

Program EVALPLOT  
(Version 2018-1)

by

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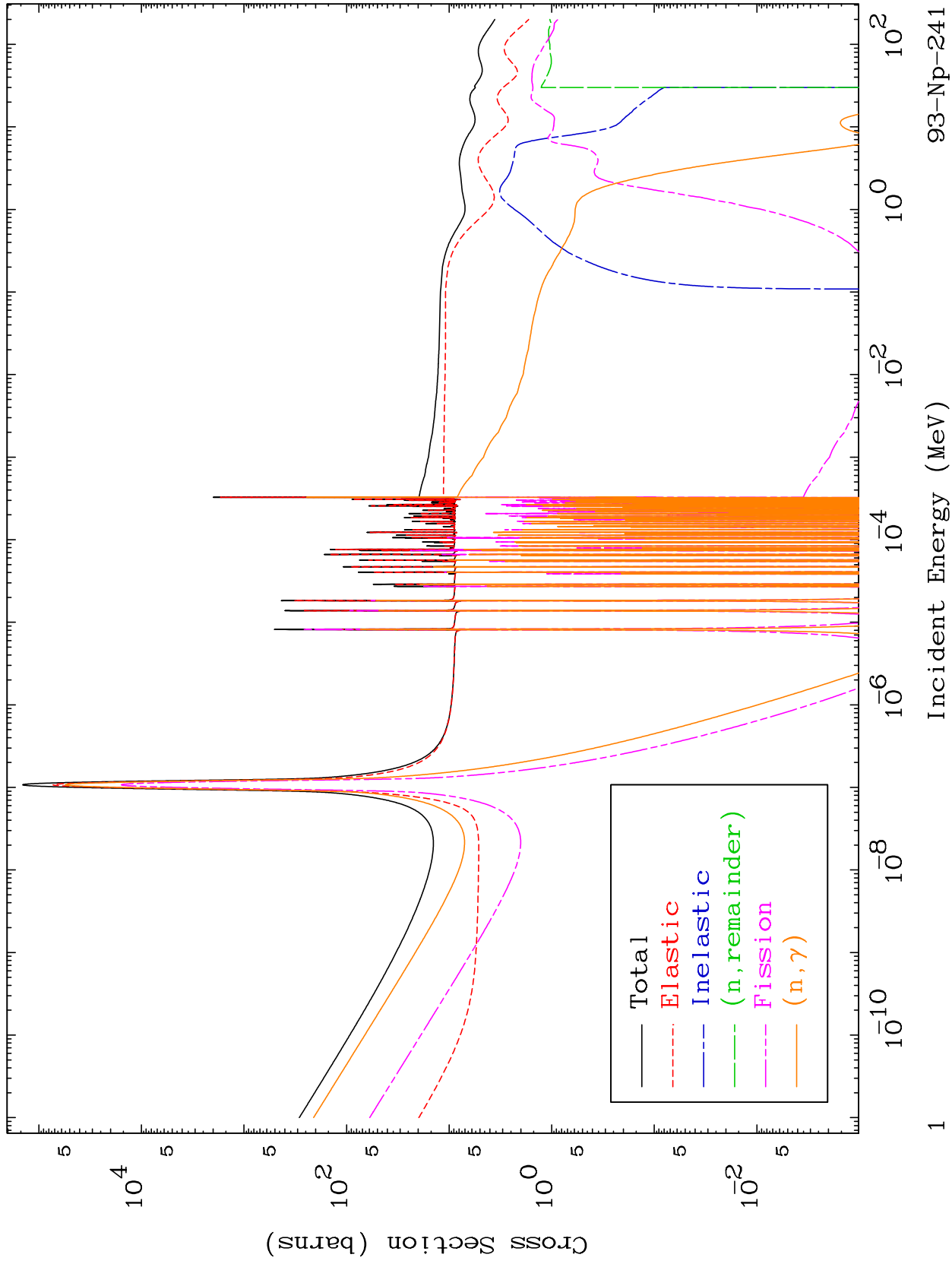
Press Mouse Button to Start

