

Program EVALPLOT
(Version 2018-1)

by

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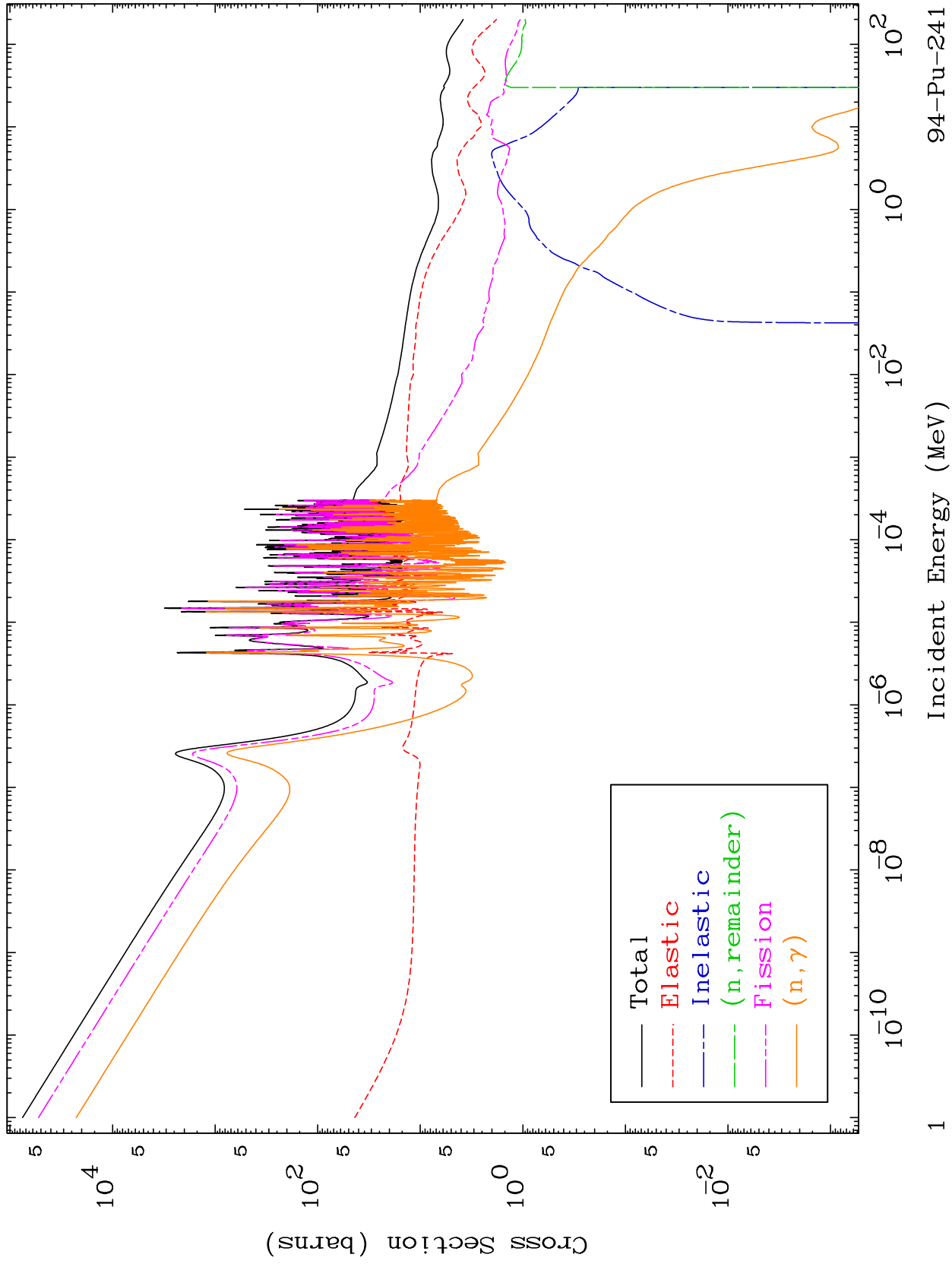
Press Mouse Button to Start

