

Program EVALPLOT  
(Version 2018-1)

by

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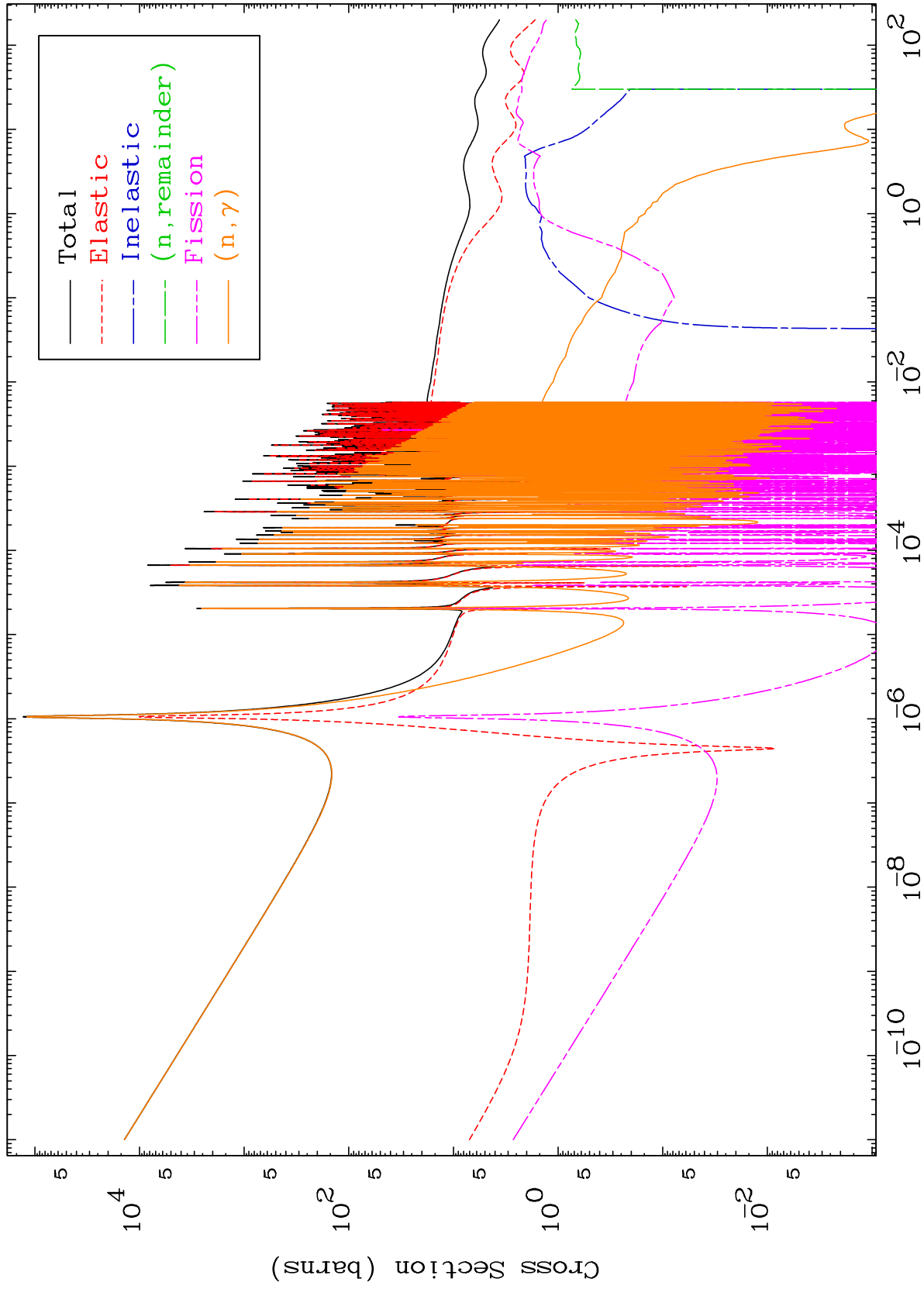
Press Mouse Button to Start