MAT 9358

Major

293 Kelvin Cross Sections

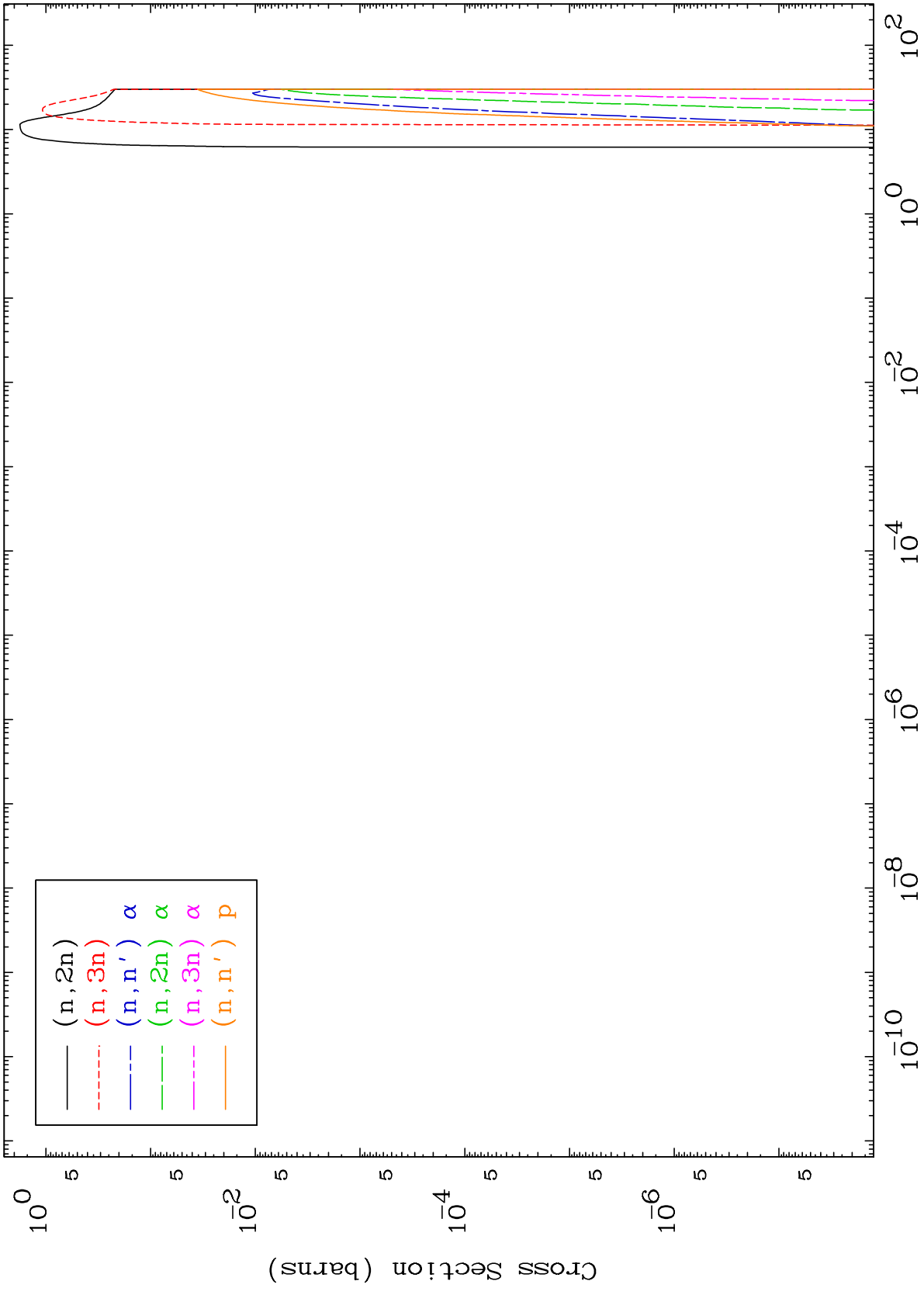
93-Np-241



MAT 9358

Neutron Production  
293 Kelvin Cross Sections

93-Np-241

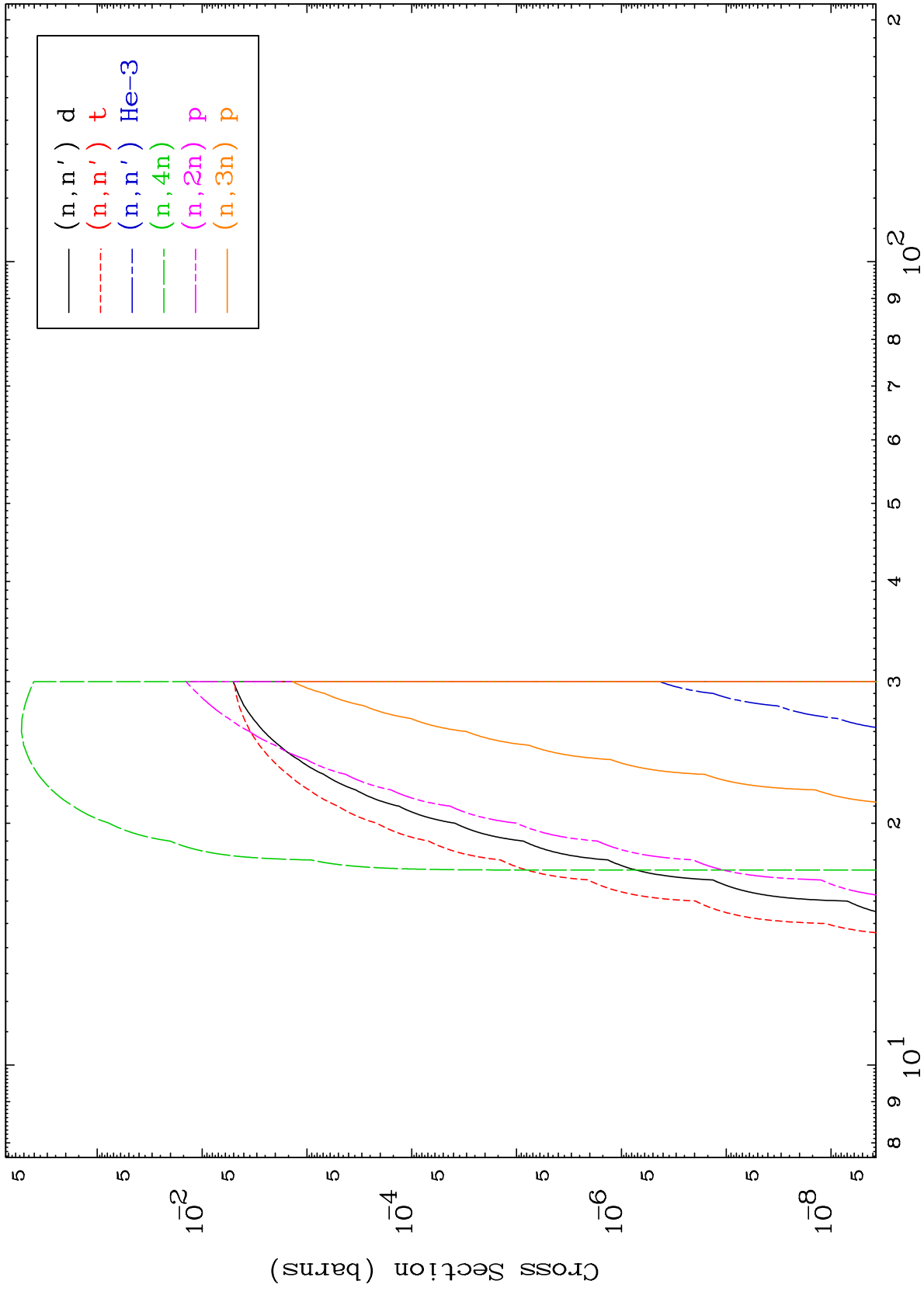


93-Np-241

MAT 9358

Neutron Production  
293 Kelvin Cross Sections

93-Np-241



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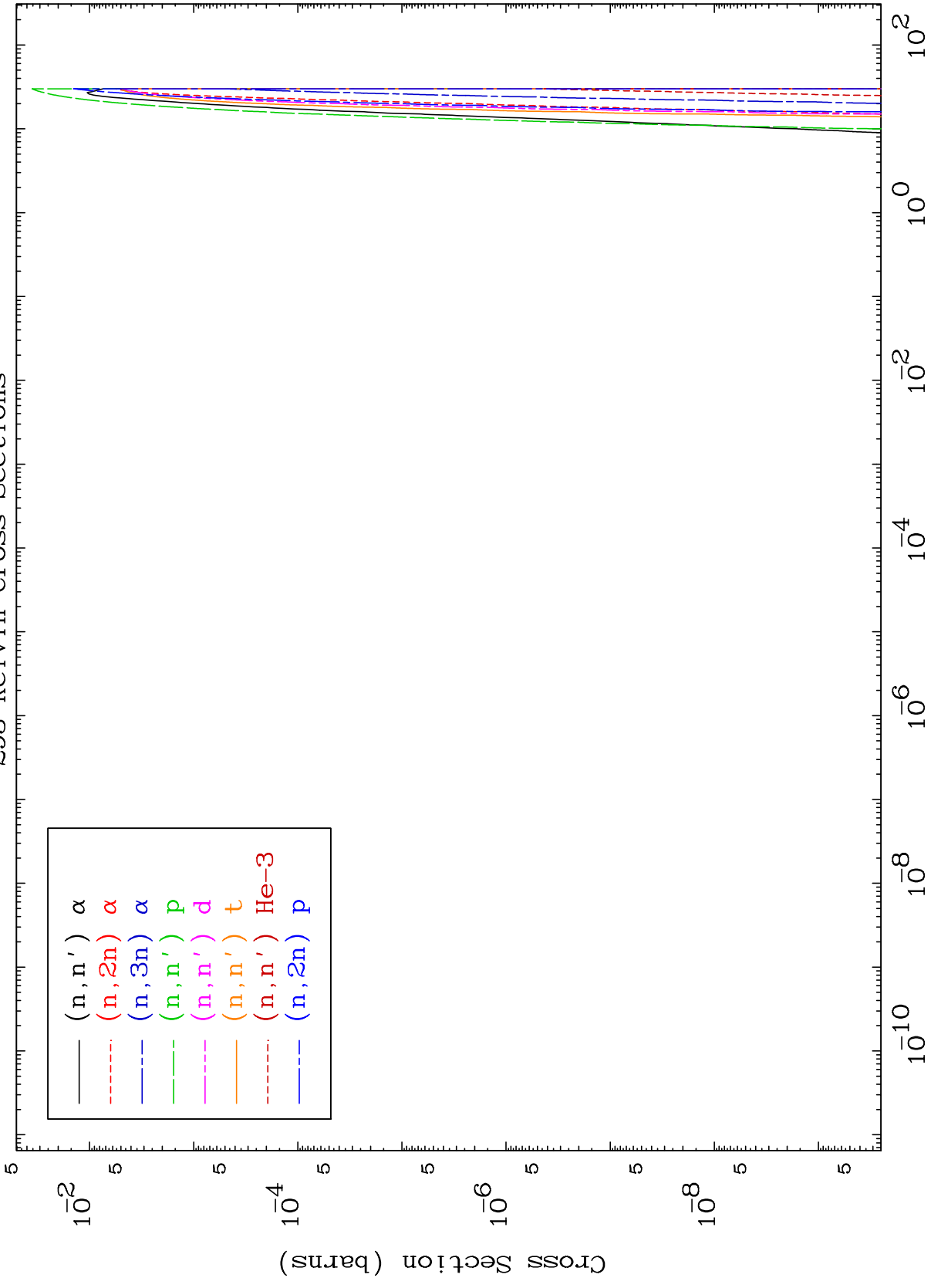
Incident Energy (MeV)

93-Np-241

MAT 9358

Charged Particle  
293 Kelvin Cross Sections

93-Np-241



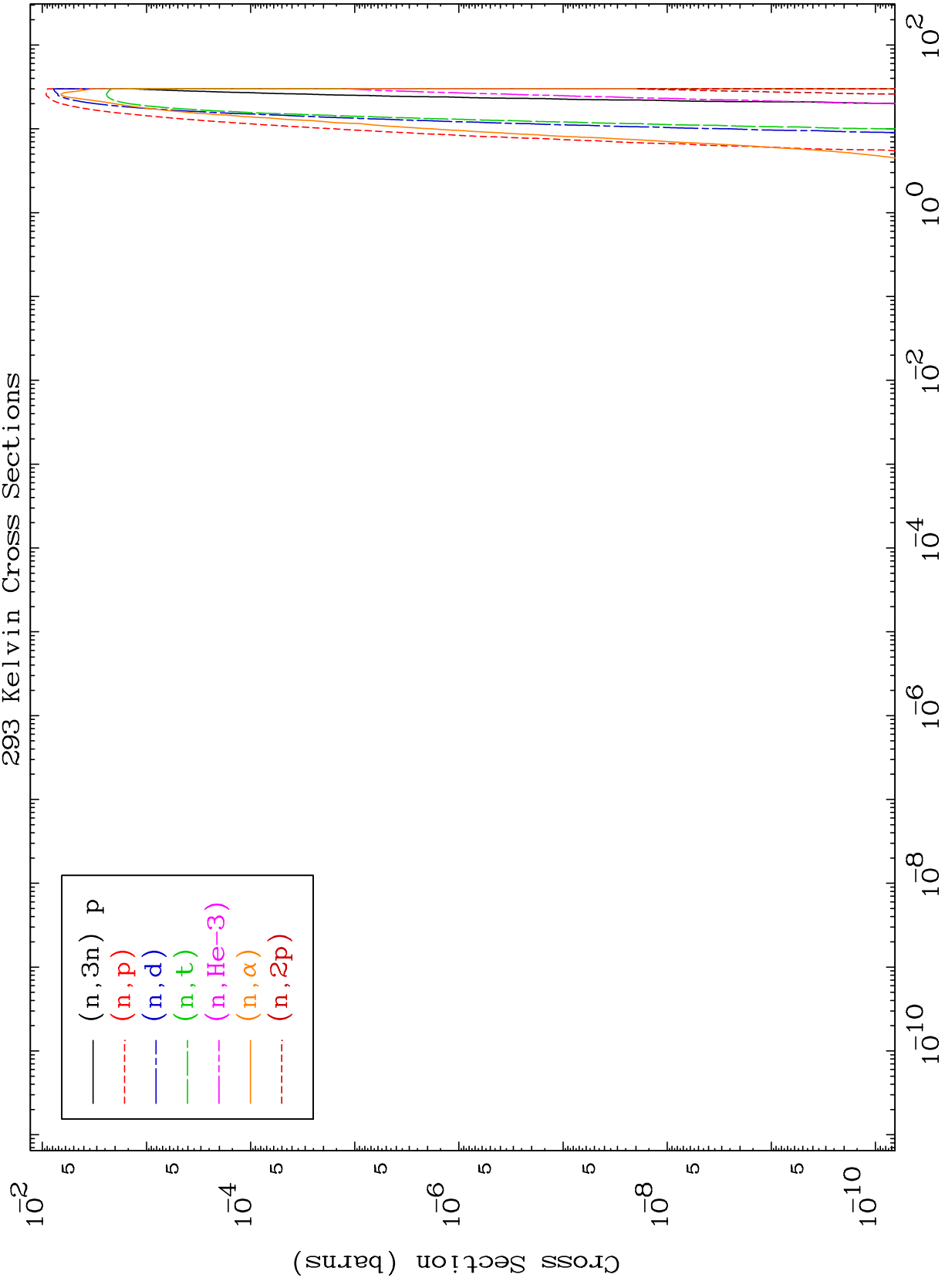
93-Np-241

Incident Energy (MeV)

MAT 9358

Charged Particle  
293 Kelvin Cross Sections

93-Np-241



5

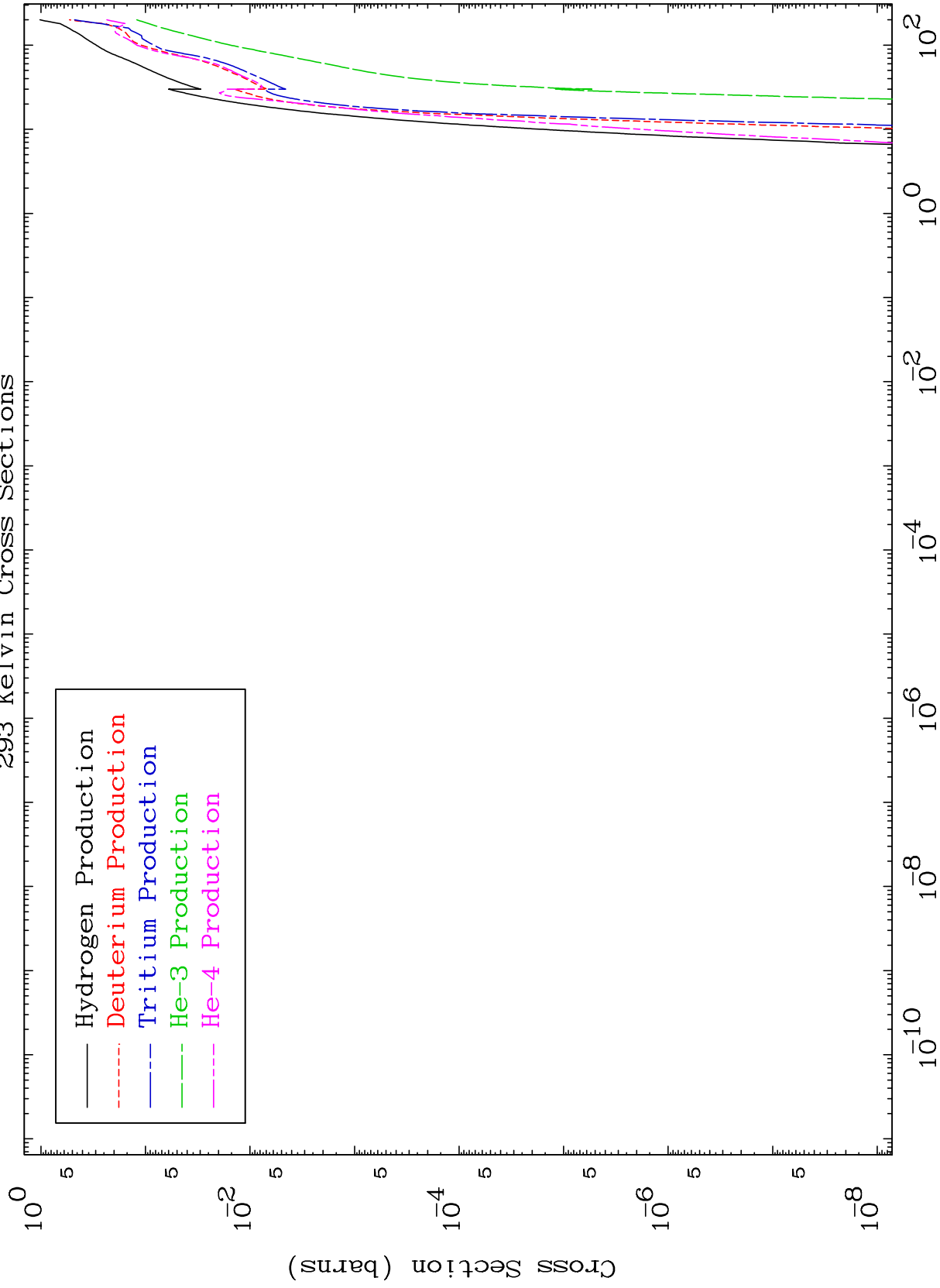
Incident Energy (MeV)

