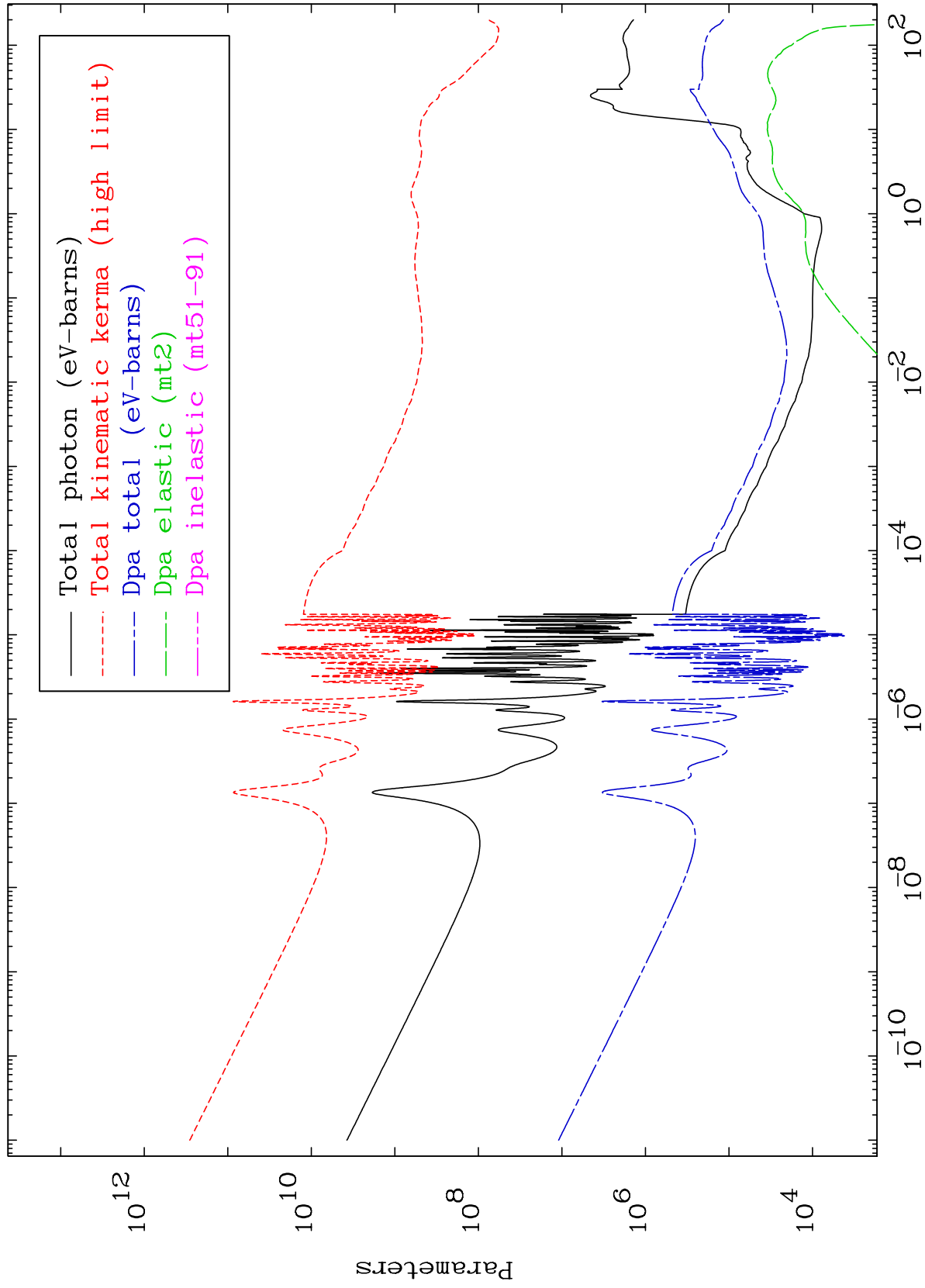
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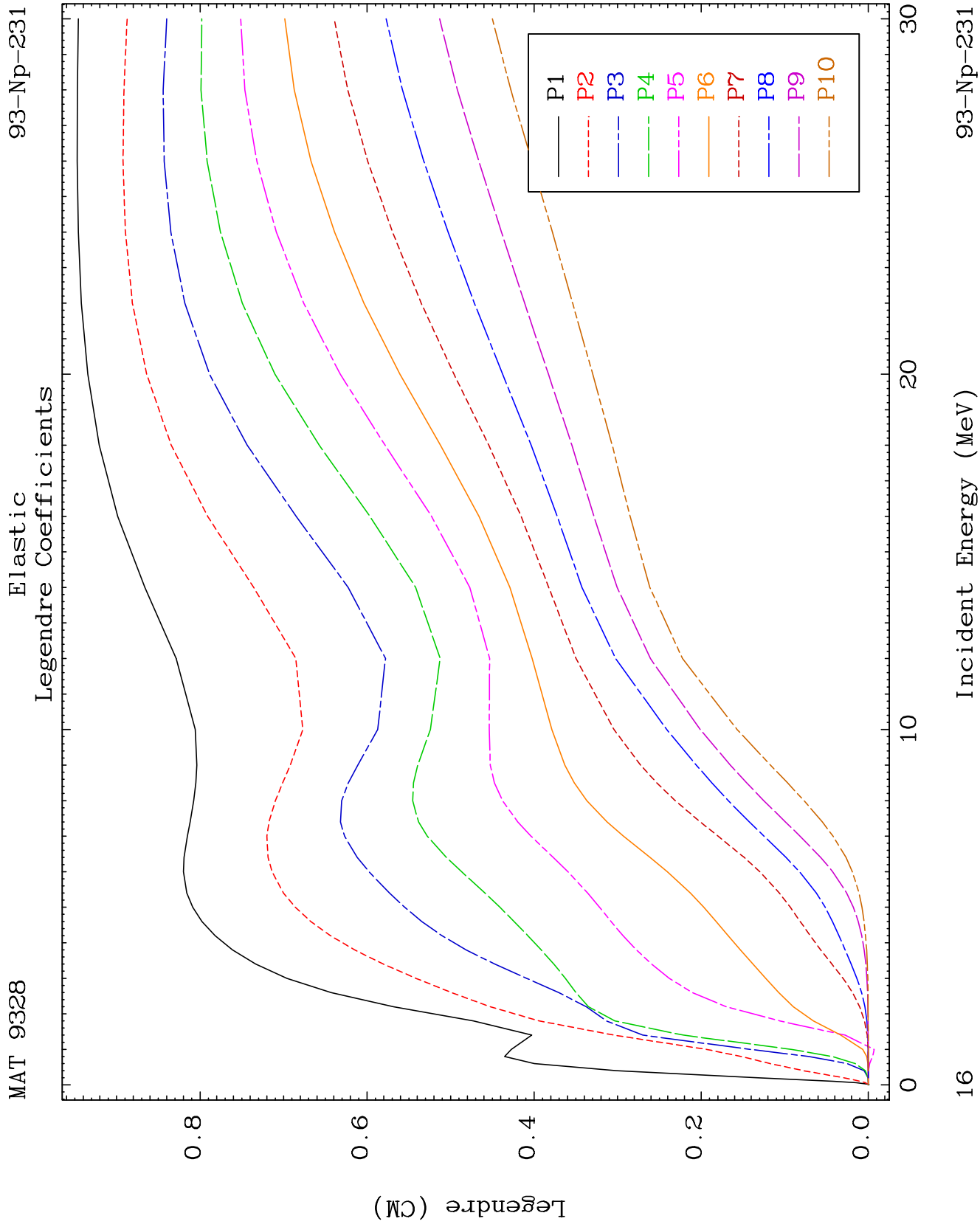


