

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

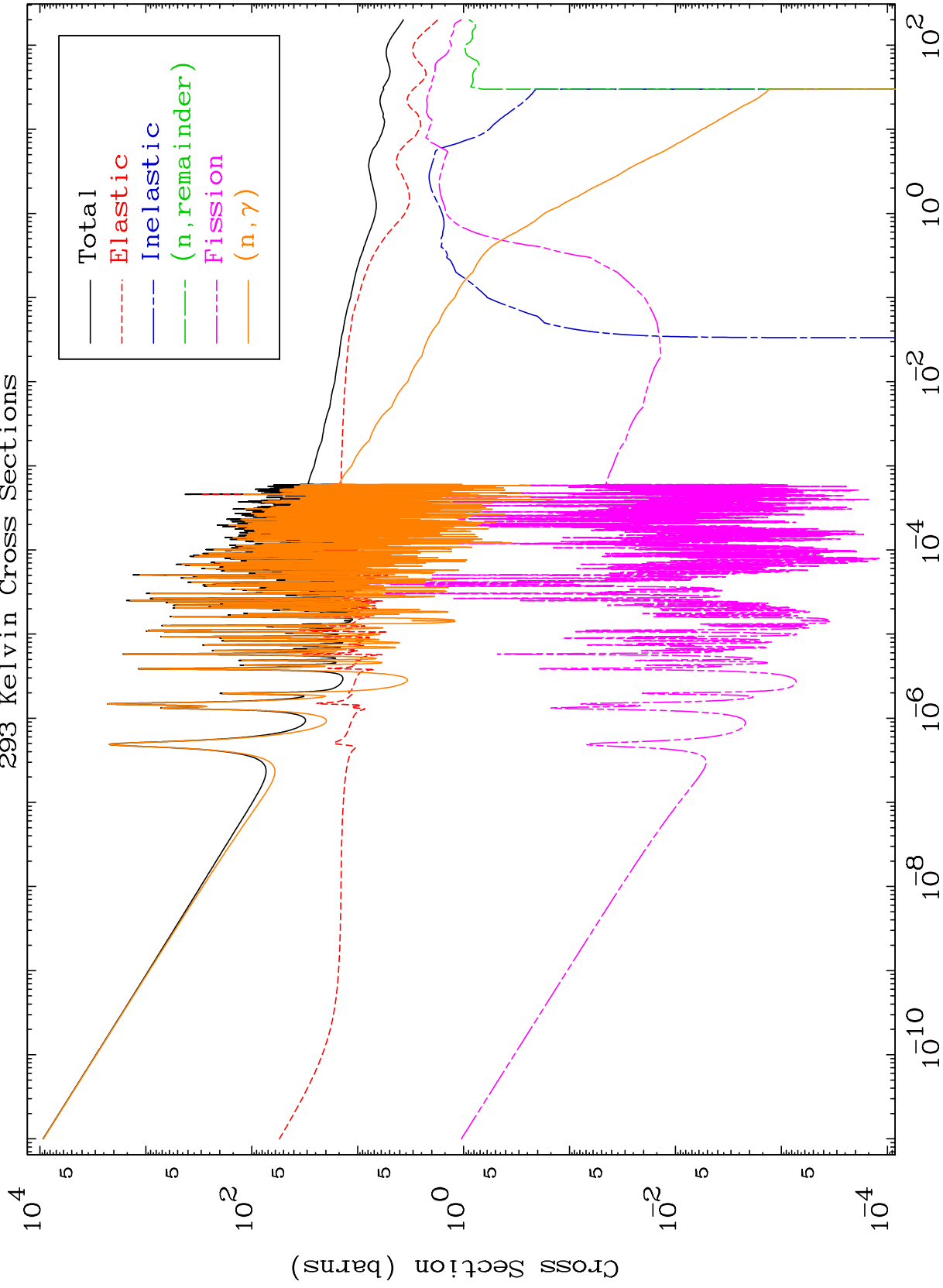
Press Mouse Button to Start

MAT 9346

Major

293 Kelvin Cross Sections

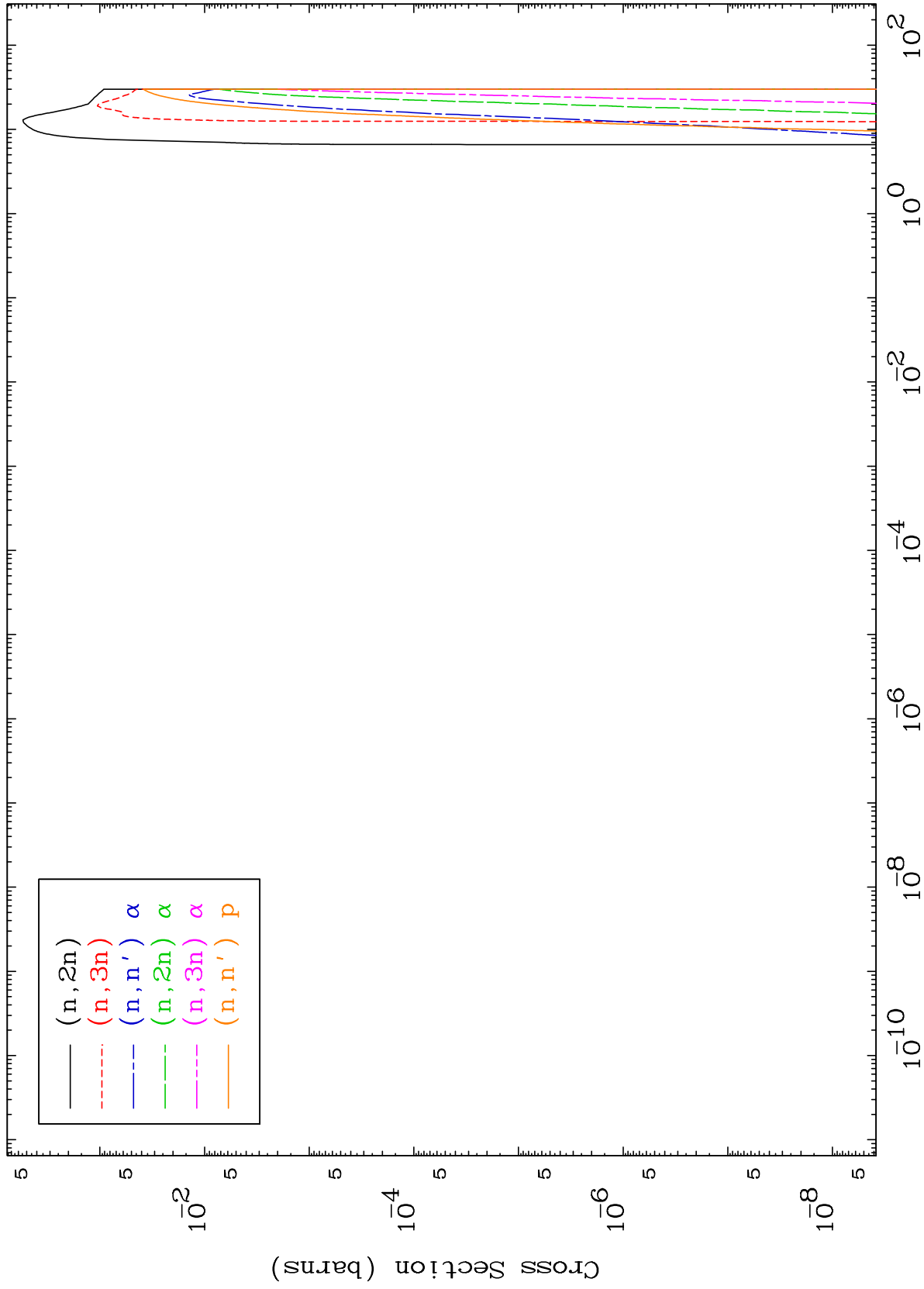
93-Np-237



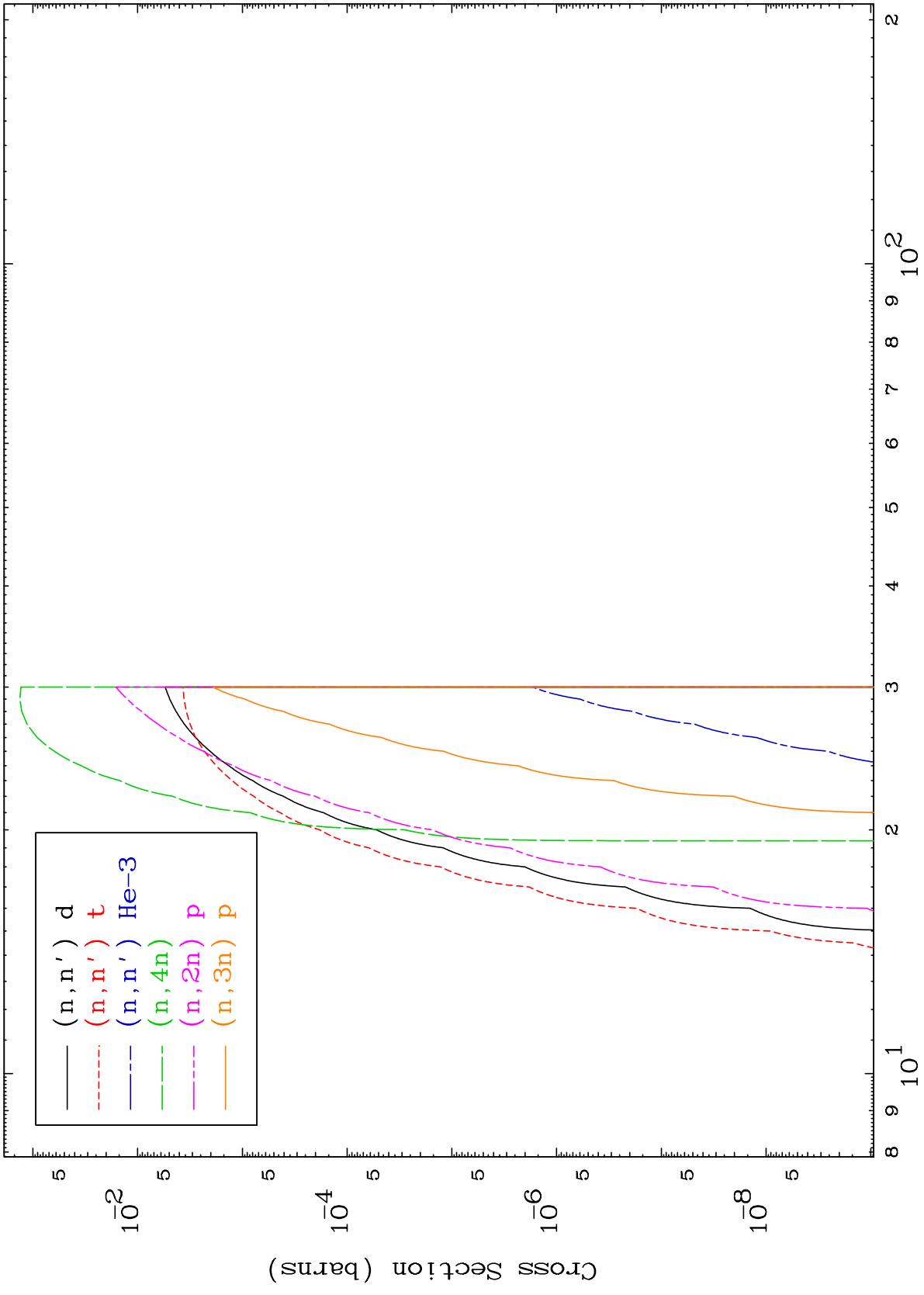
MAT 9346

Neutron Production  
293 Kelvin Cross Sections