93-Np-241

MAT 9358

Particle Production  
293 Kelvin Cross Sections

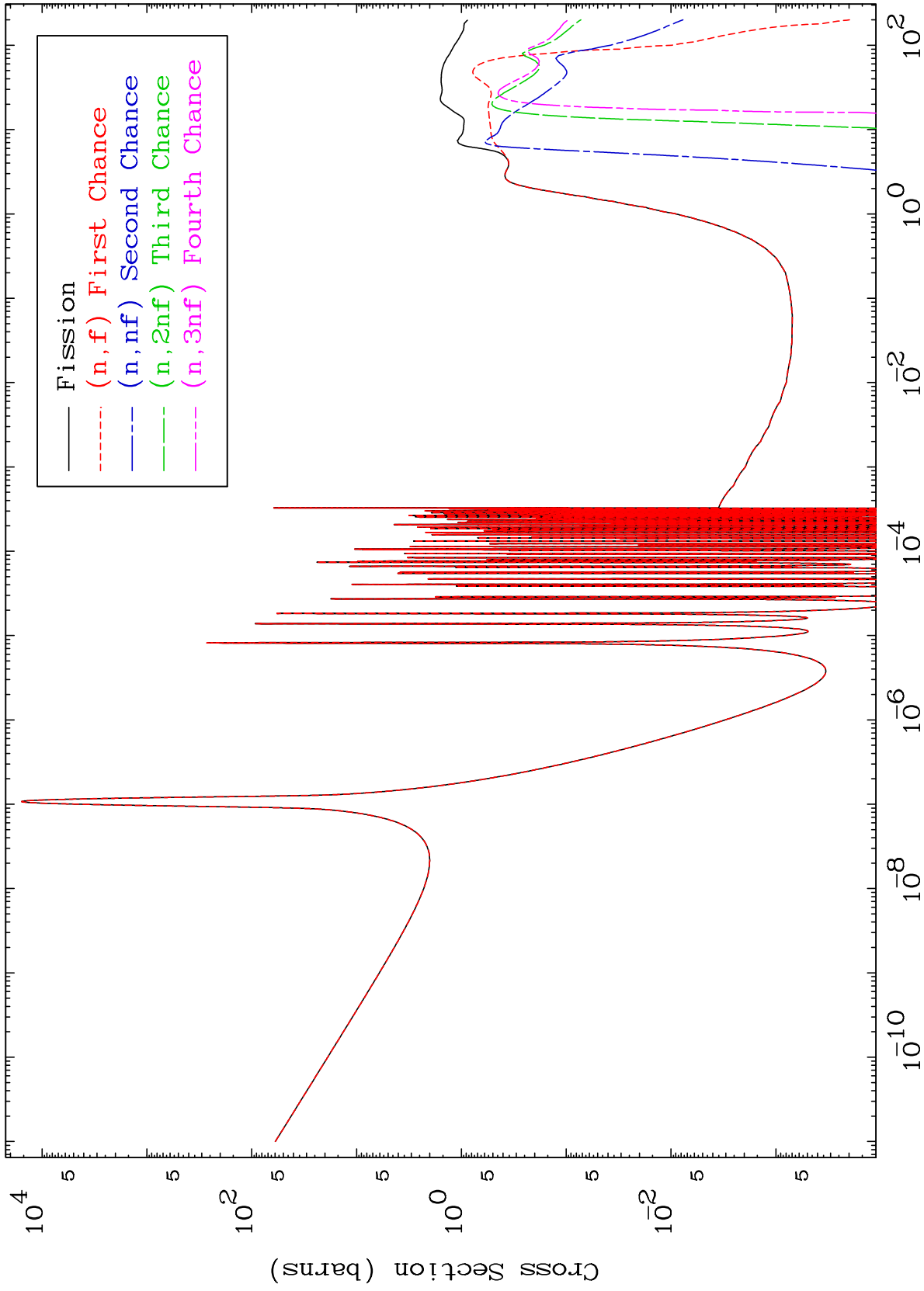
93-Np-241



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Fission  
293 Kelvin Cross Sections

93-Np-241

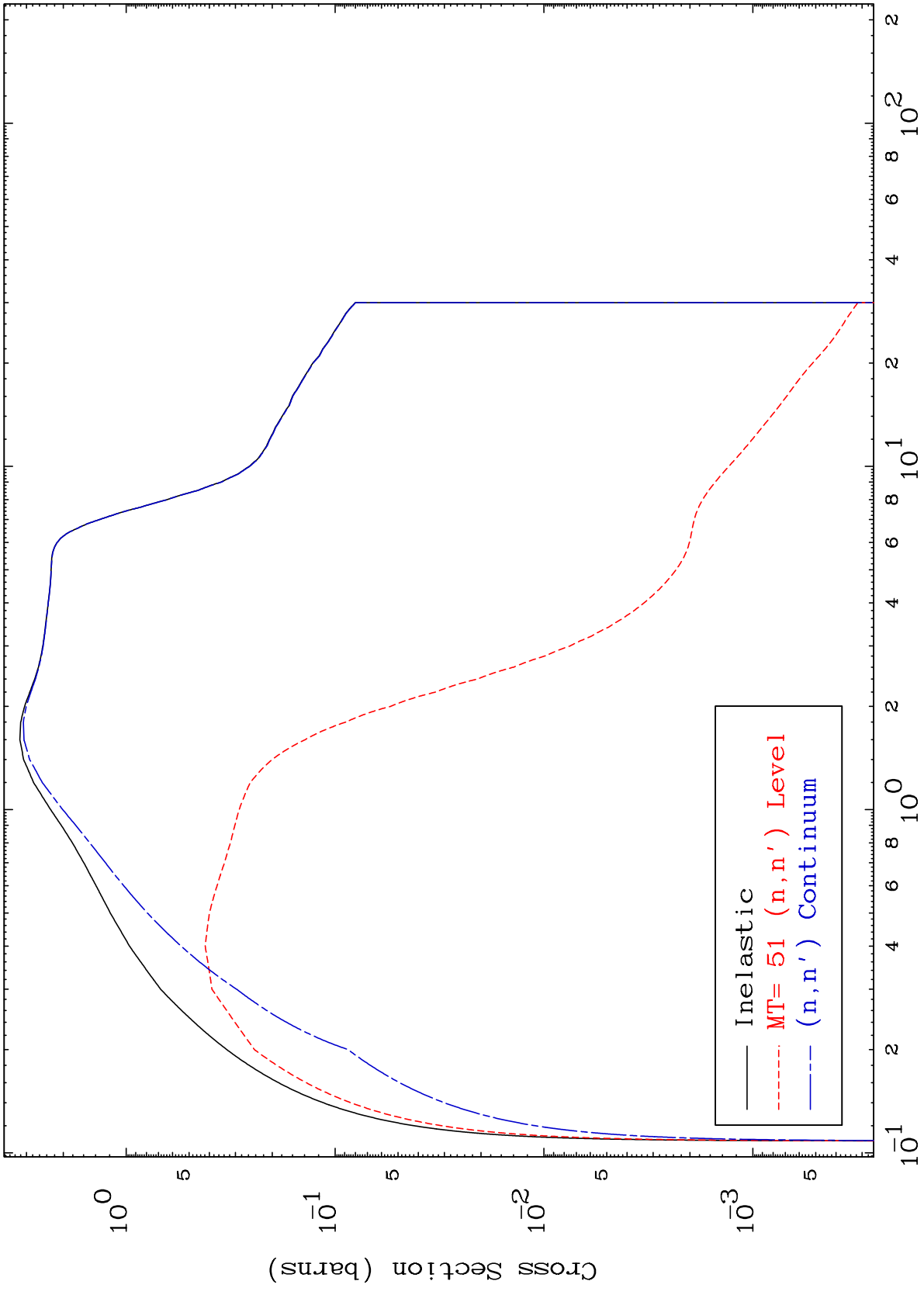




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93-Np-241

(n,n') Level  
293 Kelvin Cross Sections



93-Np-241

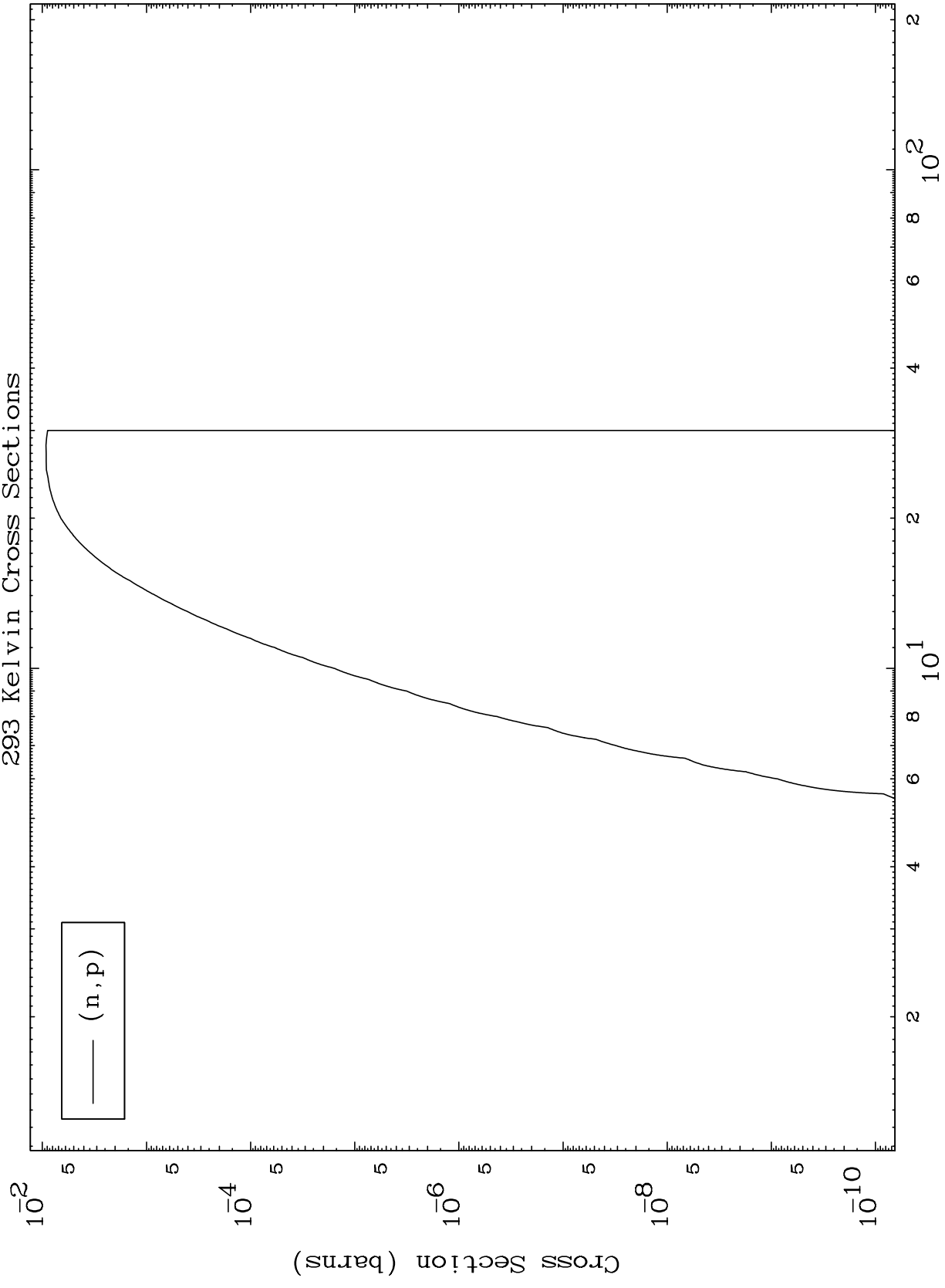
Incident Energy (MeV)

8

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(n,p) Levels  
293 Kelvin Cross Sections

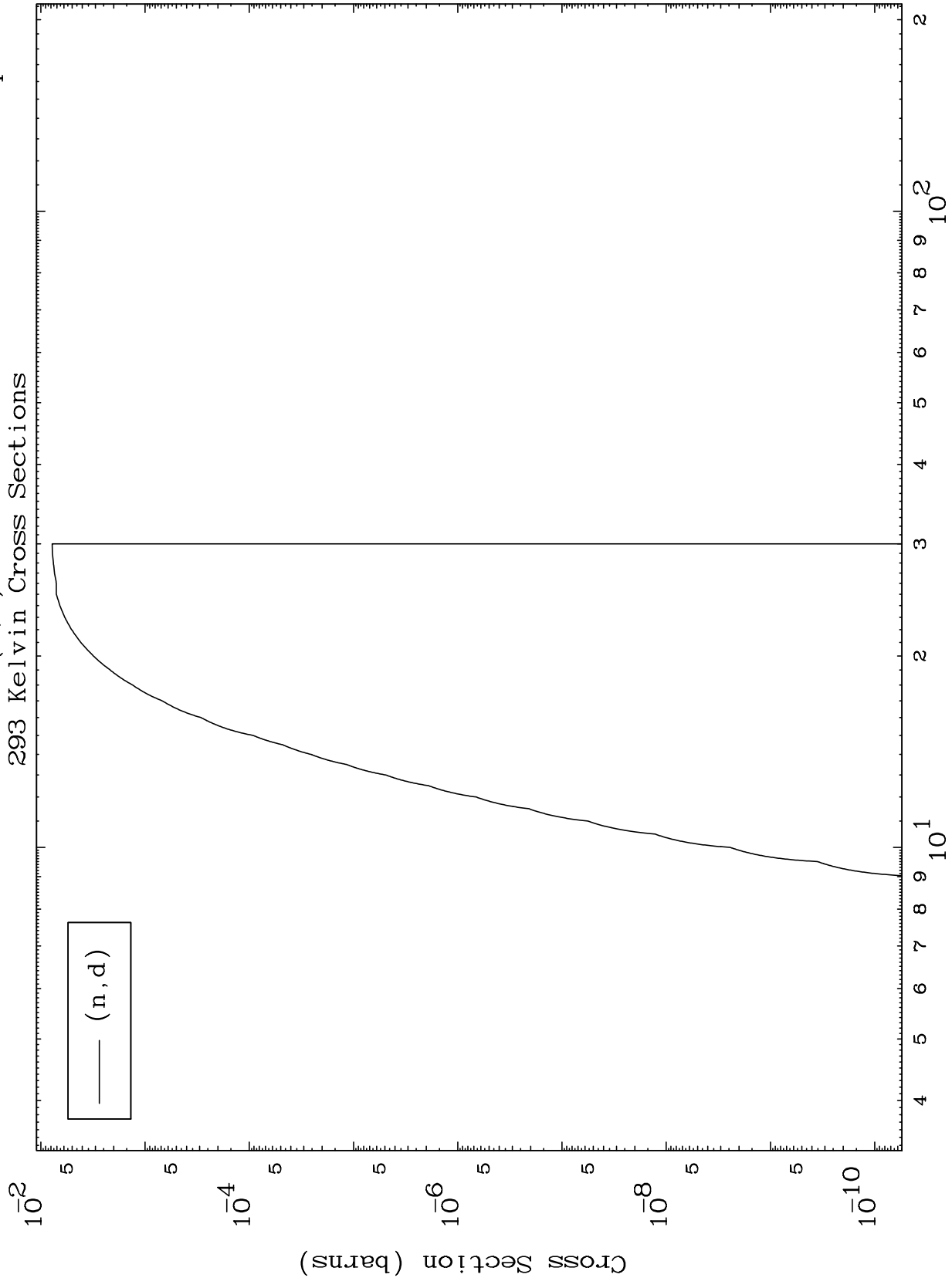
93-Np-241



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(n,d) Levels  
293 Kelvin Cross Sections