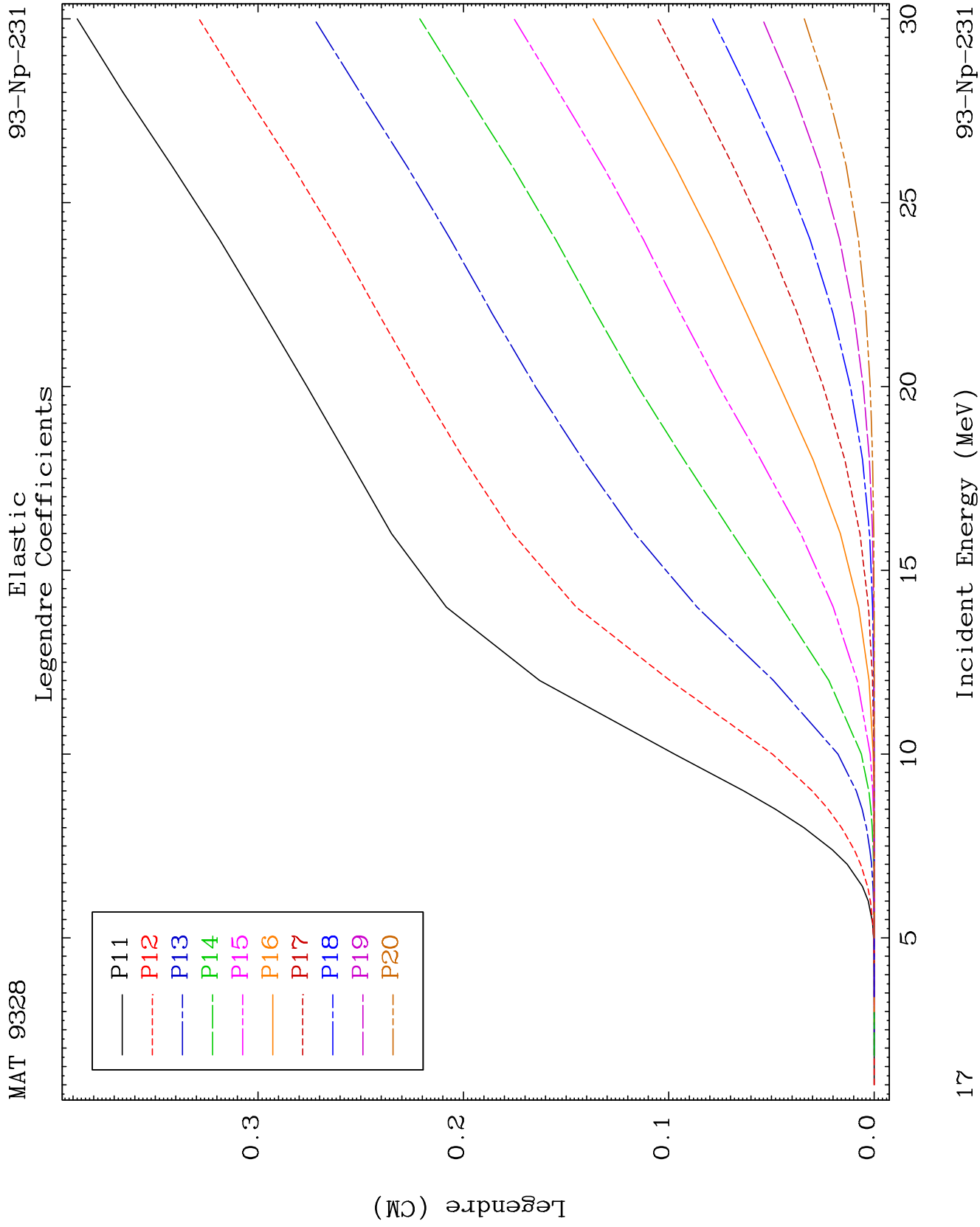
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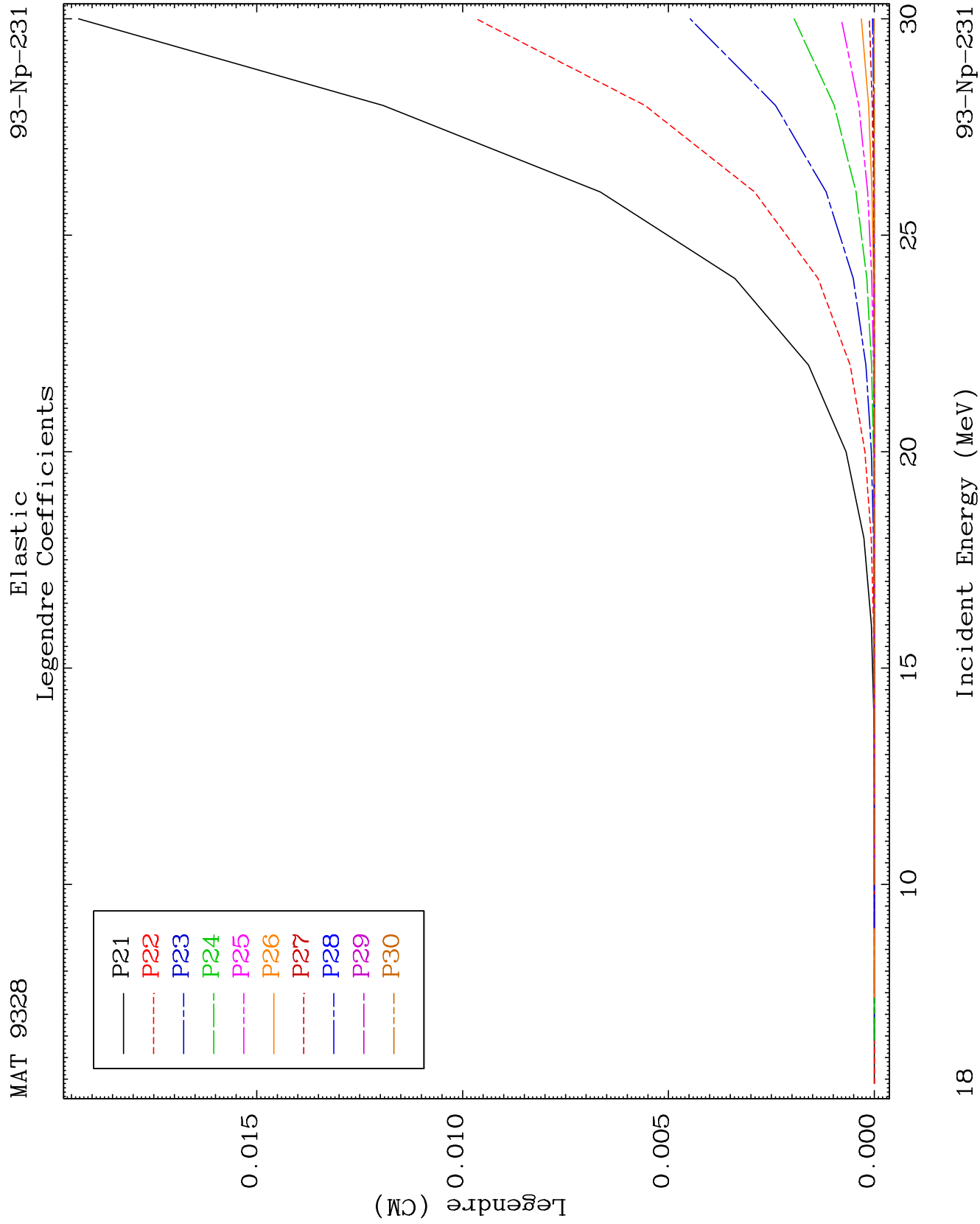
Incident Energy (MeV)