93-Np-237



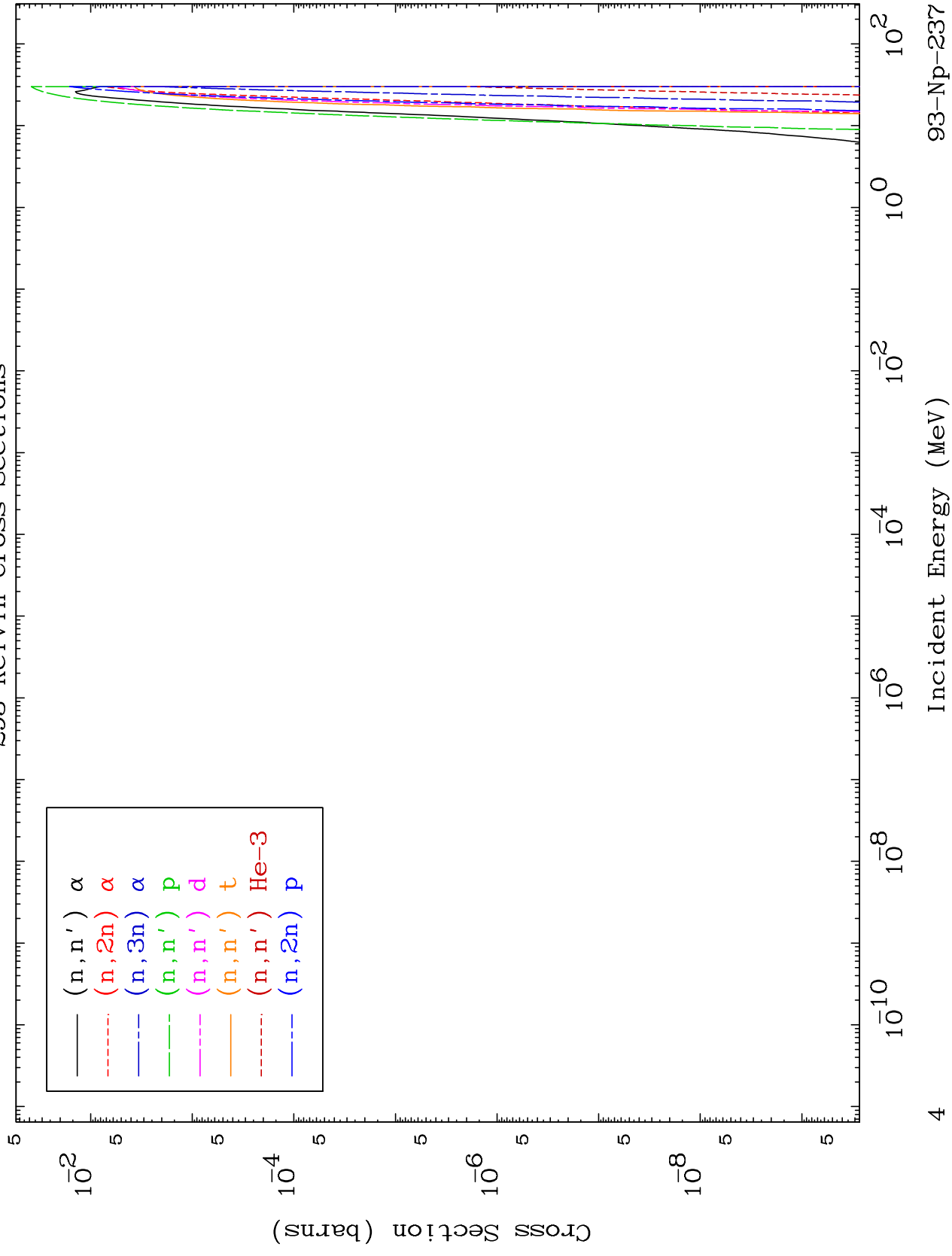
93-Np-237



MAT 9346

293 Charged Particle  
Kelvin Cross Sections

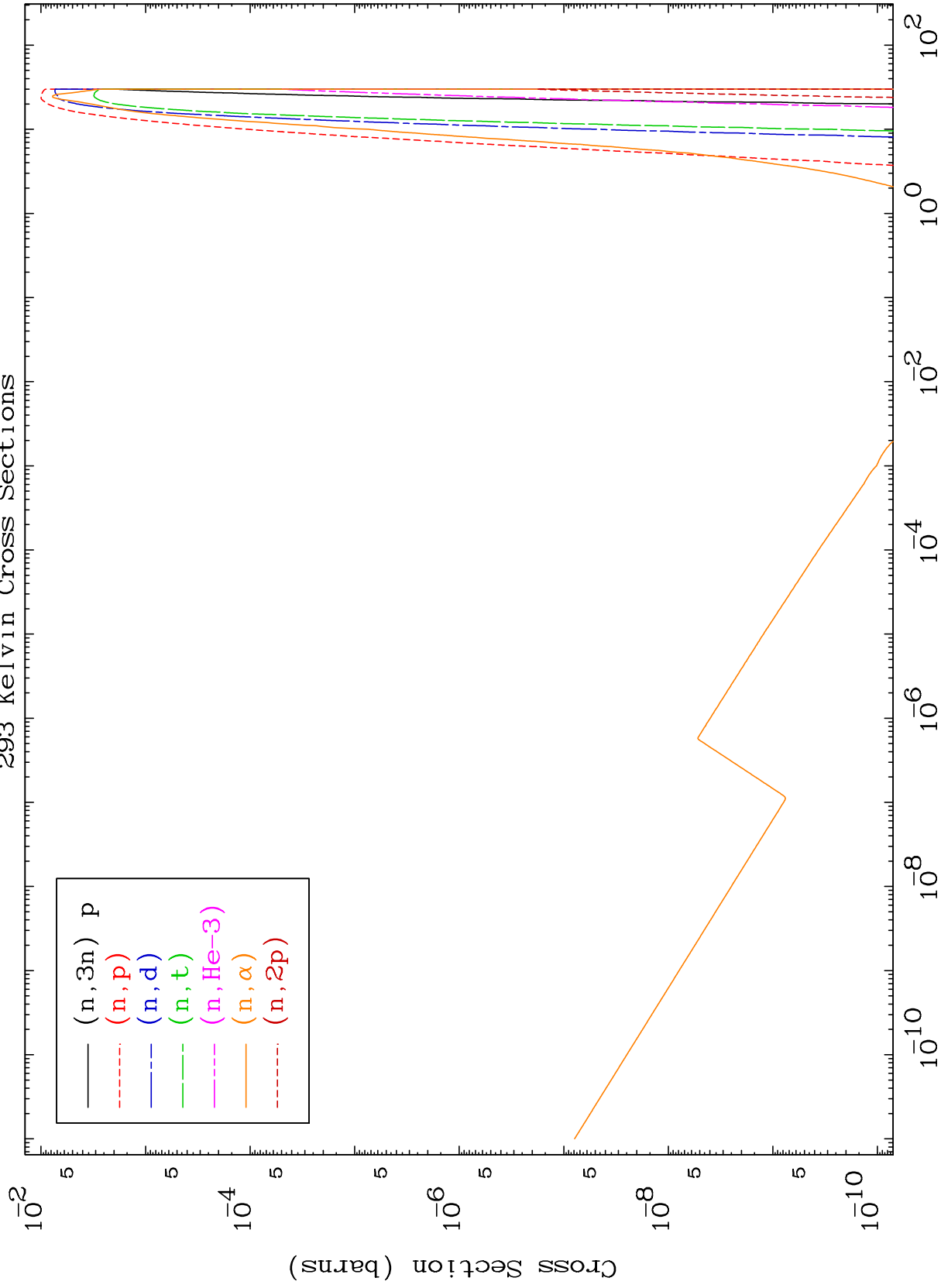
93-Np-237



MAT 9346

Charged Particle  
293 Kelvin Cross Sections

93-Np-237



5

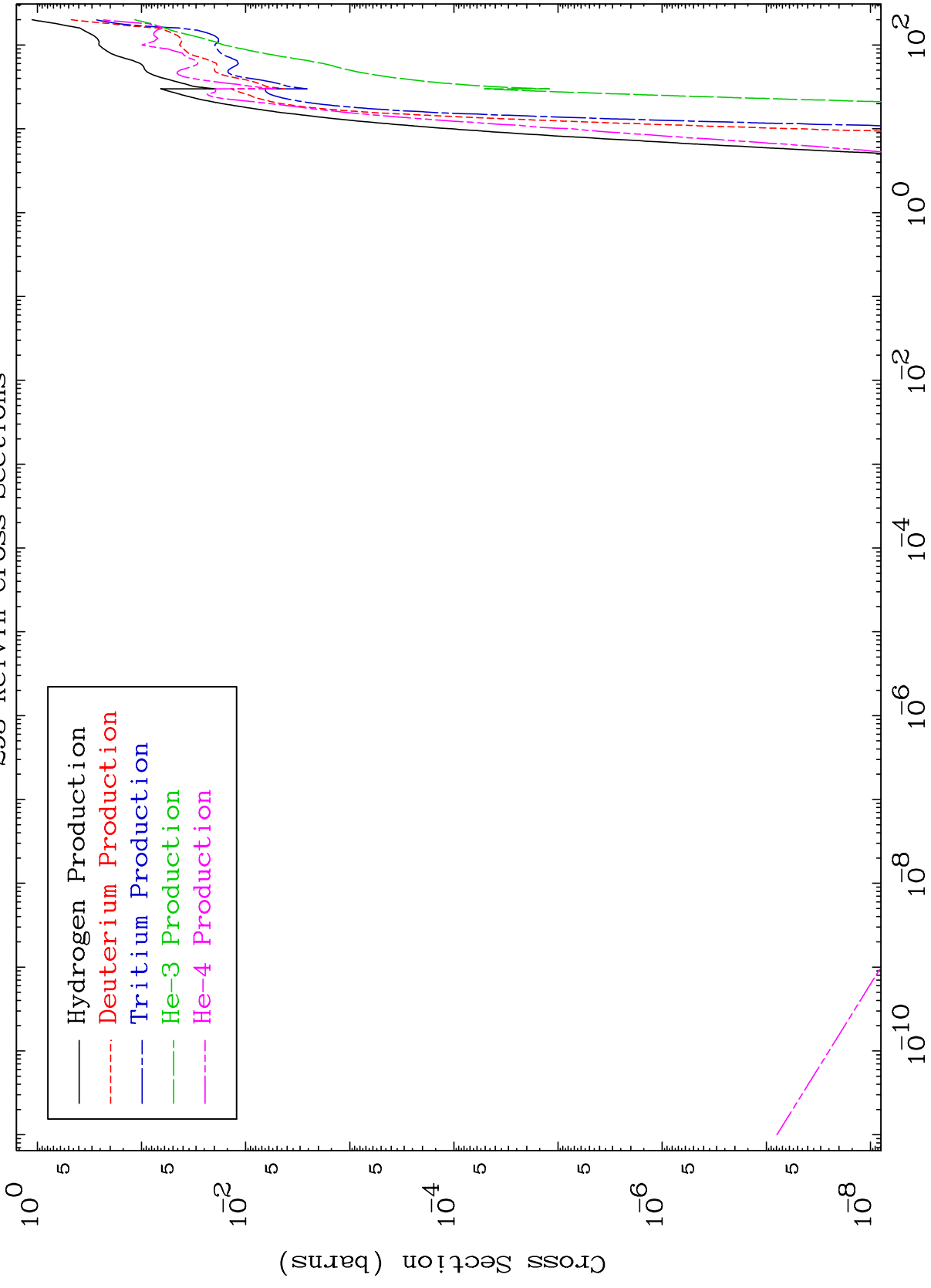
Incident Energy (MeV)