93-Np-241



10

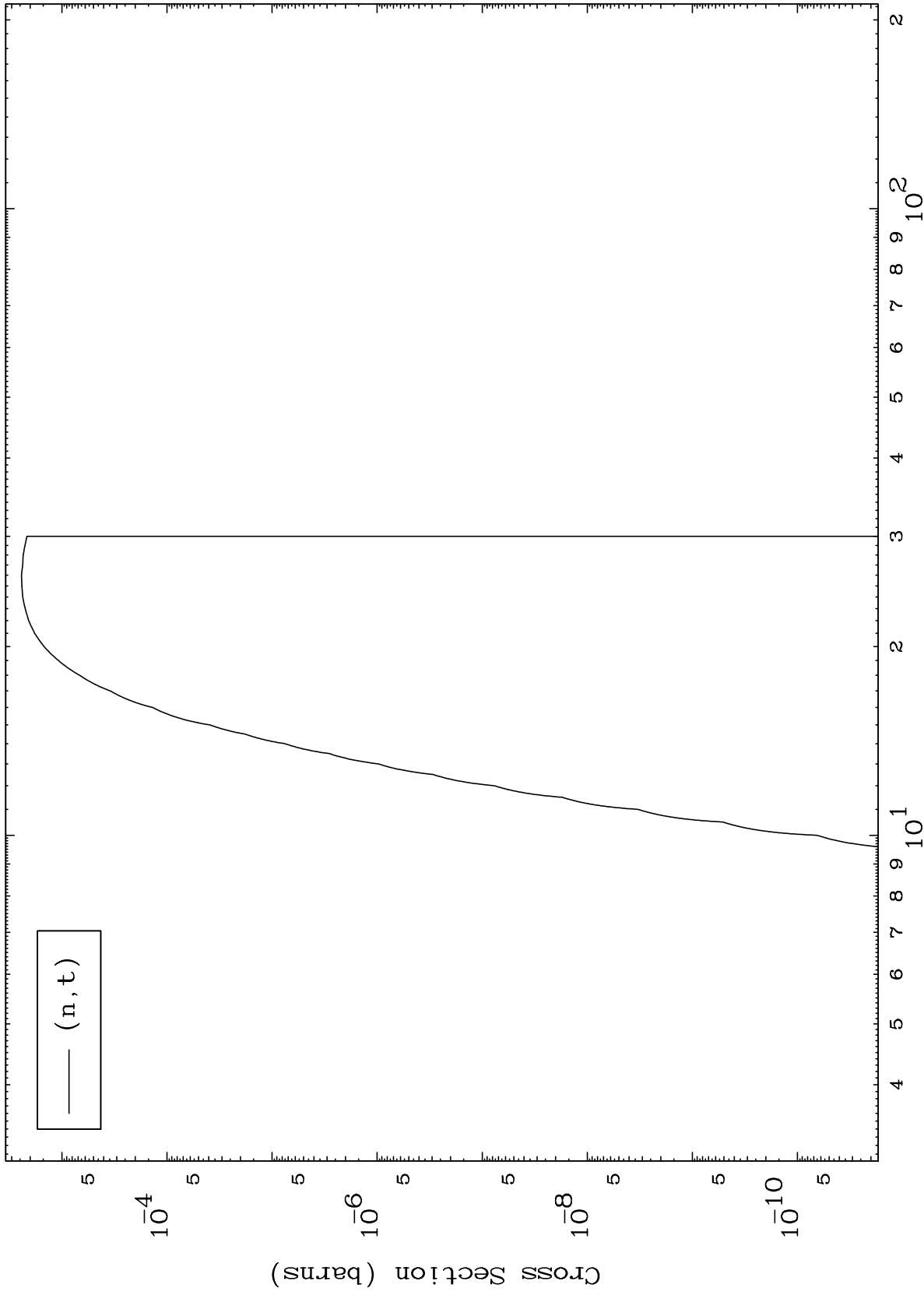
Incident Energy (MeV)

93-Np-241

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(n,t) Levels  
293 Kelvin Cross Sections

93-Np-241



11

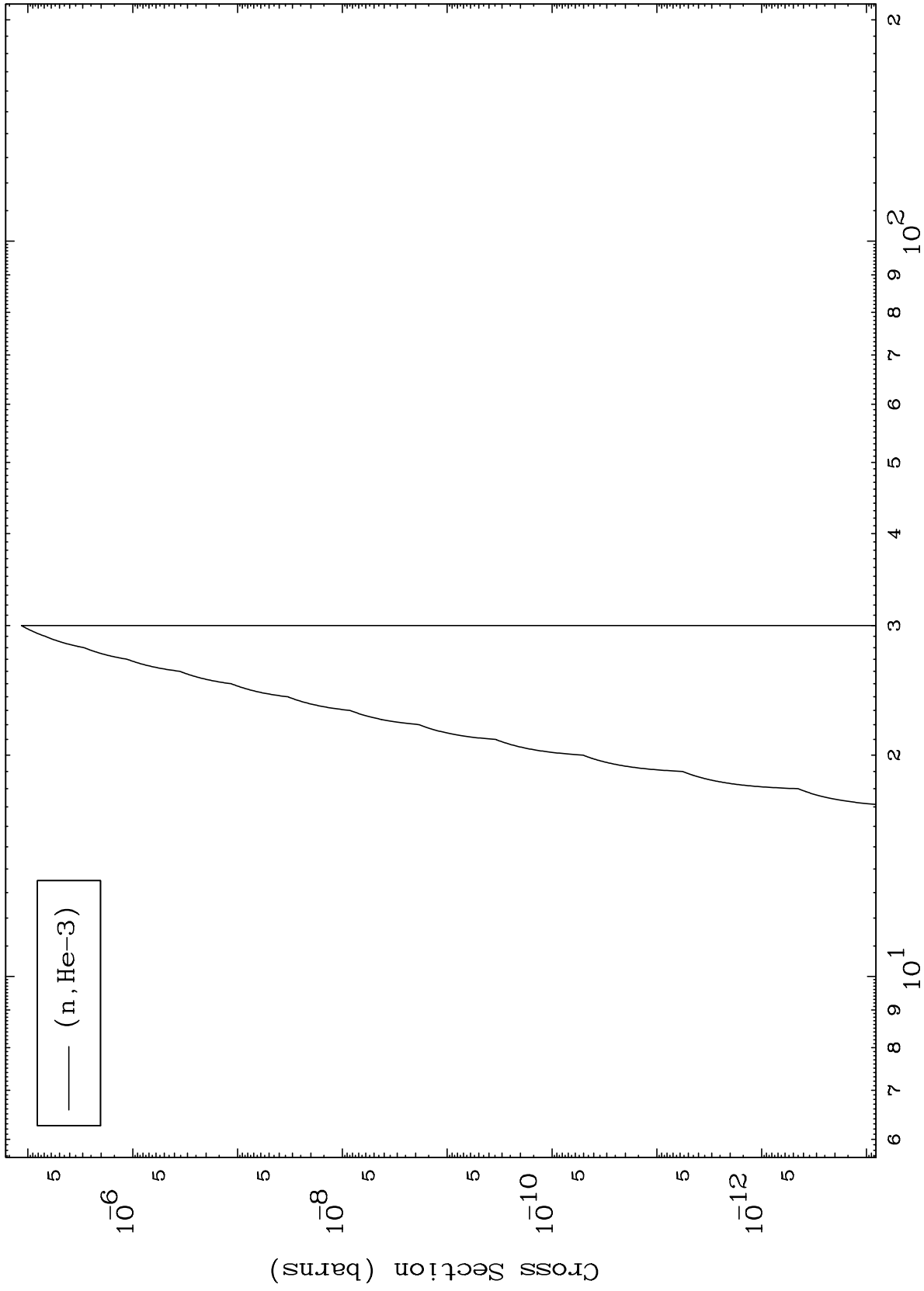
Incident Energy (MeV)

93-Np-241

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(n,He3) Levels  
293 Kelvin Cross Sections

93-Np-241



12

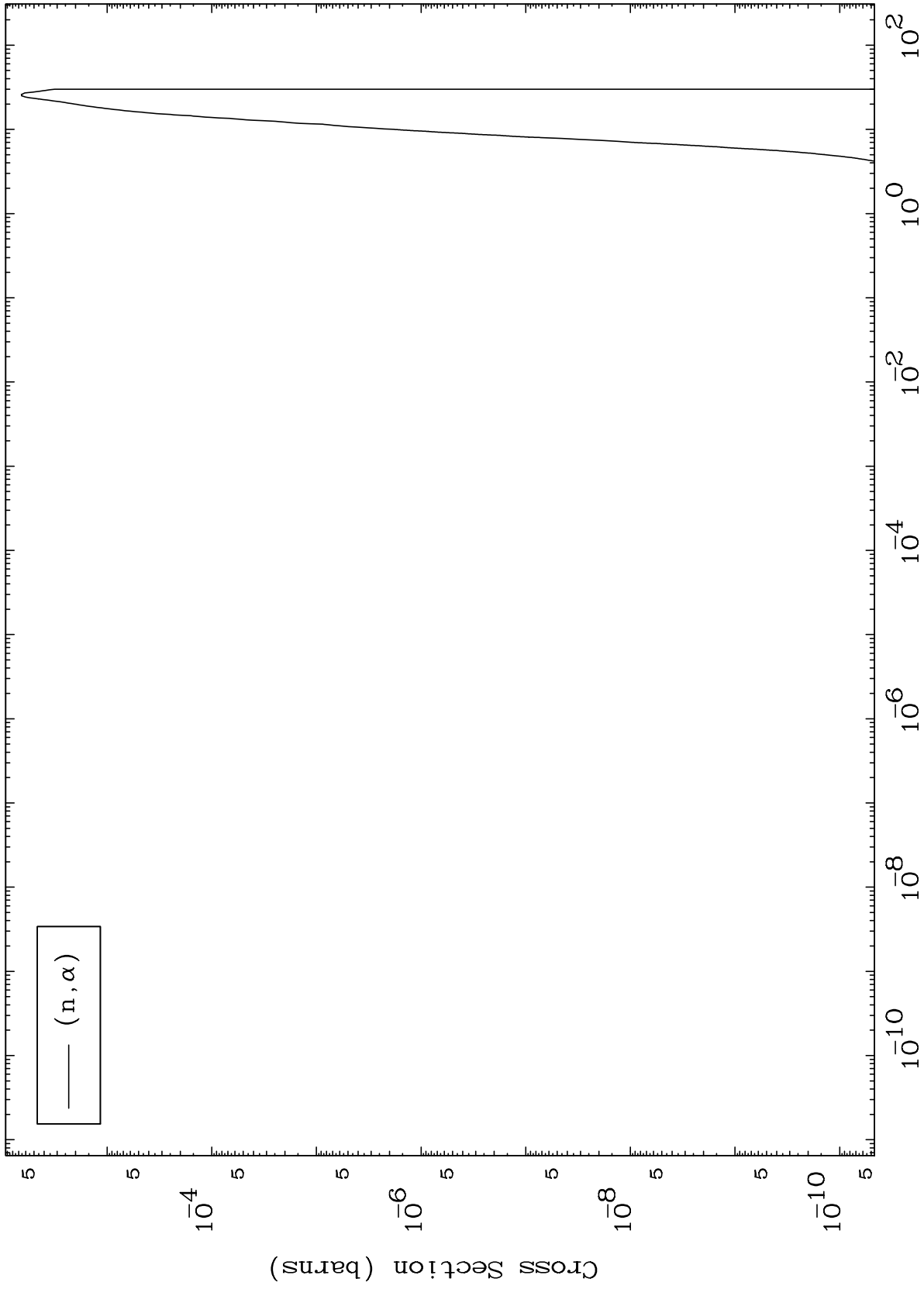
Incident Energy (MeV)