MAT 9443

Major

293 Kelvin Cross Sections

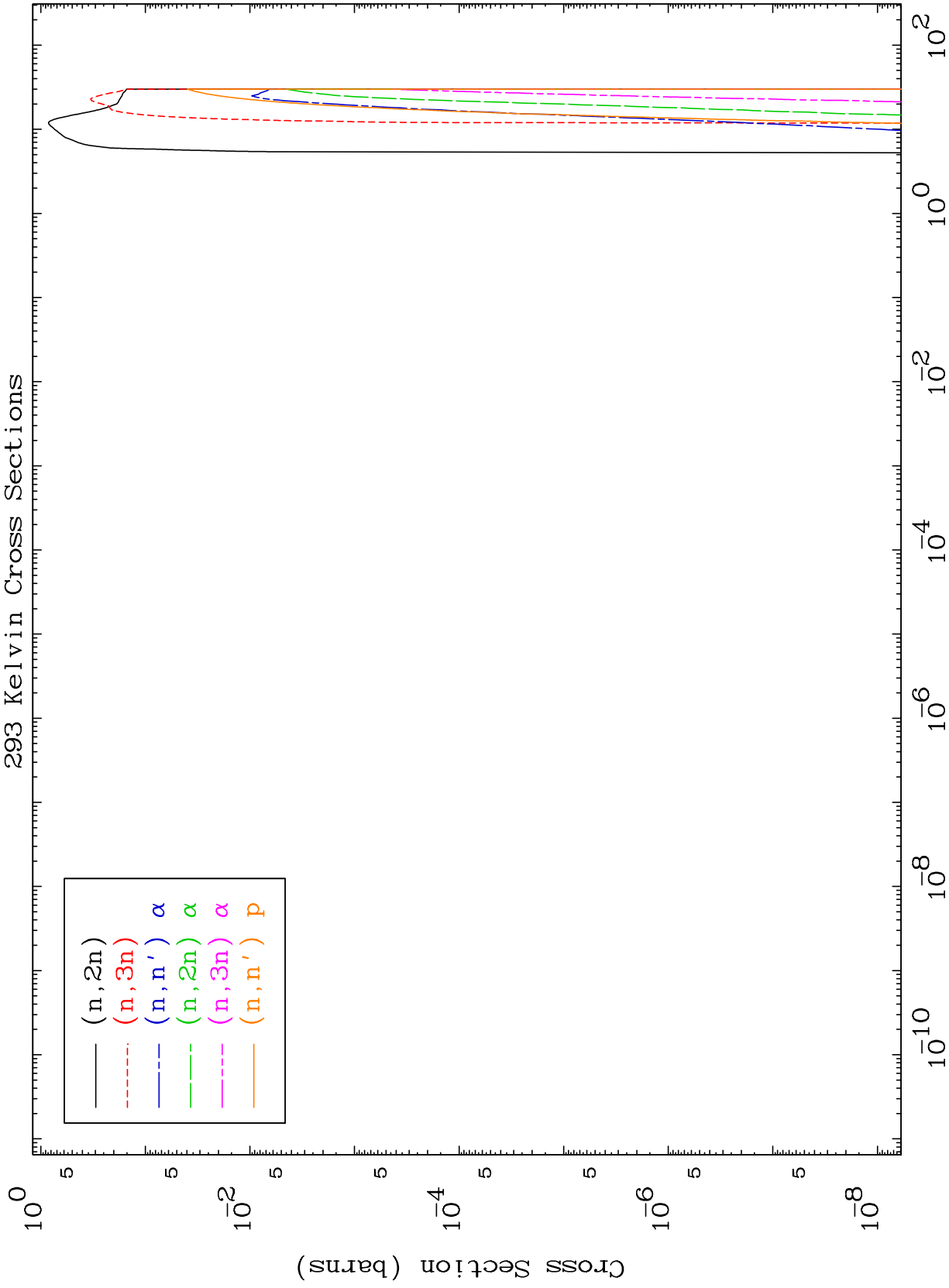
94-Pu-241



MAT 9443

Neutron Production
293 Kelvin Cross Sections

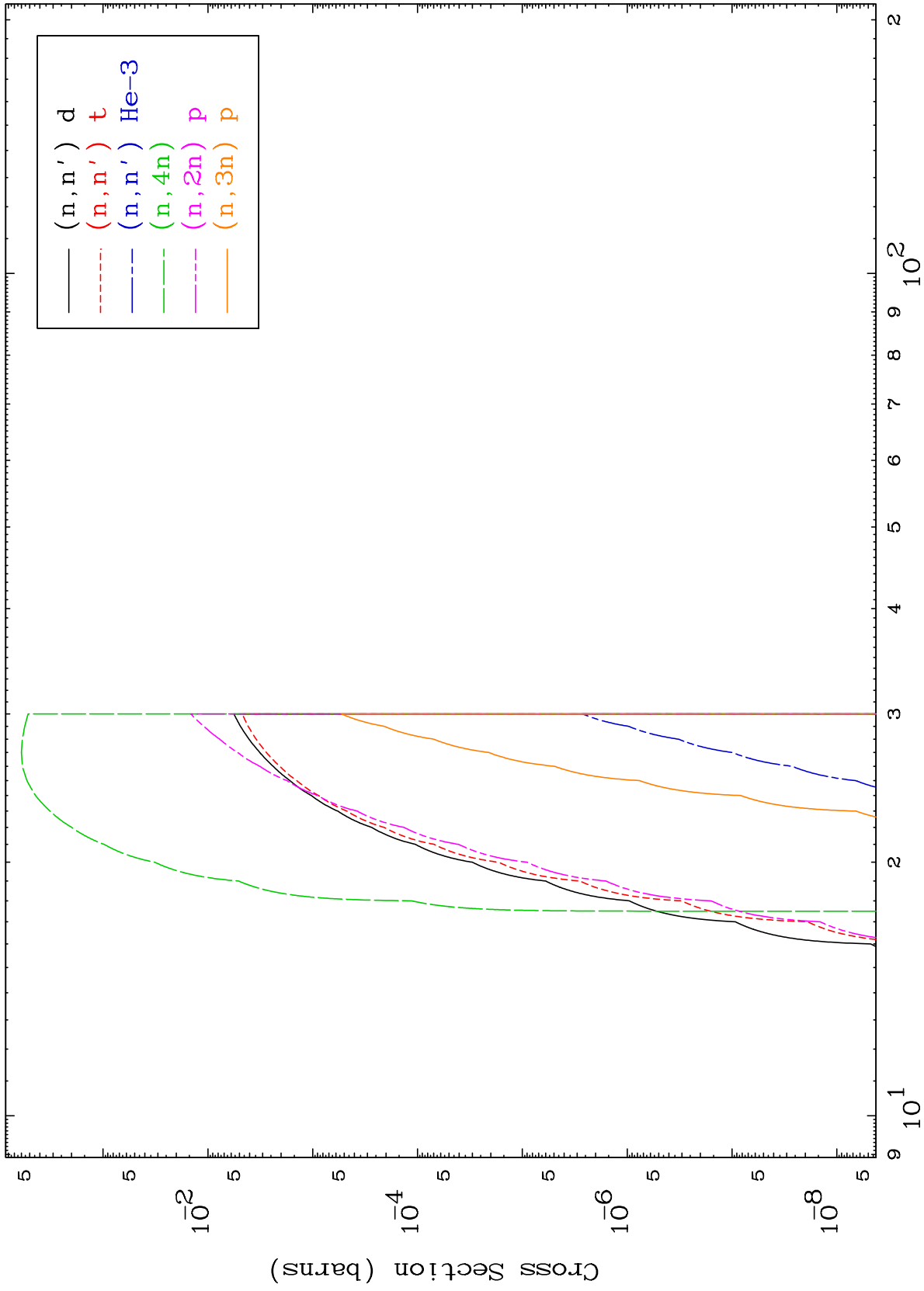
94-Pu-241



MAT 9443

Neutron Production
293 Kelvin Cross Sections

94-Pu-241



3

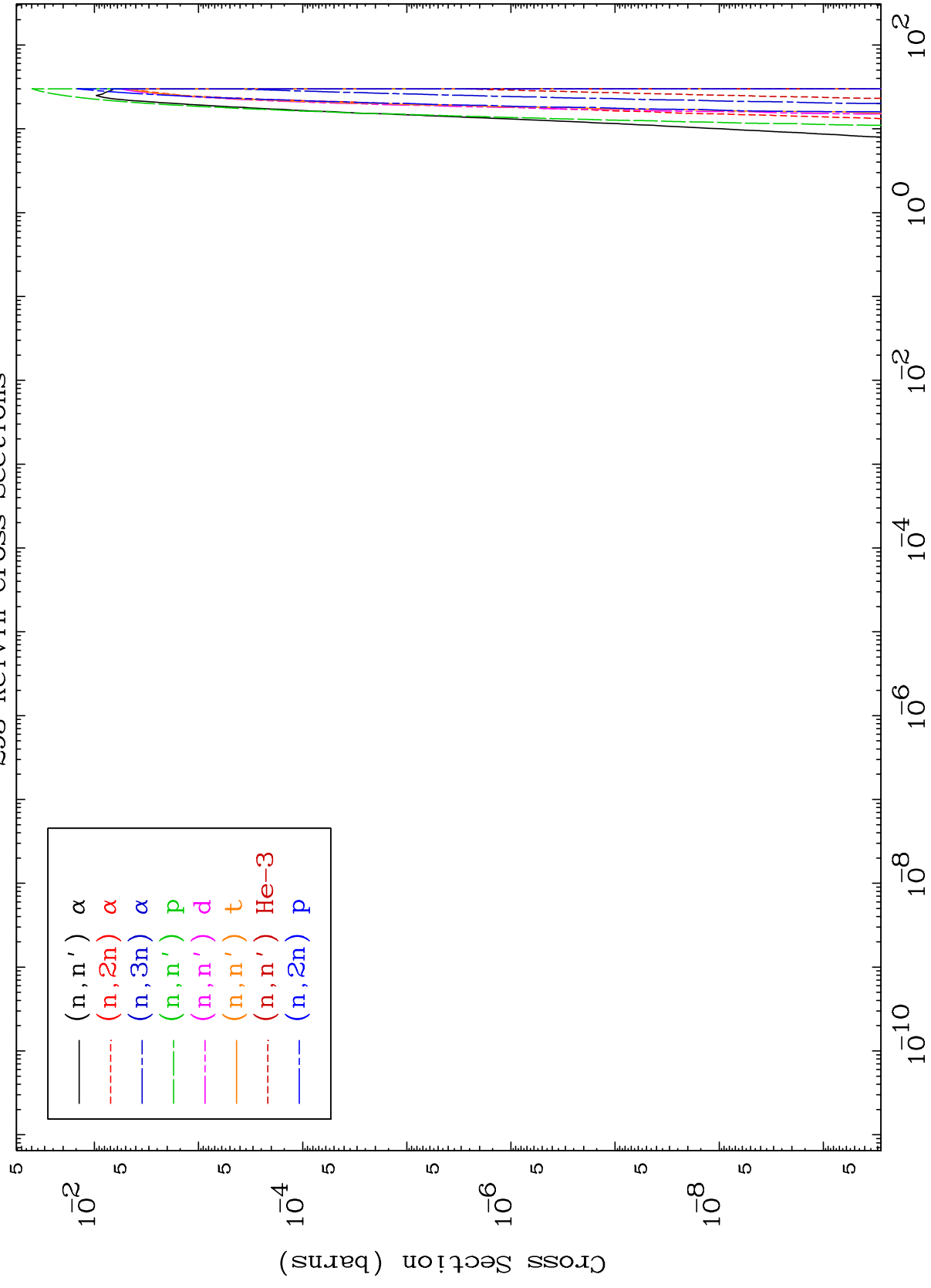
Incident Energy (MeV)