93-Np-237

MAT 9346

Particle Production  
293 Kelvin Cross Sections

93-Np-237



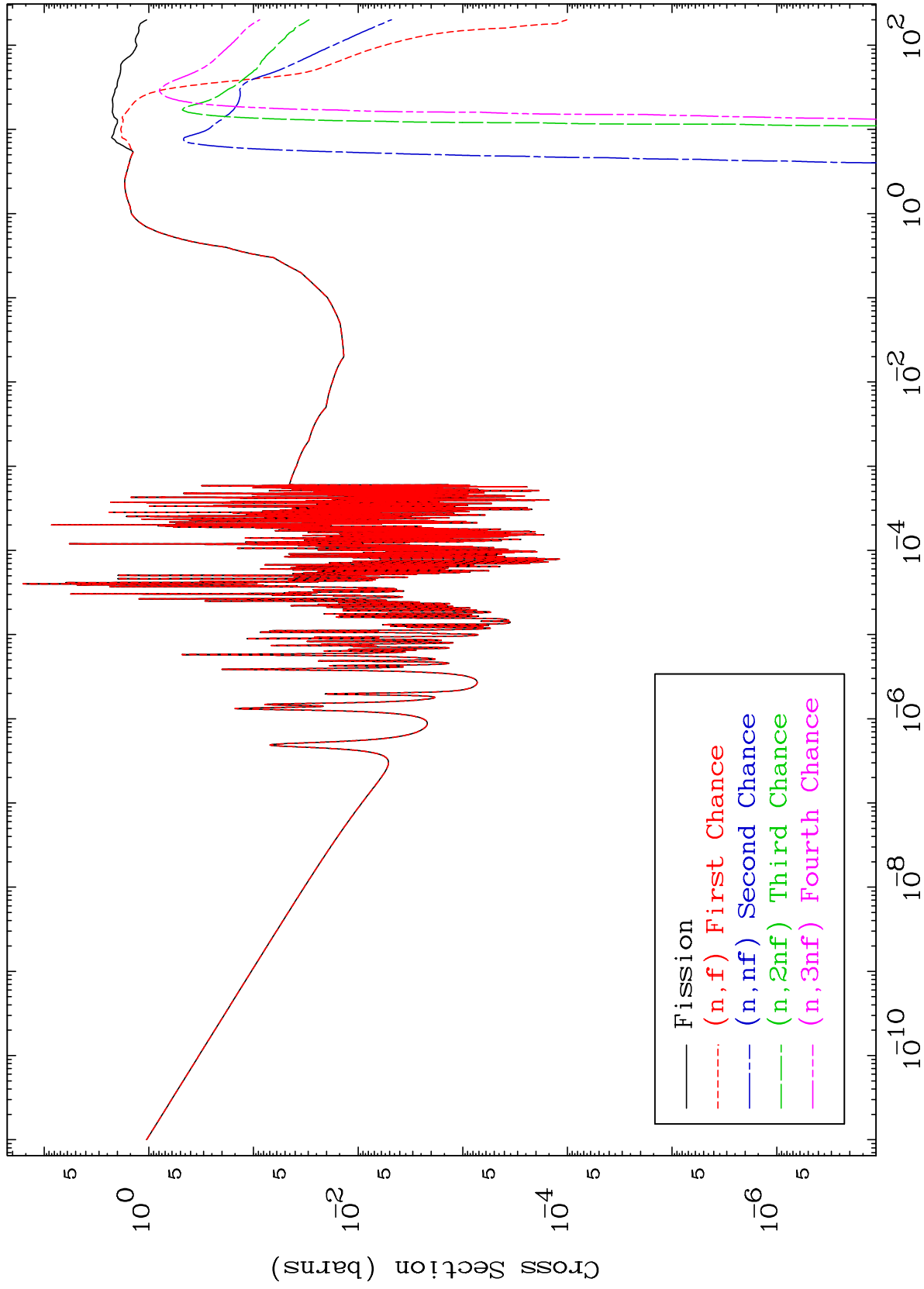
6

93-Np-237

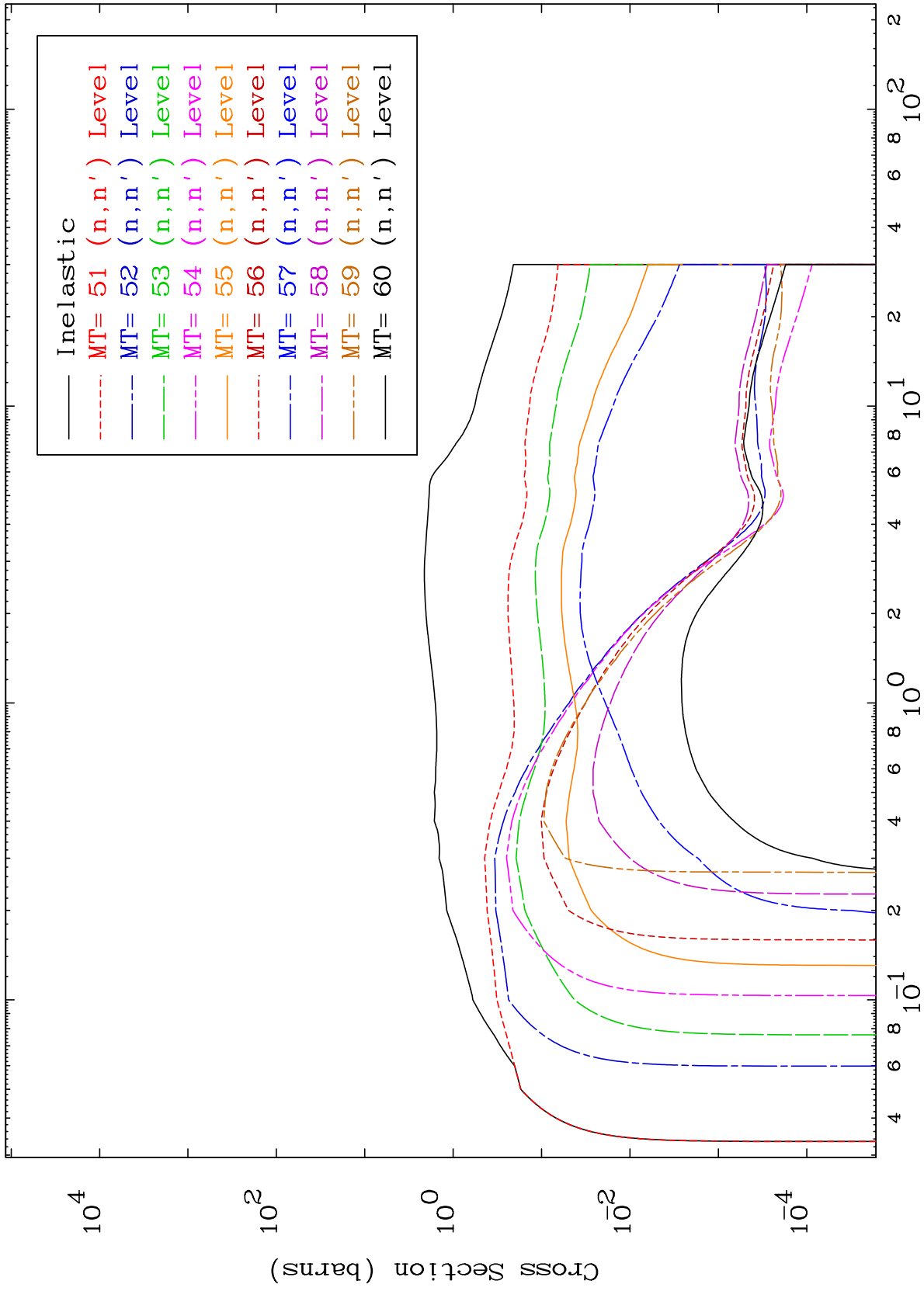
MAT 9346

Fission  
293 Kelvin Cross Sections

93-Np-237



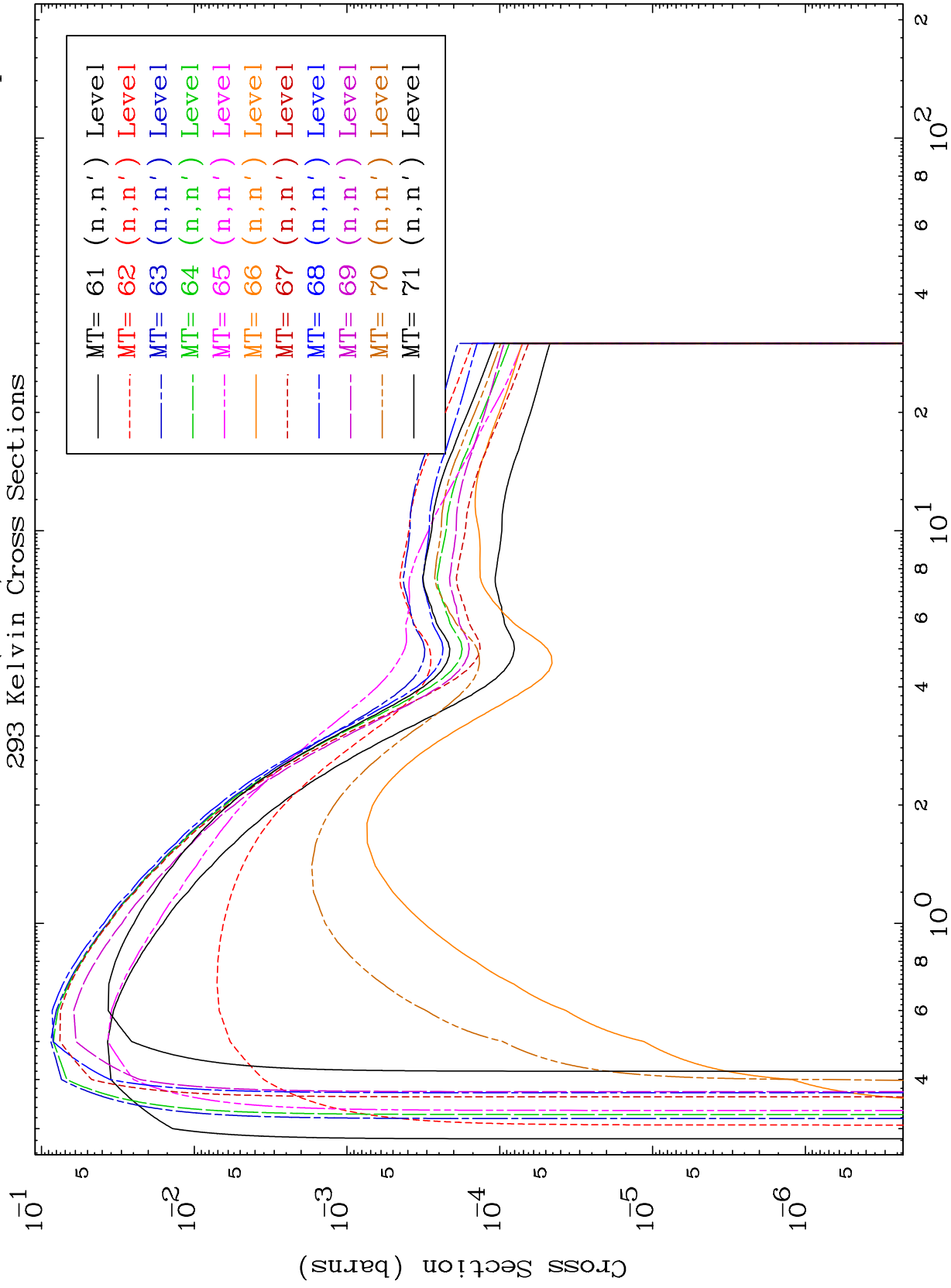




MAT 9346

(n,n') Level

93-Np-237



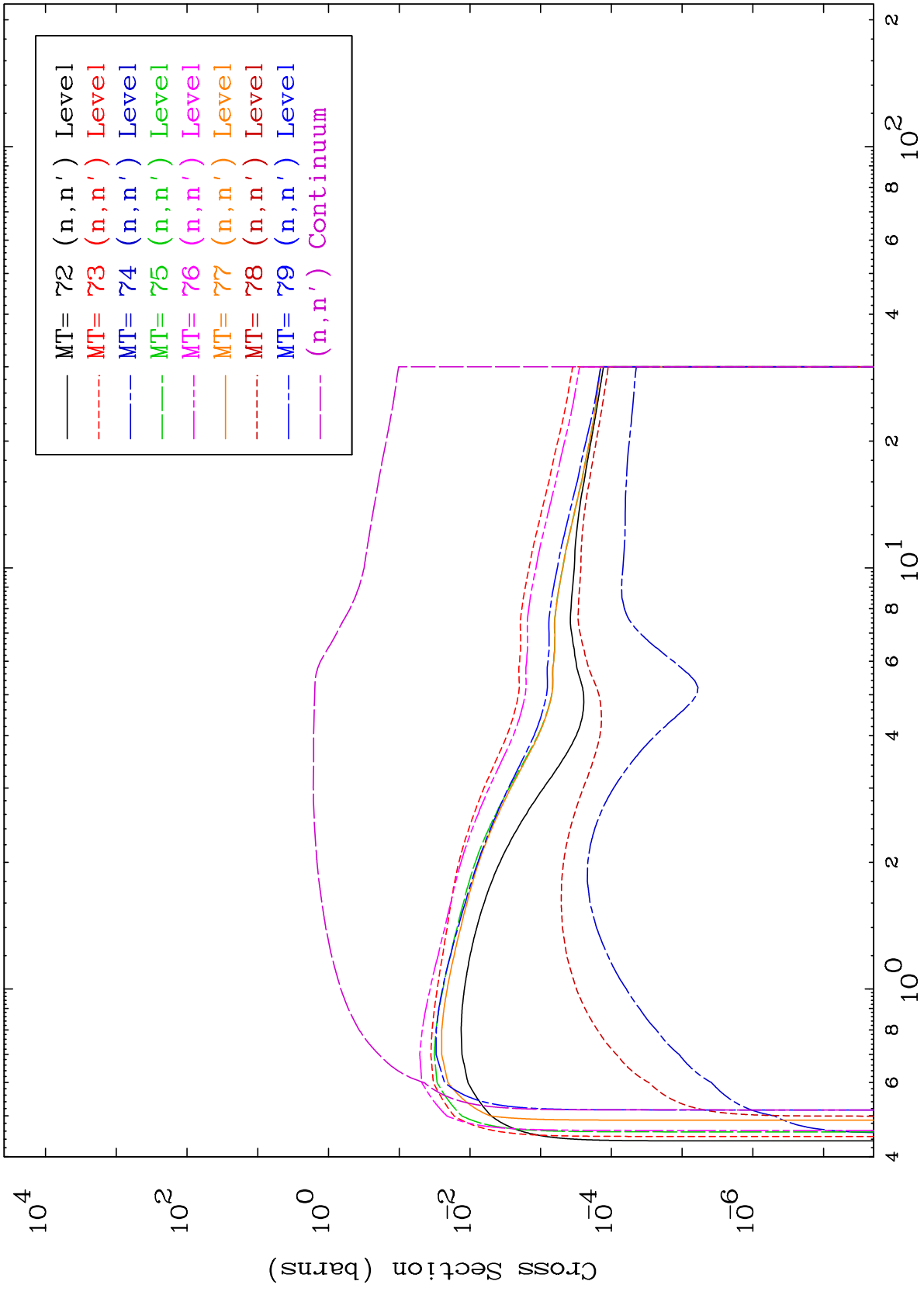
9

MAT 9346

(n,n') Level

93-Np-237

293 Kelvin Cross Sections



10

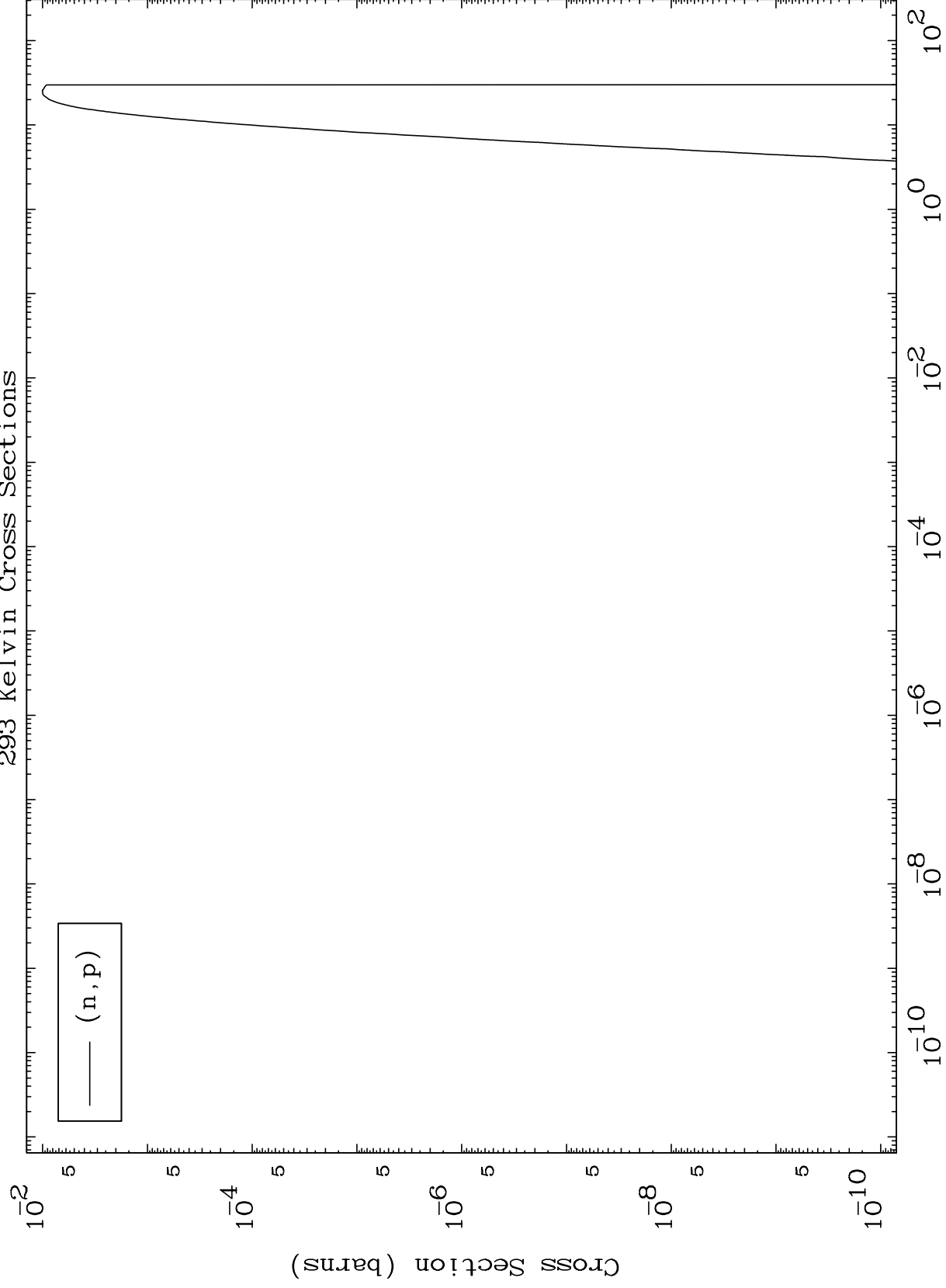
Incident Energy (MeV)