MAT 9440

Major

293 Kelvin Cross Sections

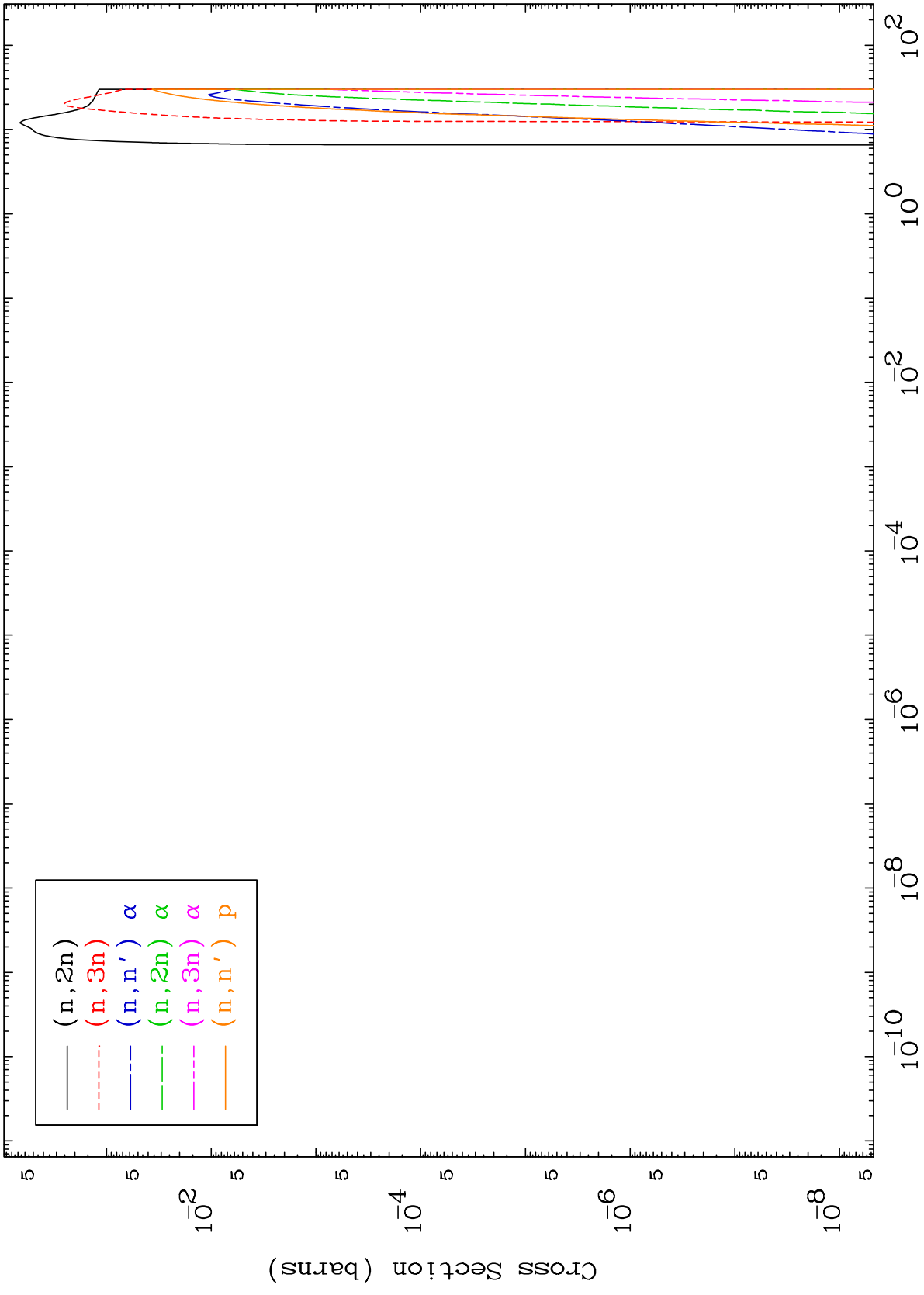
94-Pu-240



MAT 9440

Neutron Production  
293 Kelvin Cross Sections

94-Pu-240

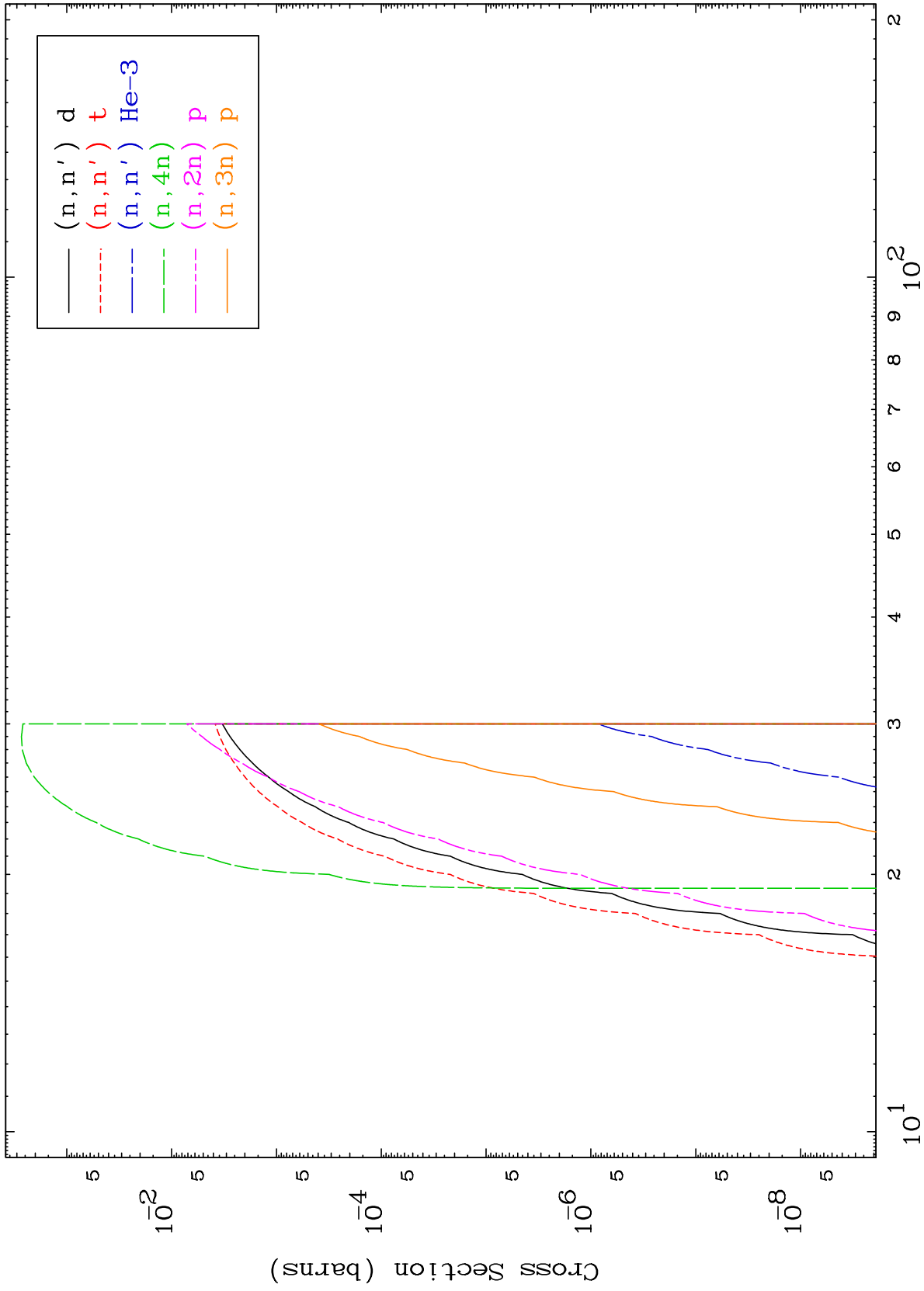


94-Pu-240

MAT 9440

### Neutron Production 293 Kelvin Cross Sections

94-Pu-240



3

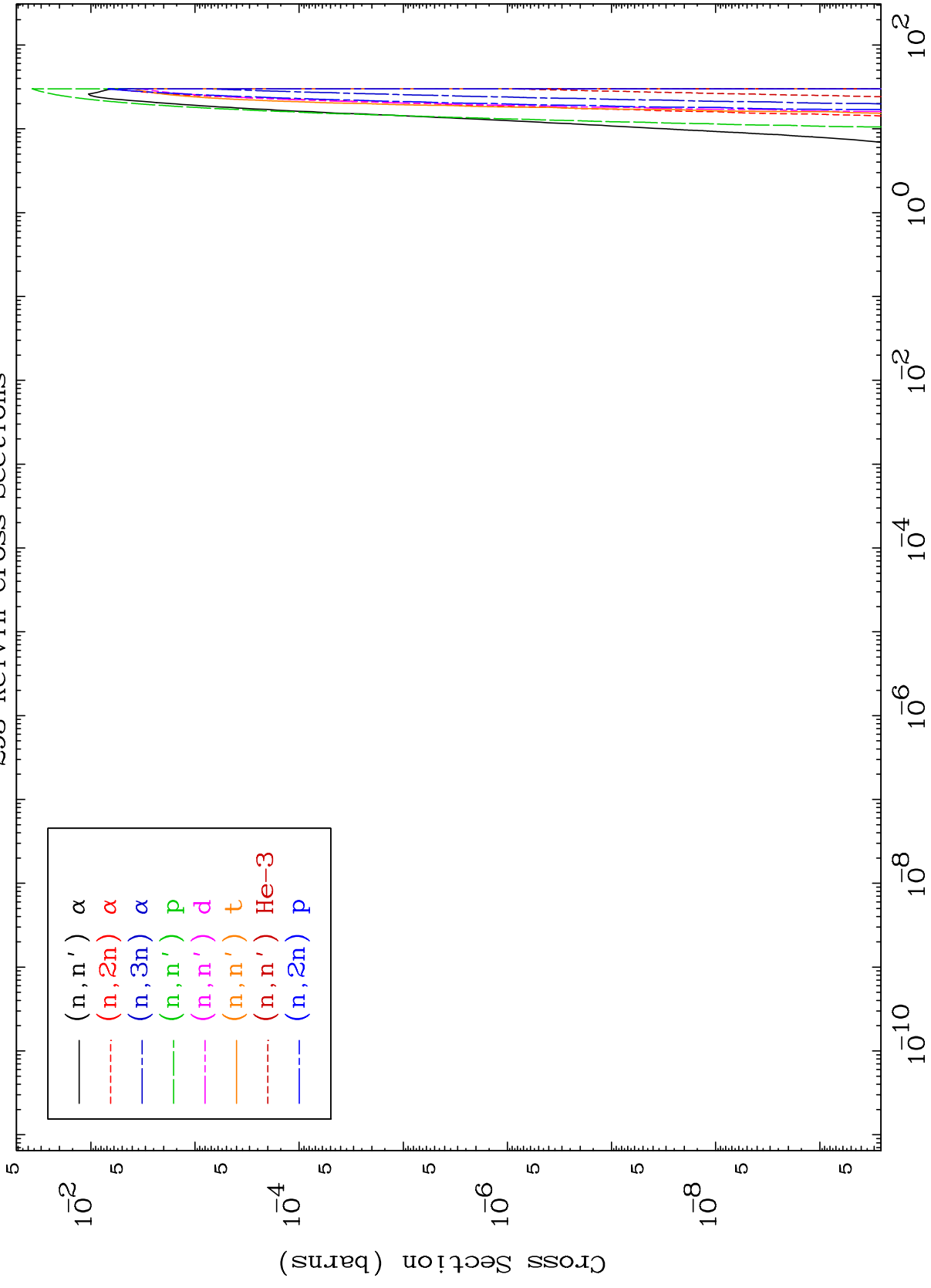
Incident Energy (MeV)