93-Np-241

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(n,α) Levels  
293 Kelvin Cross Sections

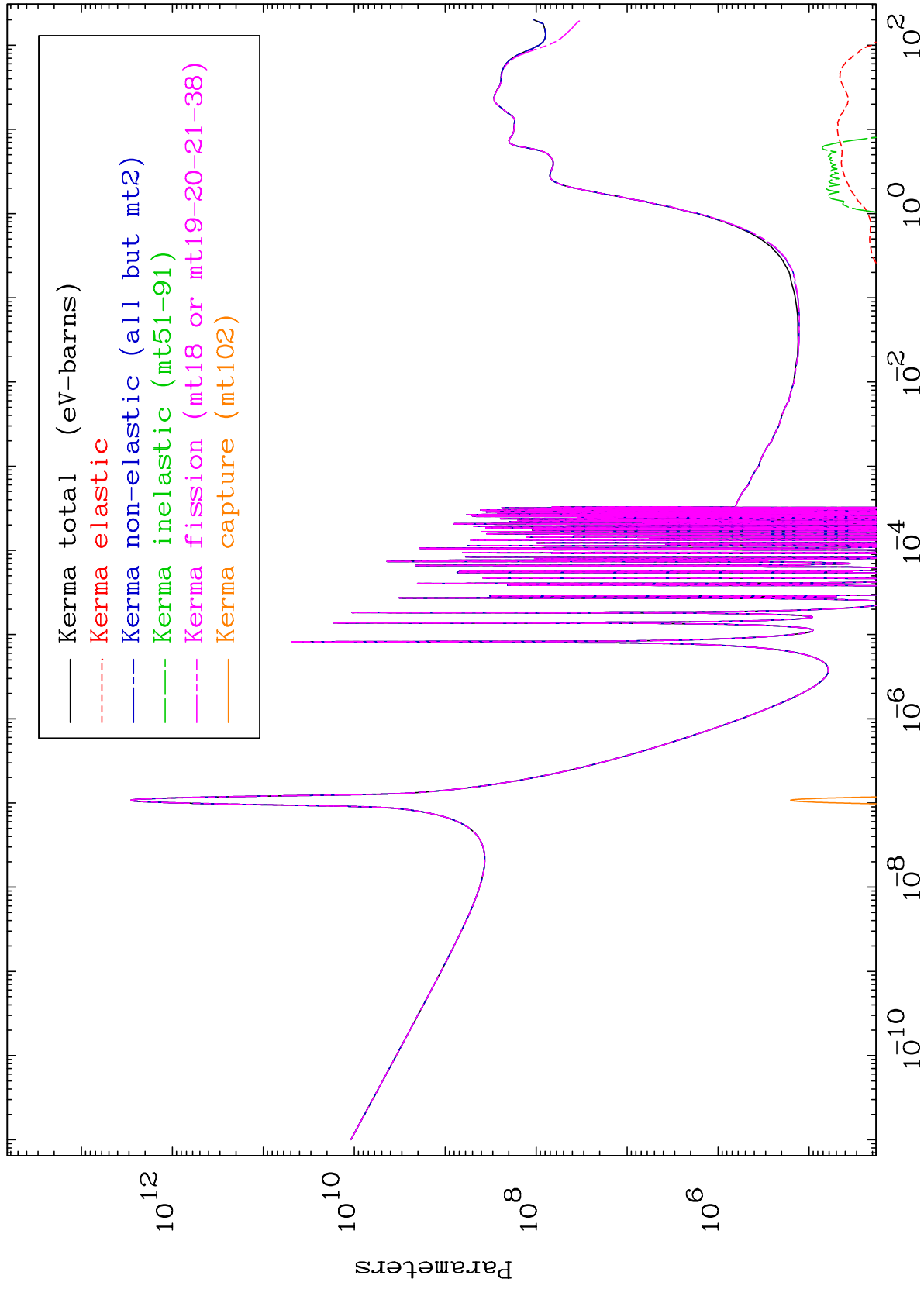
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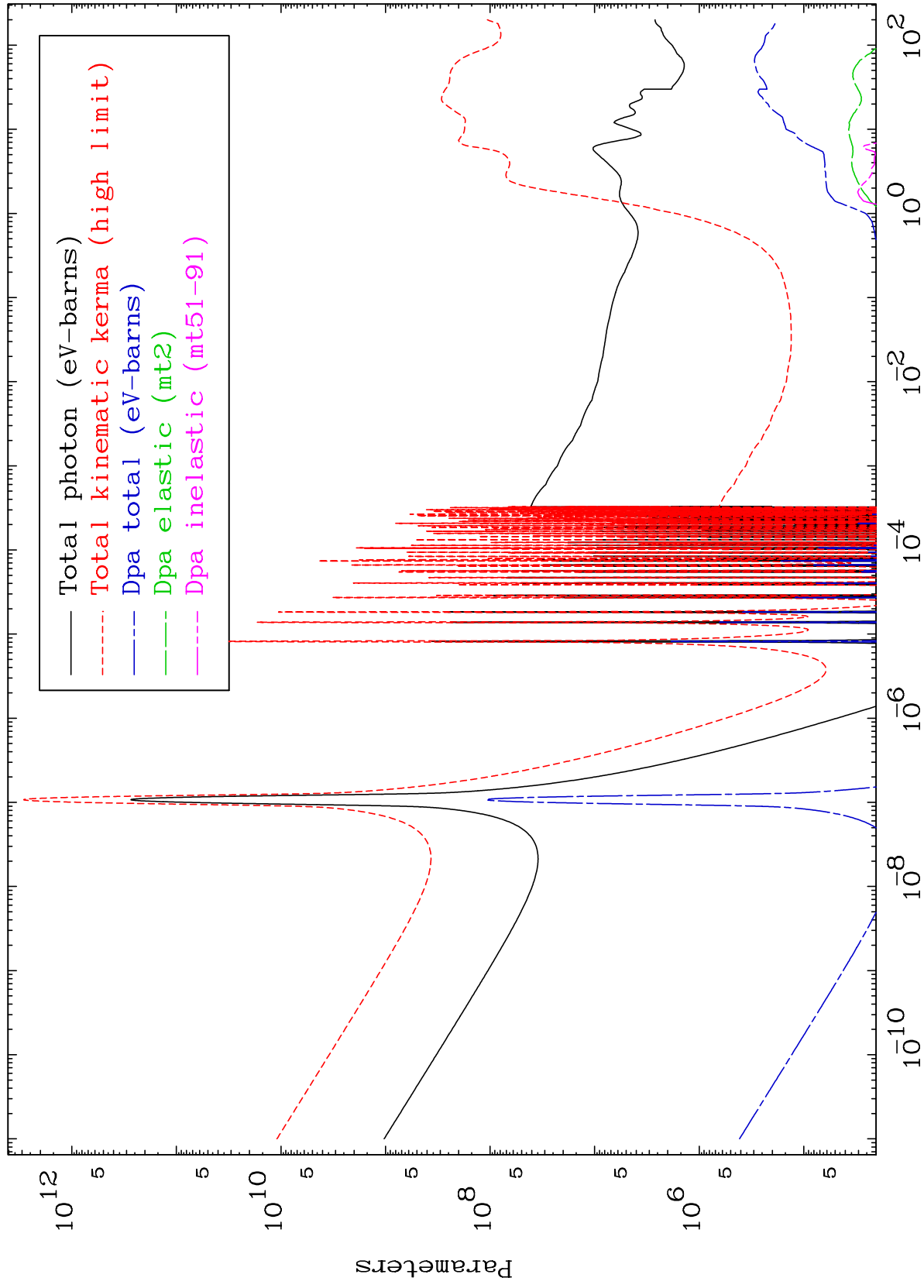


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Incident Energy (MeV)

93-Np-241



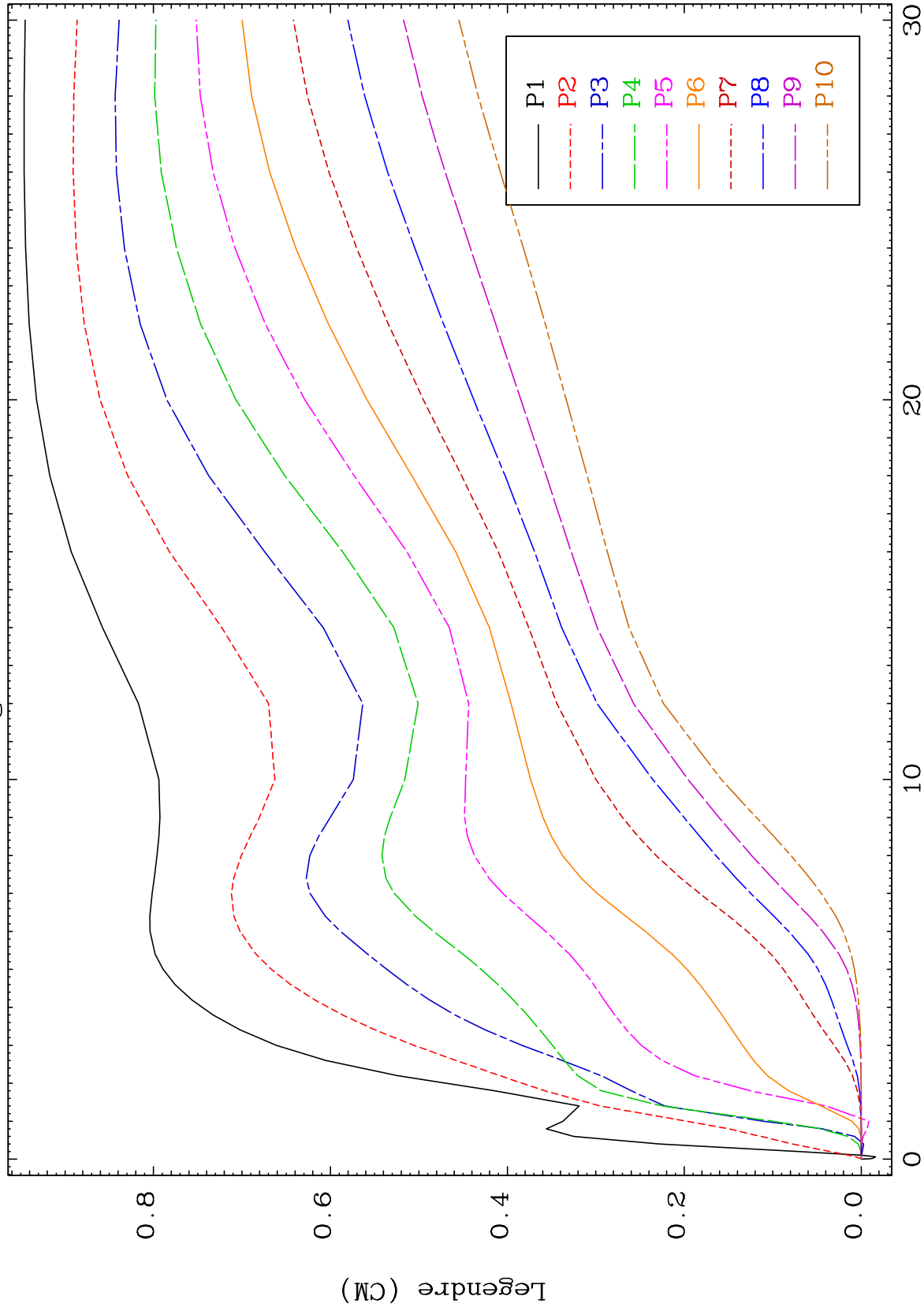




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Elastic Legendre Coefficients