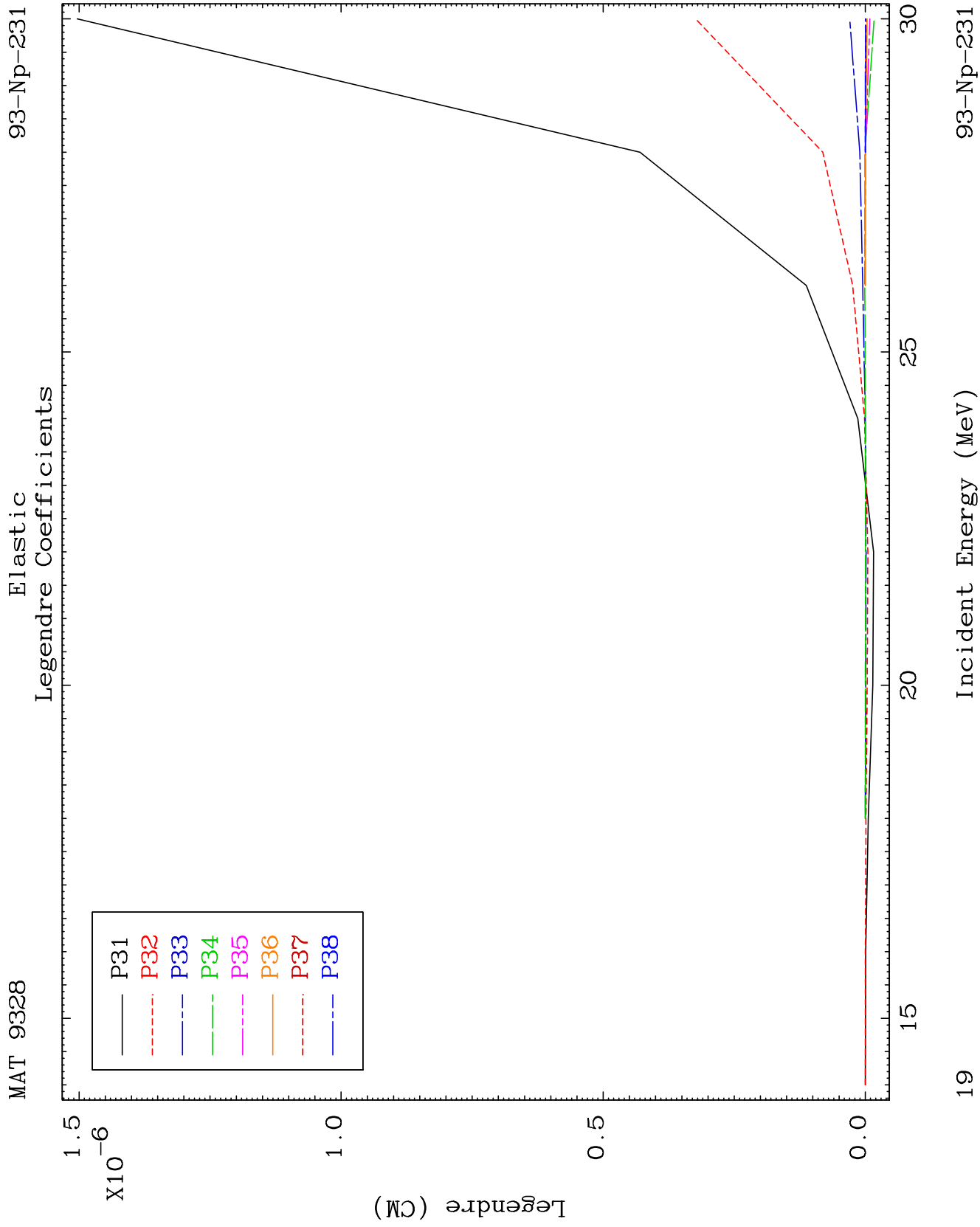
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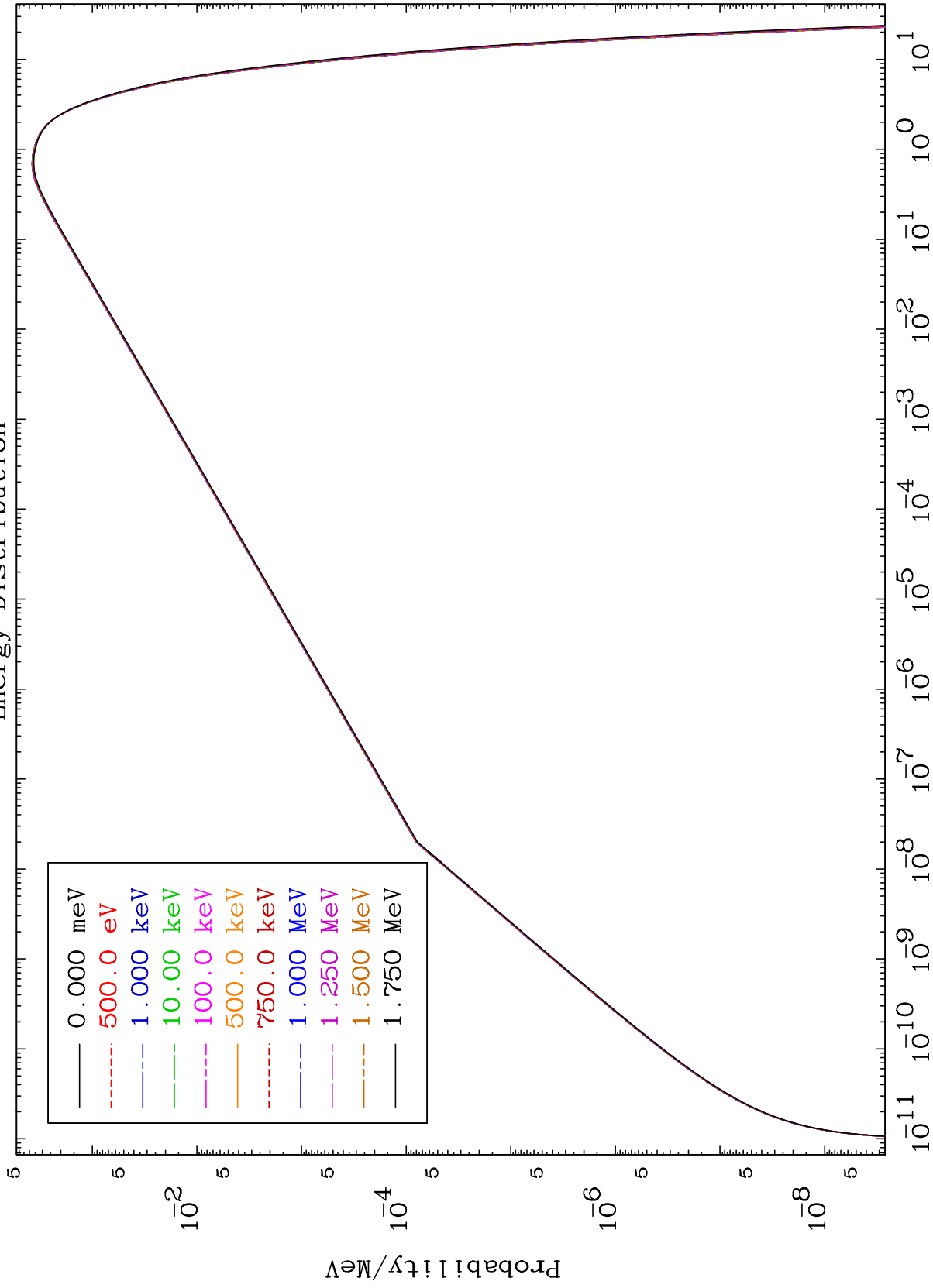