93-Np-237

MAT 9346

(n,p) Levels  
293 Kelvin Cross Sections

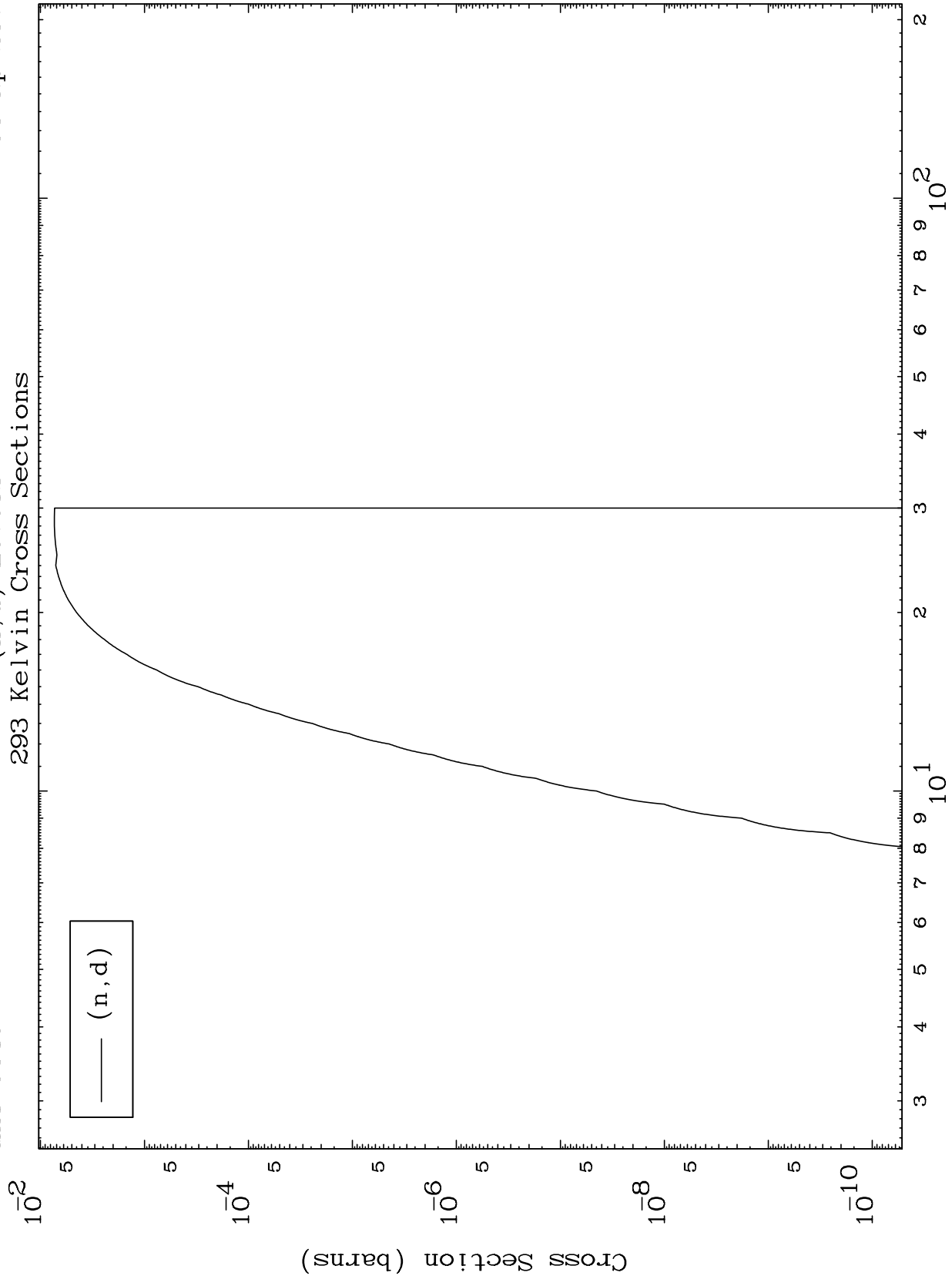
93-Np-237



MAT 9346

(n,d) Levels  
293 Kelvin Cross Sections

93-Np-237



12

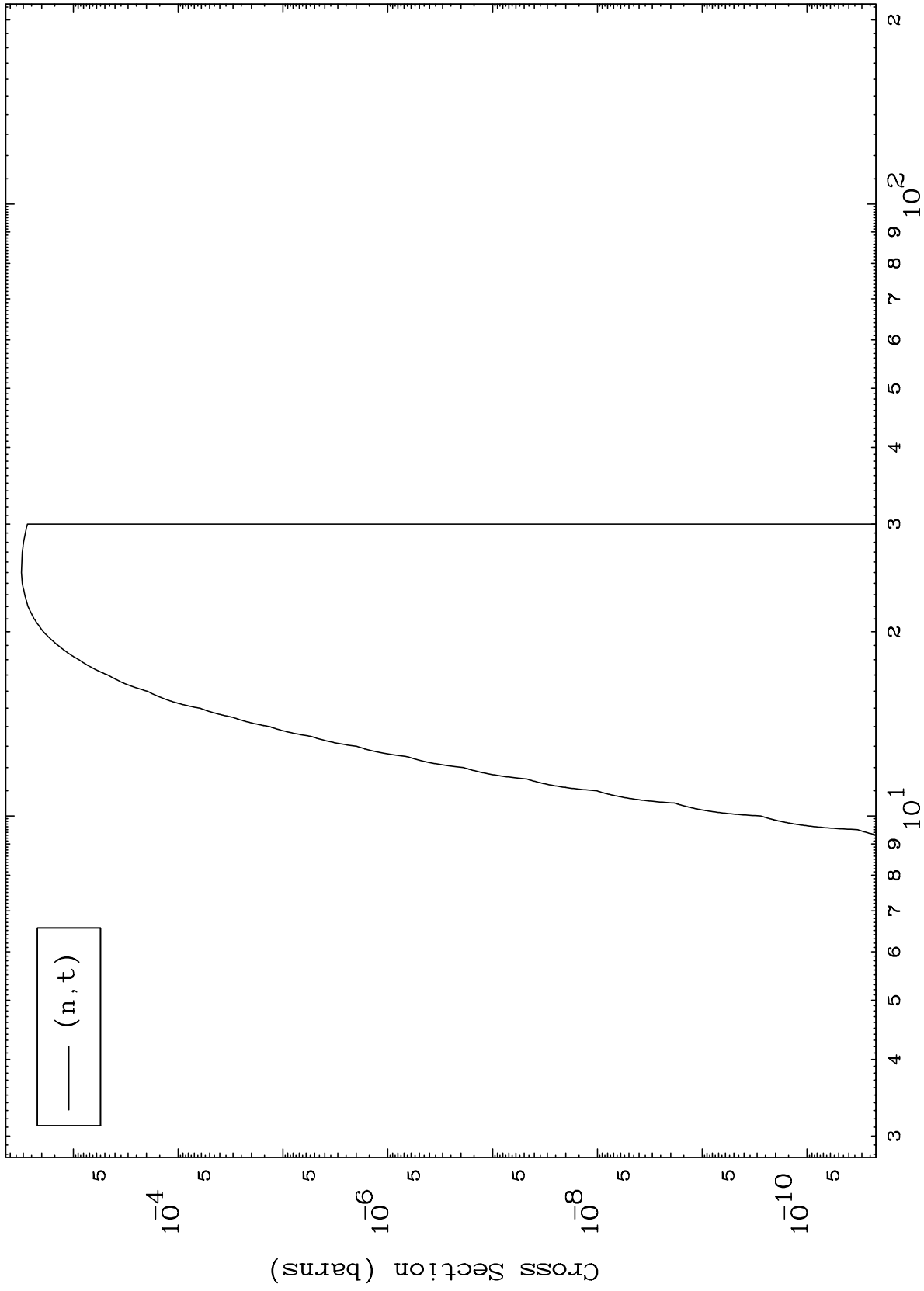
Incident Energy (MeV)

93-Np-237

MAT 9346

(n,t) Levels  
293 Kelvin Cross Sections

93-Np-237



13

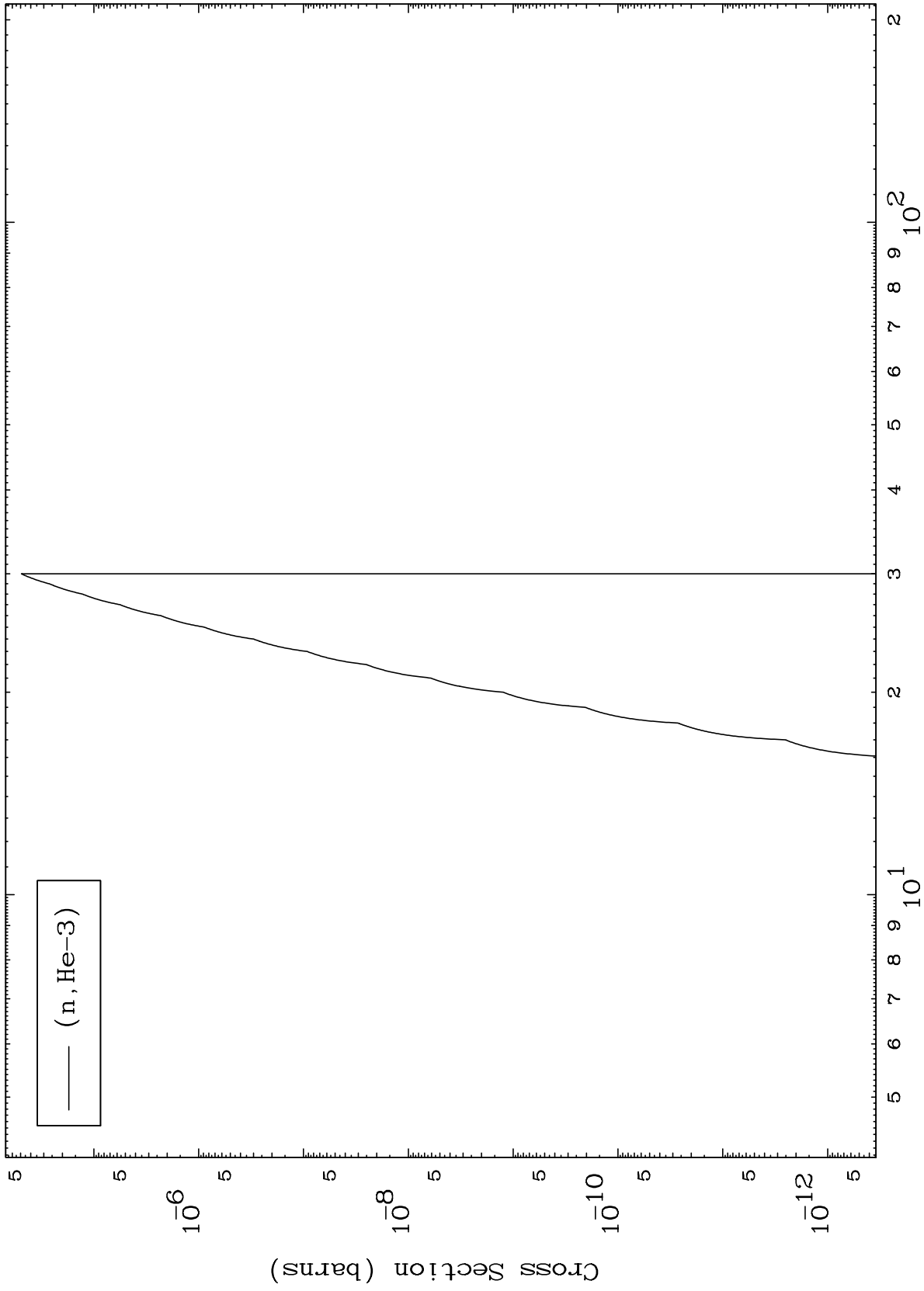
Incident Energy (MeV)

93-Np-237

MAT 9346

(n,He3) Levels  
293 Kelvin Cross Sections

93-Np-237



14

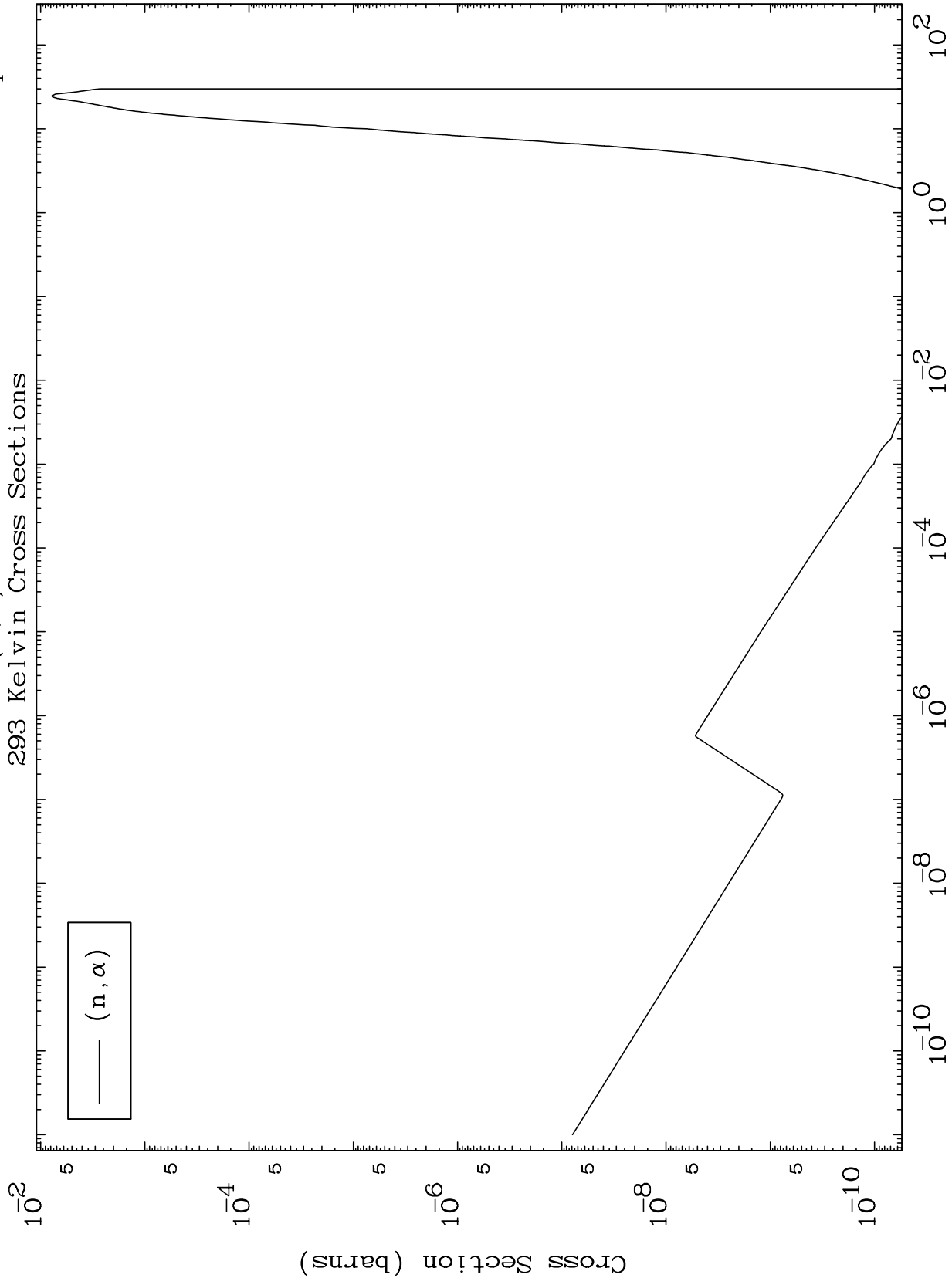
Incident Energy (MeV)