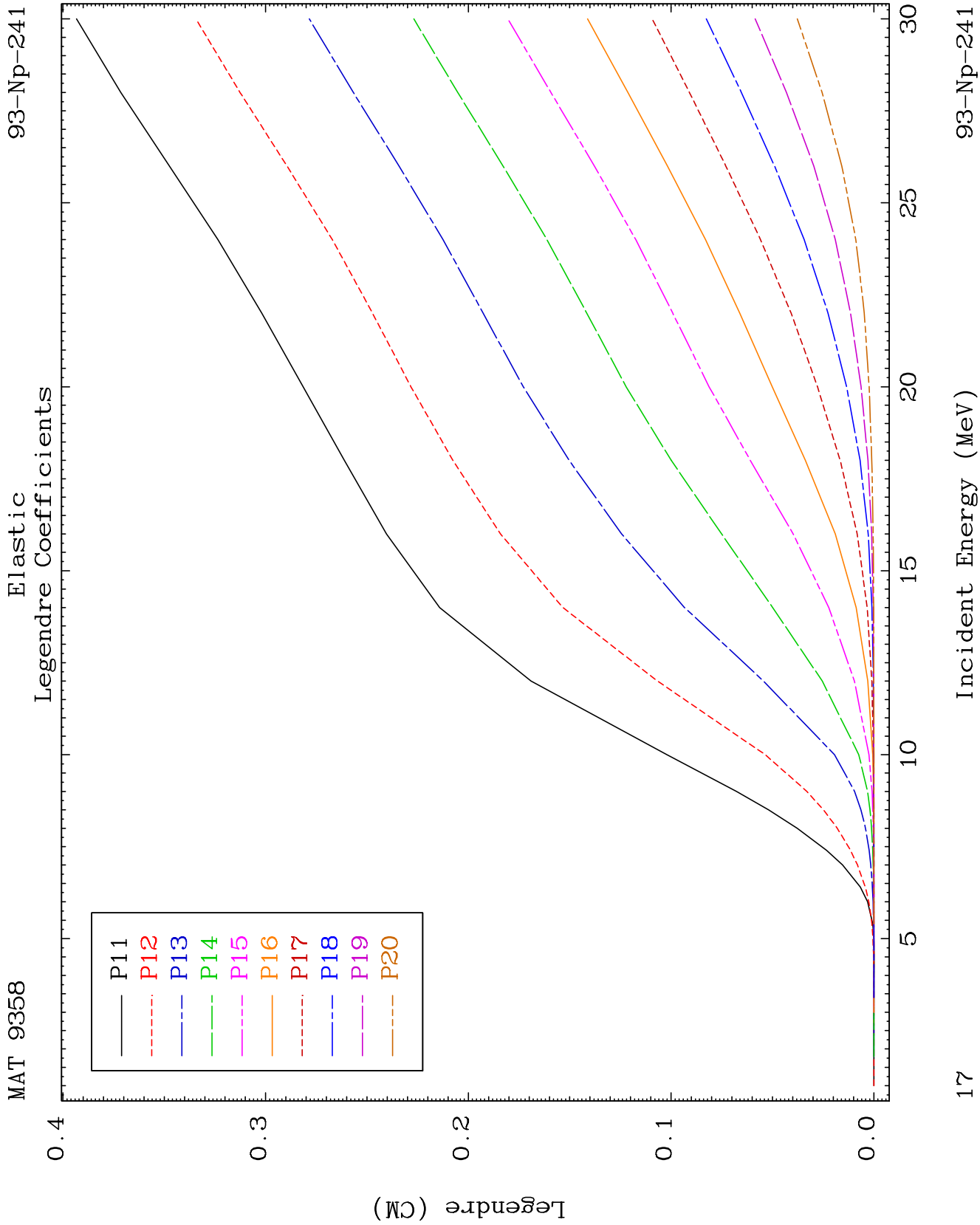
93-Np-241



16

Incident Energy (MeV)

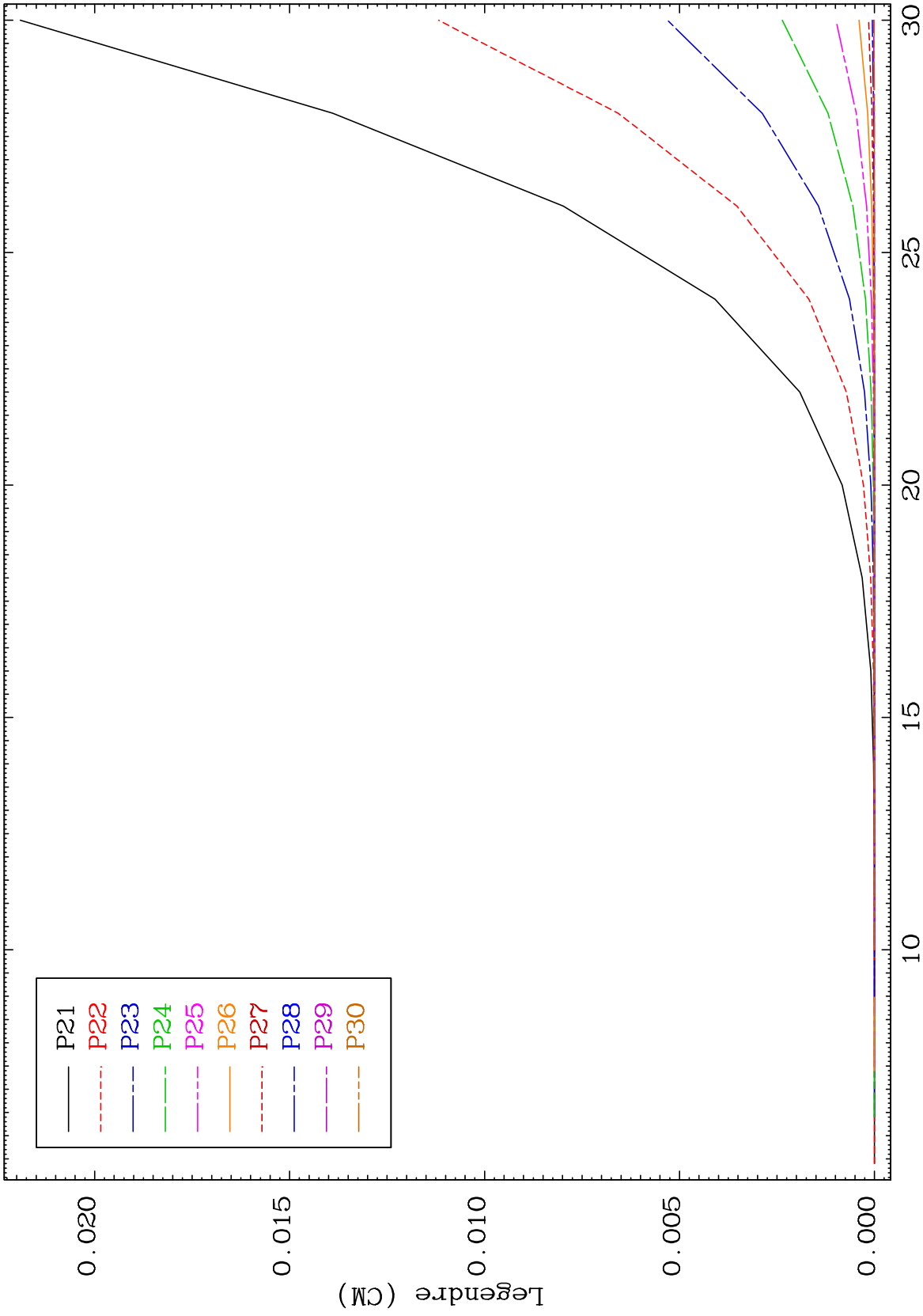
93-Np-241



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### Elastic Legendre Coefficients

93-Np-241



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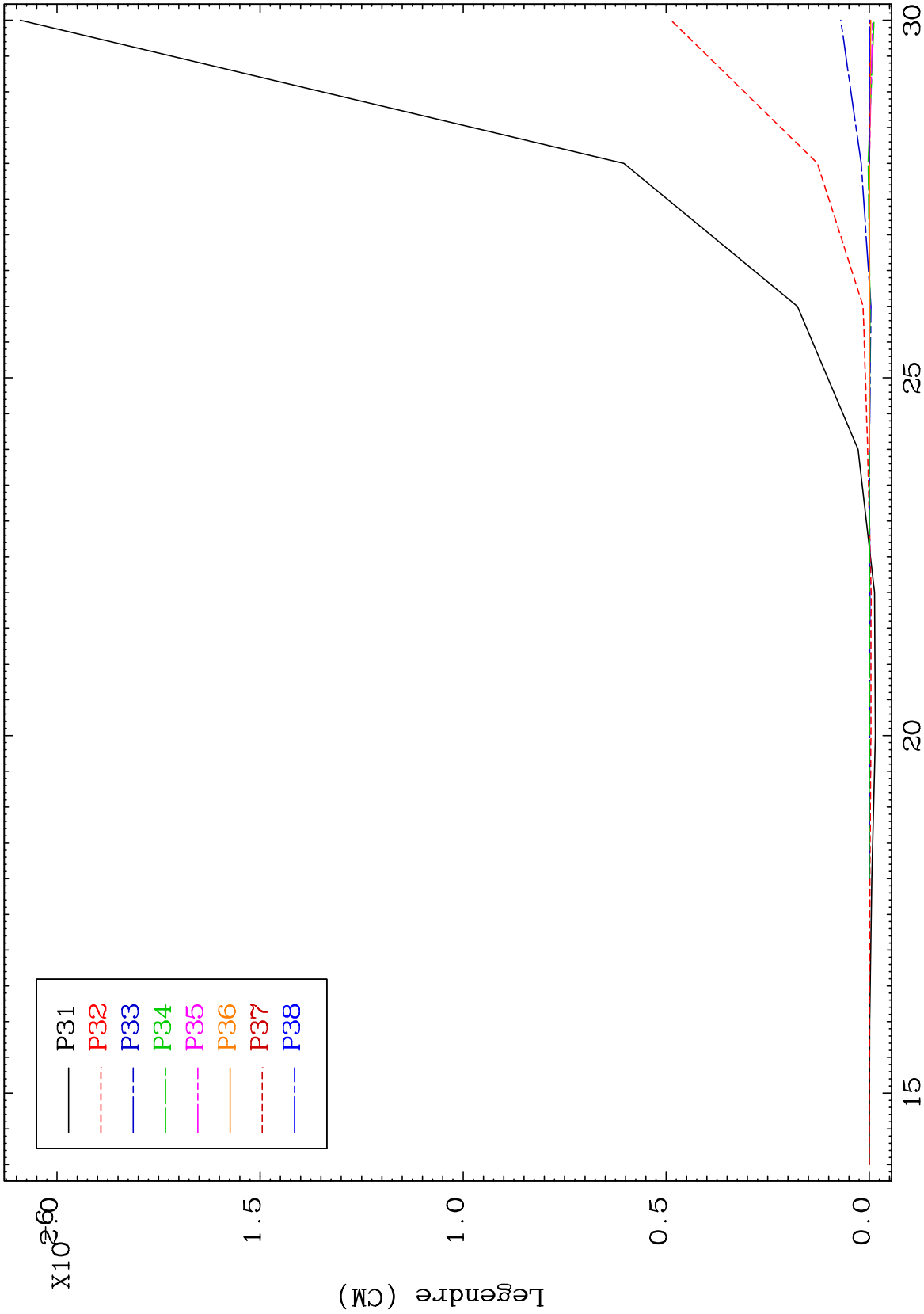
Incident Energy (MeV)

93-Np-241

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Elastic Legendre Coefficients