94-Pu-241

MAT 9443

Charged Particle
293 Kelvin Cross Sections

94-Pu-241

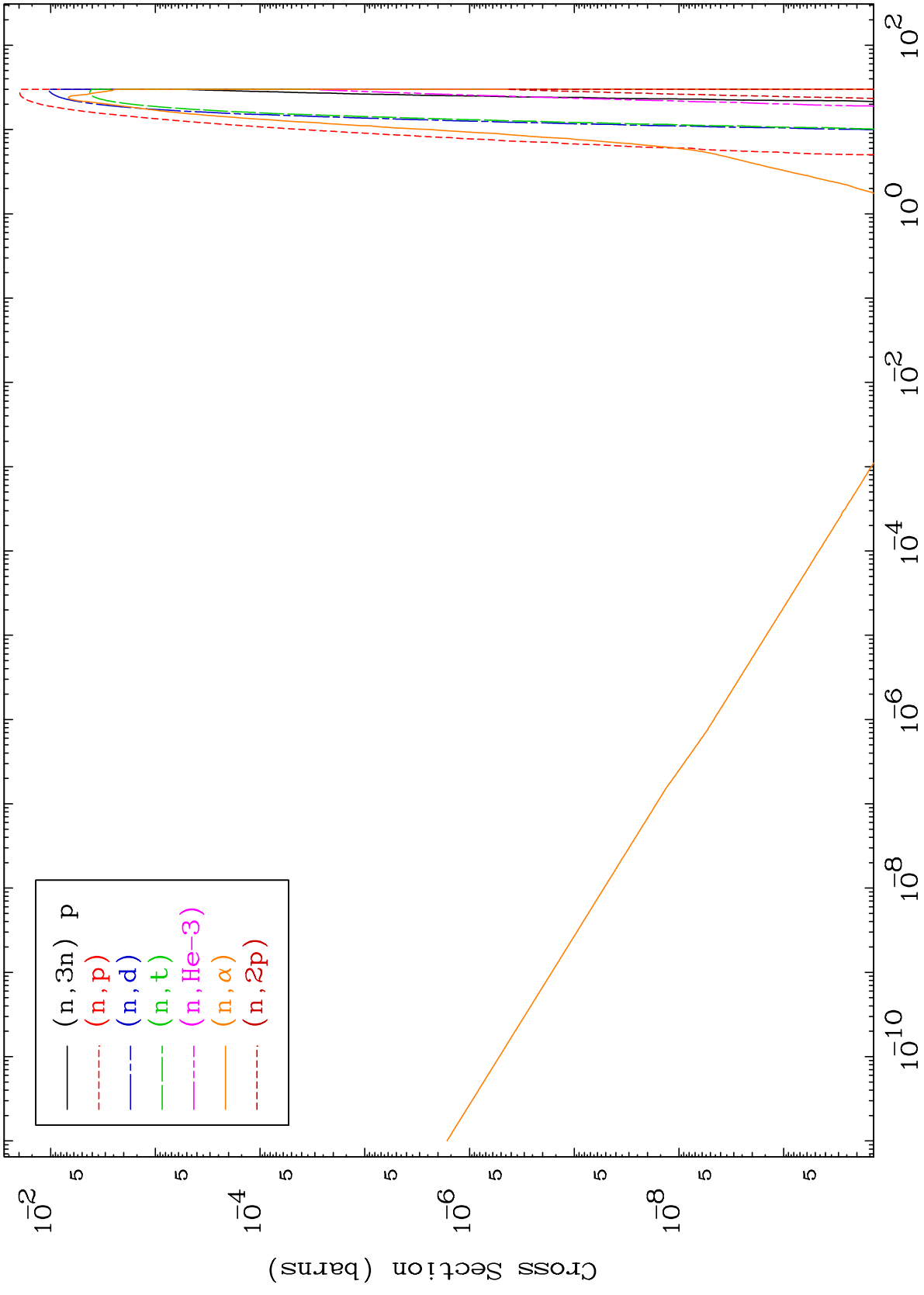


94-Pu-241

MAT 9443

Charged Particle
293 Kelvin Cross Sections

94-Pu-241

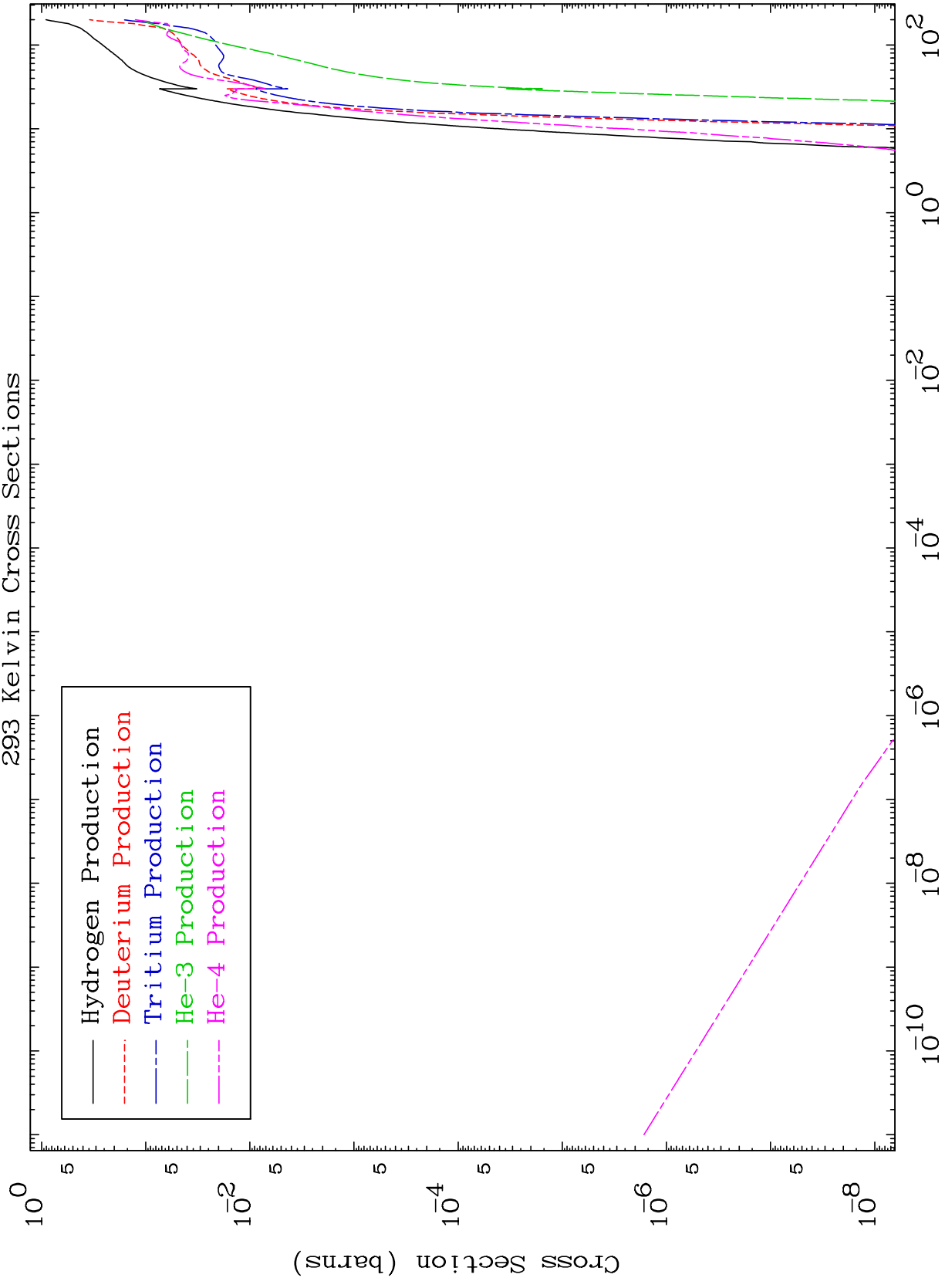


94-Pu-241

MAT 9443

Particle Production
293 Kelvin Cross Sections

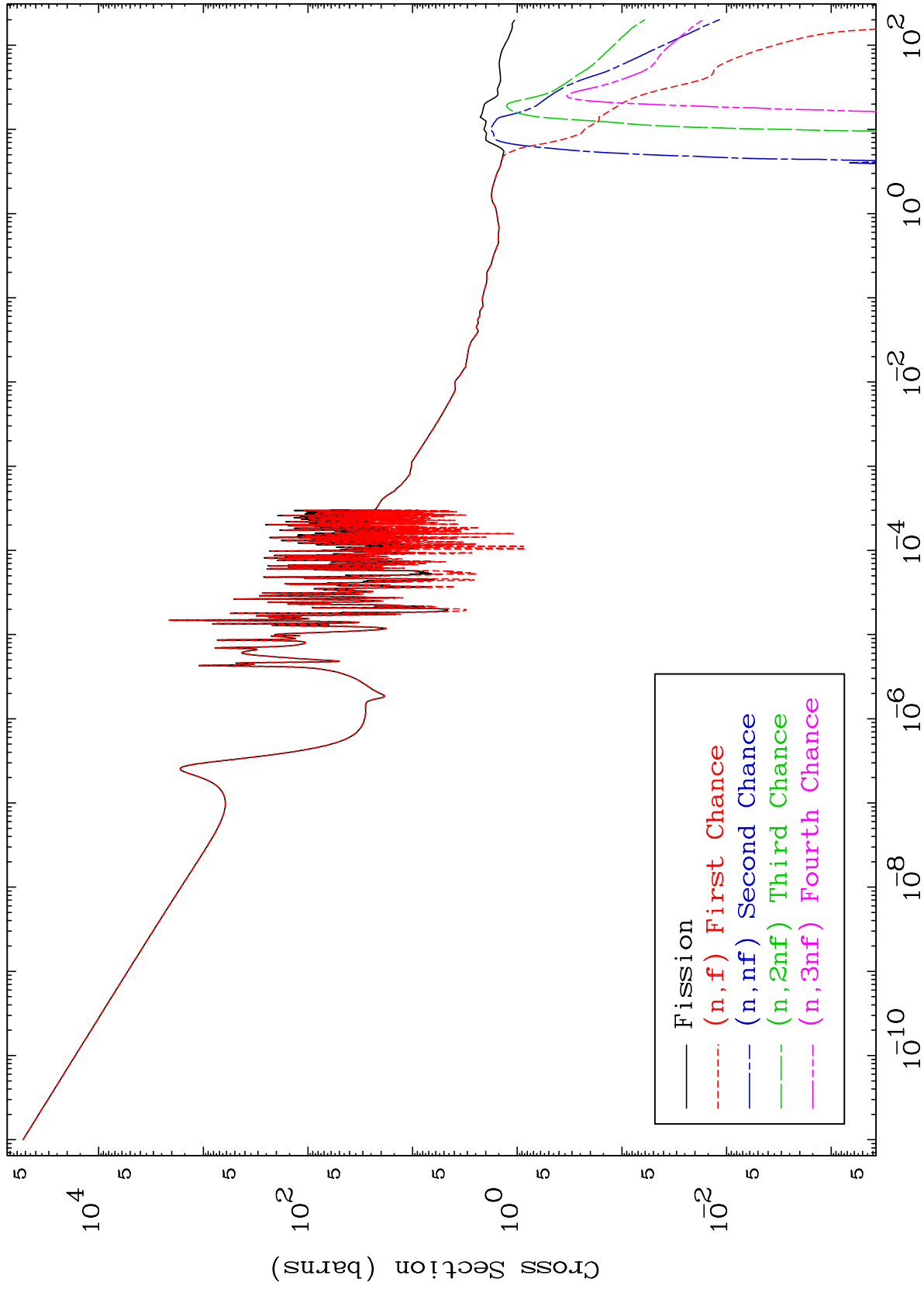
94-Pu-241



MAT 9443

Fission
293 Kelvin Cross Sections

94-Pu-241



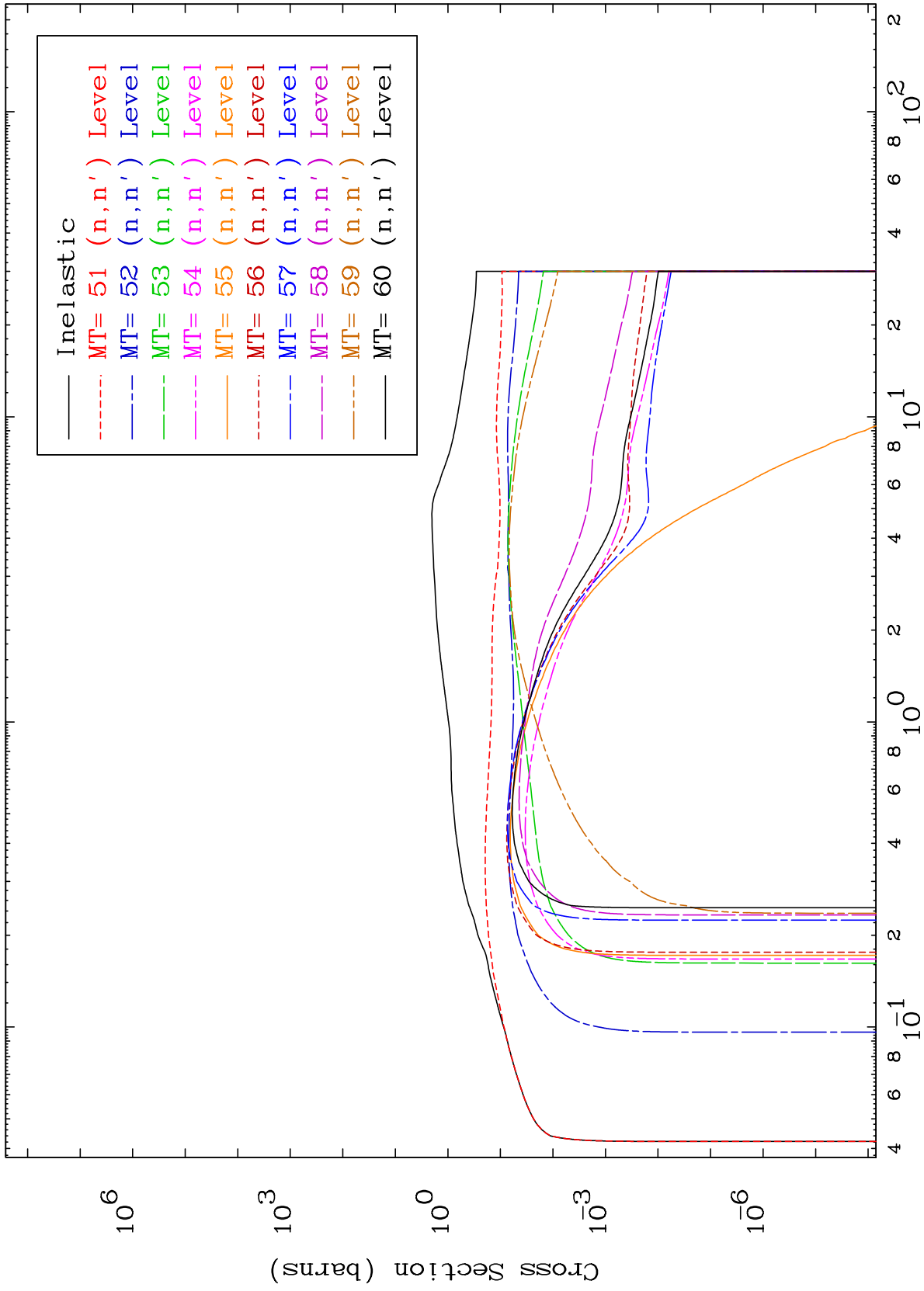
94-Pu-241

MAT 9443

(n,n') Level

94-Pu-241