93-Np-237

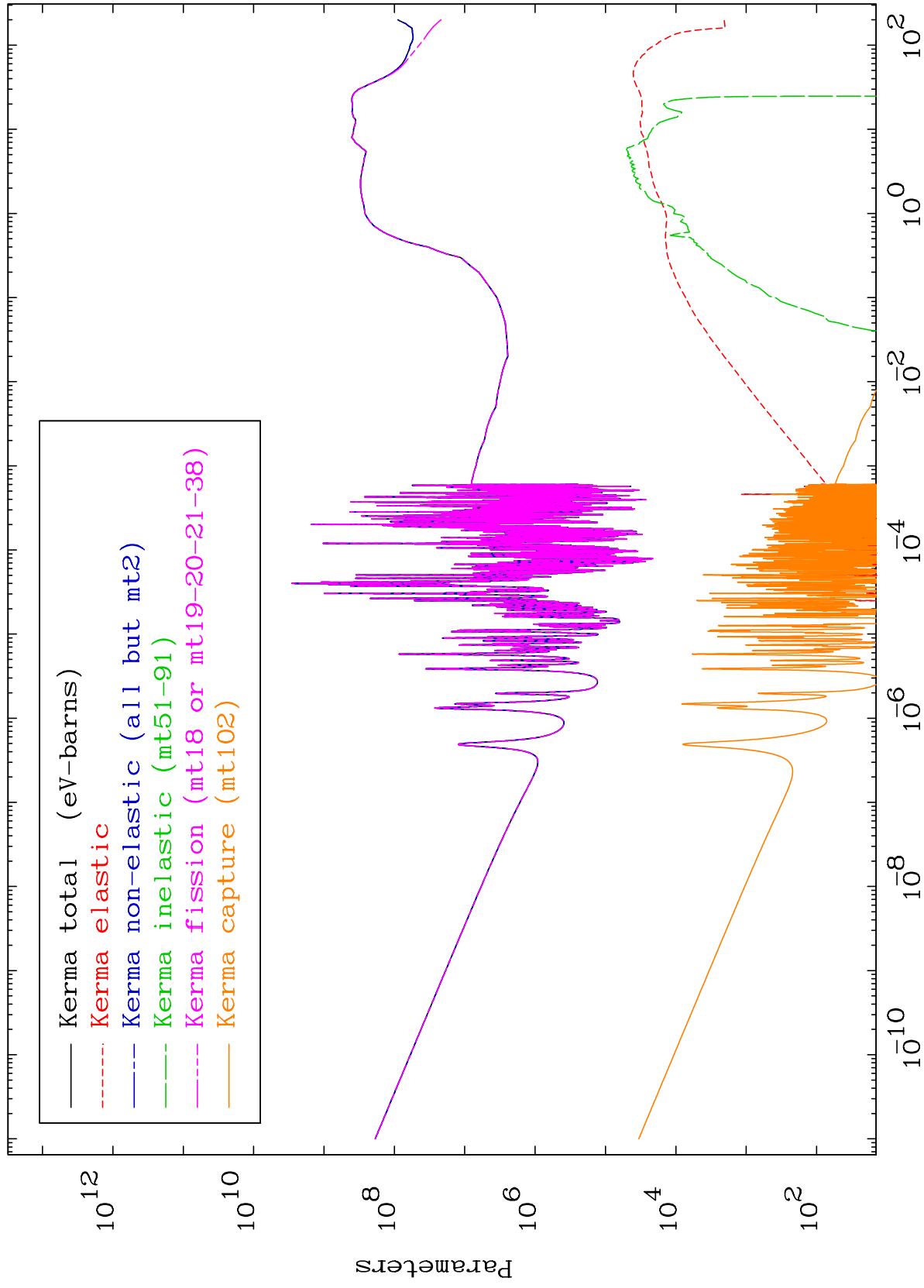
MAT 9346

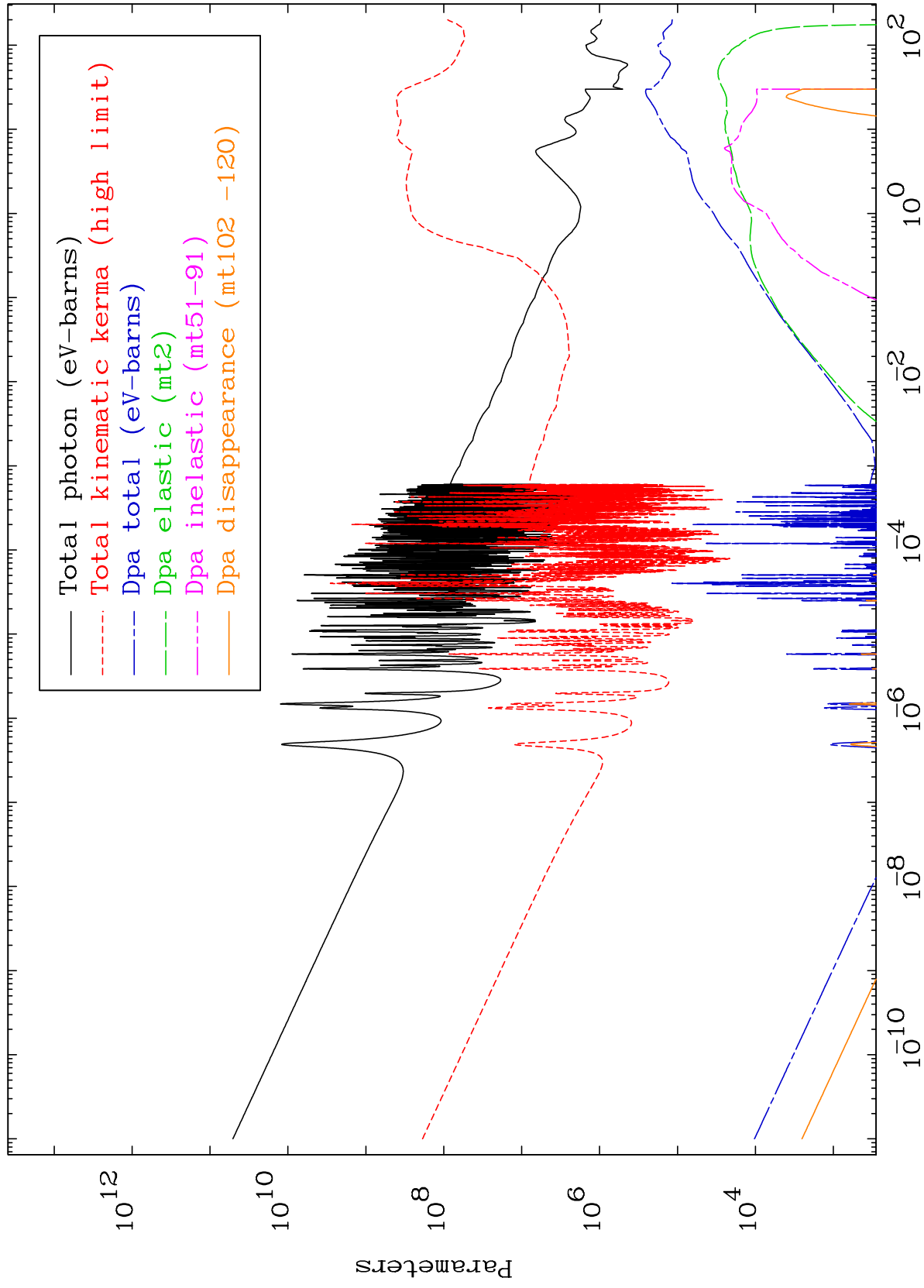
(n,α) Levels  
293 Kelvin Cross Sections

93-Np-237





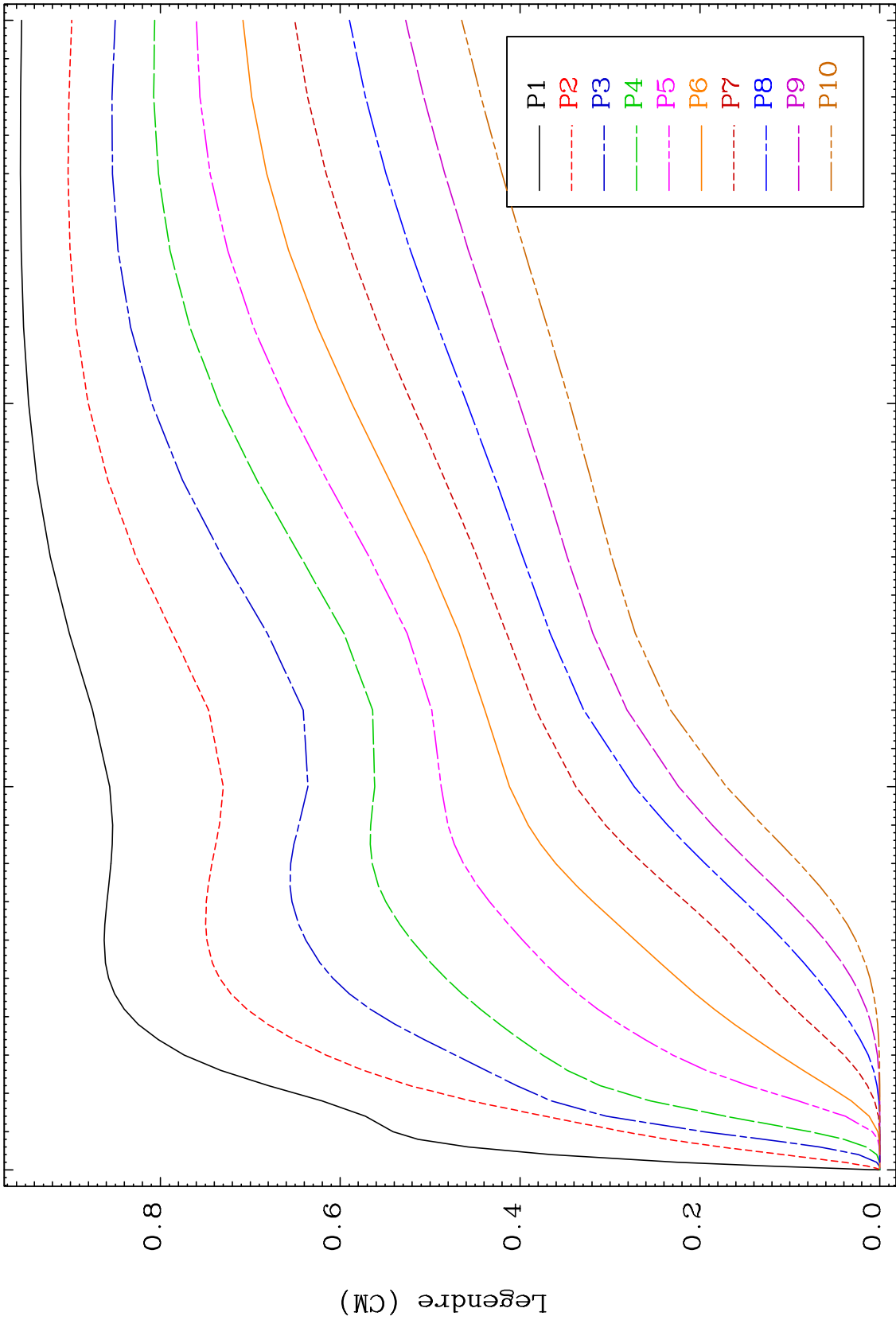




MAT 9346

93-Np-237

Elastic Legendre Coefficients



18

Incident Energy (MeV)

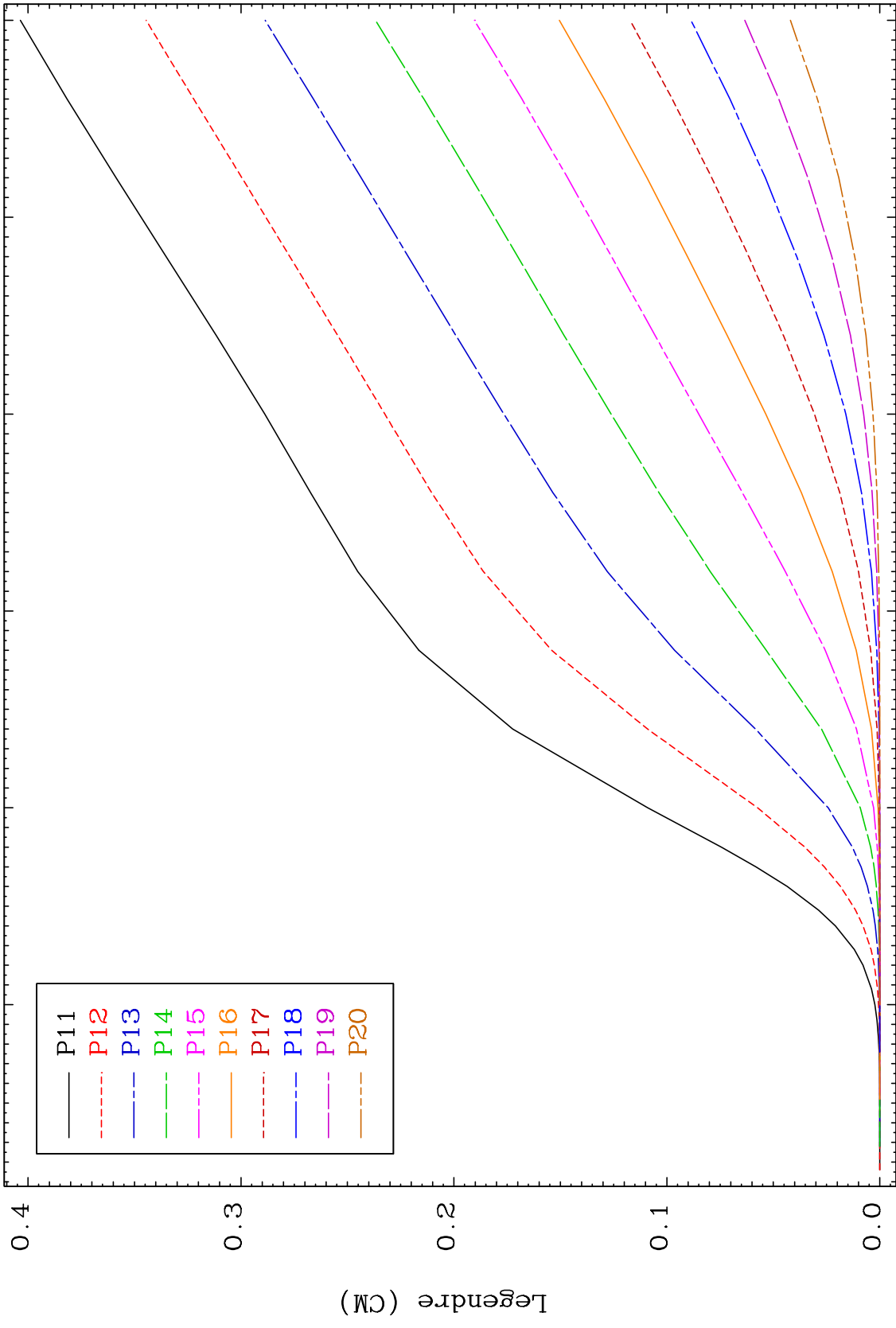
93-Np-237

30

MAT 9346

Elastic Legendre Coefficients

<sup>93</sup>Np-237



19

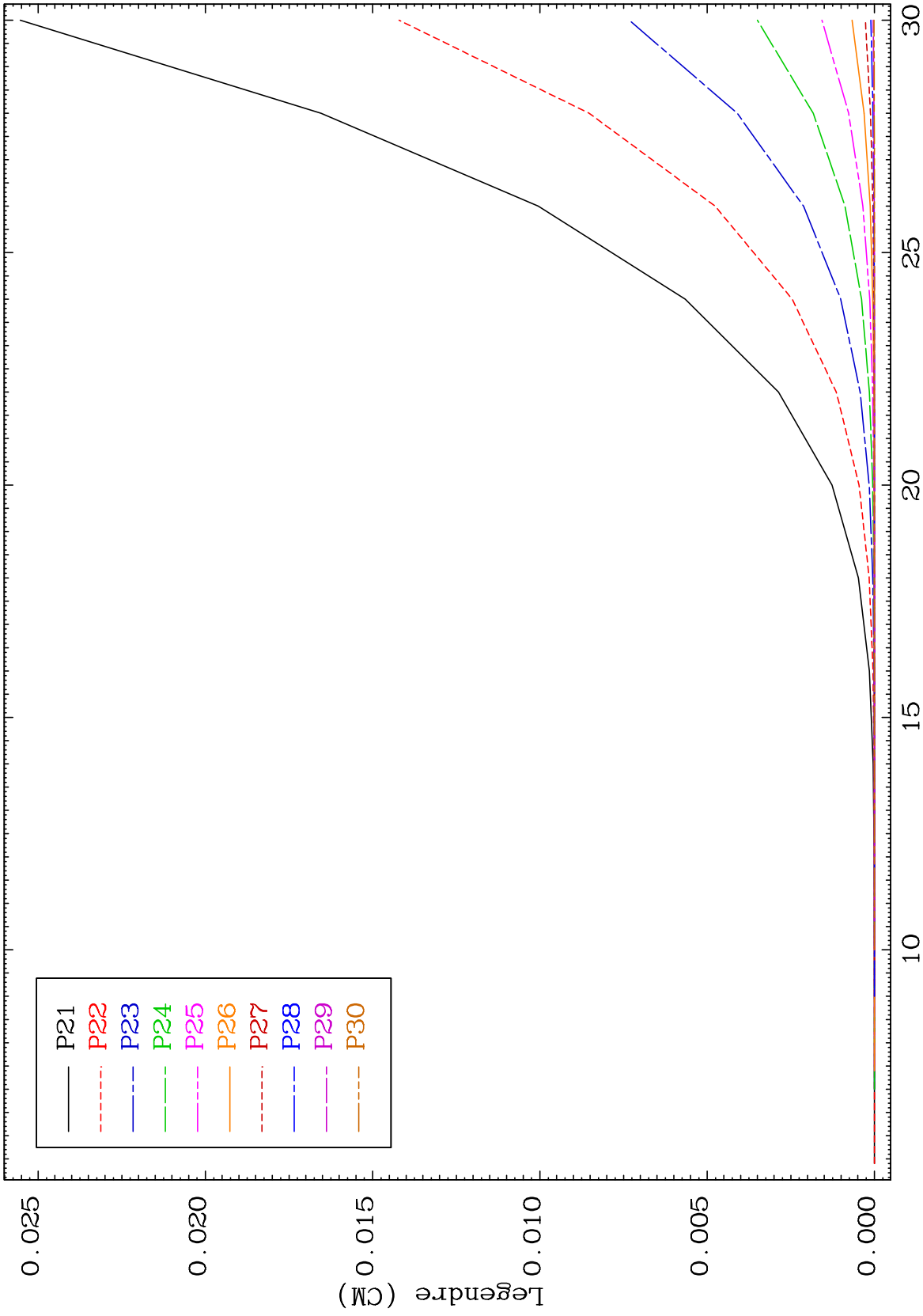
Incident Energy (MeV)