94-Pu-240

MAT 9440

293 Kelvin Cross Sections

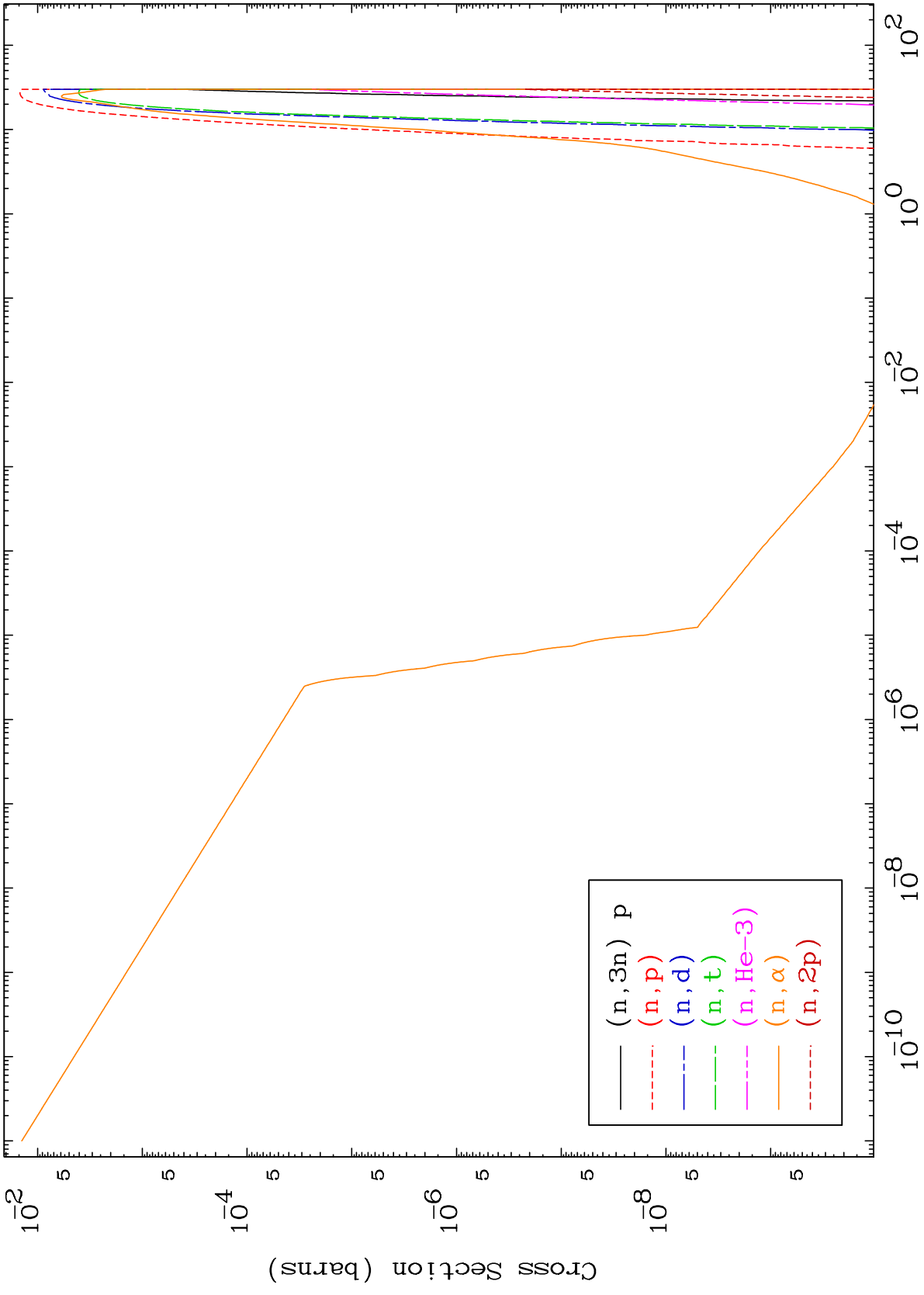
94-Pu-240



MAT 9440

293 Kelvin Cross Sections

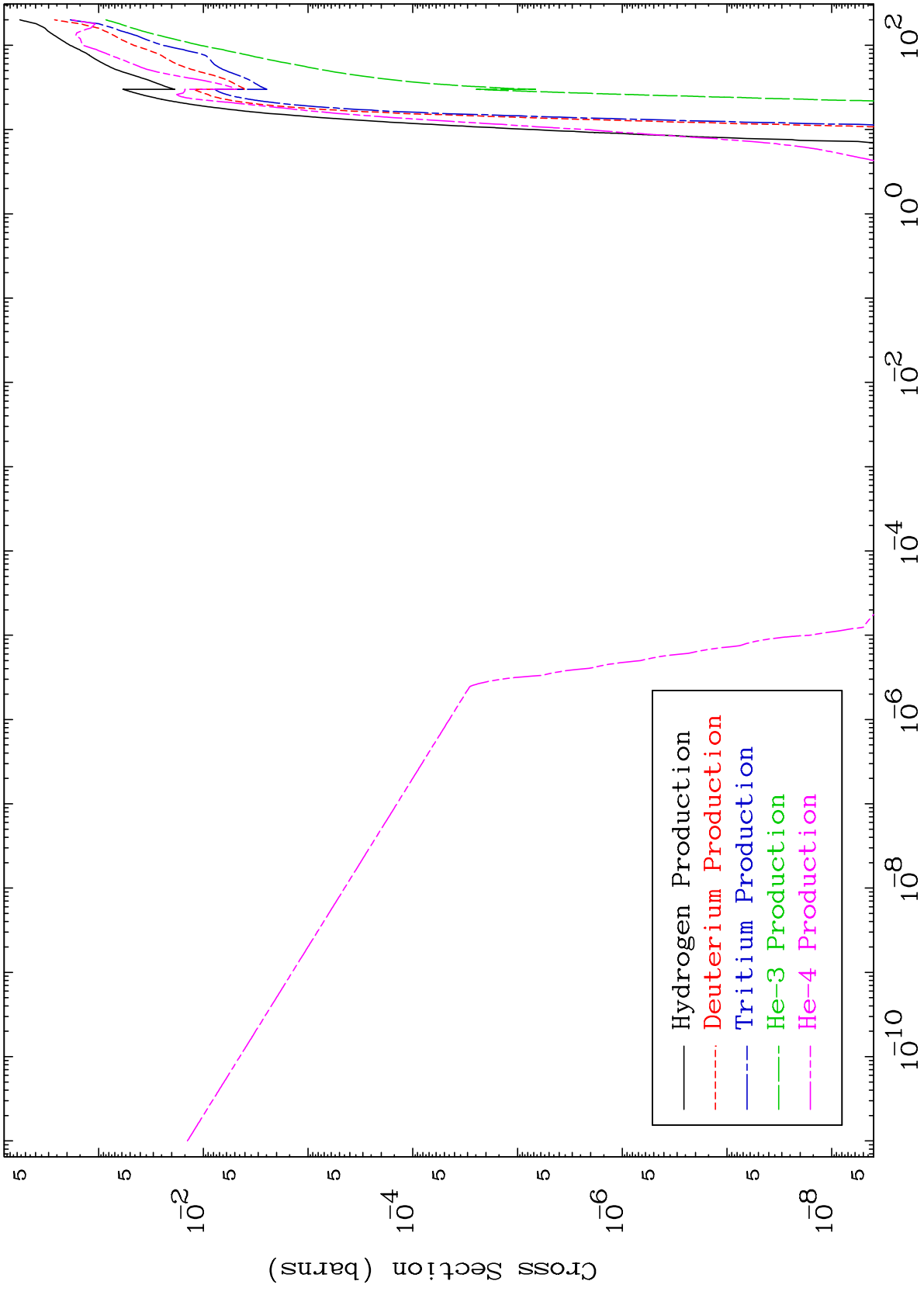
94-Pu-240



MAT 9440

Particle Production  
293 Kelvin Cross Sections

94-Pu-240

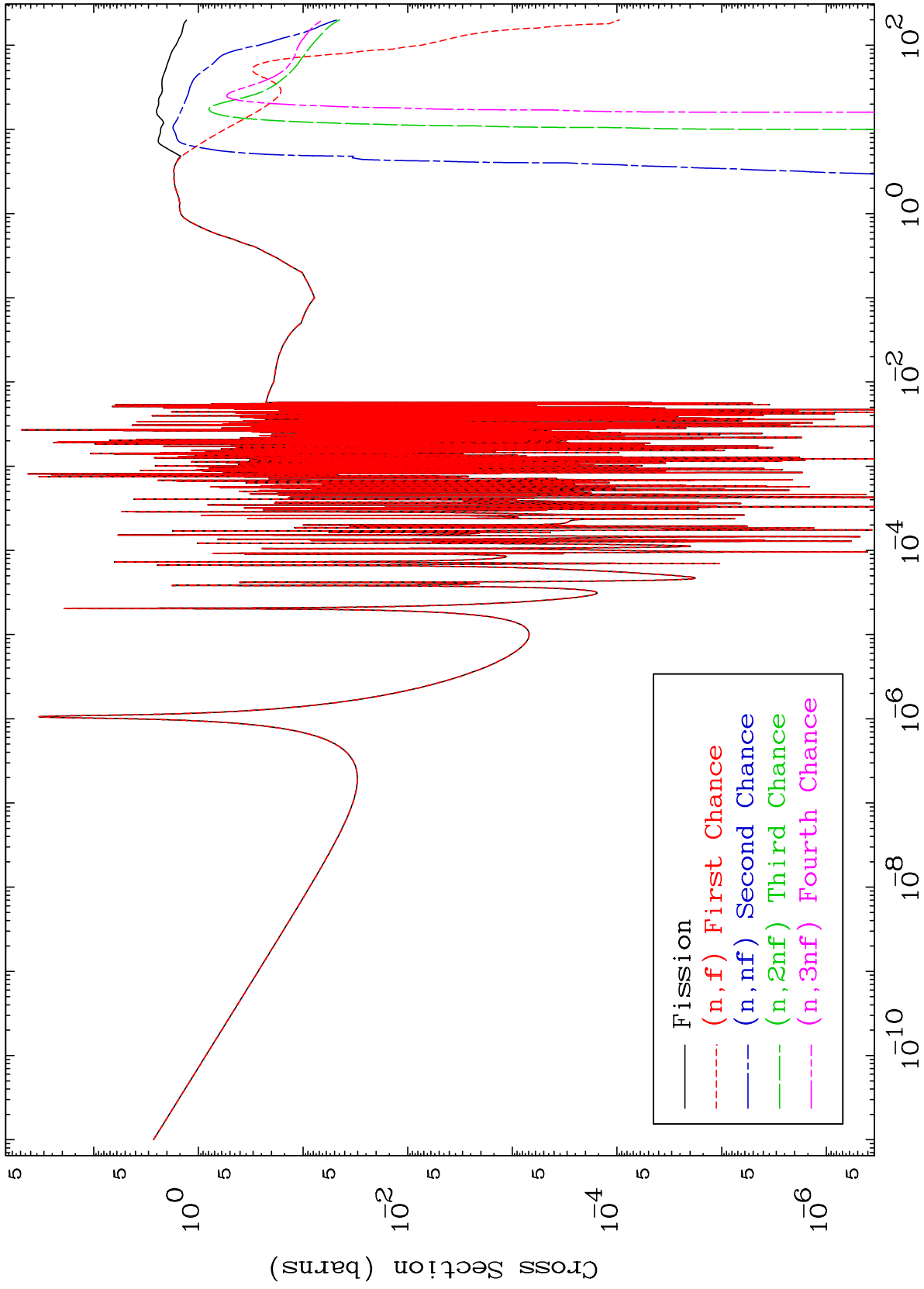


94-Pu-240

MAT 9440

Fission  
293 Kelvin Cross Sections

94-Pu-240



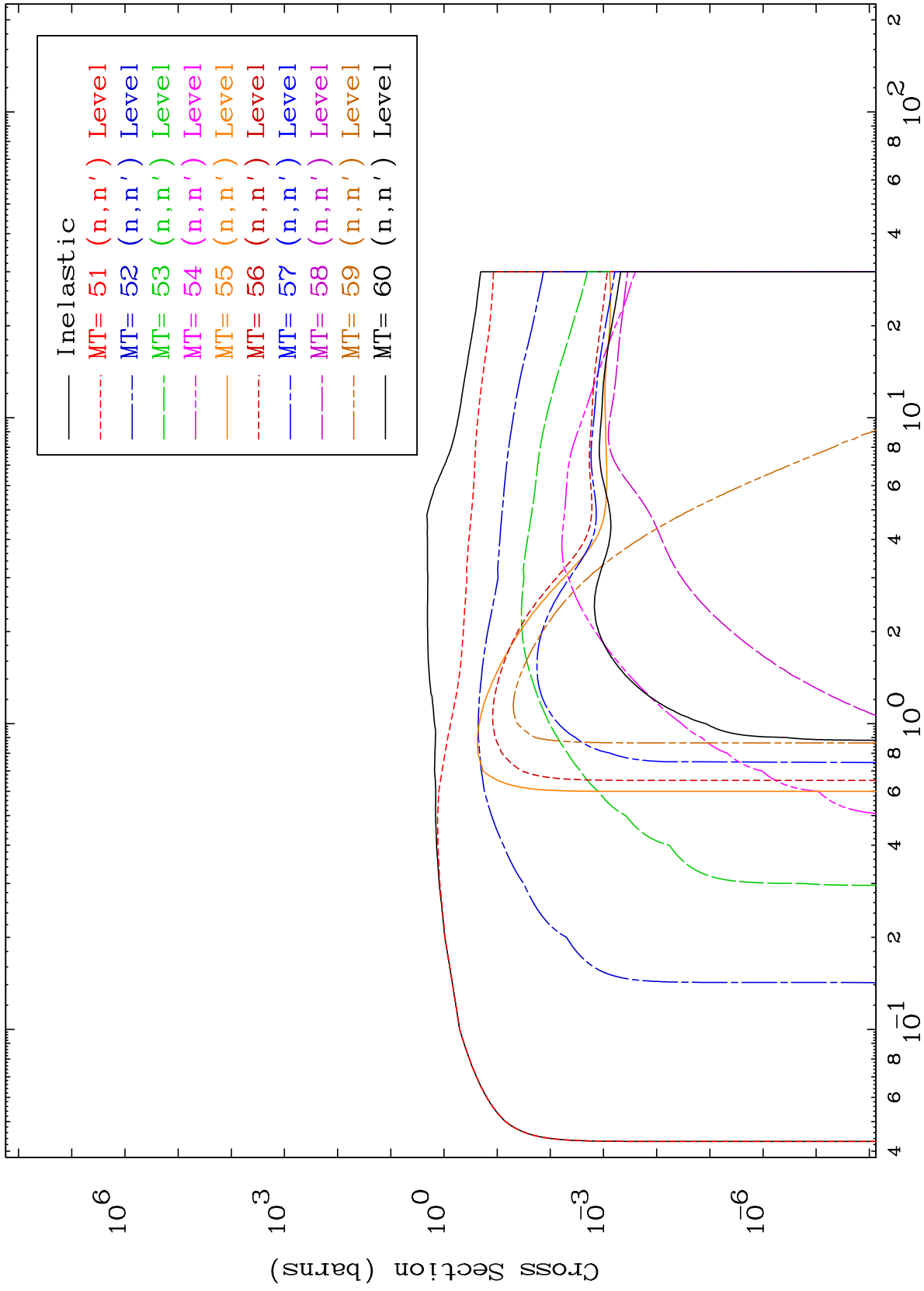


MAT 9440

(n,n') Level

94-Pu-240

293 Kelvin Cross Sections

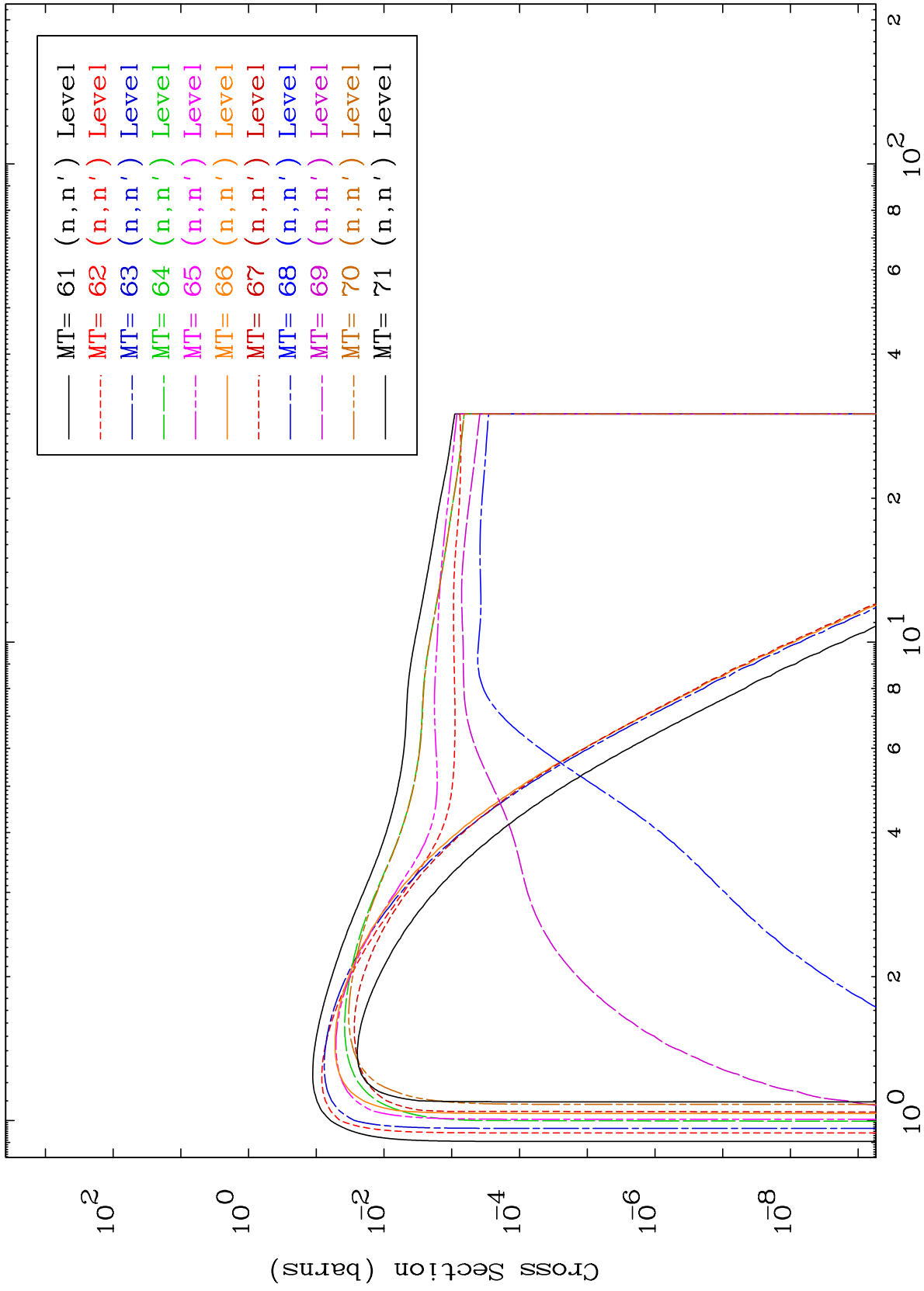


MAT 9440

(n,n') Level

94-Pu-240

293 Kelvin Cross Sections



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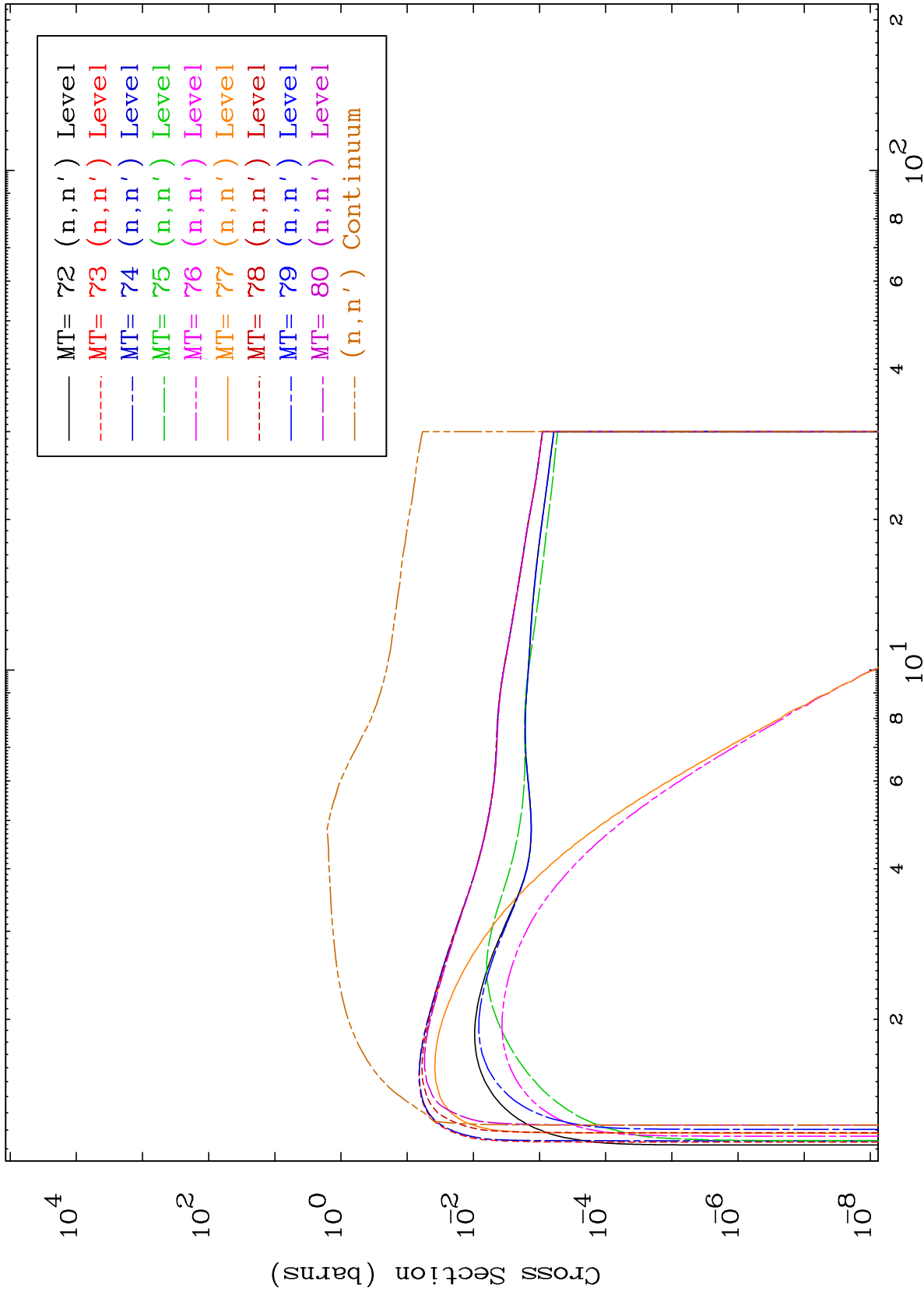
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,n') Level  
293 Kelvin Cross Sections