293 Kelvin Cross Sections

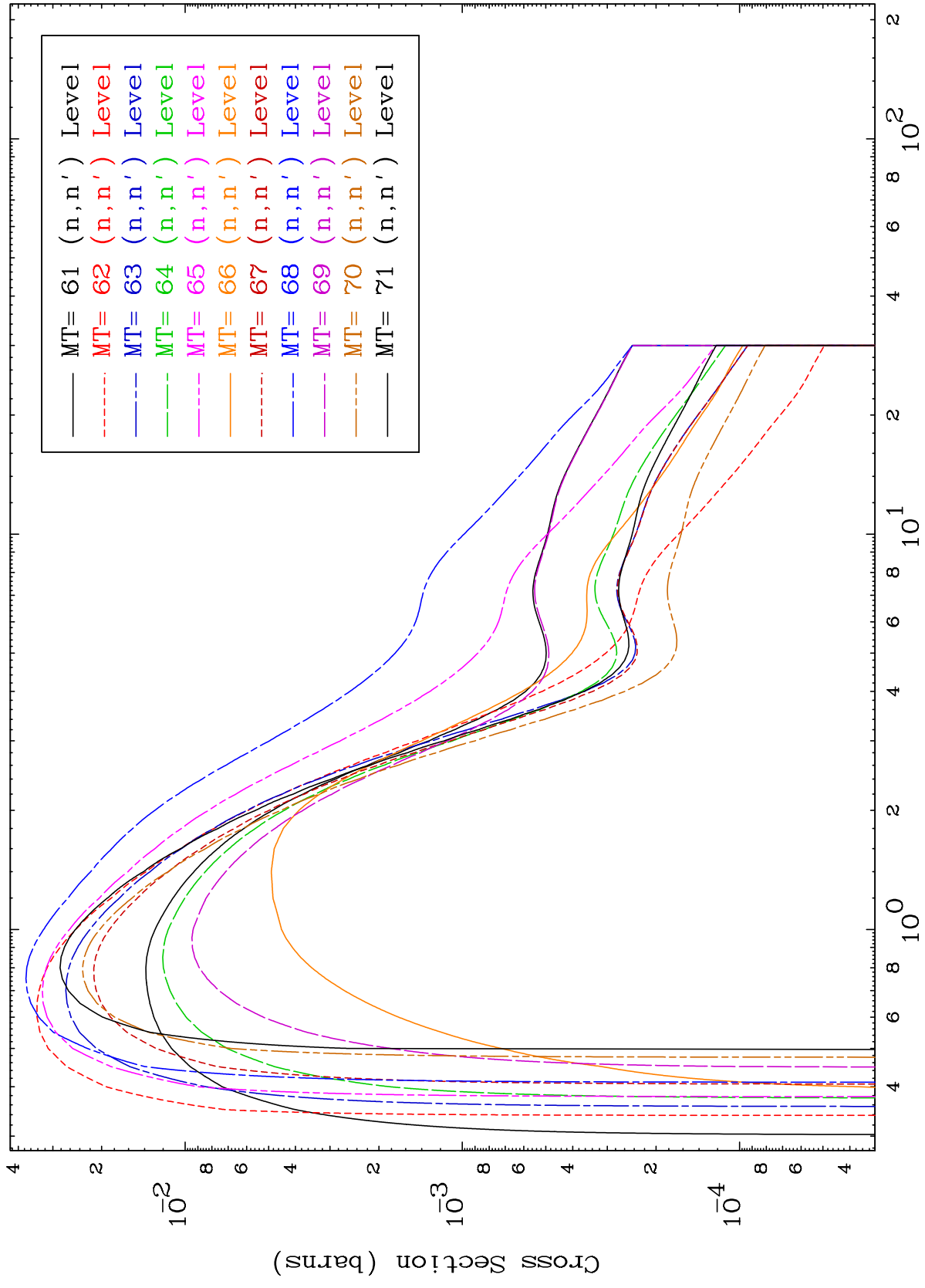


MAT 9443

(n,n') Level

94-Pu-241

293 Kelvin Cross Sections



9

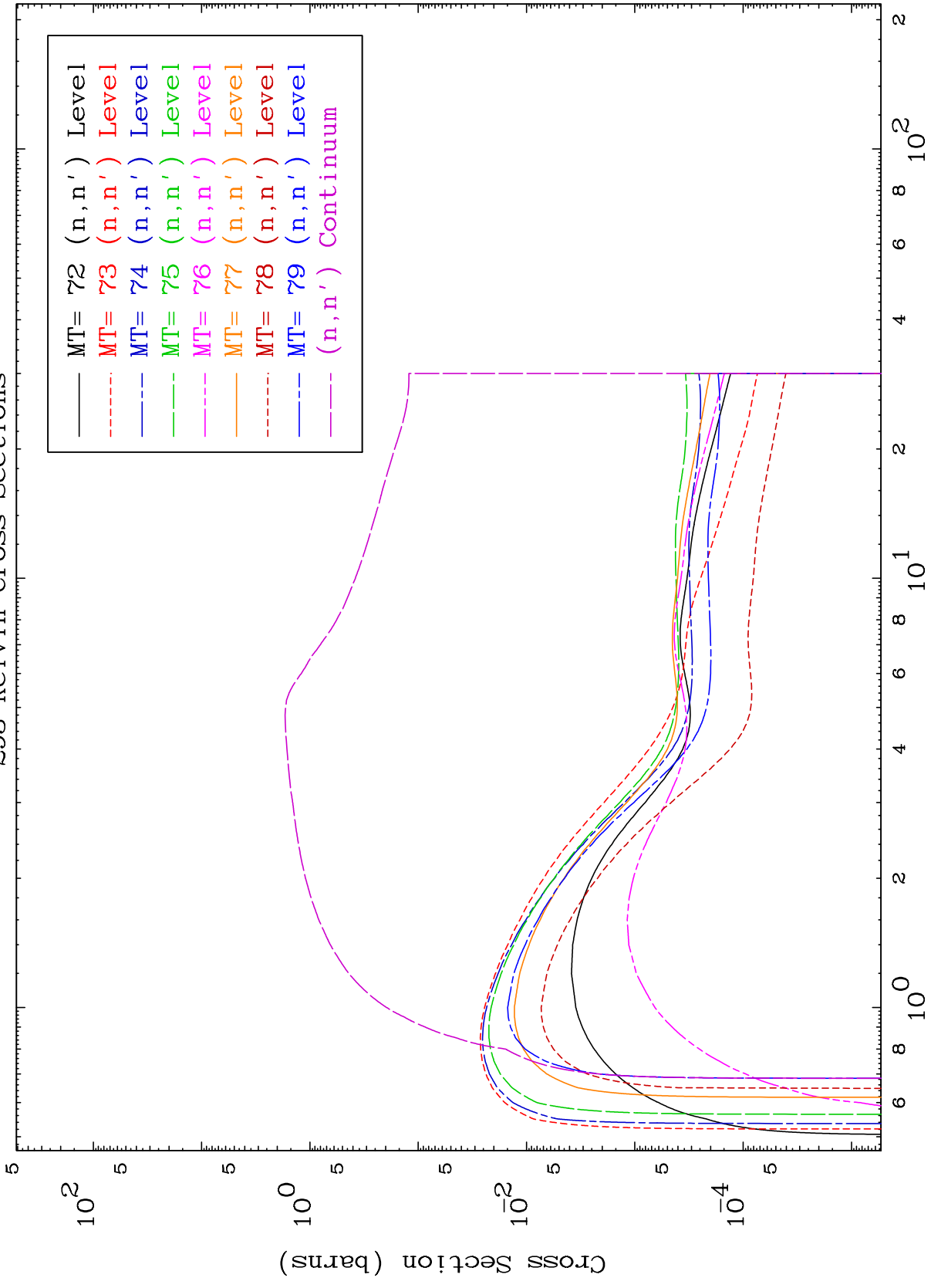
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,n') Level
293 Kelvin Cross Sections

94-Pu-241



10

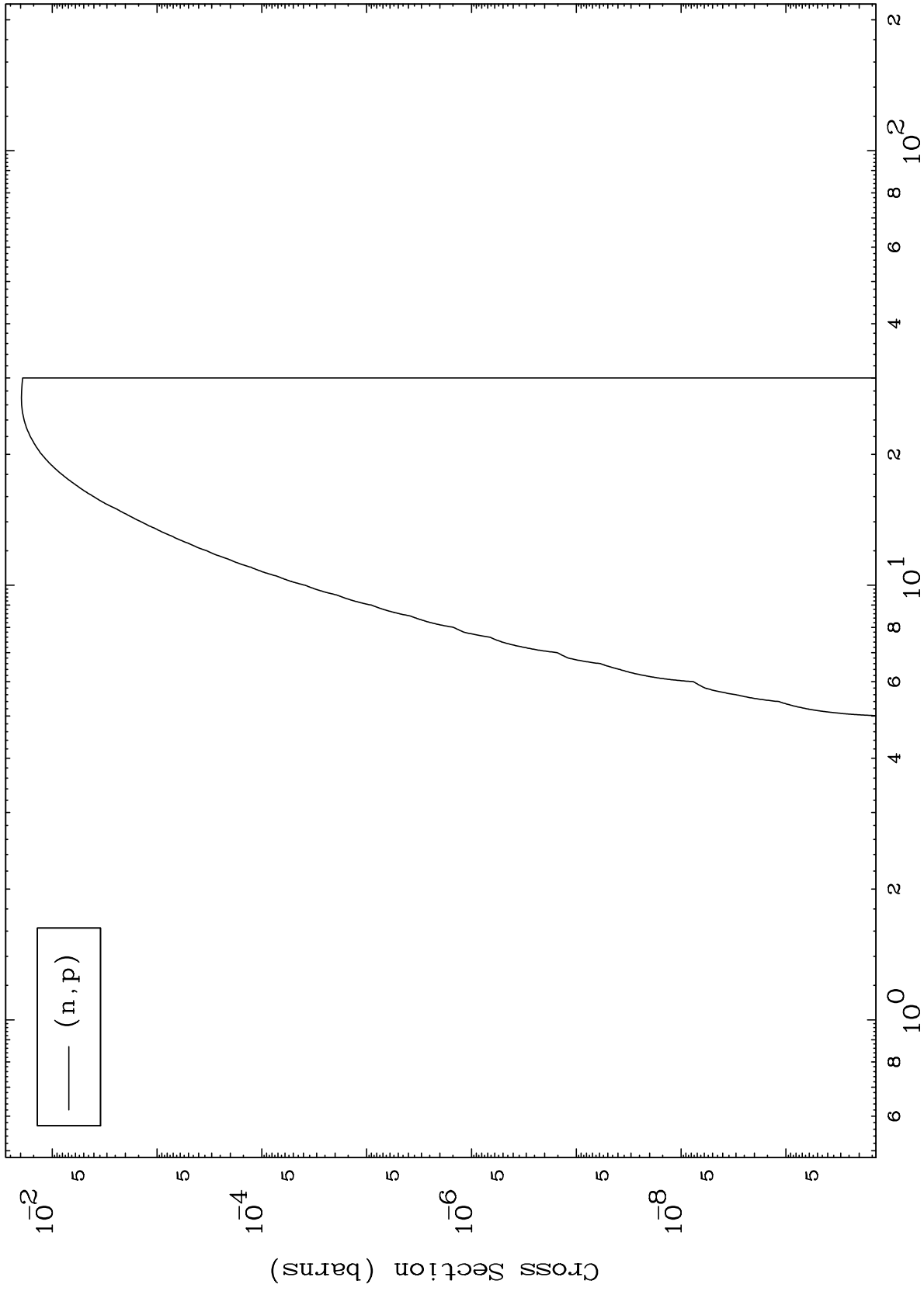
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,p) Levels
293 Kelvin Cross Sections

94-Pu-241



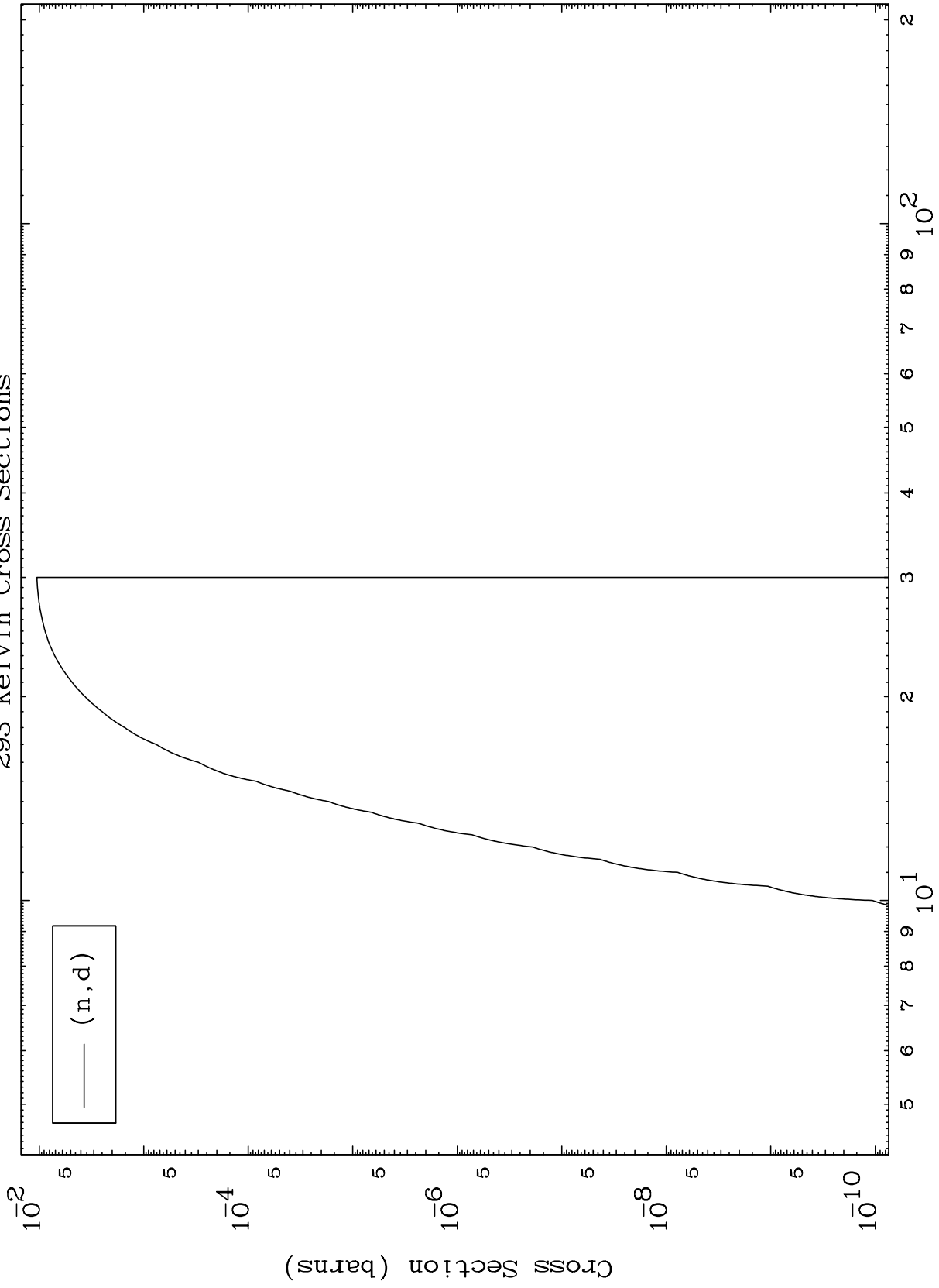
94-Pu-241

Incident Energy (MeV)