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Fission Energy Distribution

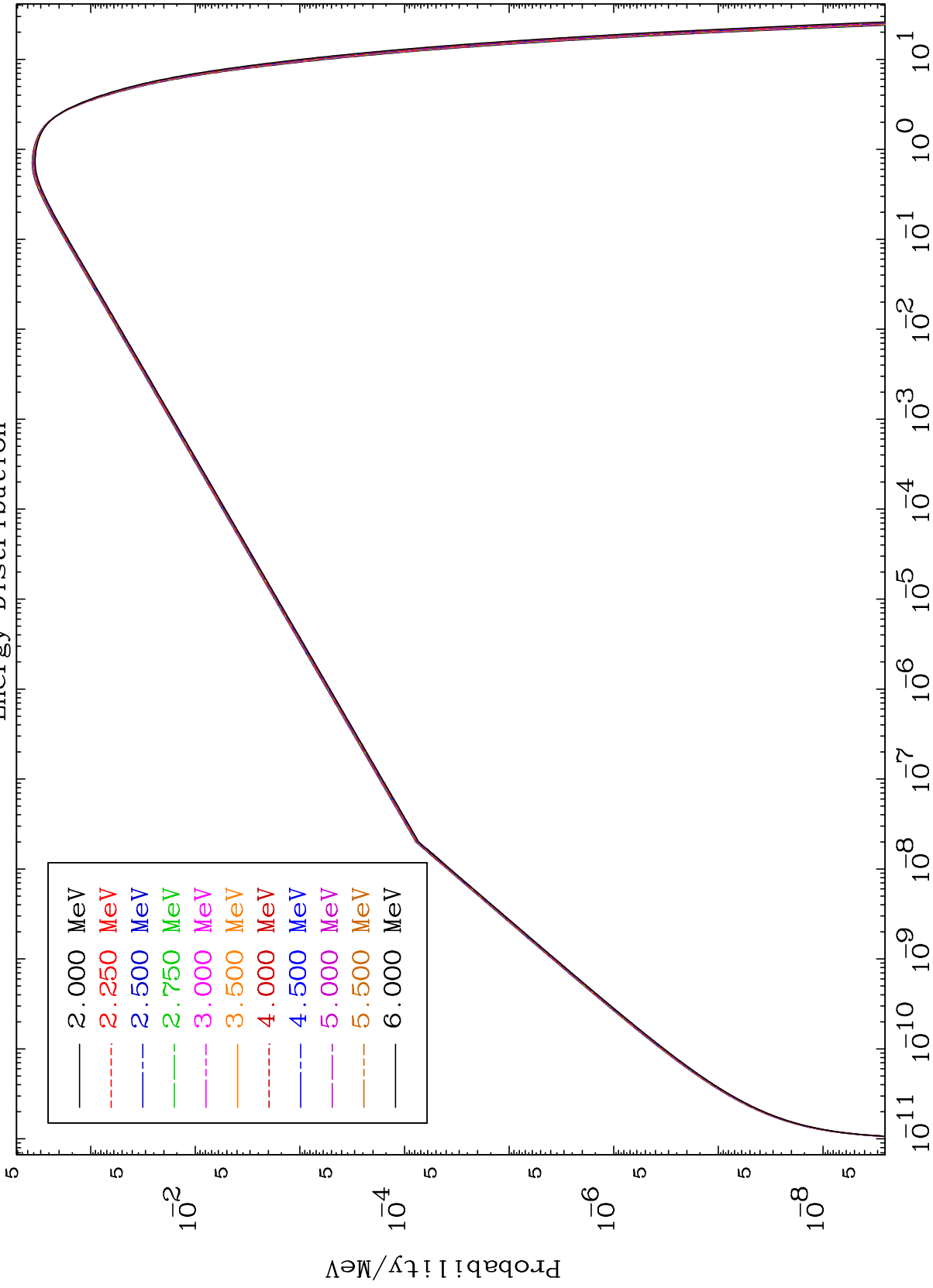
93-Np-231



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