94-Pu-240



10

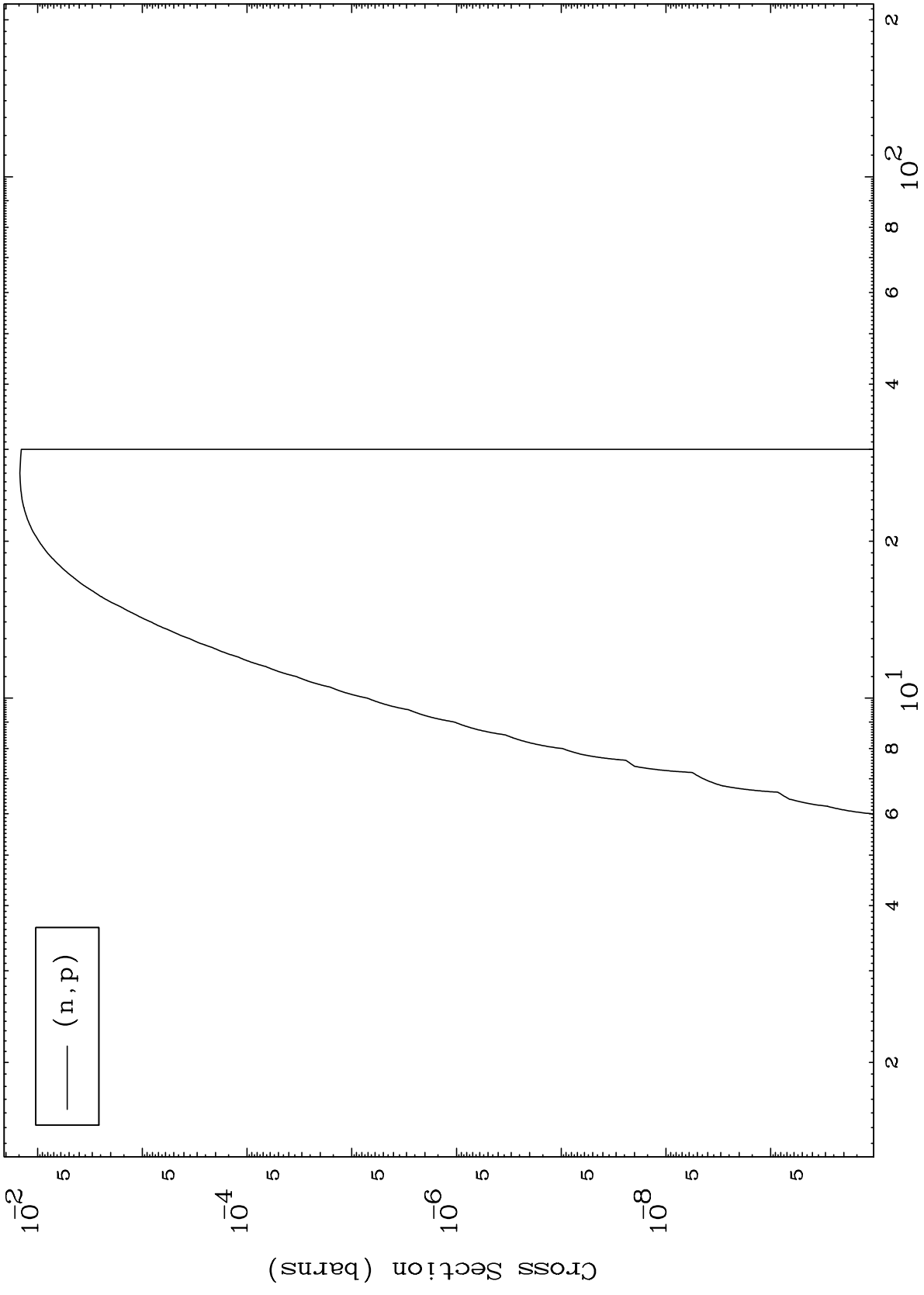
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,p) Levels  
293 Kelvin Cross Sections

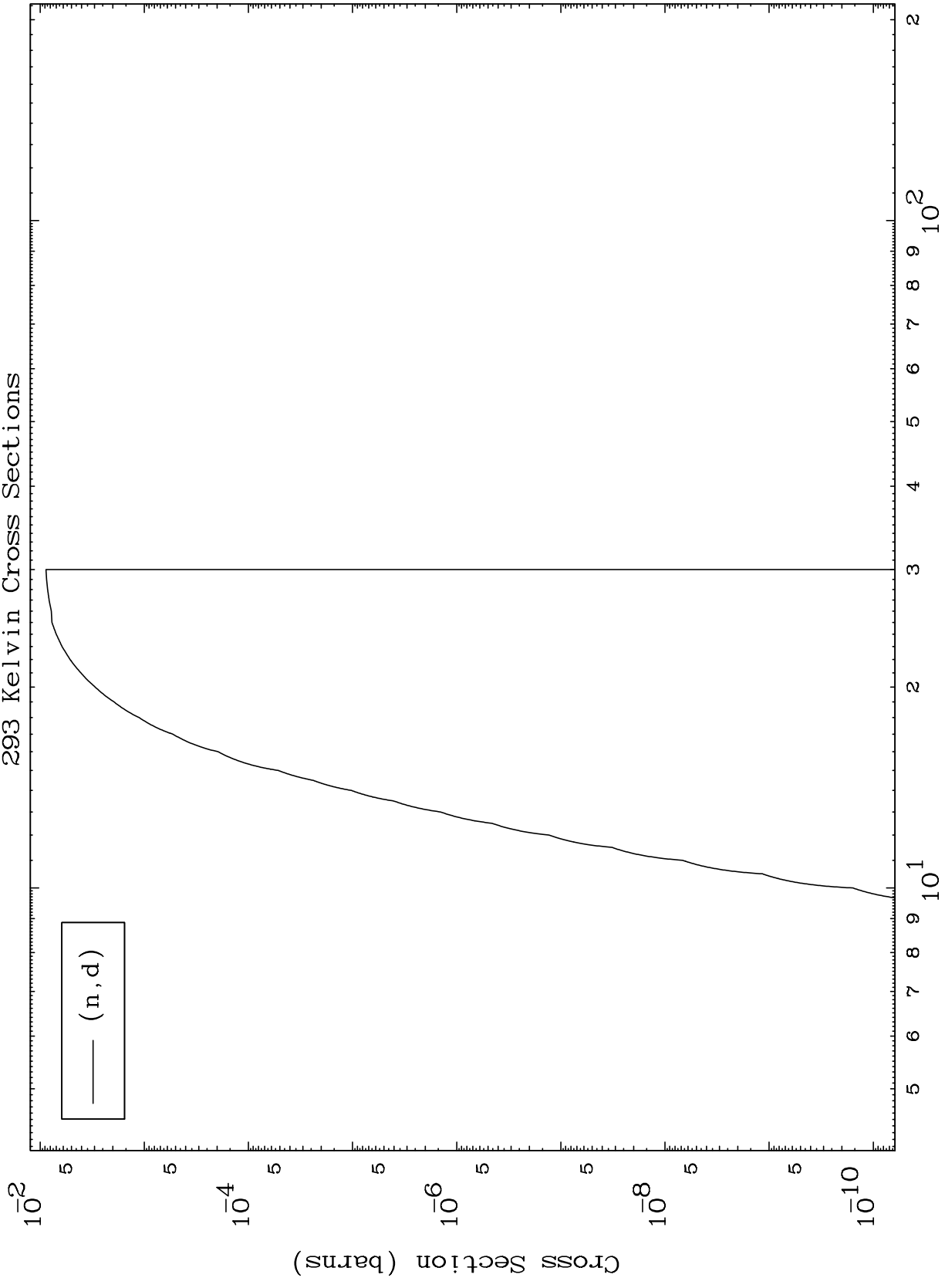
94-Pu-240



MAT 9440

(n,d) Levels  
293 Kelvin Cross Sections

94-Pu-240



12

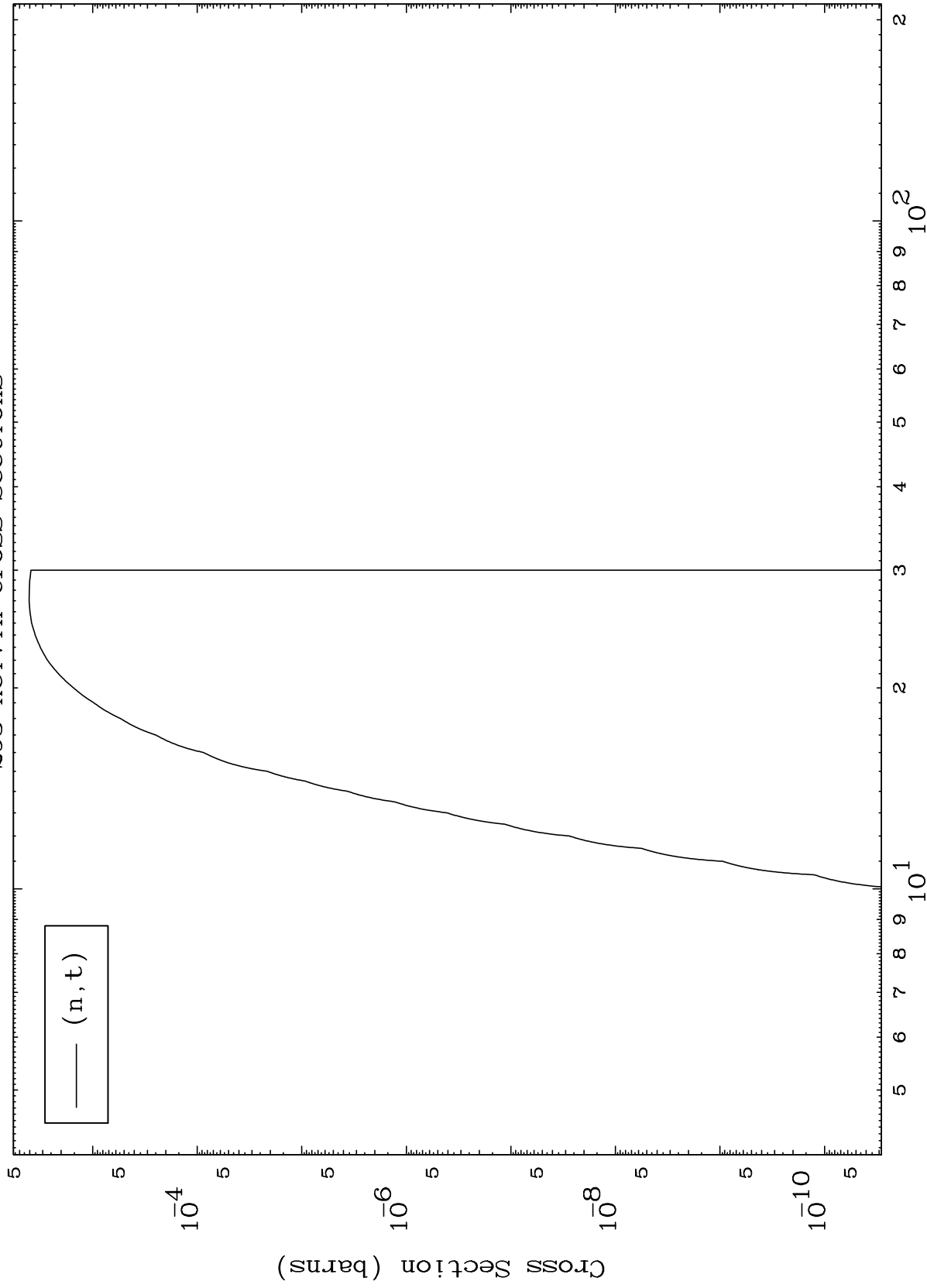
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,t) Levels  
293 Kelvin Cross Sections