MAT 9443

(n,d) Levels
293 Kelvin Cross Sections

94-Pu-241



12

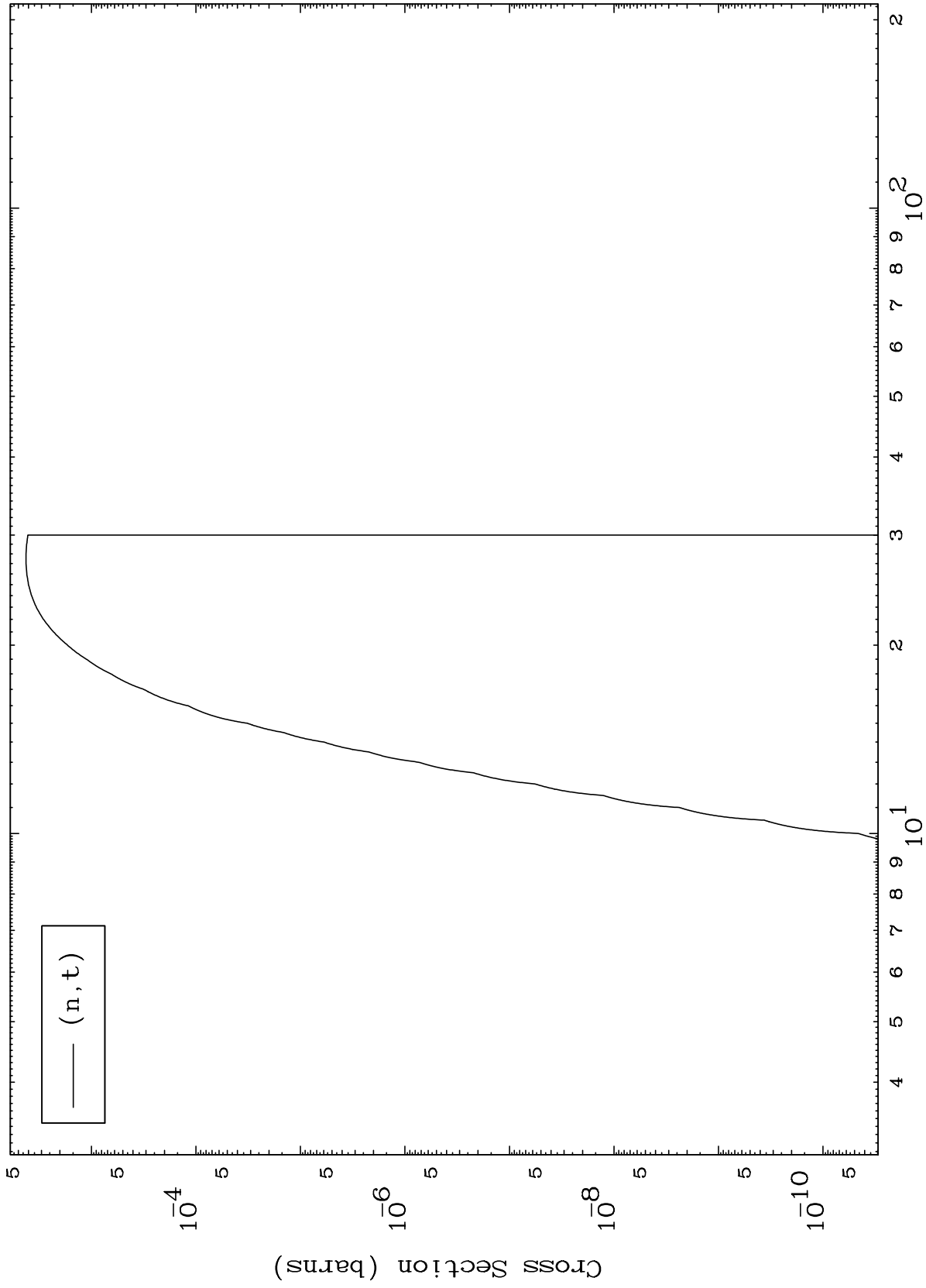
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,t) Levels
293 Kelvin Cross Sections

94-Pu-241



13

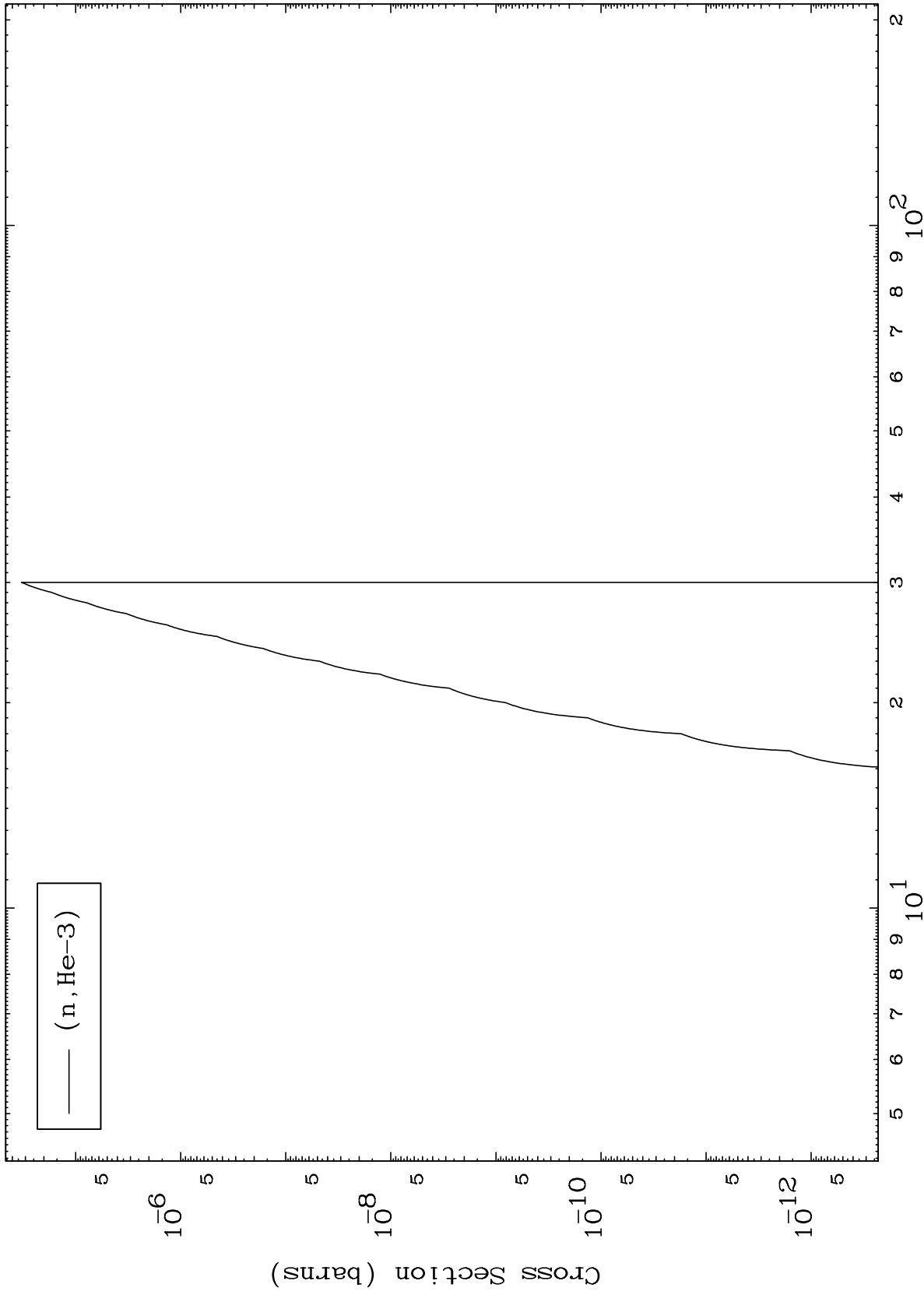
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,He3) Levels
293 Kelvin Cross Sections

94-Pu-241



14

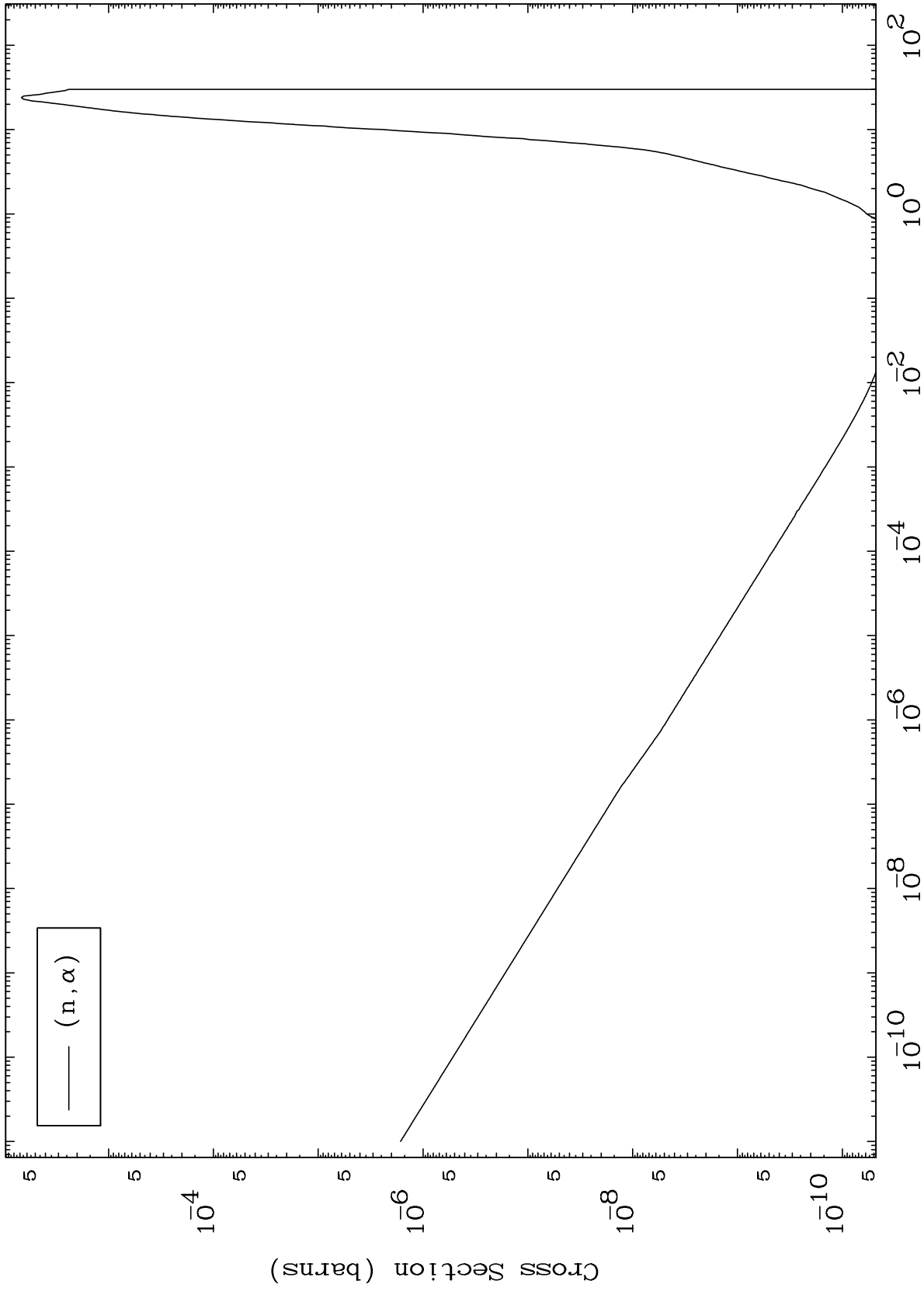
Incident Energy (MeV)