Fission
Energy Distribution

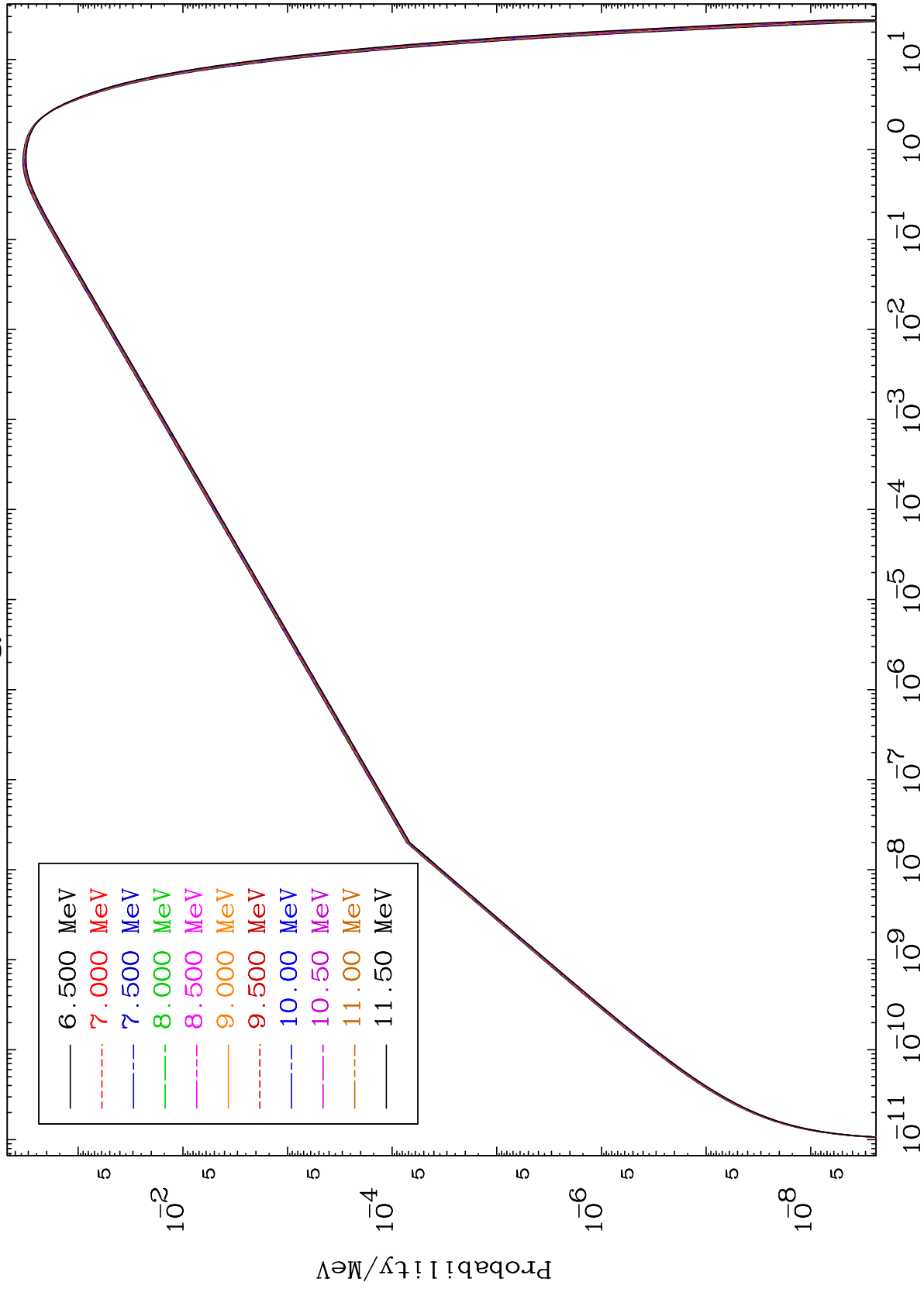
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Fission
Energy Distribution