94-Pu-240



13

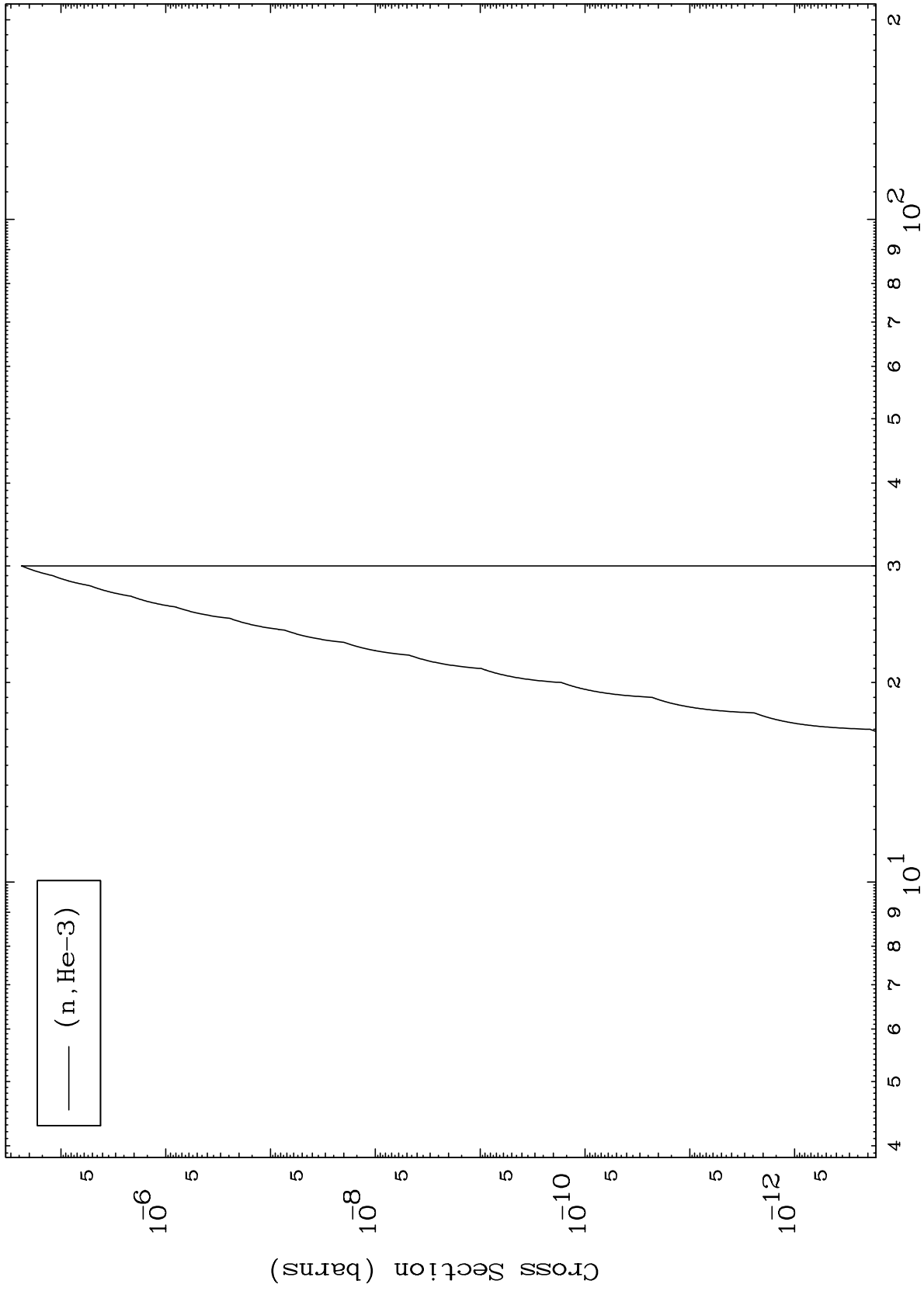
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,He3) Levels  
293 Kelvin Cross Sections

94-Pu-240



14

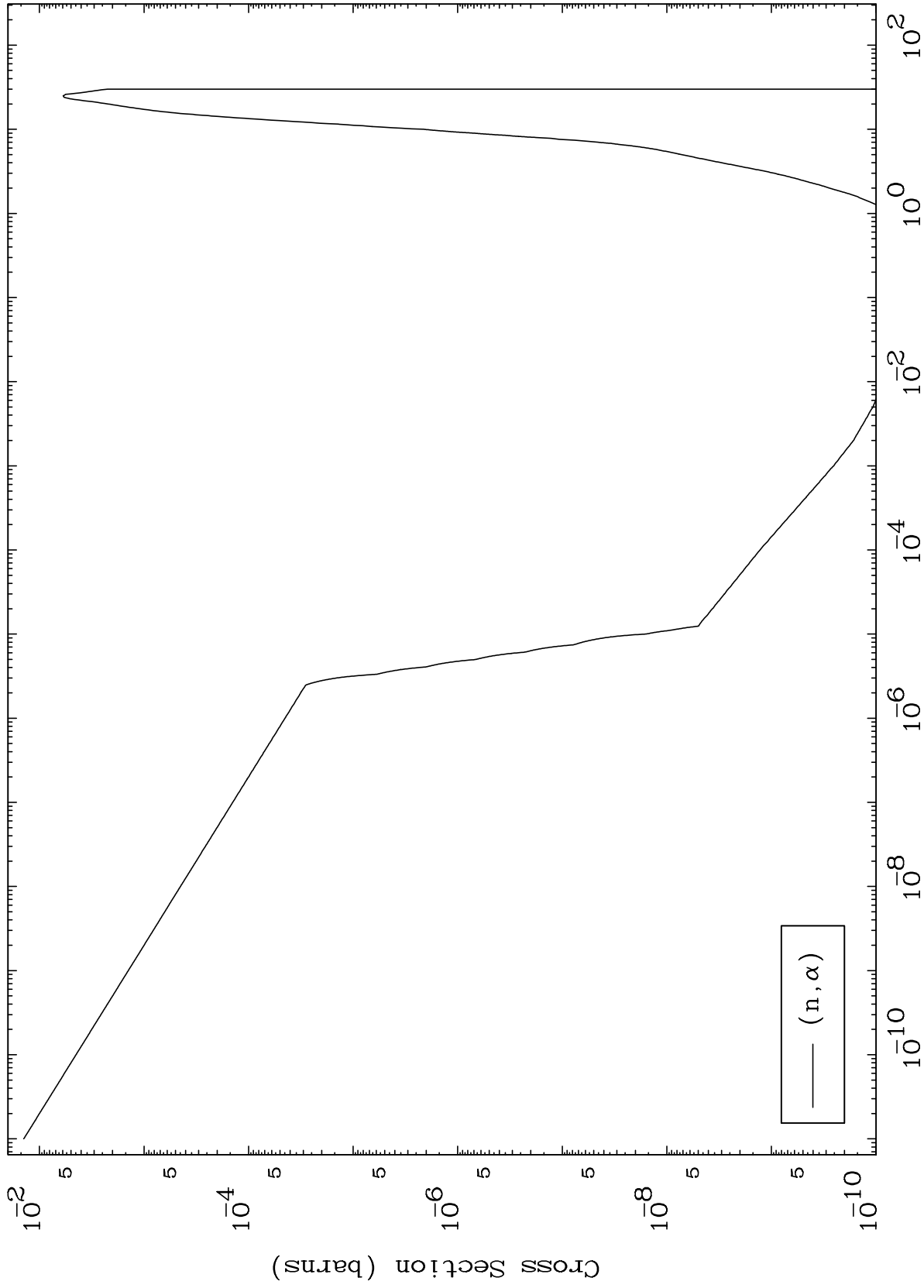
Incident Energy (MeV)