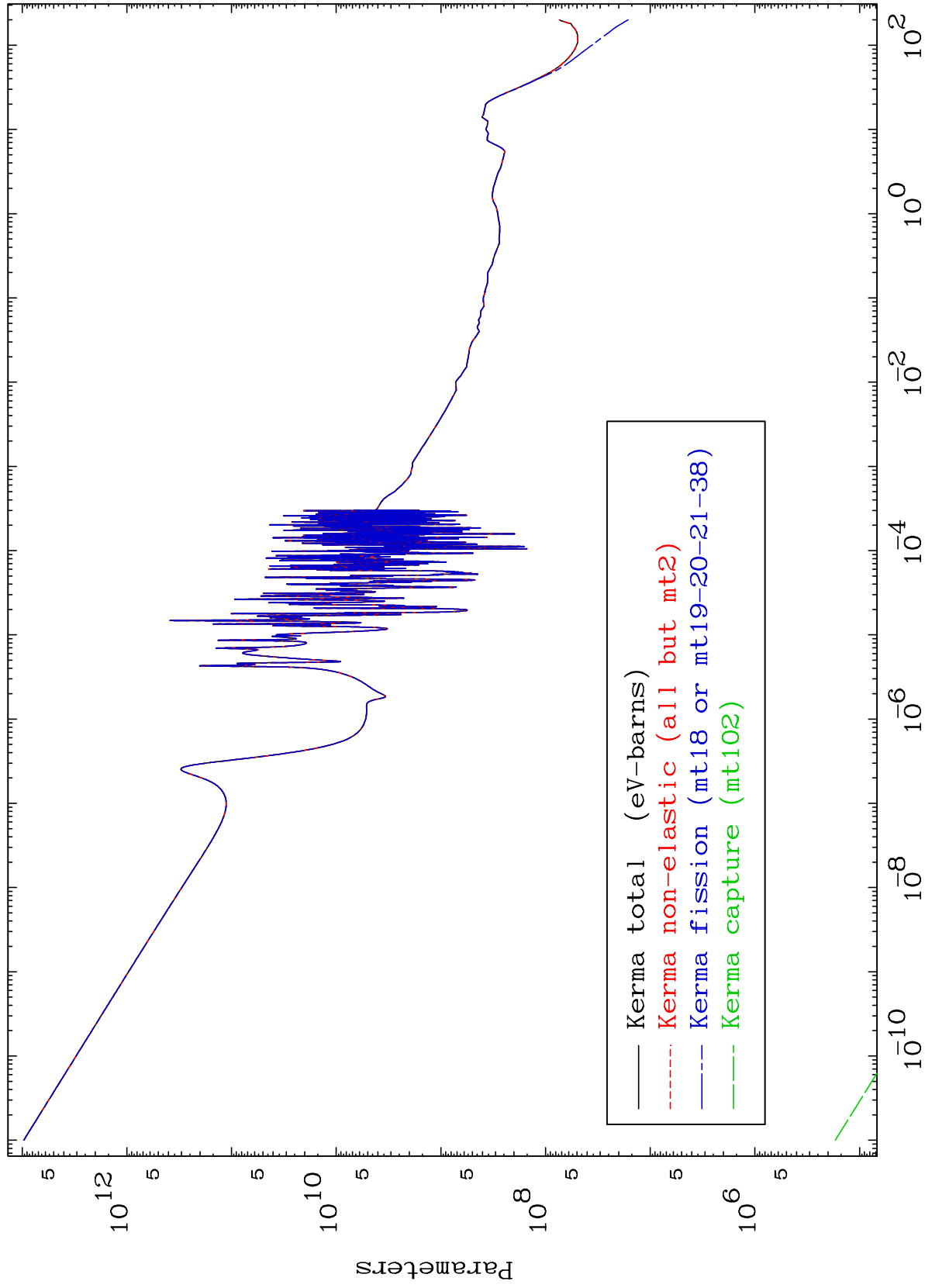
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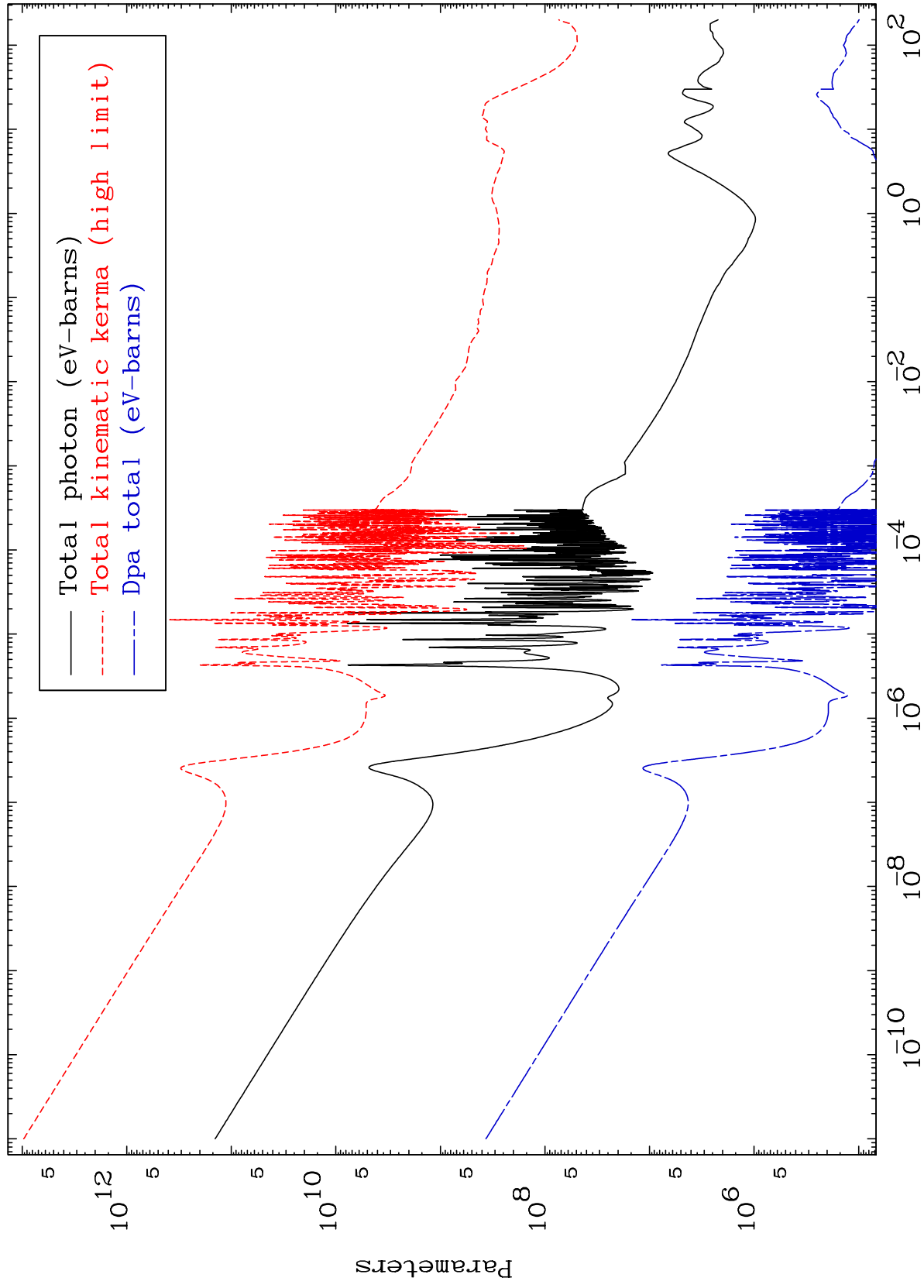
MAT 9443