93-Np-241



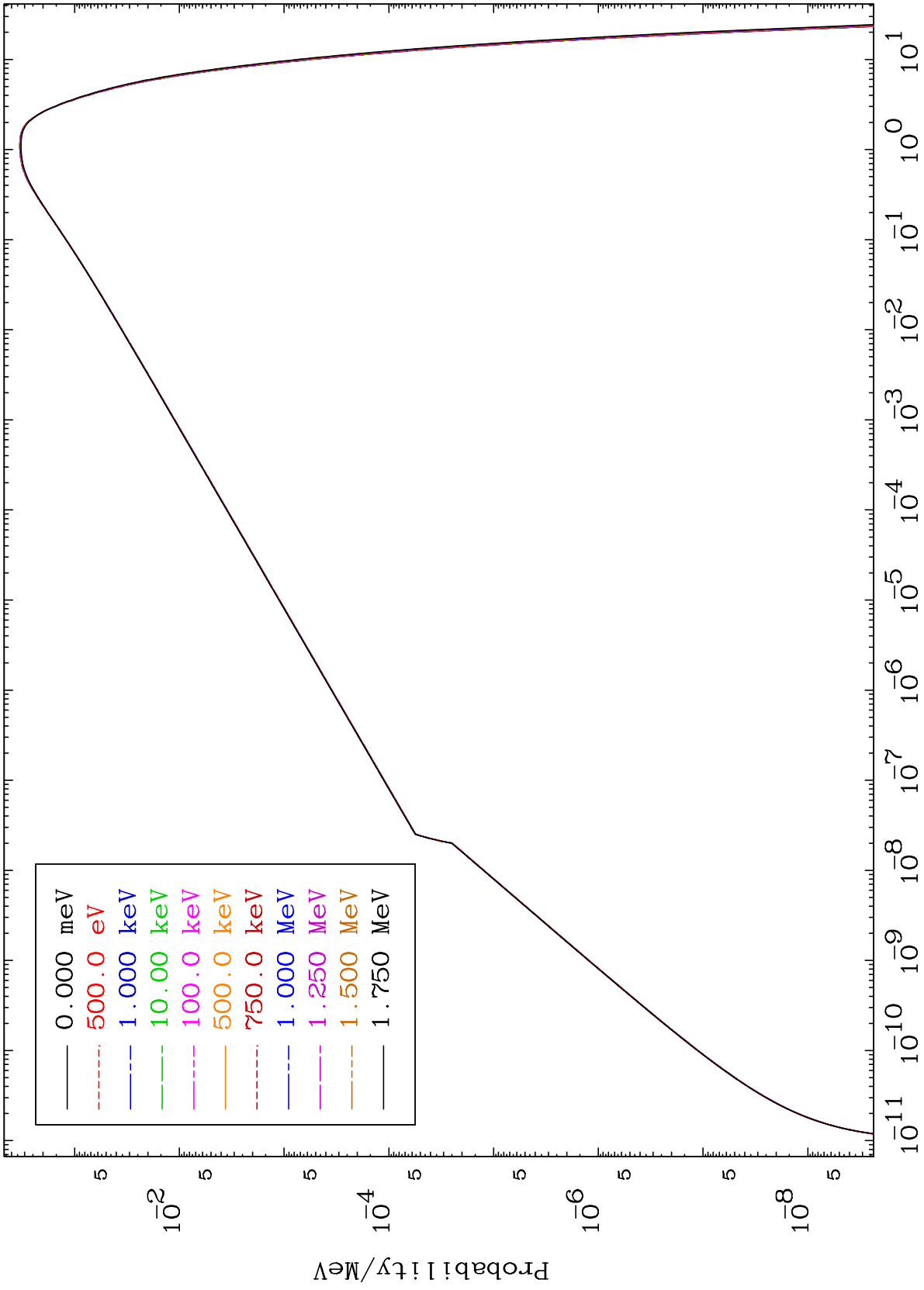
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93-Np-241

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# Fission Energy Distribution

93-Np-241



- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

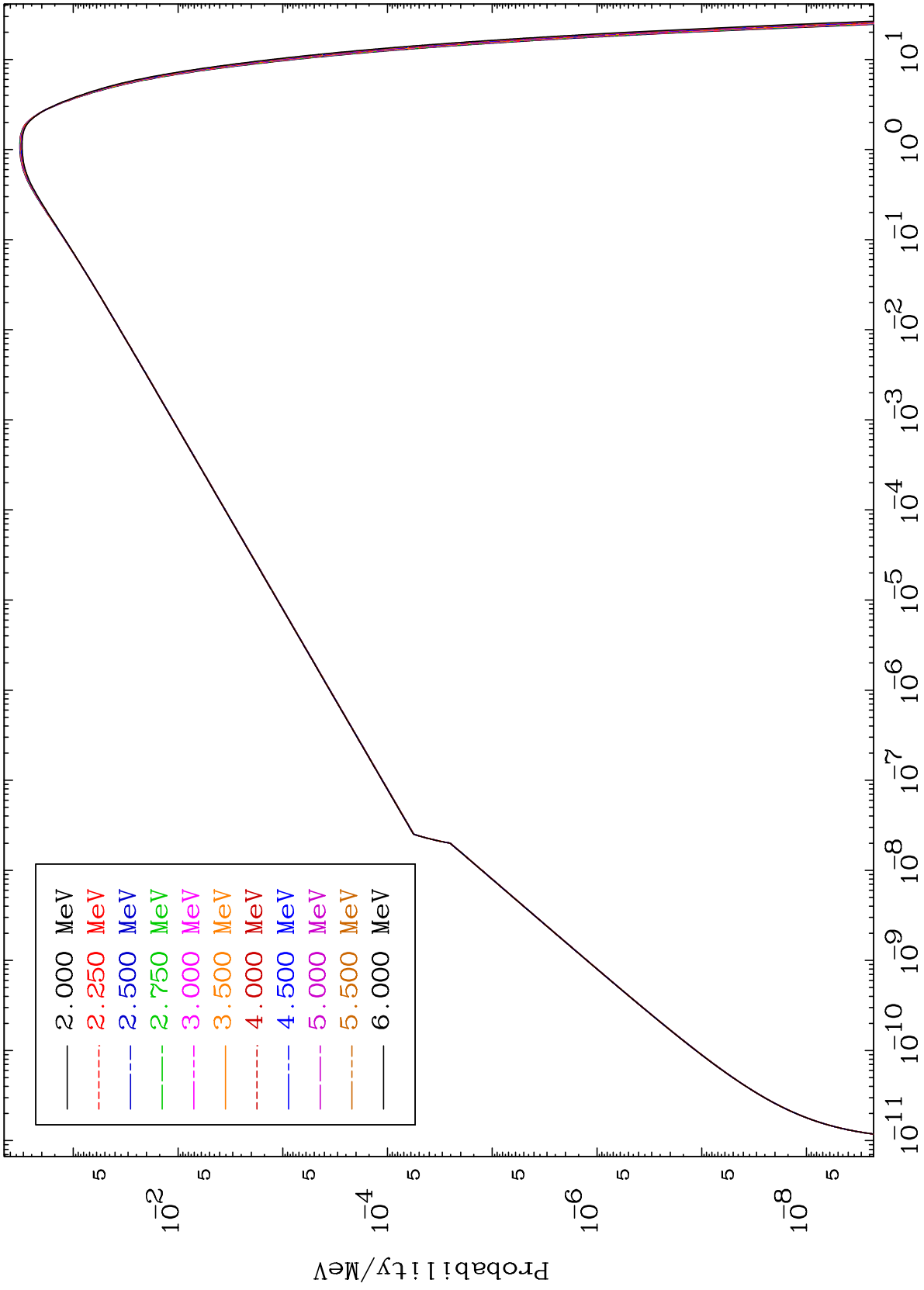
93-Np-241

Secondary Energy (MeV)

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# Fission Energy Distribution

93-Np-241

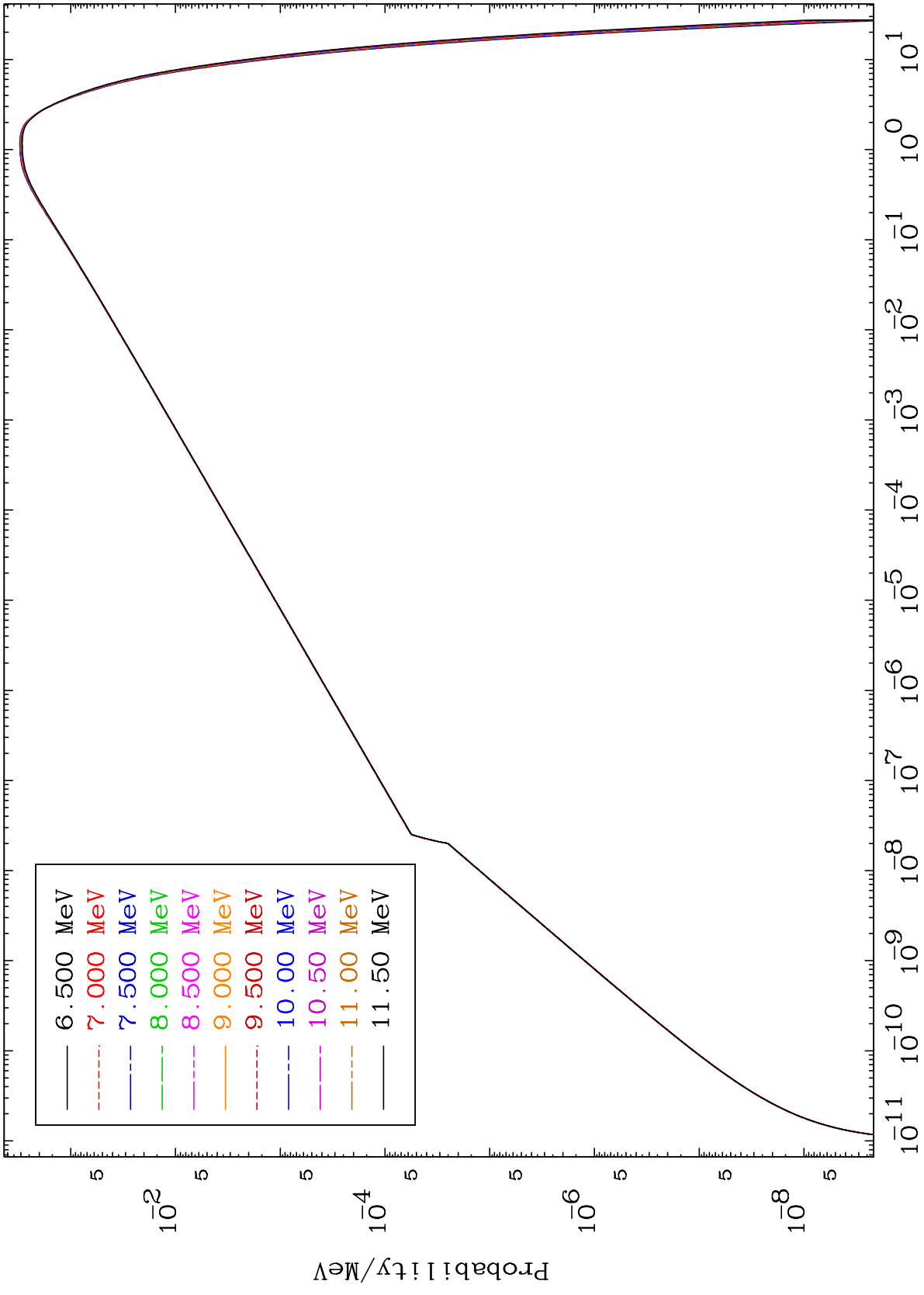


93-Np-241

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### Fission Energy Distribution

93-Np-241

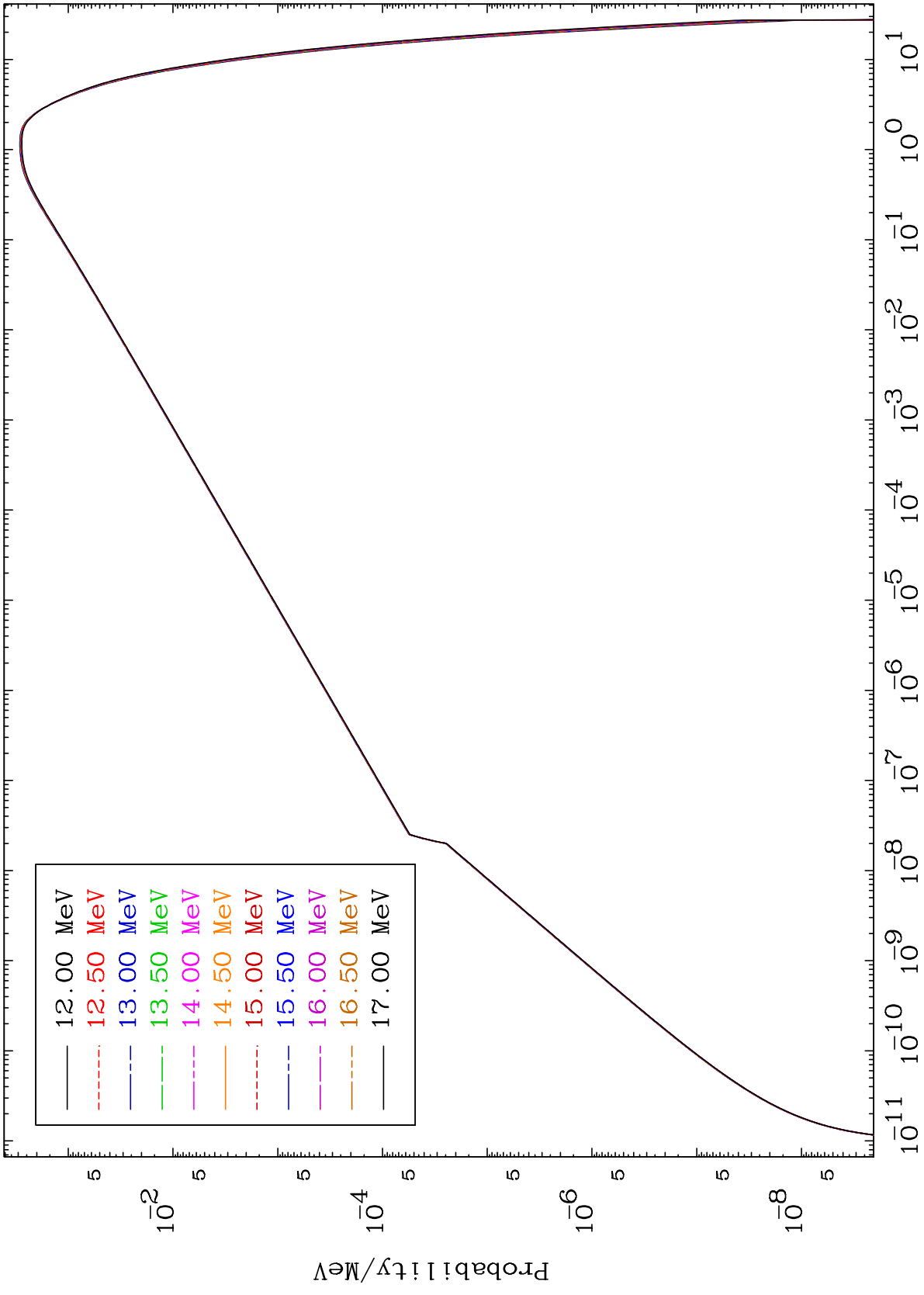


93-Np-241

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# Fission Energy Distribution