<sup>93</sup>Np-237

MAT 9346

### Elastic Legendre Coefficients

93-Np-237



20

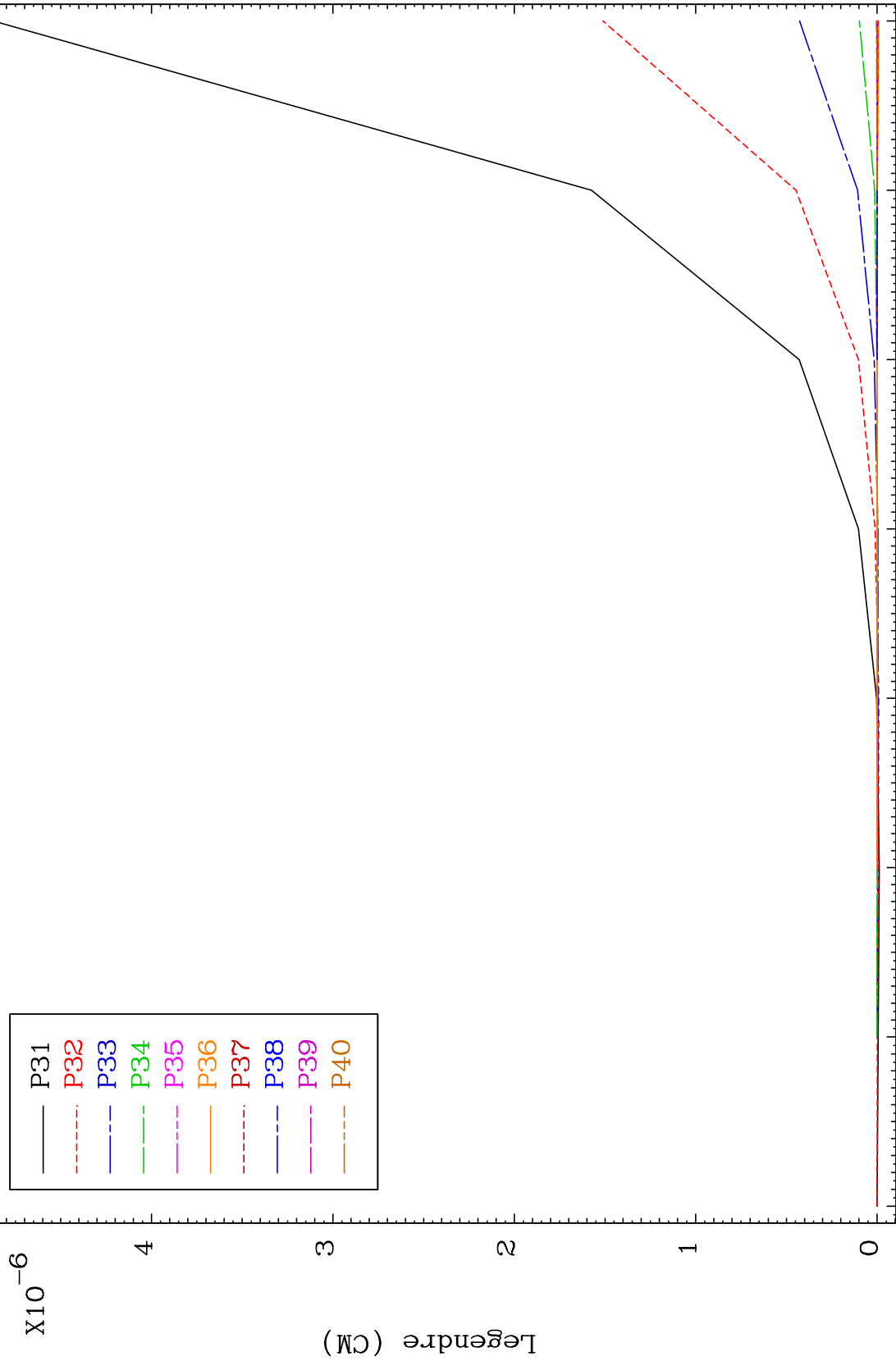
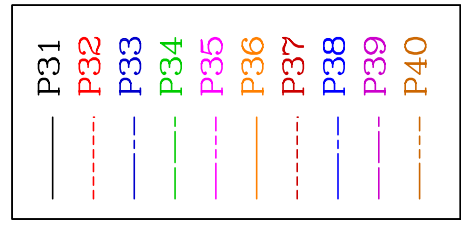
Incident Energy (MeV)

93-Np-237

MAT 9346

Elastic Legendre Coefficients

93-Np-237



21

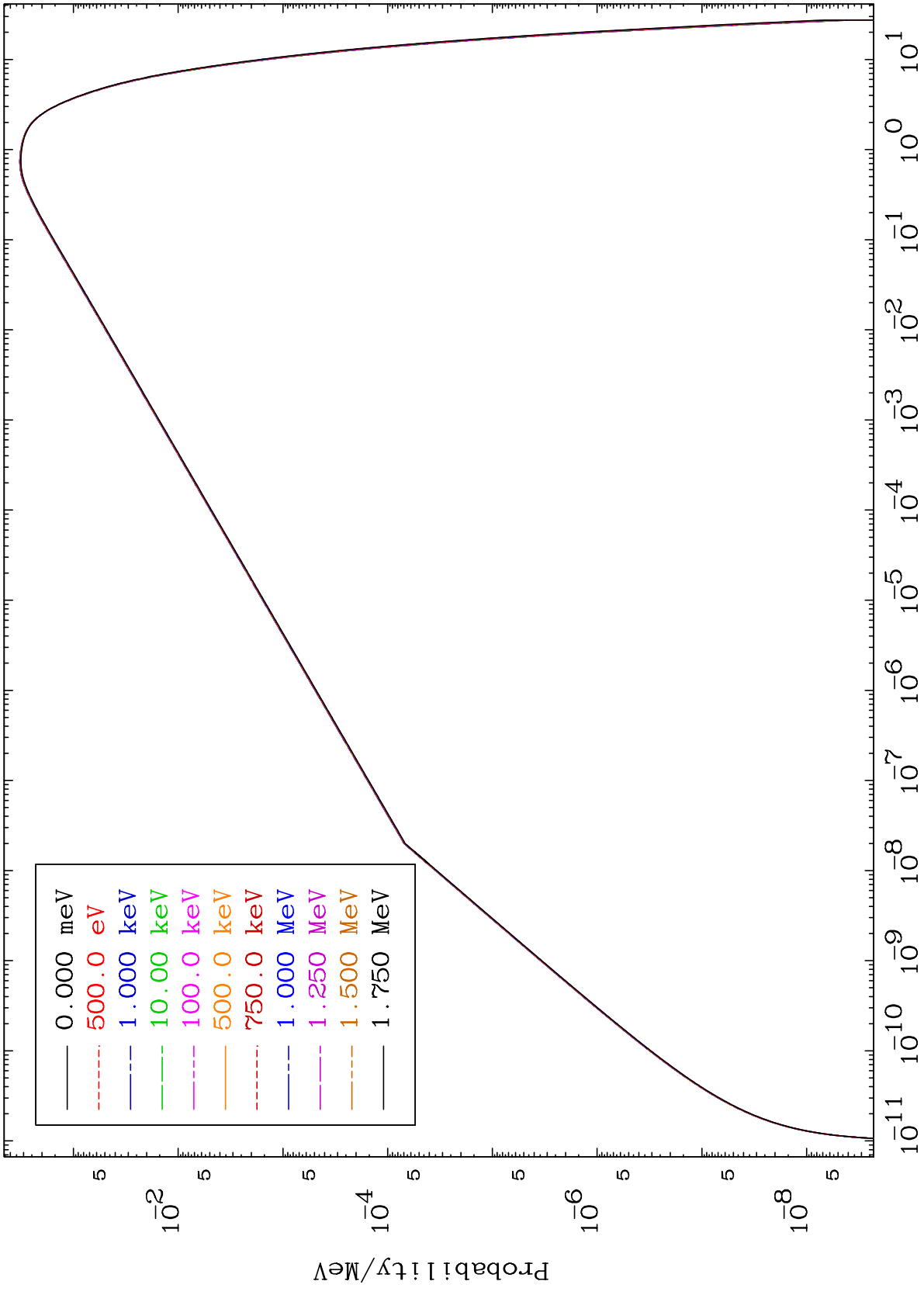
Incident Energy (MeV)

93-Np-237

MAT 9346

### Fission Energy Distribution

93-Np-237



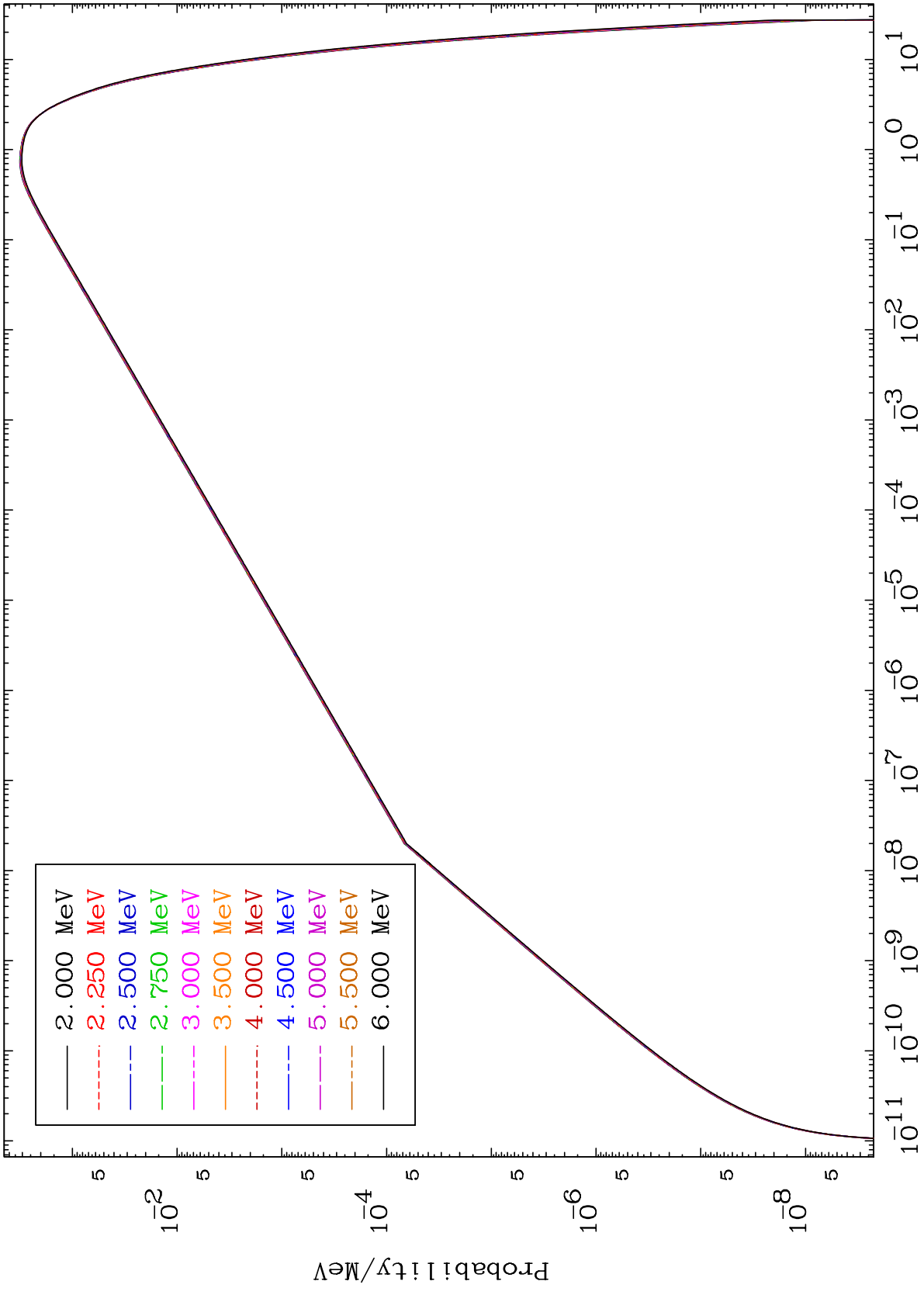
93-Np-237

Secondary Energy (MeV)