94-Pu-240

MAT 9440

(n,α) Levels  
293 Kelvin Cross Sections

94-Pu-240

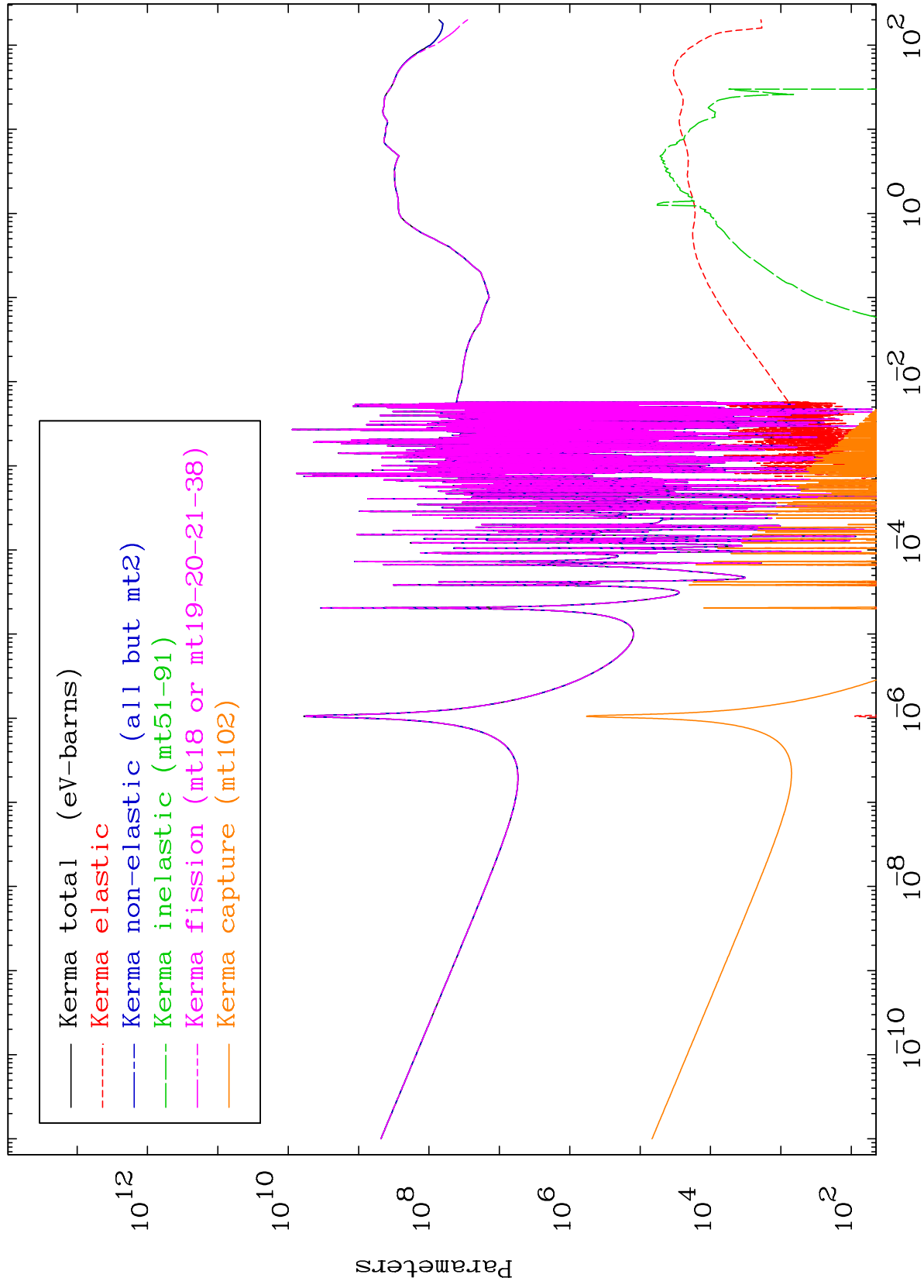


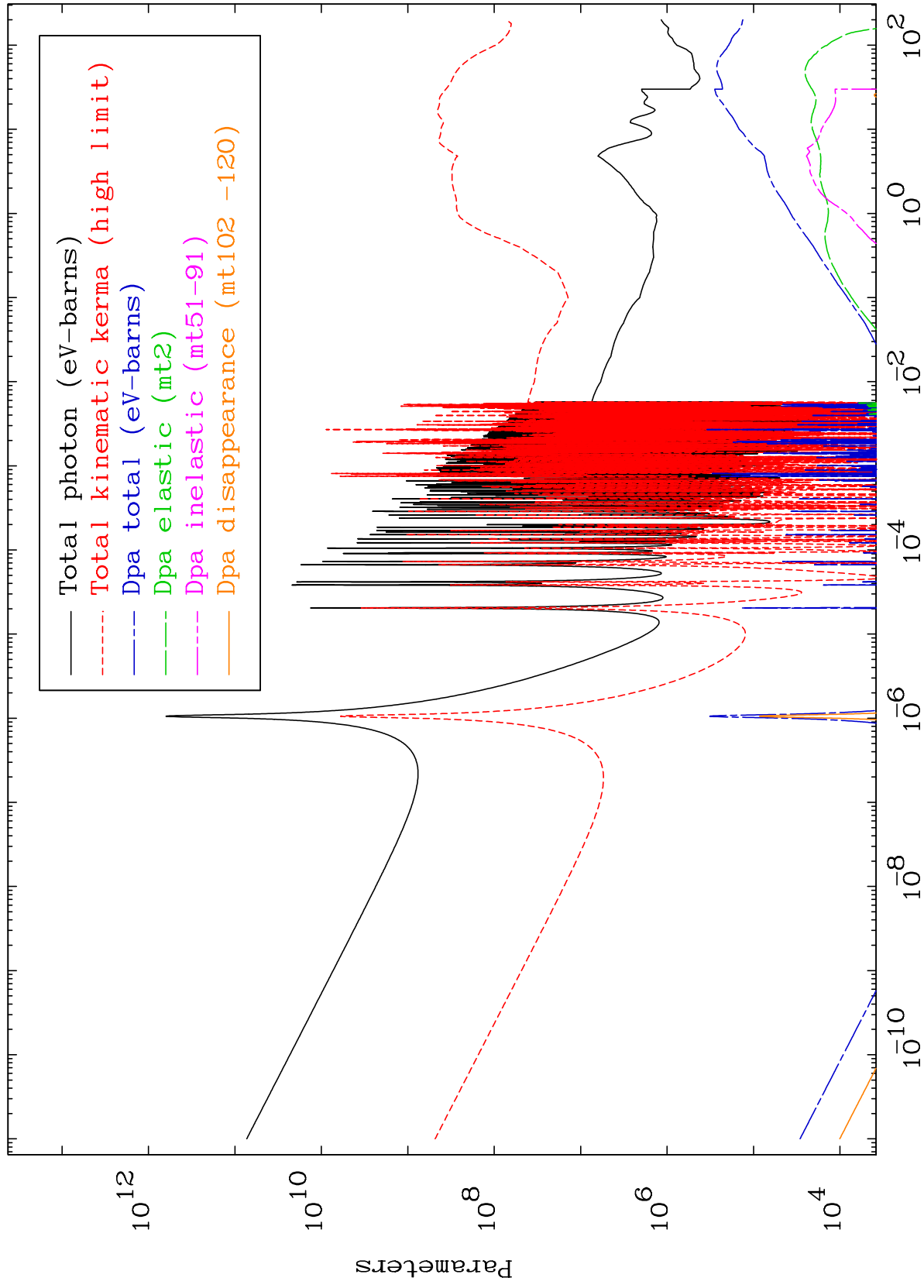
15

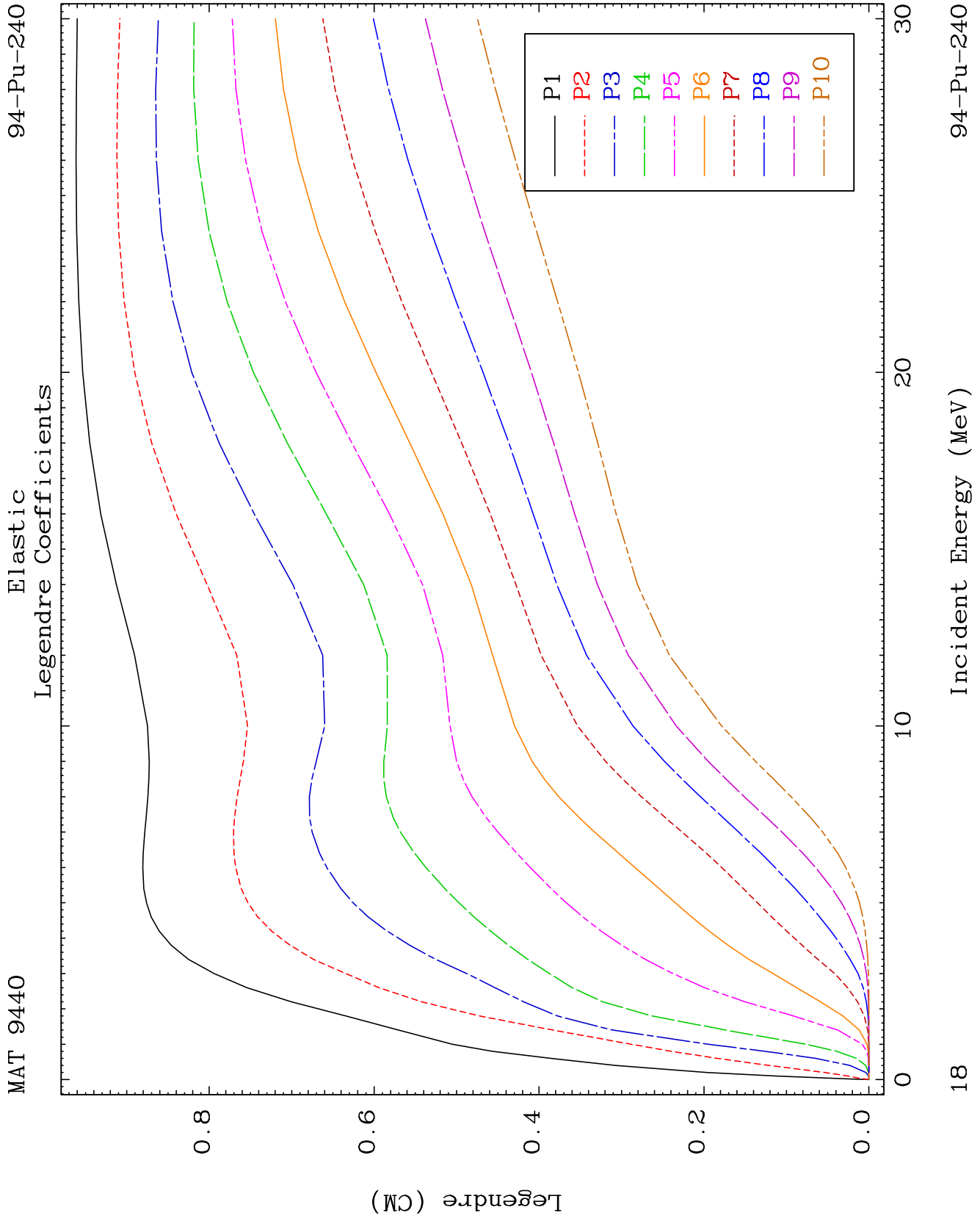
Incident Energy (MeV)