(n,α) Levels
293 Kelvin Cross Sections

94-Pu-241



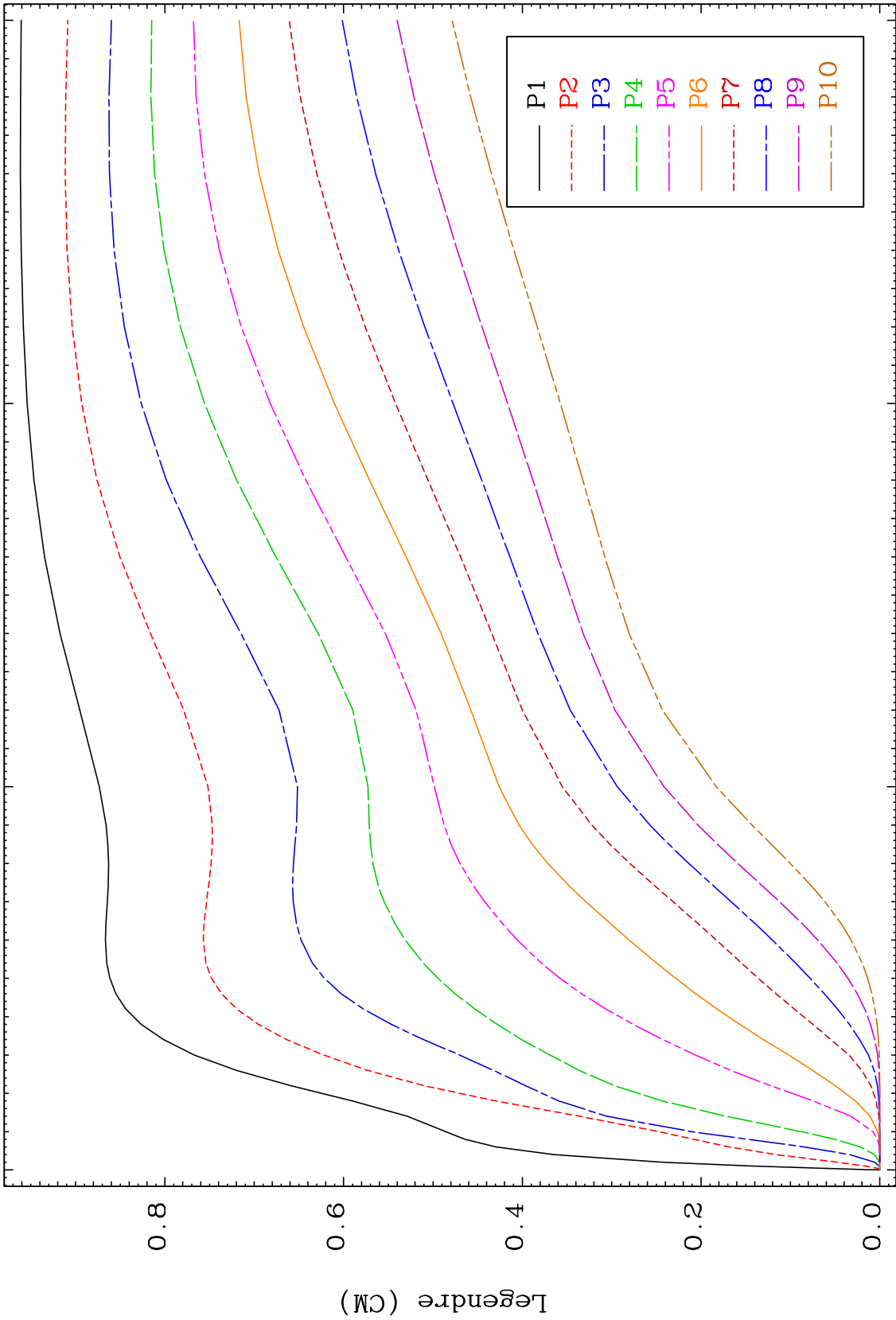




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94-Pu-241

Elastic Legendre Coefficients



18

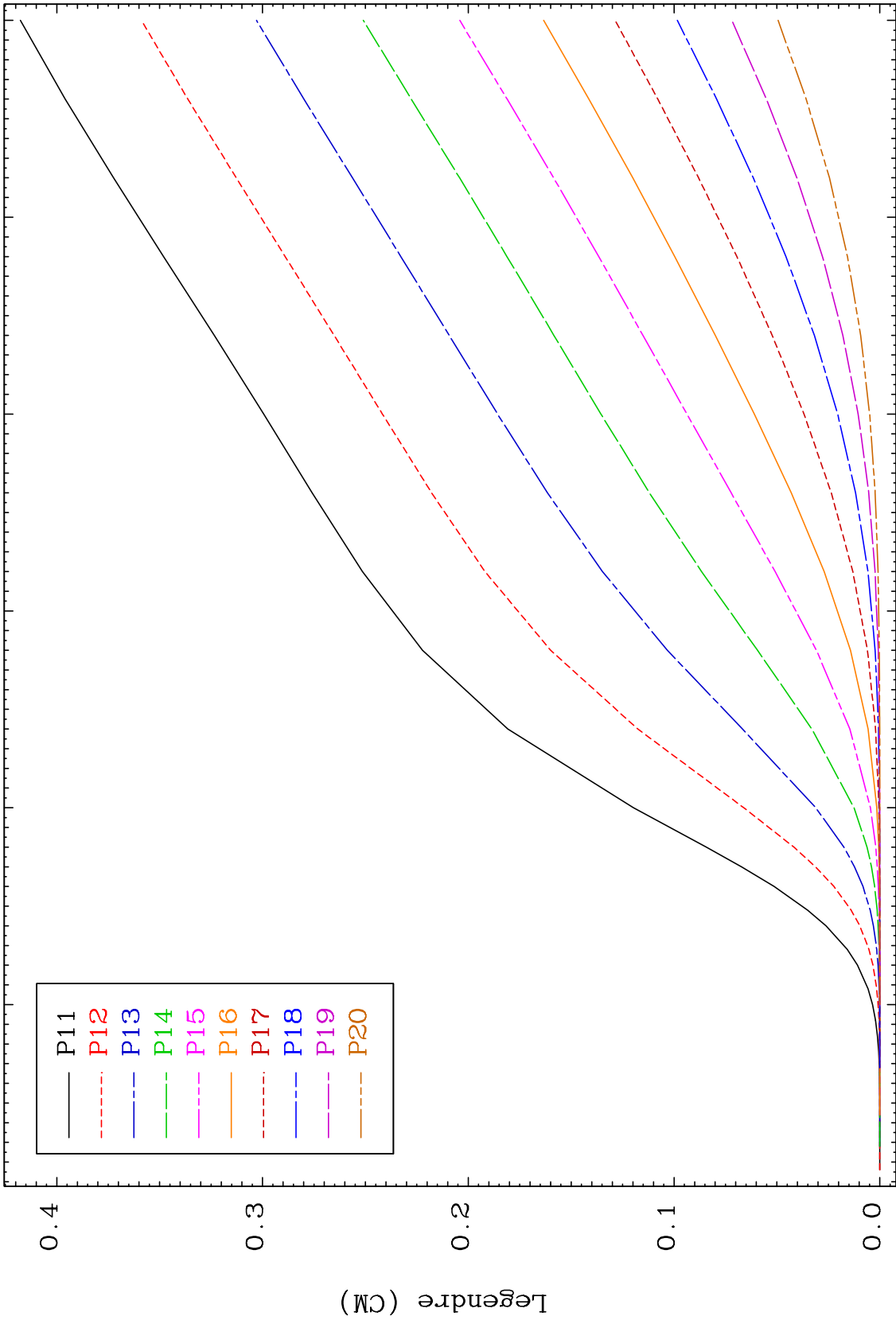
Incident Energy (MeV)

94-Pu-241

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



19

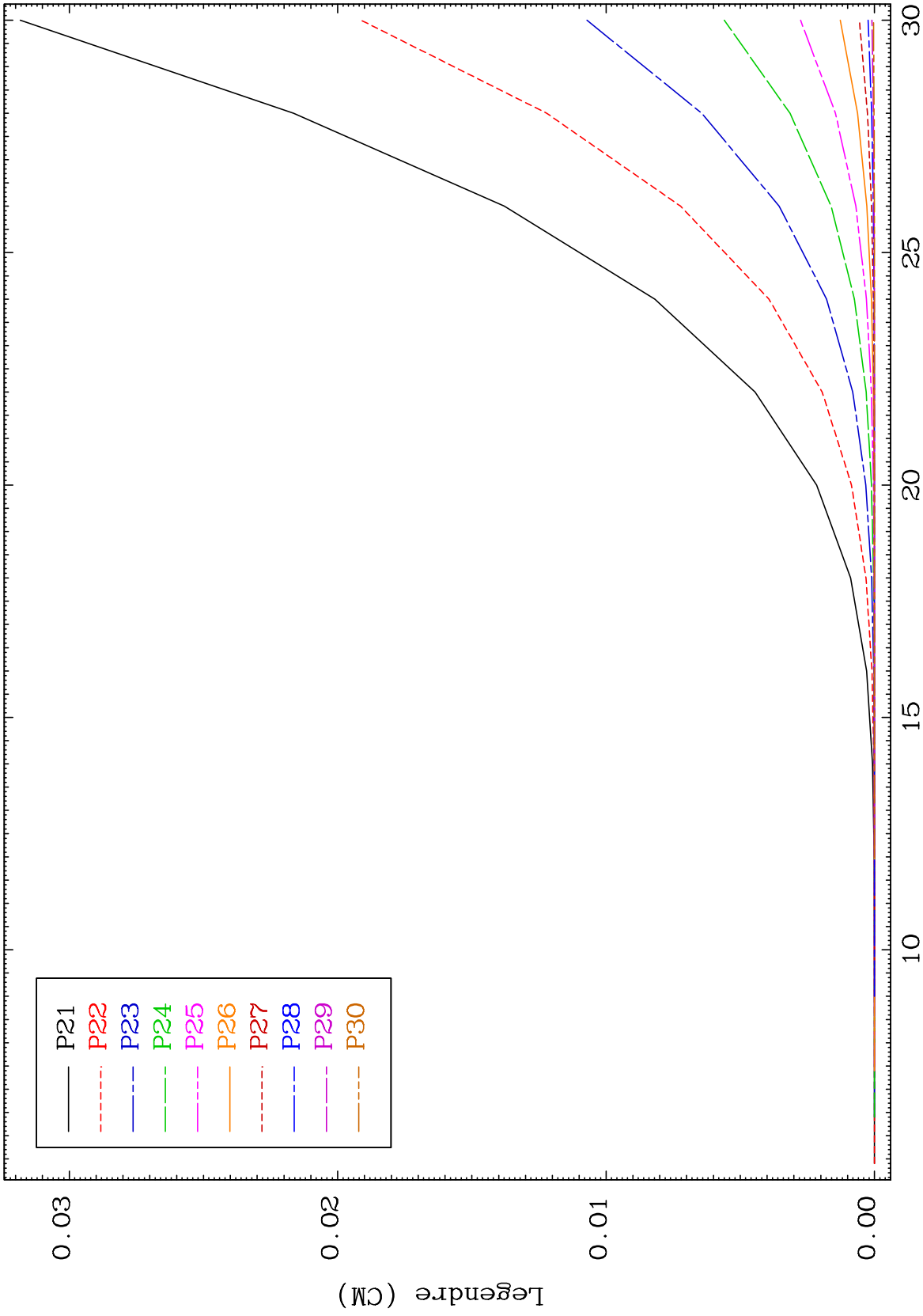
Incident Energy (MeV)

94-Pu-241

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



94-Pu-241

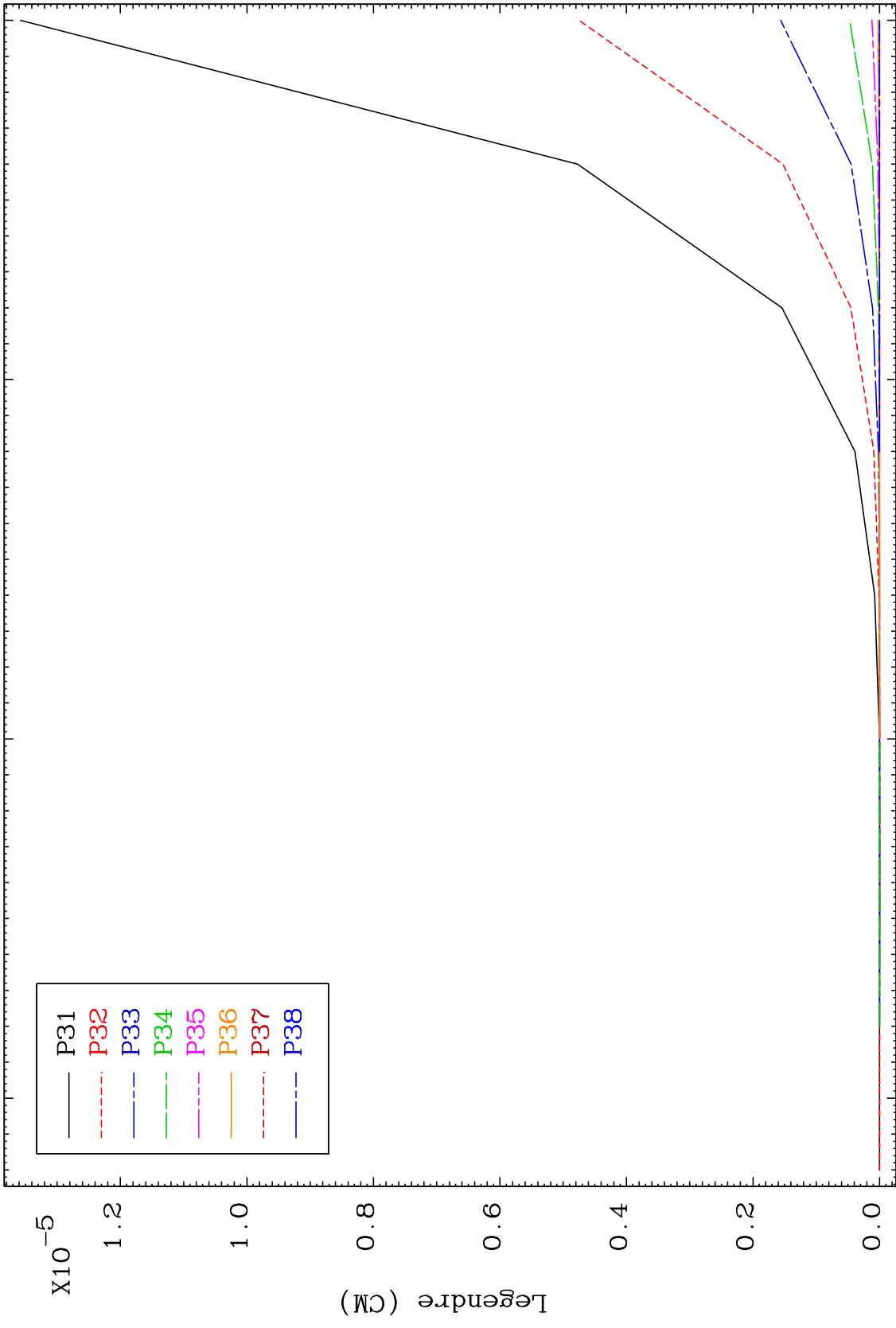
Incident Energy (MeV)