93-Np-241

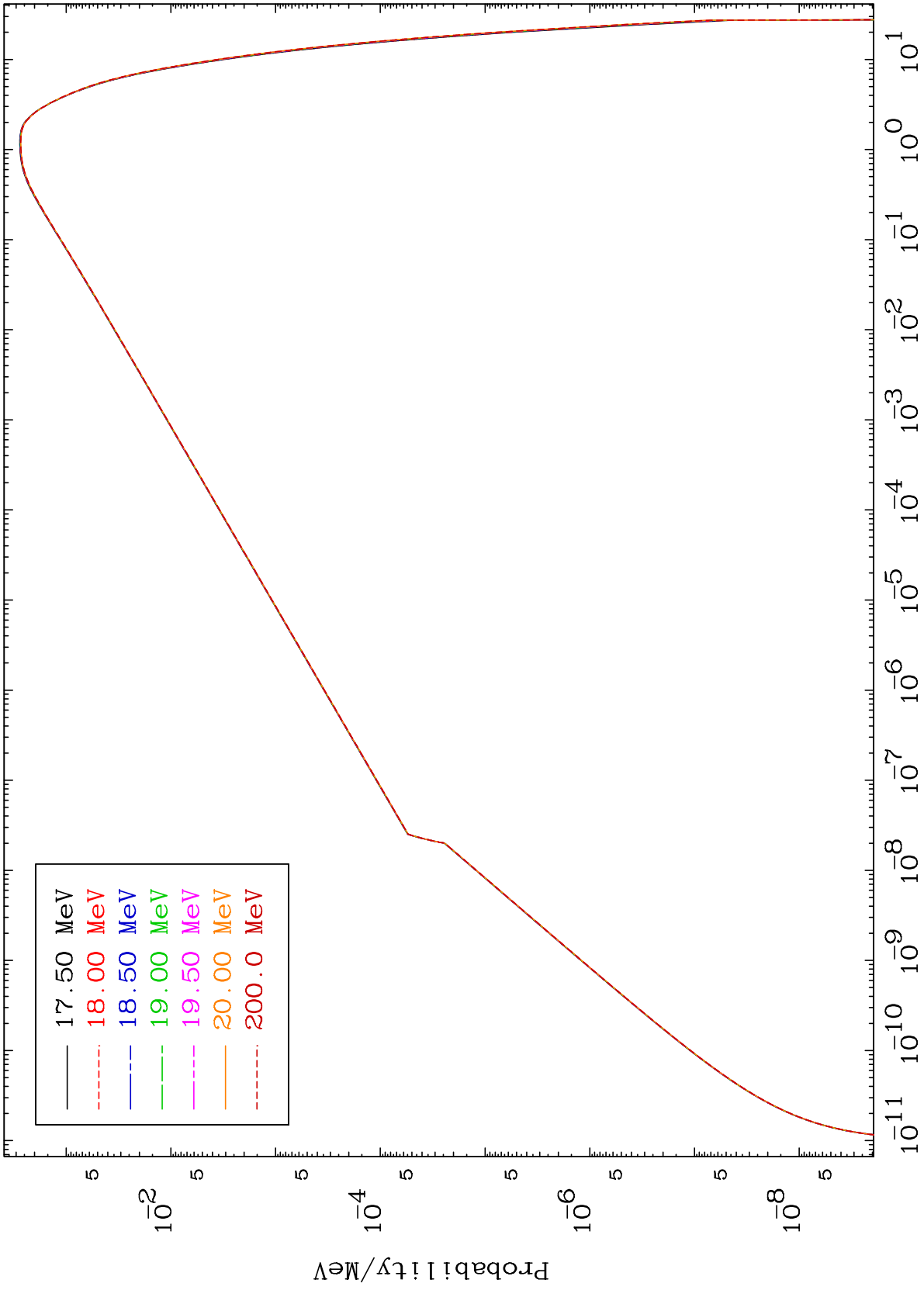




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# Fission Energy Distribution

93-Np-241



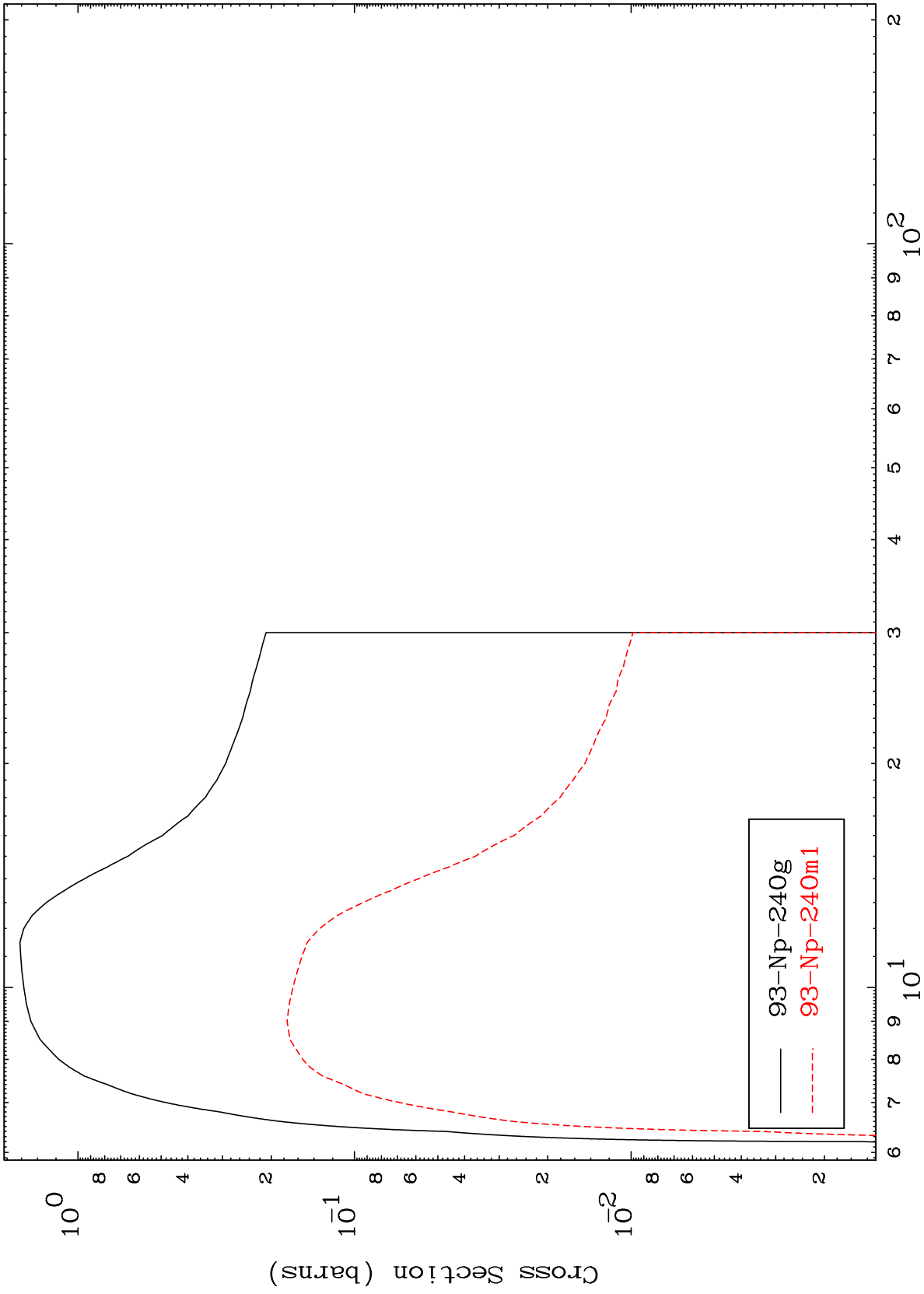
24

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93-Np-241

(n,2n)  
Radionuclide Production Cross Section



25

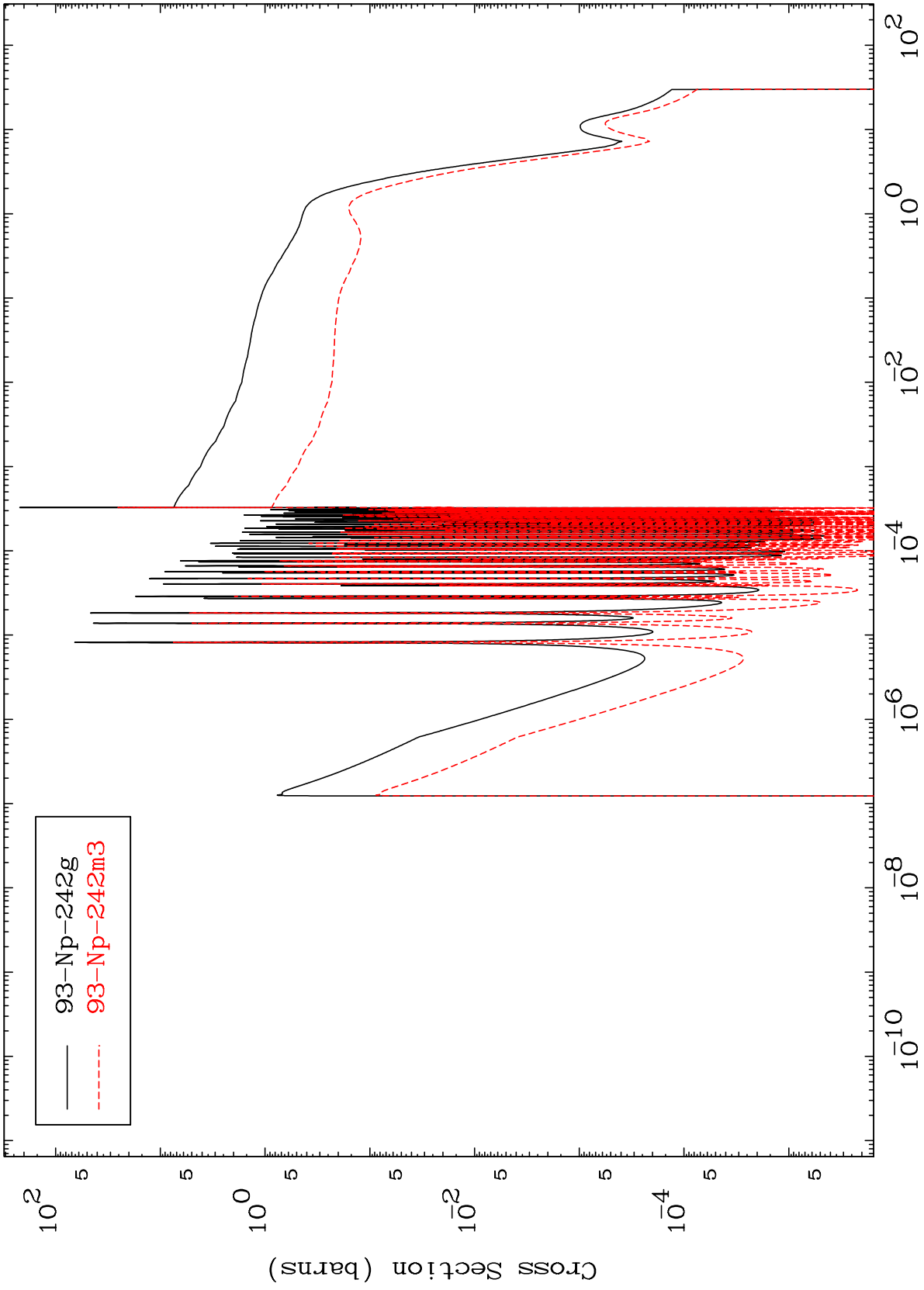
Incident Energy (MeV)

93-Np-241

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(n,  $\gamma$ )  
Radionuclide Production Cross Section



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