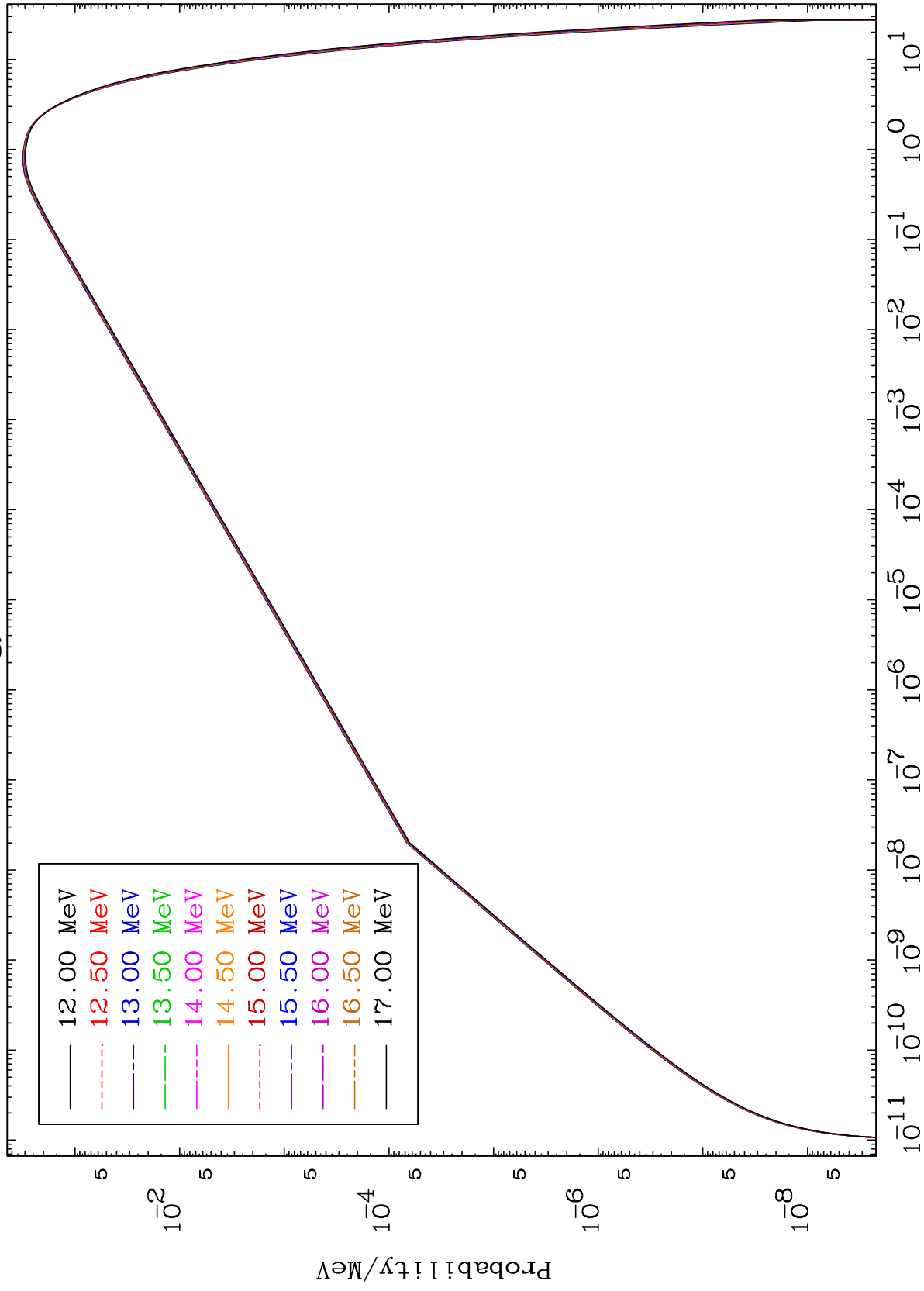
93-Np-231



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Fission