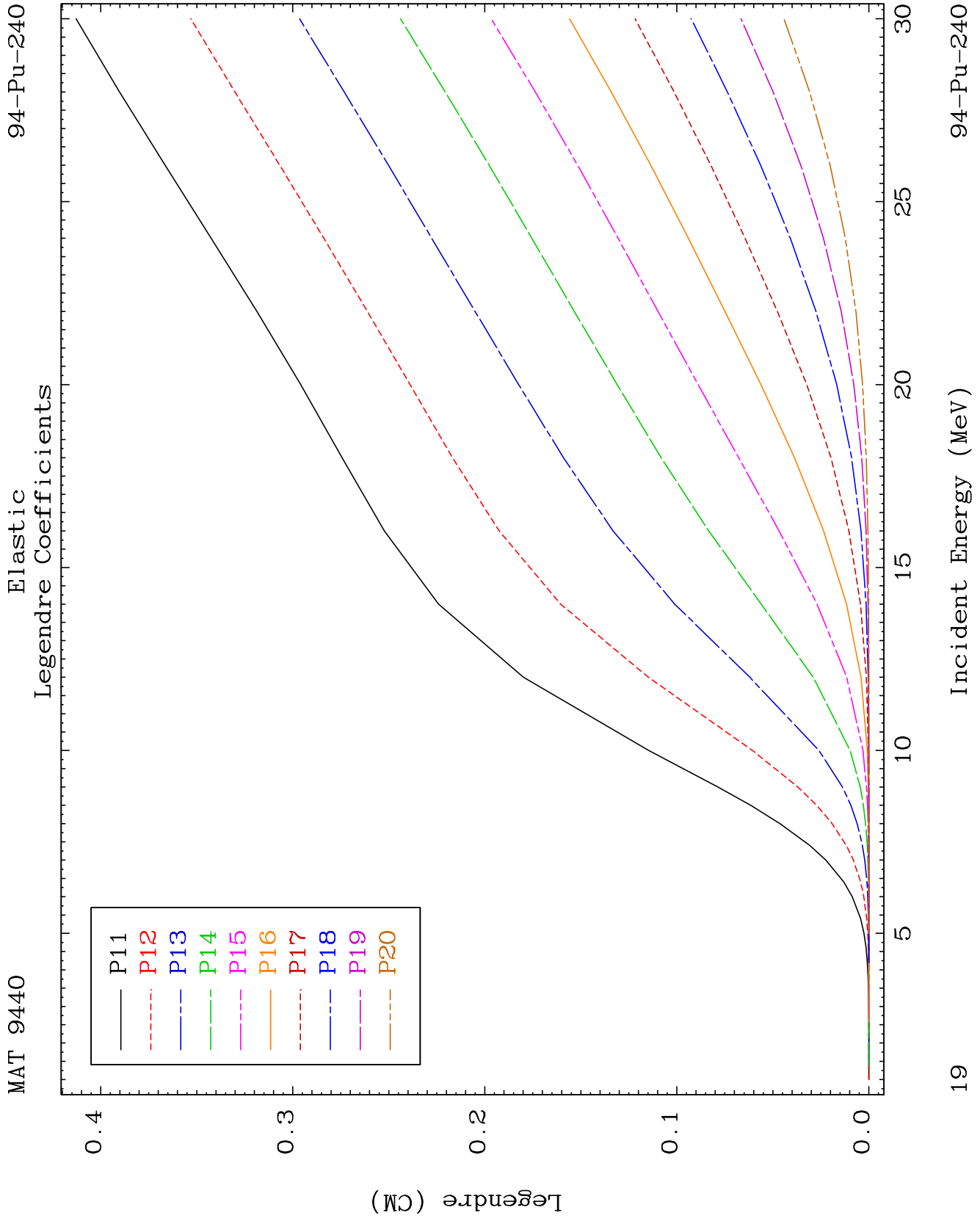
94-Pu-240







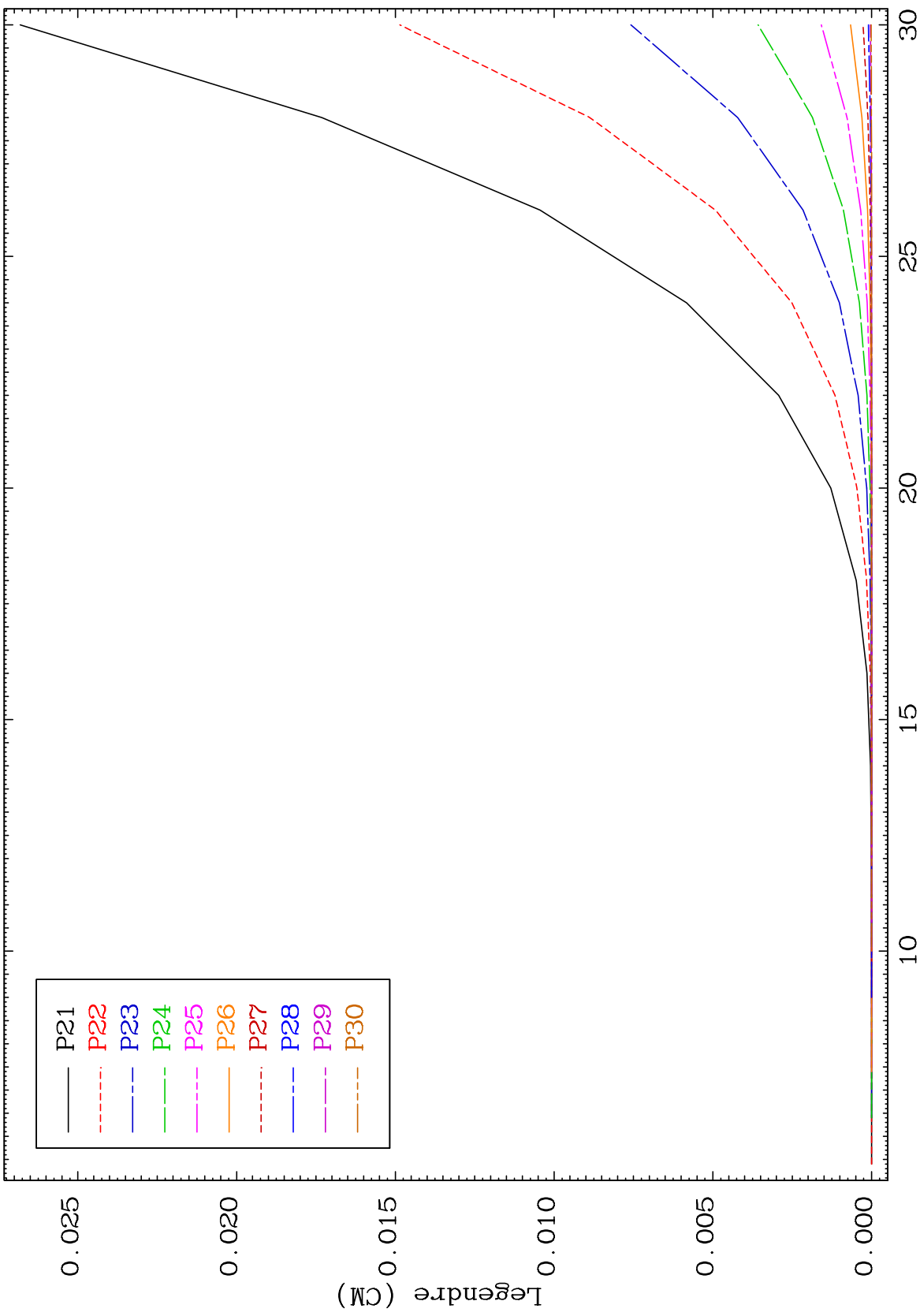




MAT 9440

Elastic Legendre Coefficients

94-Pu-240



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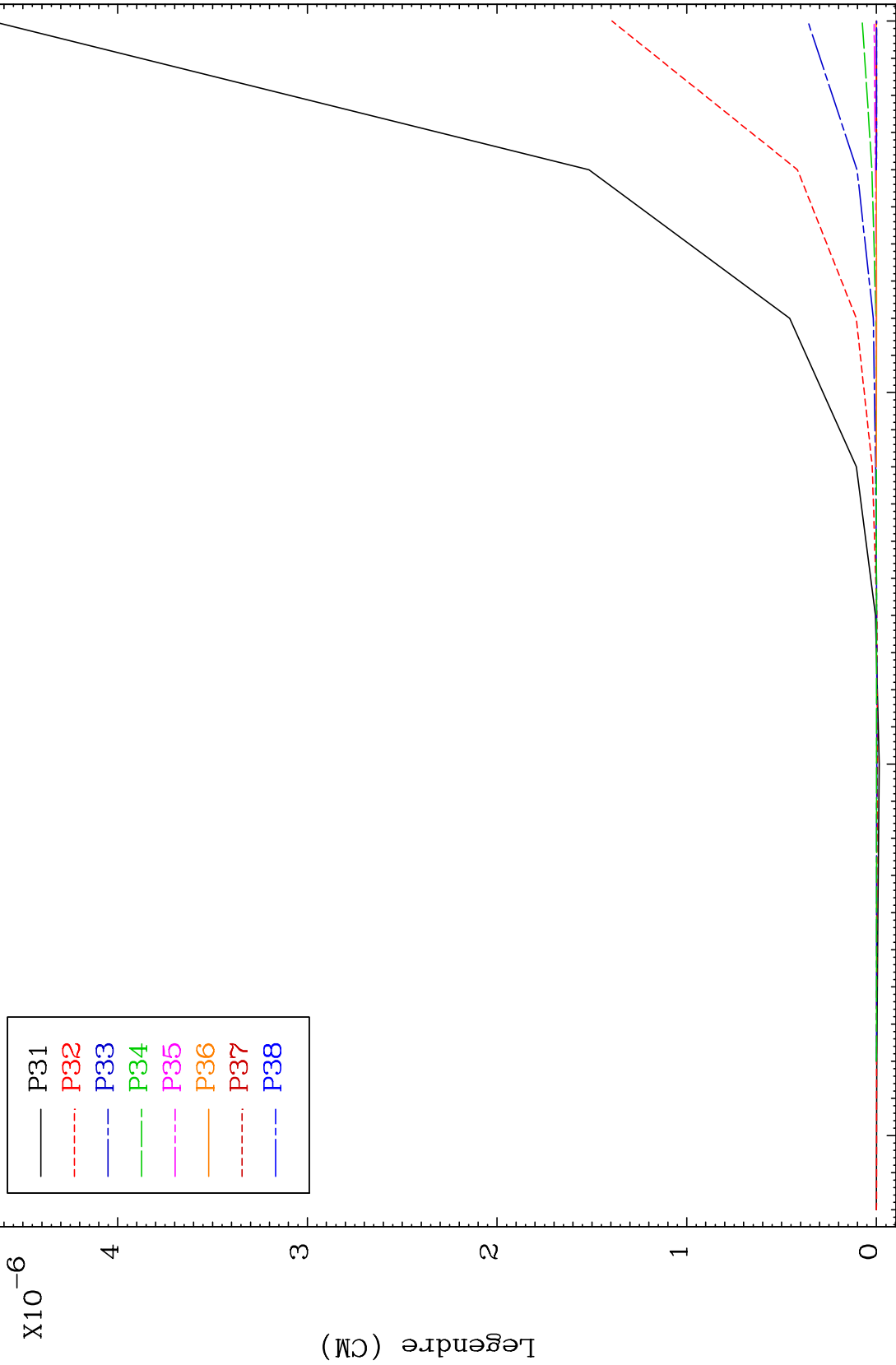
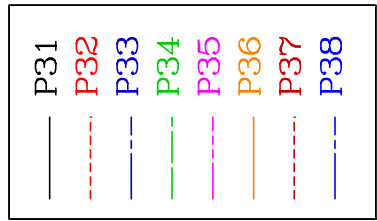
Incident Energy (MeV)

94-Pu-240

MAT 9440

Elastic Legendre Coefficients

94-Pu-240



21

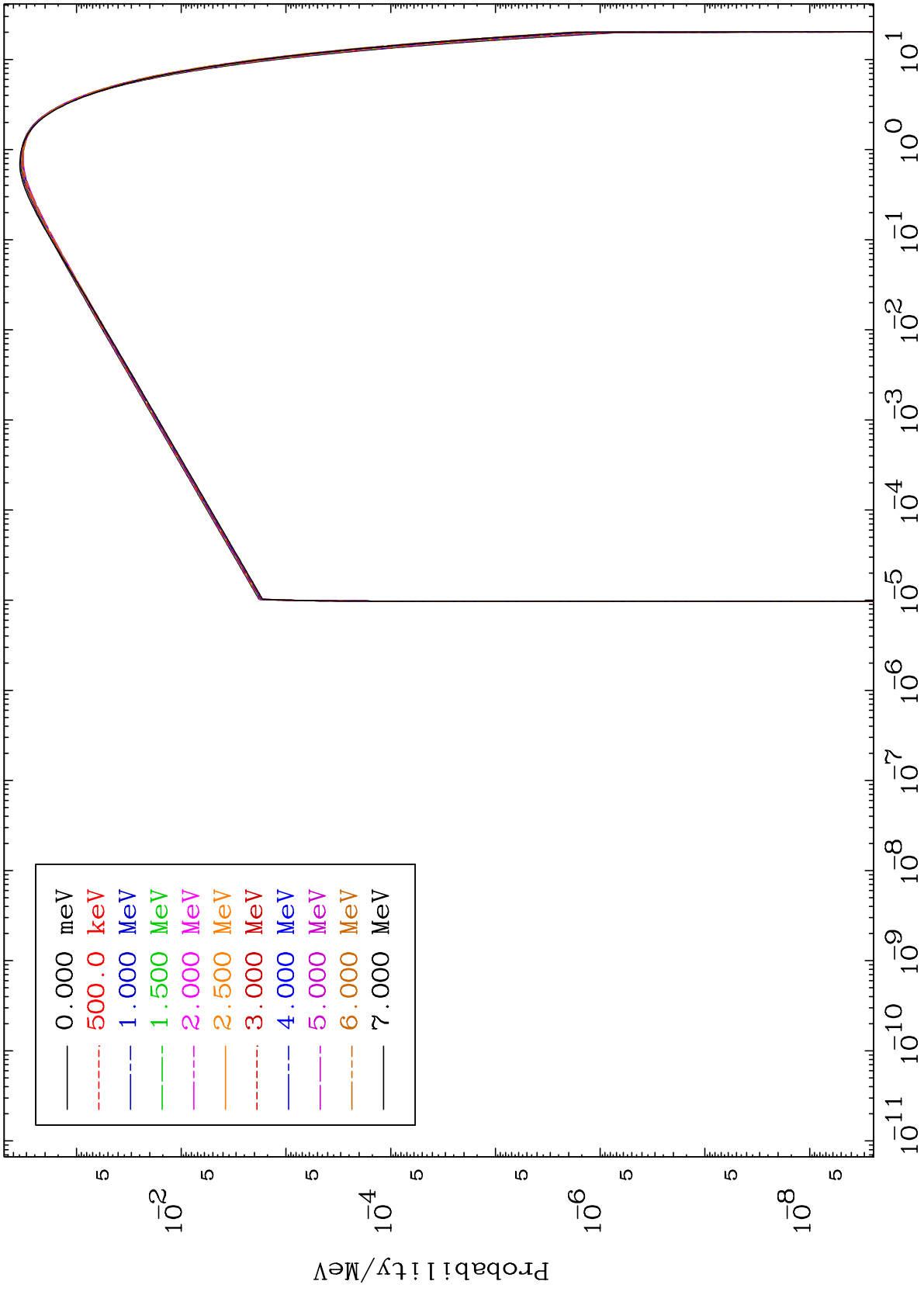
Incident Energy (MeV)