MAT 9346

# Fission Energy Distribution

93-Np-237



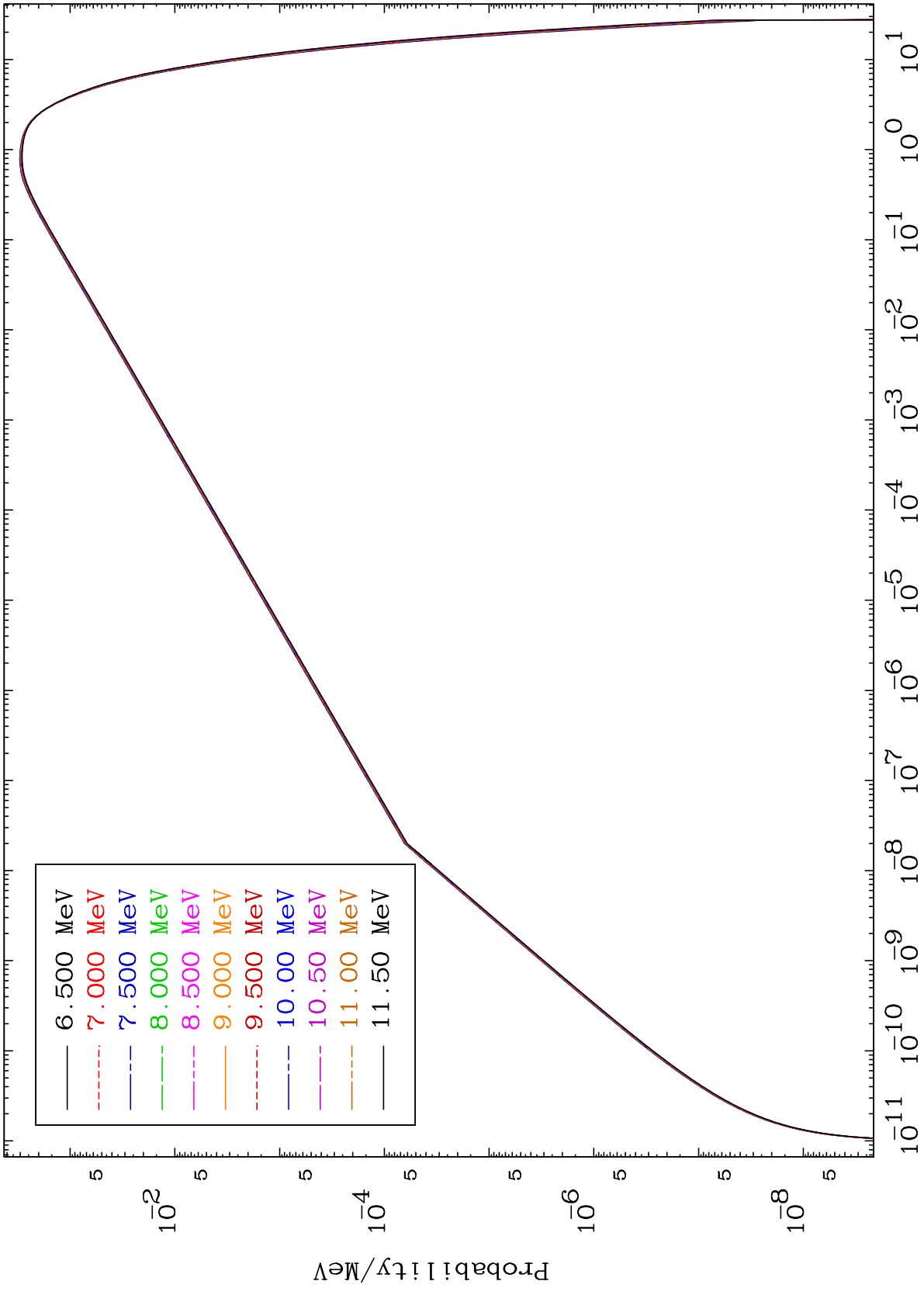
93-Np-237



MAT 9346

### Fission Energy Distribution

93-Np-237

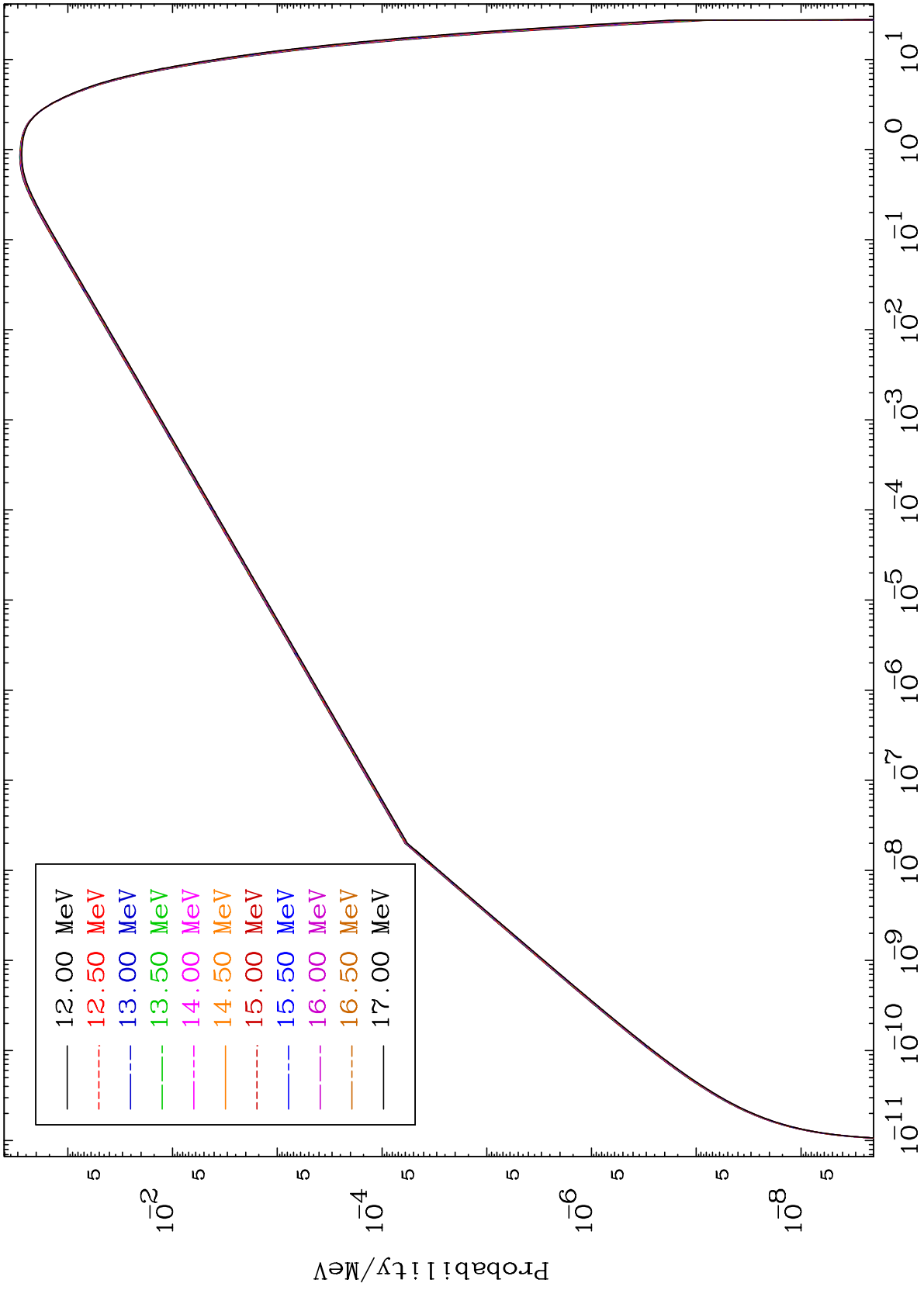


93-Np-237

MAT 9346

# Fission Energy Distribution

93-Np-237



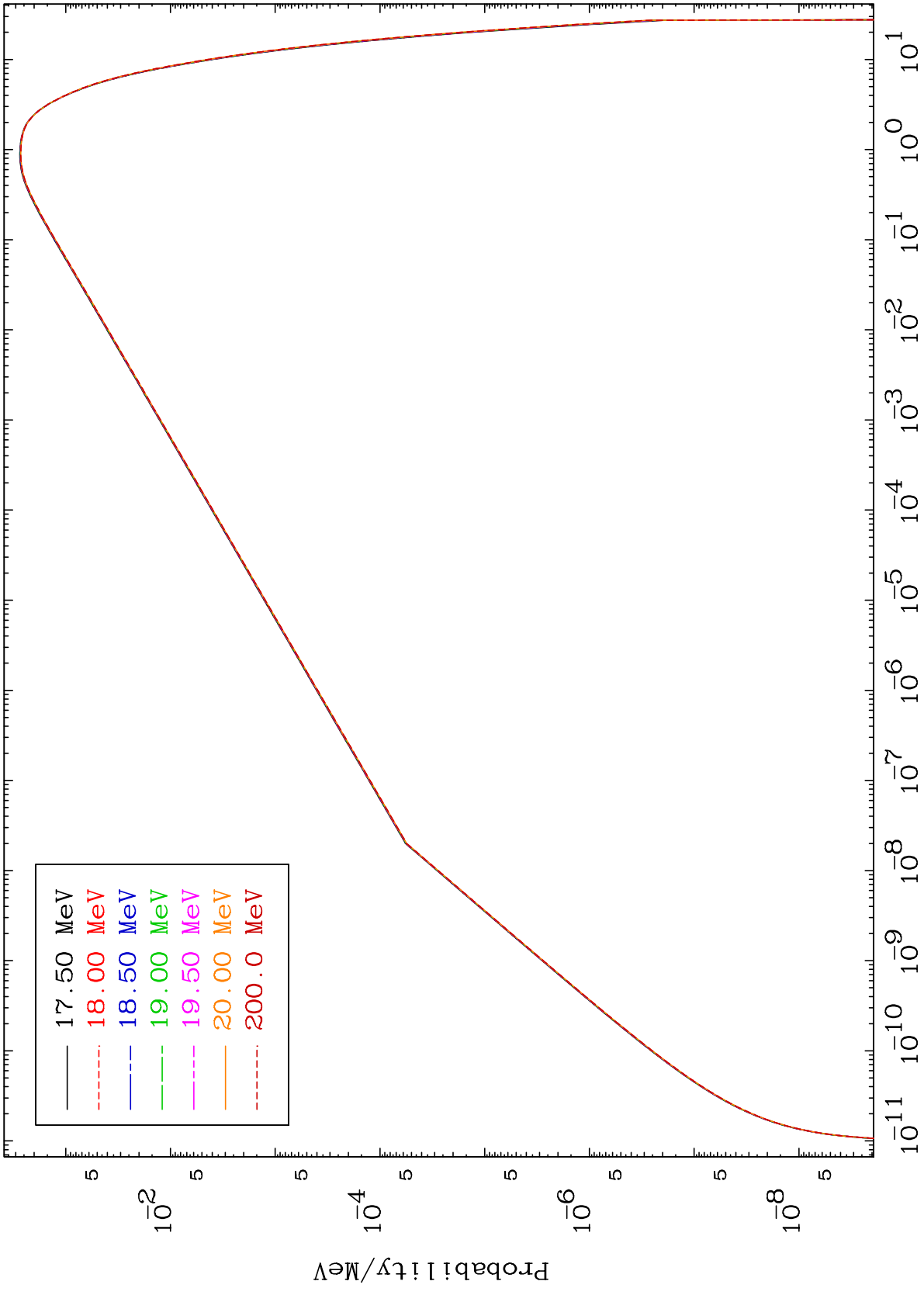
25

93-Np-237

MAT 9346

### Fission Energy Distribution

93-Np-237



- 17.50 MeV
- 18.00 MeV
- 18.50 MeV
- 19.00 MeV
- 19.50 MeV
- 20.00 MeV
- 200.0 MeV

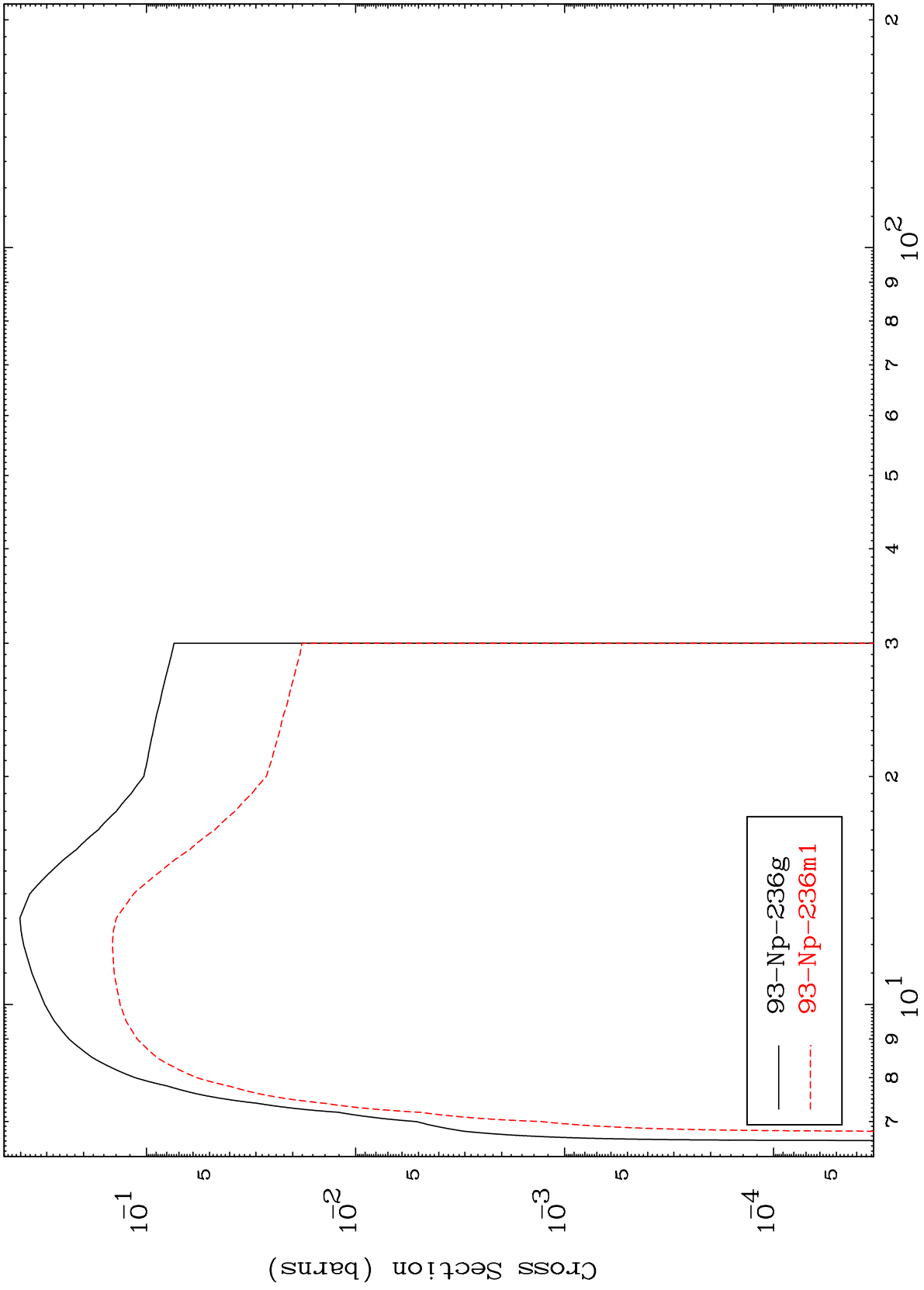
93-Np-237

Secondary Energy (MeV)

MAT 9346

93-Np-237

(n,2n)  
Radionuclide Production Cross Section



27

Incident Energy (MeV)

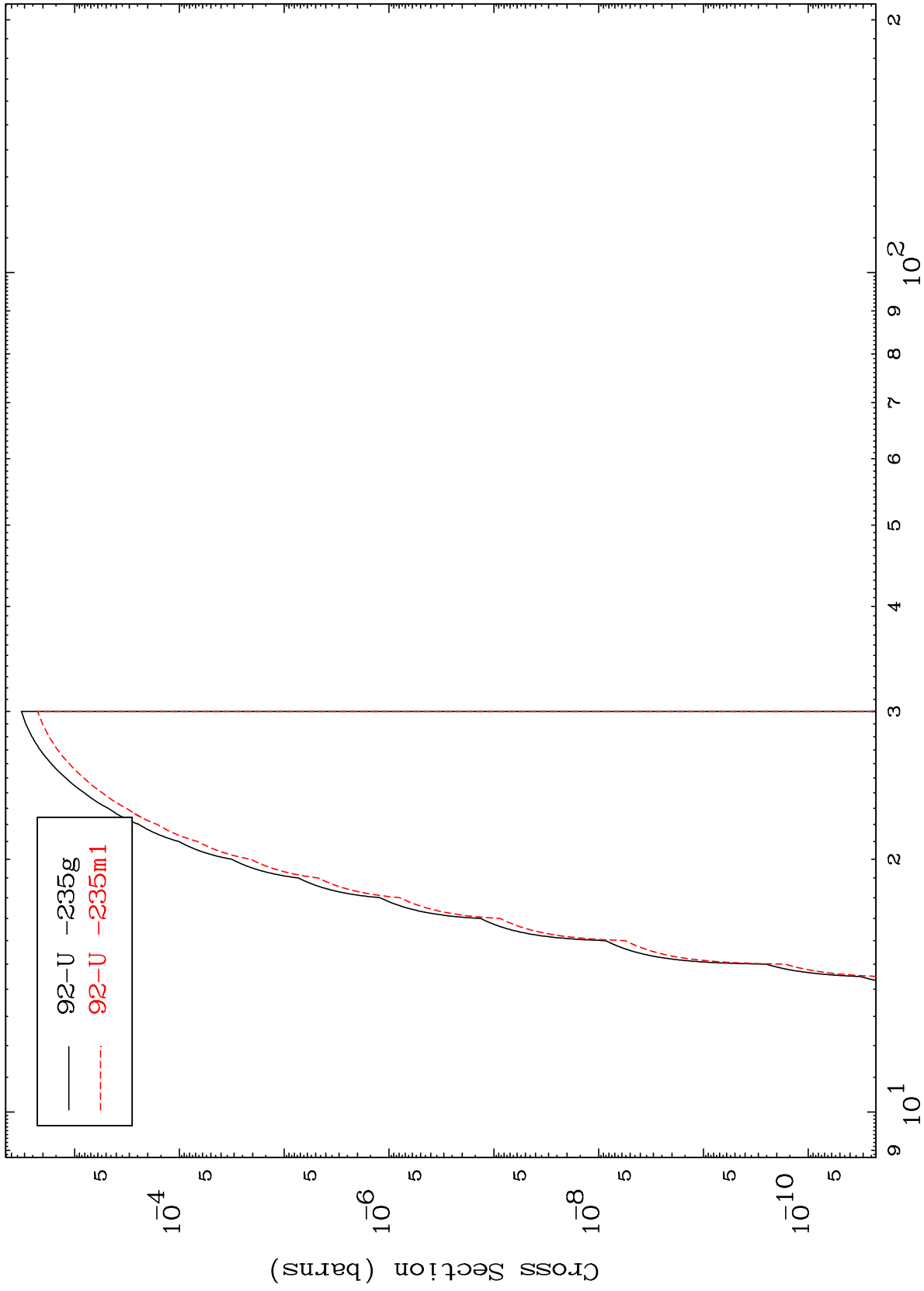
93-Np-237

MAT 9346

(n,n') d

93-Np-237

Radionuclide Production Cross Section



92-U -235g  
92-U -235m1

28

Incident Energy (MeV)