Energy Distribution

93-Np-231



23

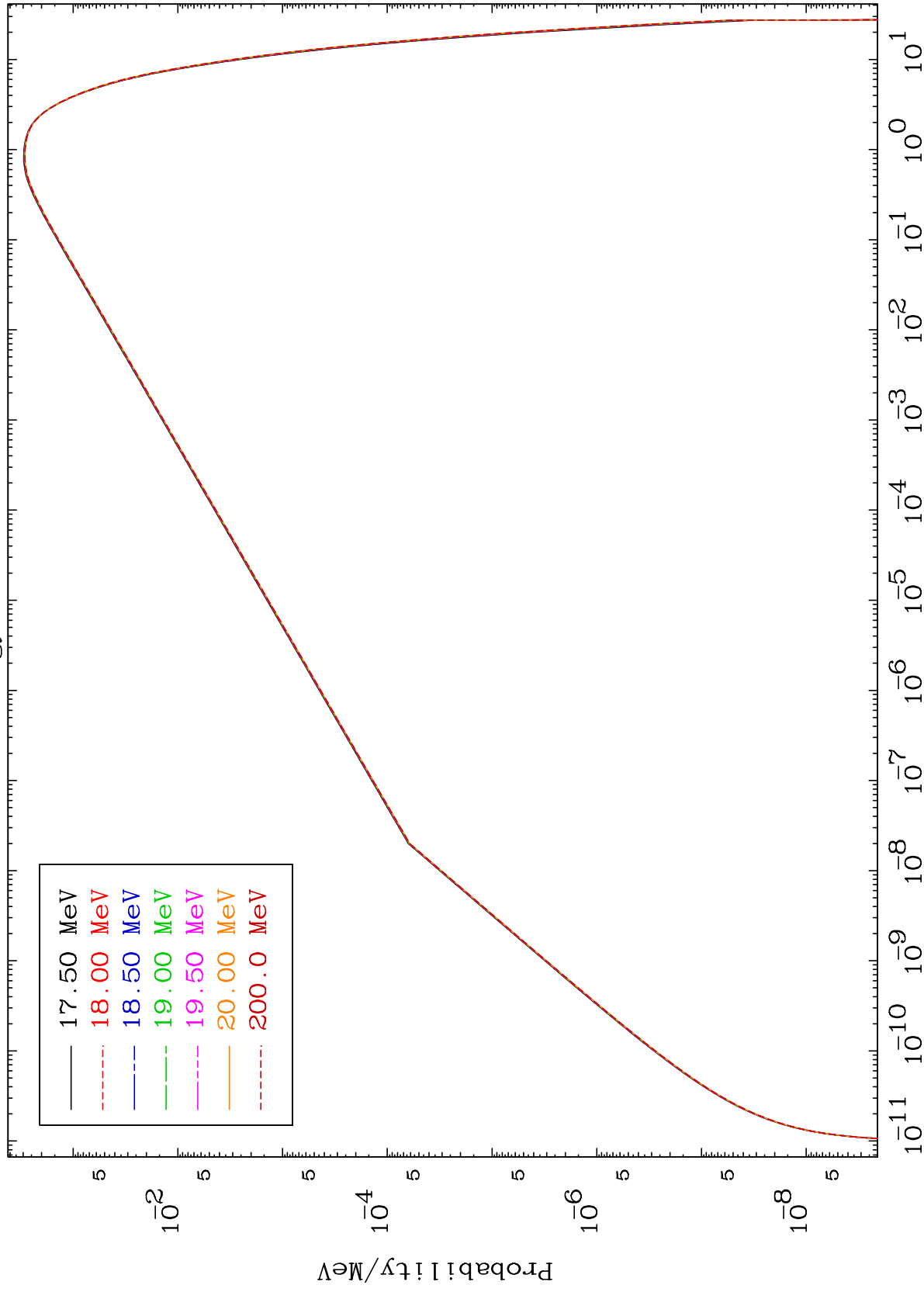
Secondary Energy (MeV)