94-Pu-240

MAT 9440

### Fission Energy Distribution

94-Pu-240

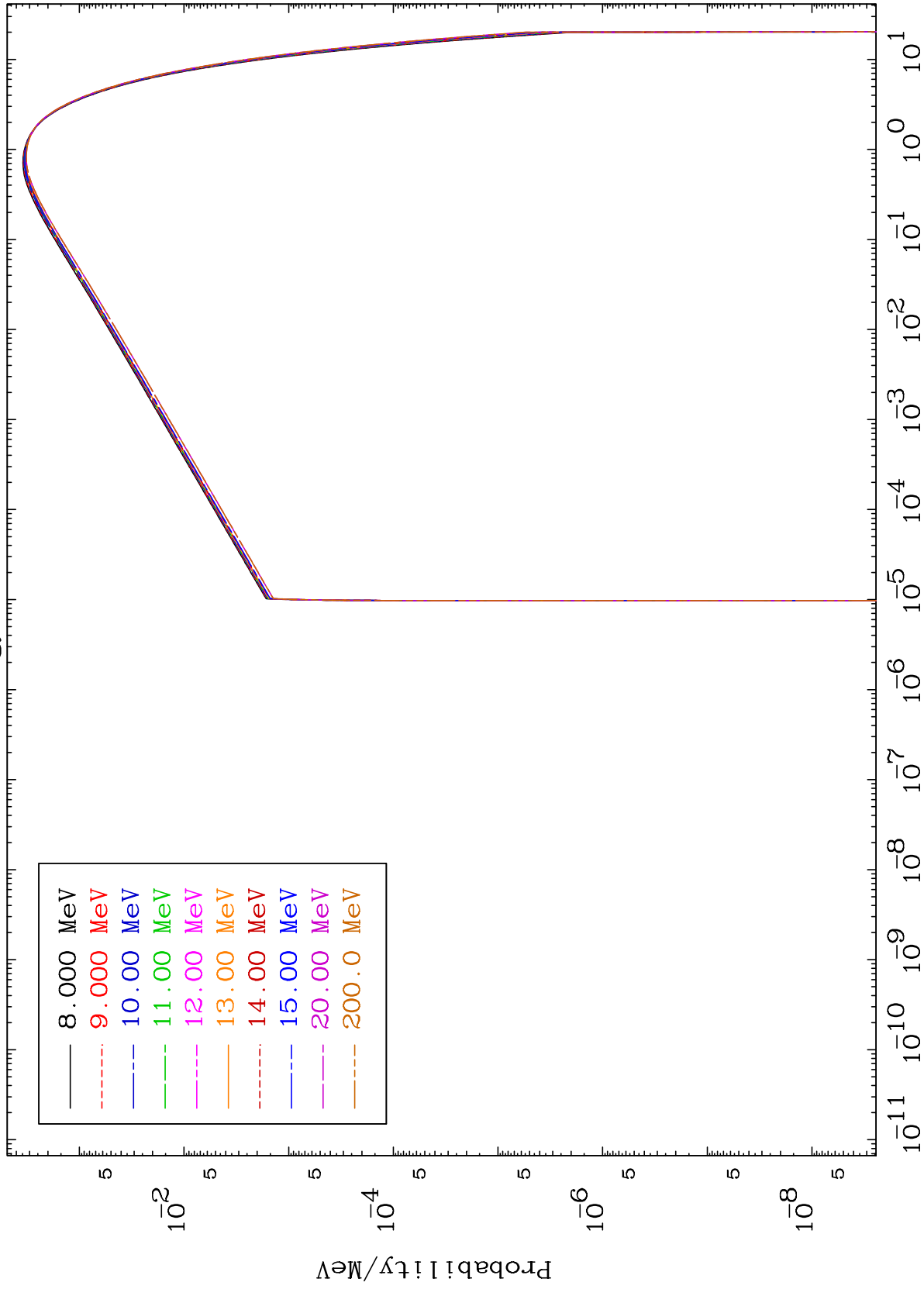


94-Pu-240

MAT 9440

### Fission Energy Distribution

94-Pu-240



94-Pu-240

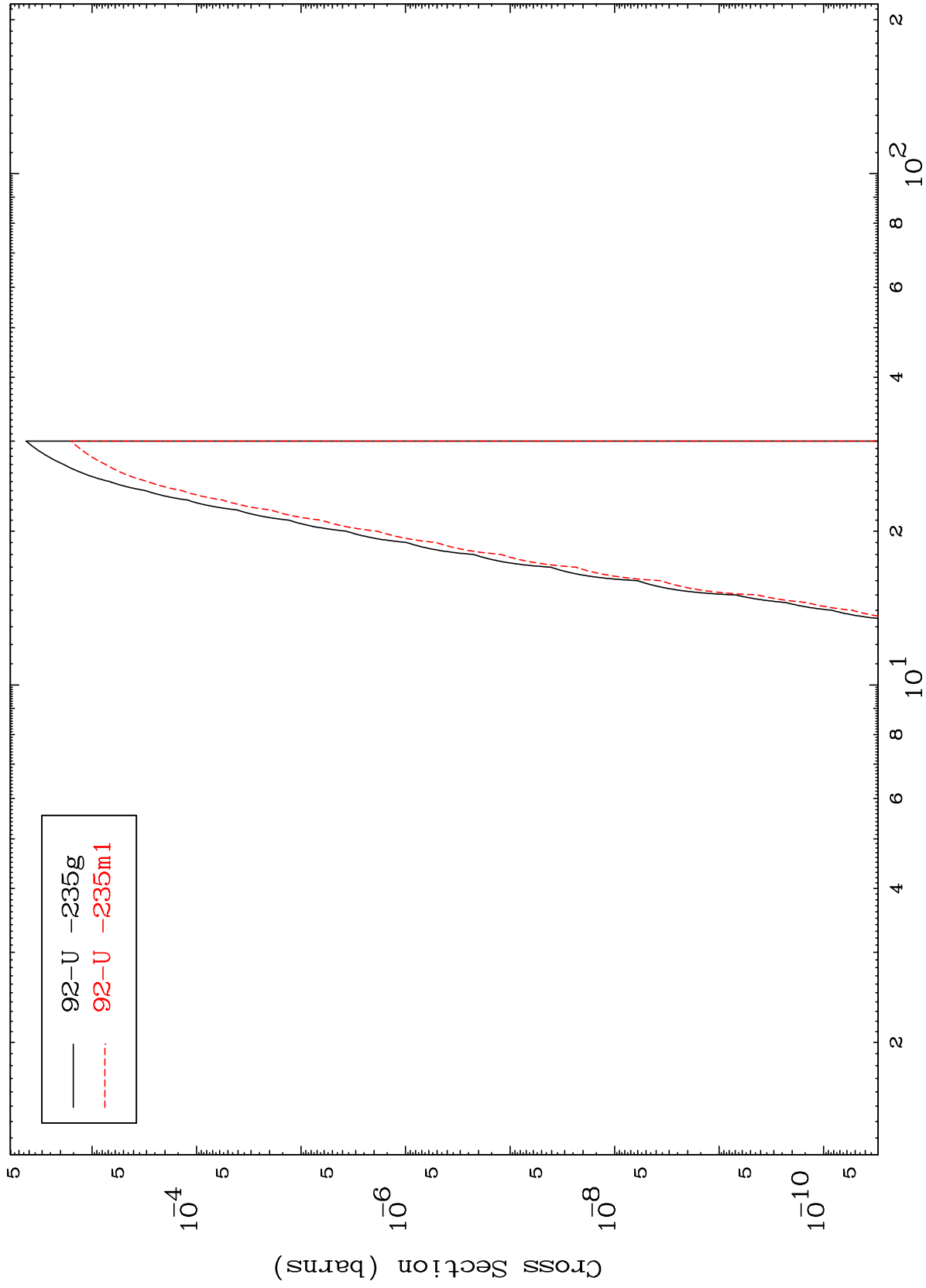


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(n,2n)  $\alpha$

94-Pu-240

Radionuclide Production Cross Section



24

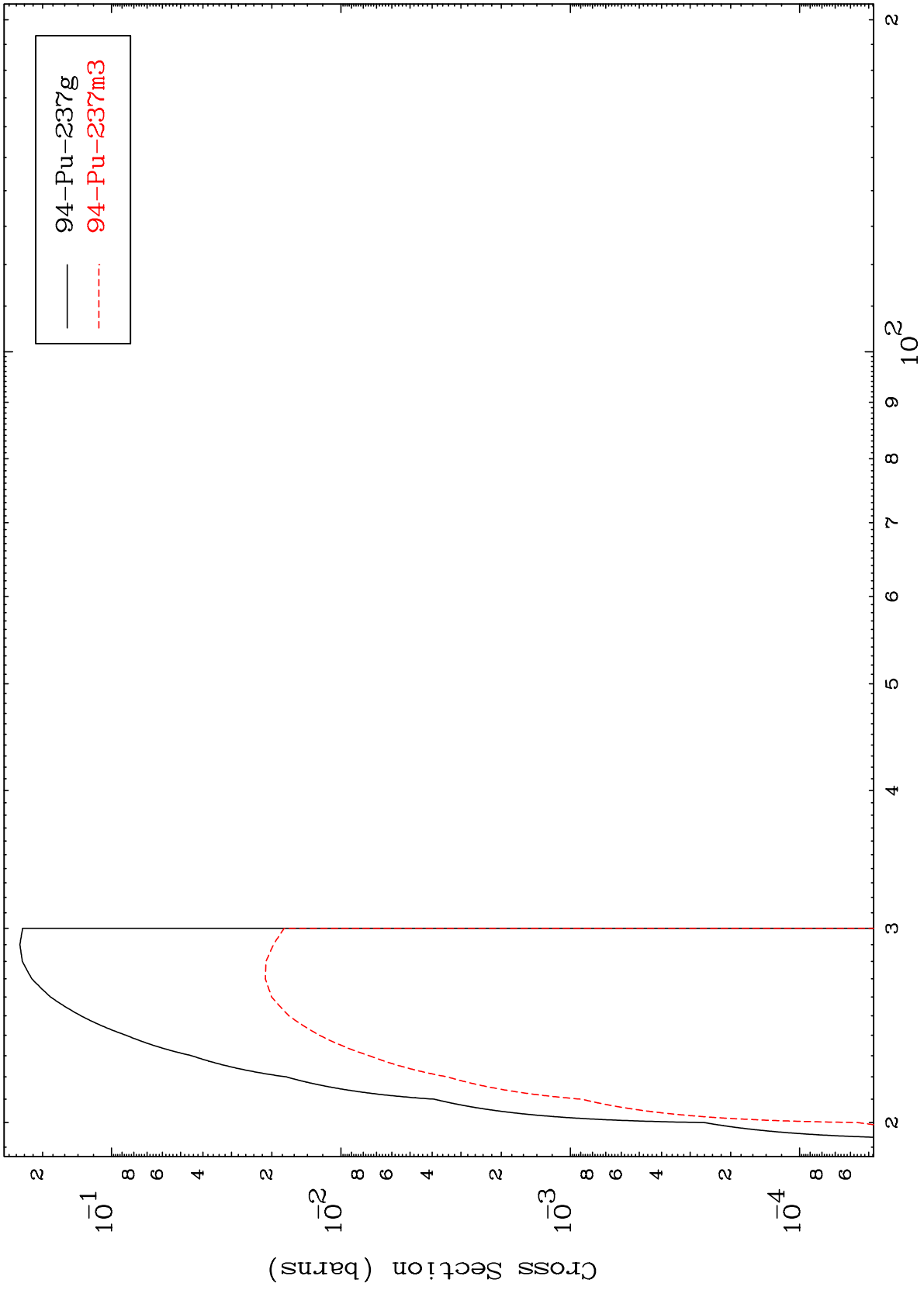
Incident Energy (MeV)

94-Pu-240

MAT 9440

94-Pu-240

(n,4n)  
Radionuclide Production Cross Section



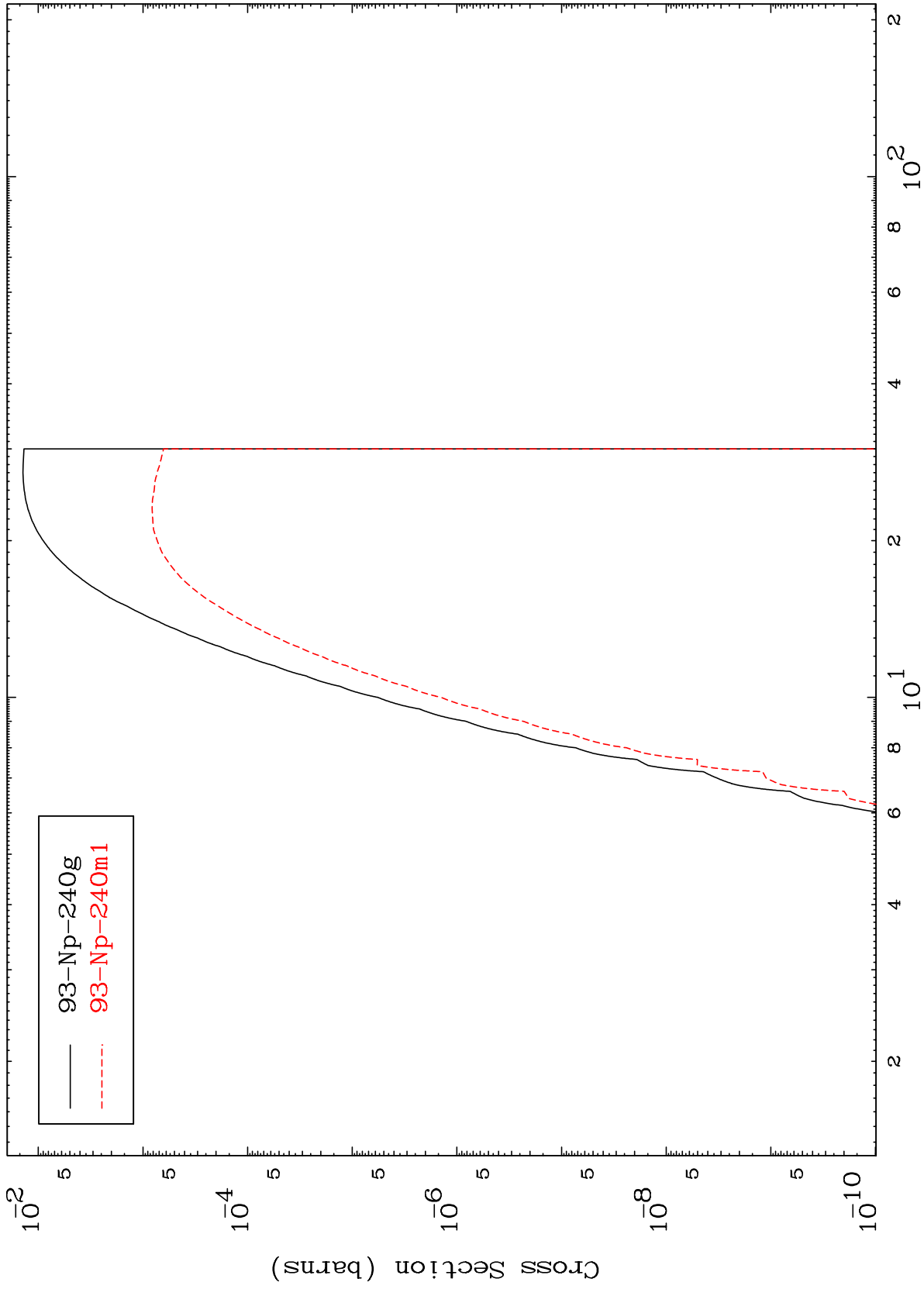
25

94-Pu-240

MAT 9440

94-Pu-240

(n,p)  
Radionuclide Production Cross Section



26

Incident Energy (MeV)

94-Pu-240