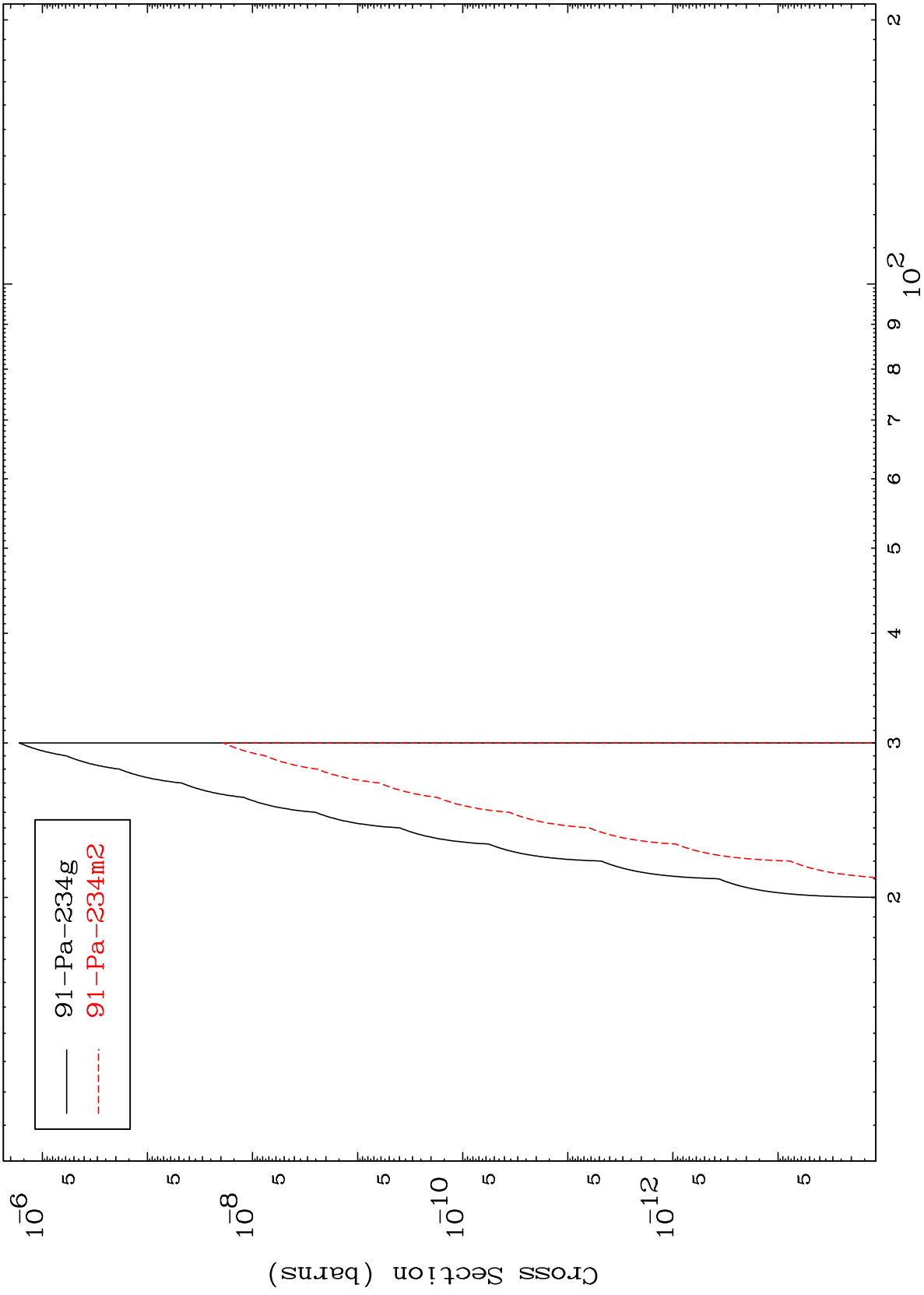
93-Np-237

MAT 9346

(n, n') He-3

93-Np-237

Radionuclide Production Cross Section



29

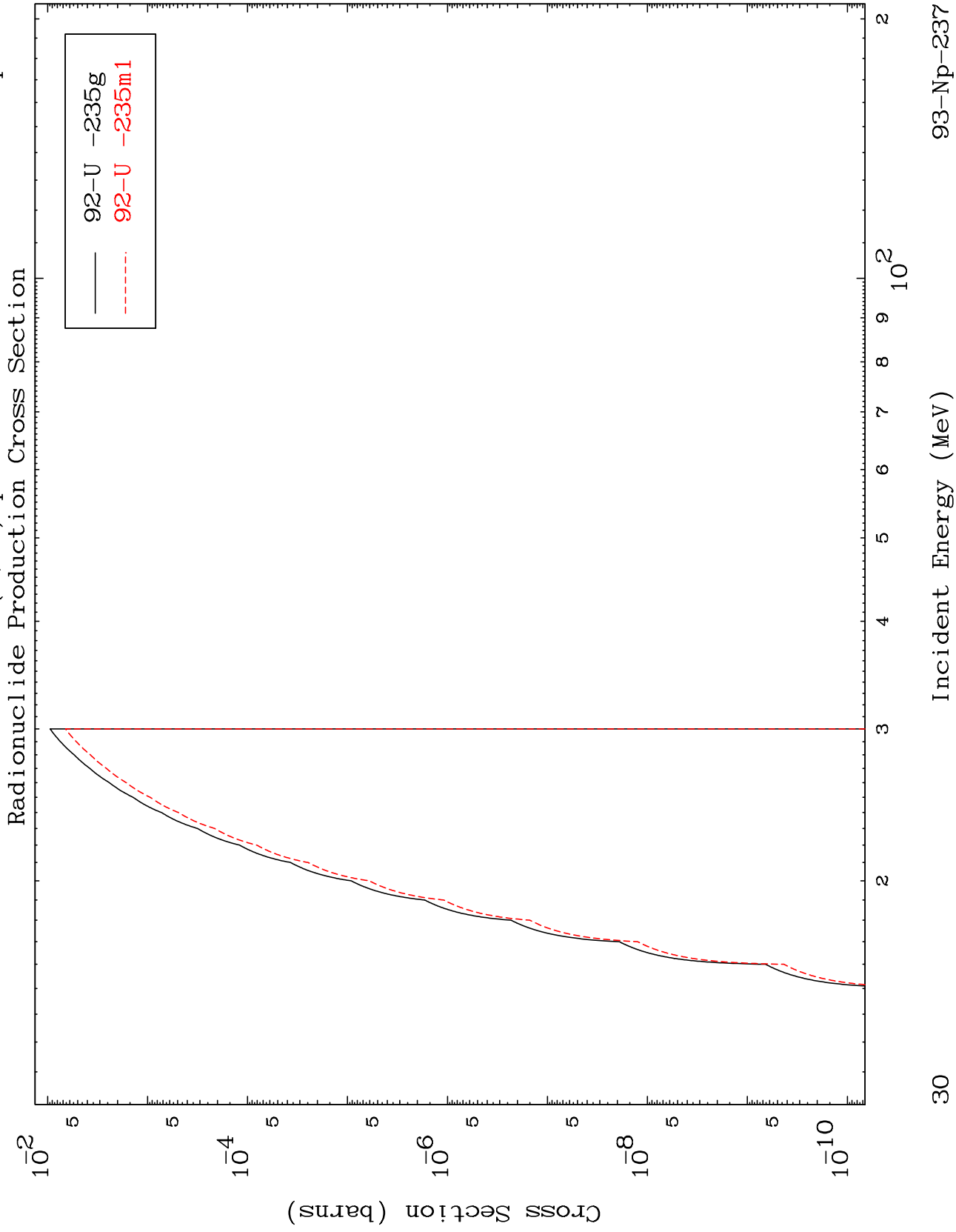
Incident Energy (MeV)

93-Np-237

MAT 9346

(n,2n) p

93-Np-237



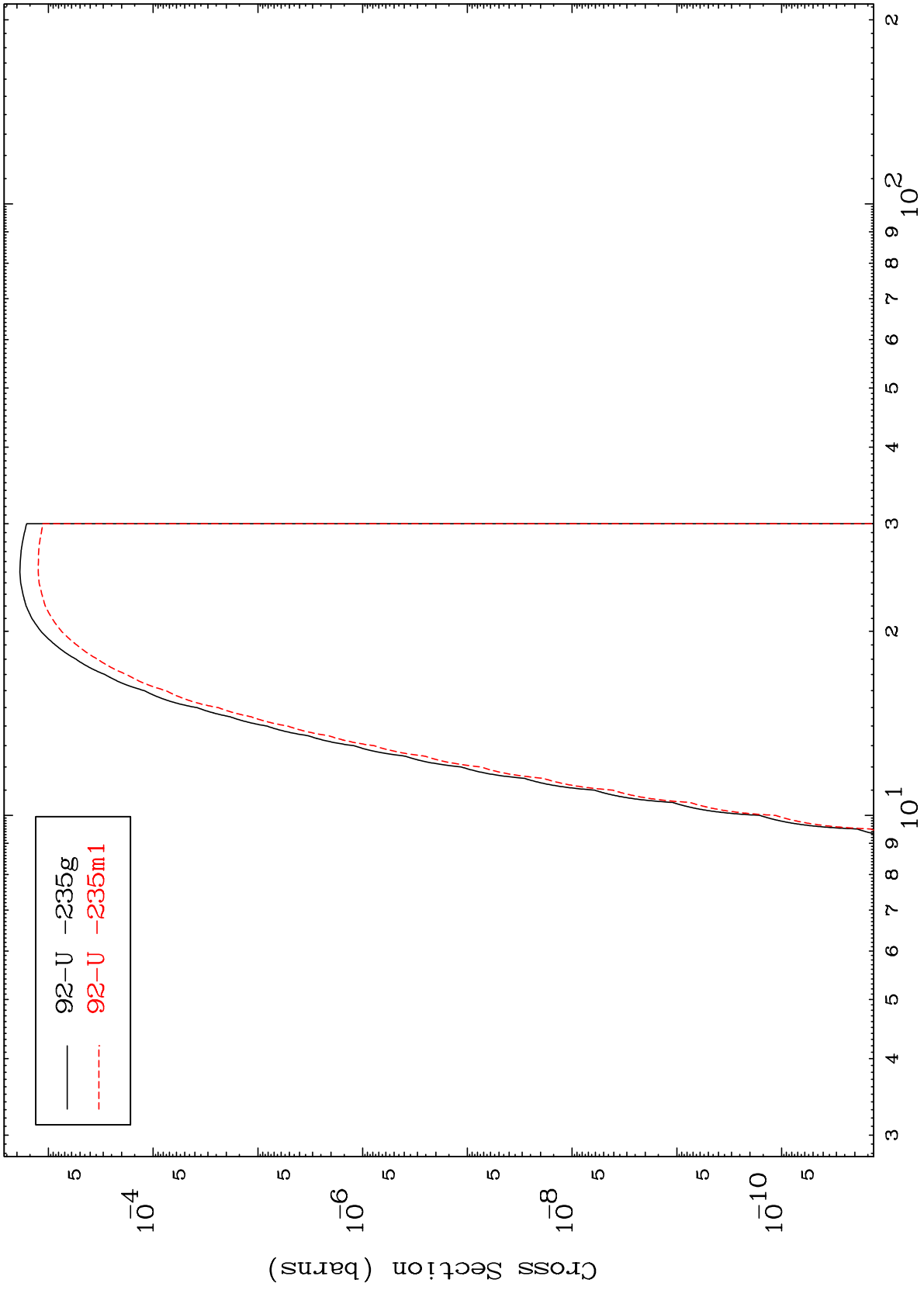
30

93-Np-237

MAT 9346

93-Np-237

(n, t)  
Radionuclide Production Cross Section



— 92-U -235g  
- - - 92-U -235m1

93-Np-237

Incident Energy (MeV)

31

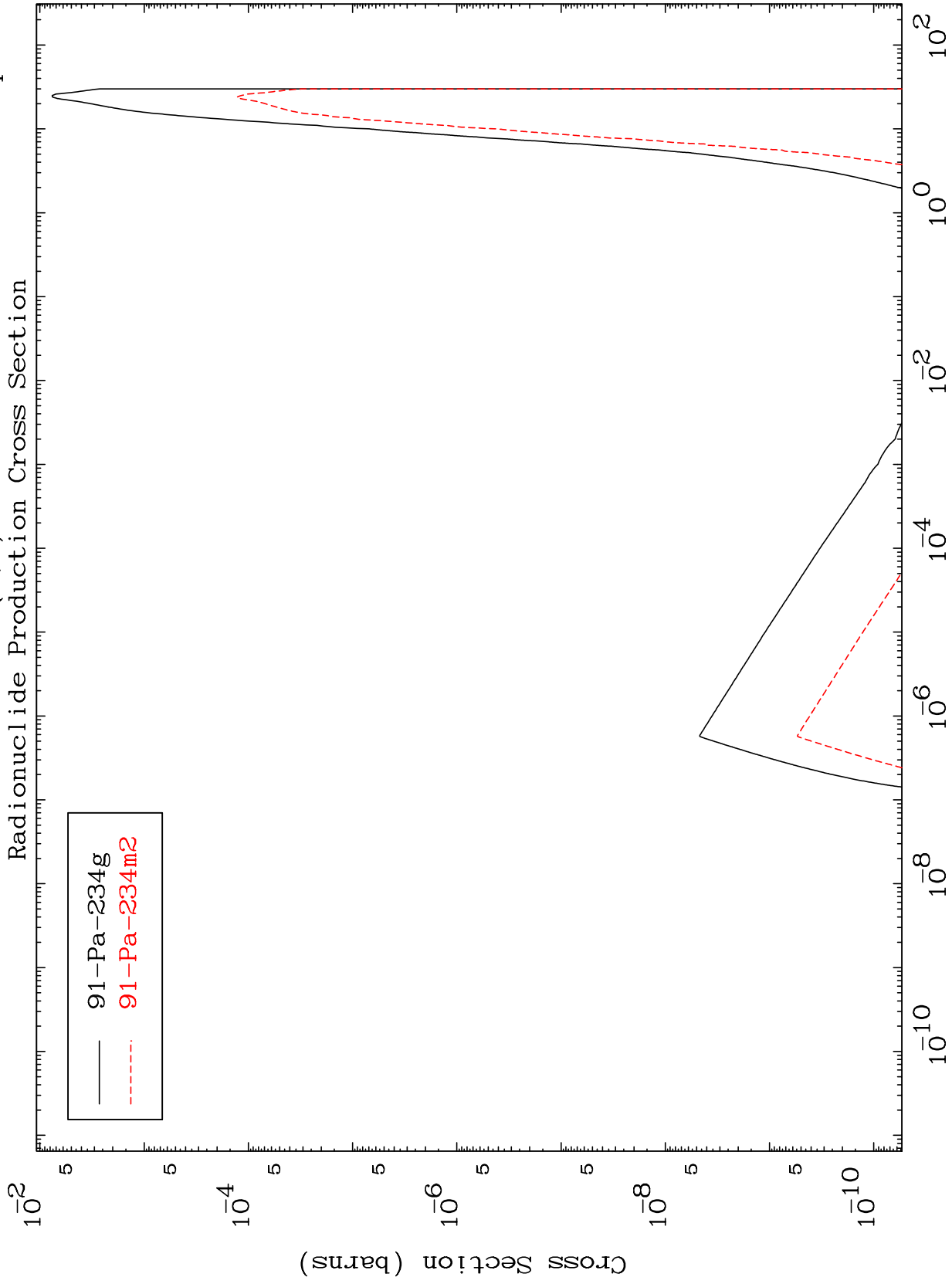


MAT 9346

(n,  $\alpha$ )

93-Np-237

Radionuclide Production Cross Section



32

Incident Energy (MeV)

93-Np-237