93-Np-231

MAT 9328

Fission
Energy Distribution

93-Np-231



24

Secondary Energy (MeV)

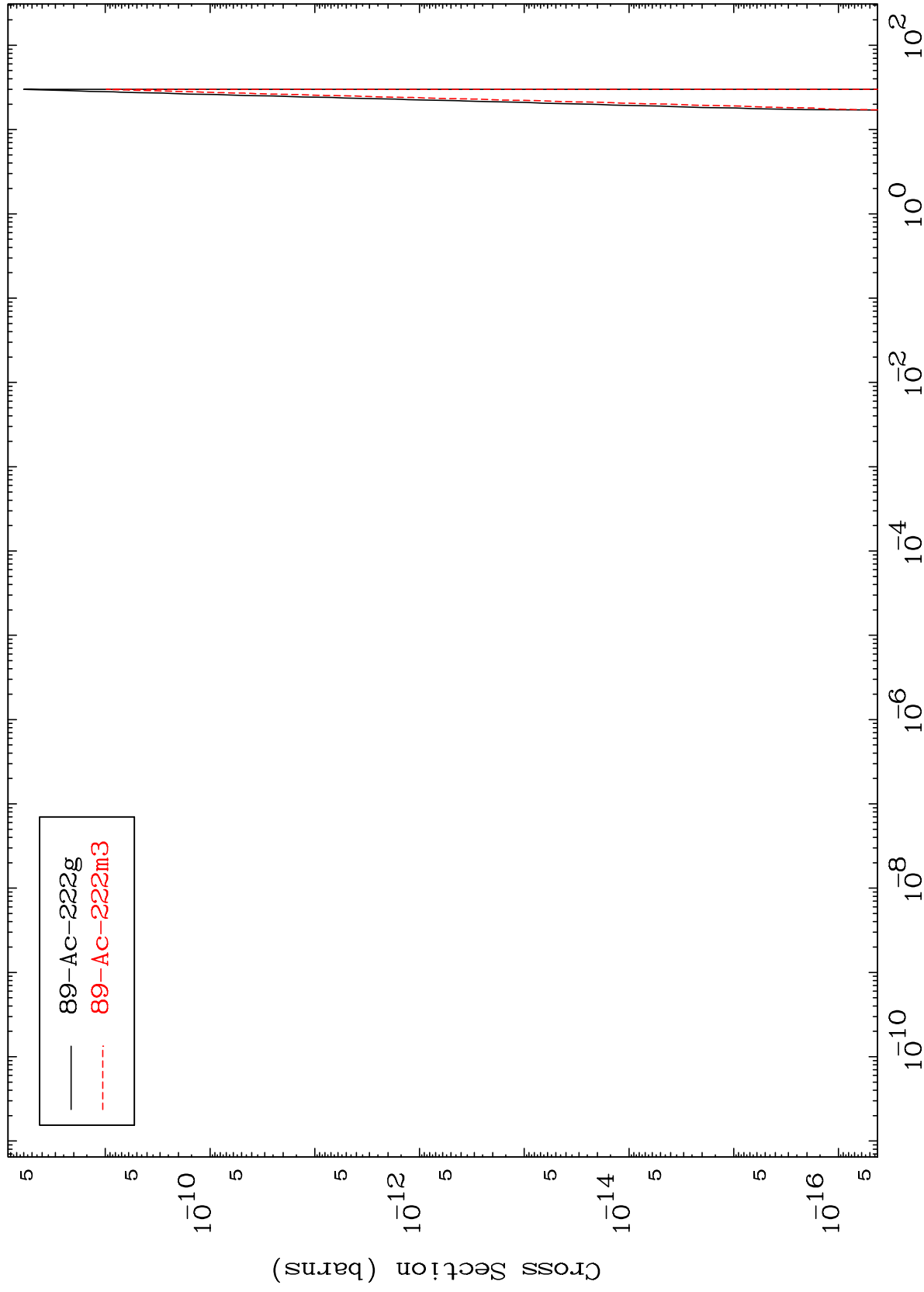
93-Np-231

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(n,2n) 2α

93-Np-231

Radionuclide Production Cross Section



25

Incident Energy (MeV)

93-Np-231