20

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



21

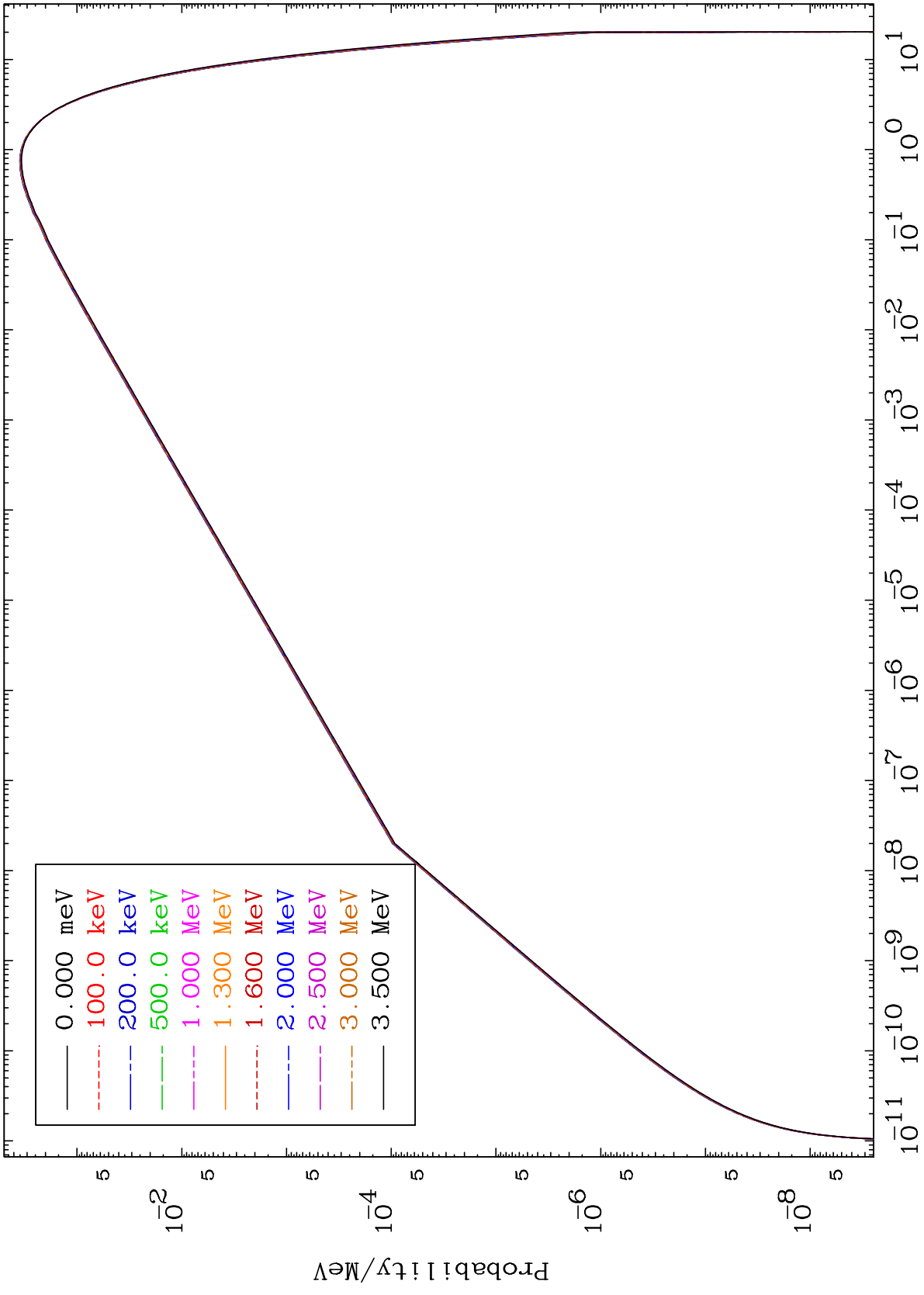
Incident Energy (MeV)

94-Pu-241

MAT 9443

Fission Energy Distribution

94-Pu-241

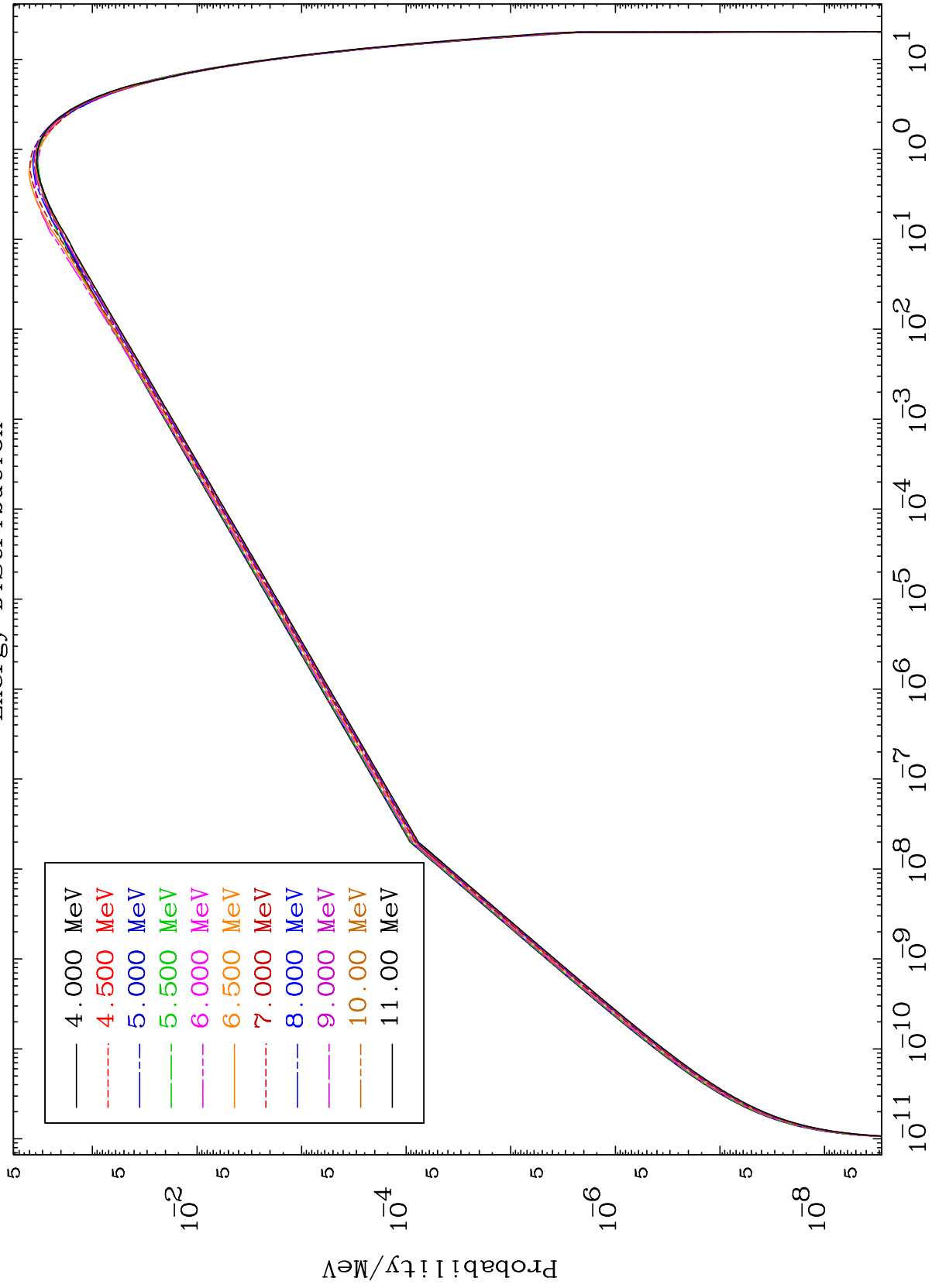


94-Pu-241

MAT 9443

Fission Energy Distribution

94-Pu-241



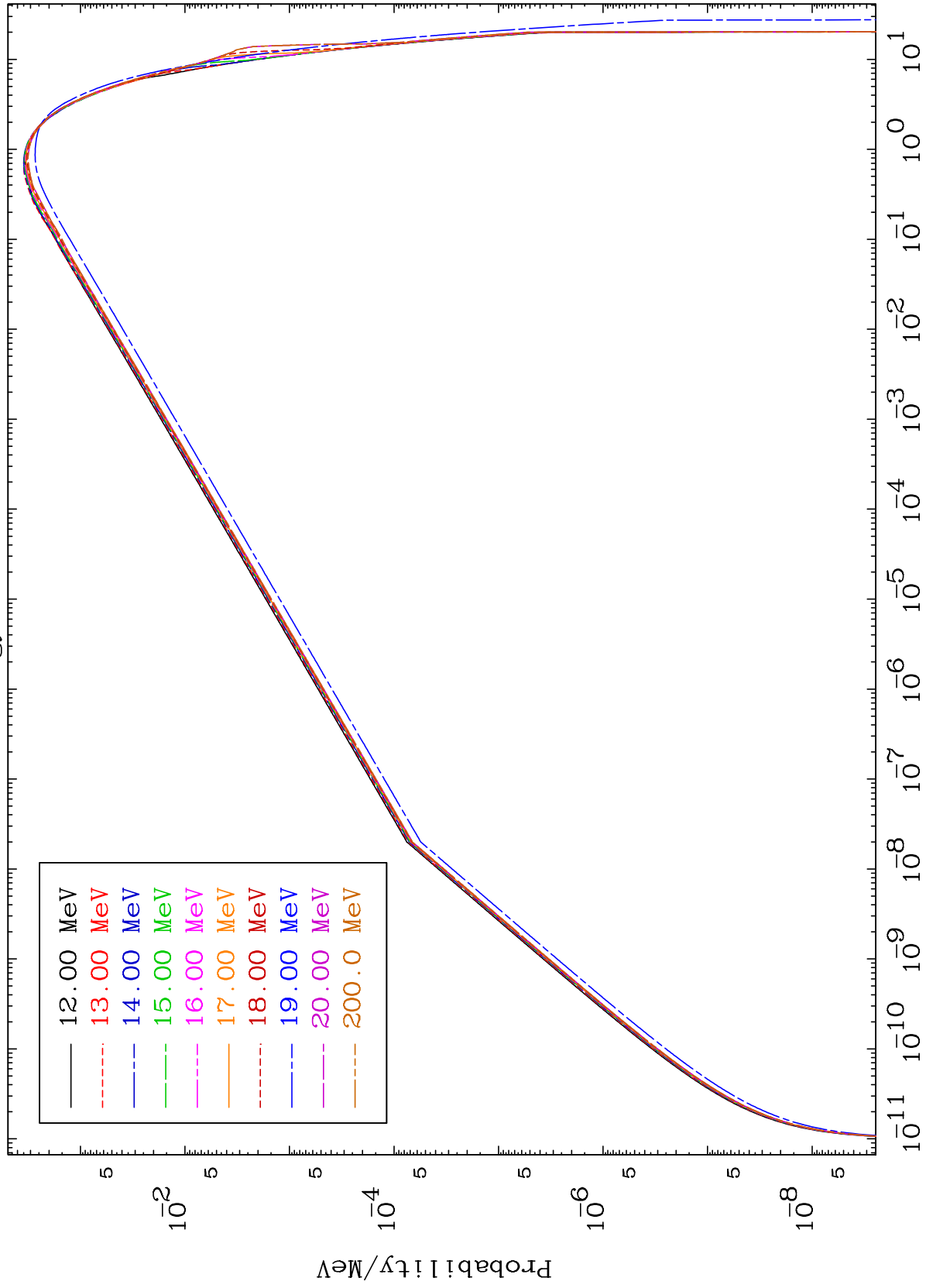
94-Pu-241

Secondary Energy (MeV)

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Fission Energy Distribution

94-Pu-241



24

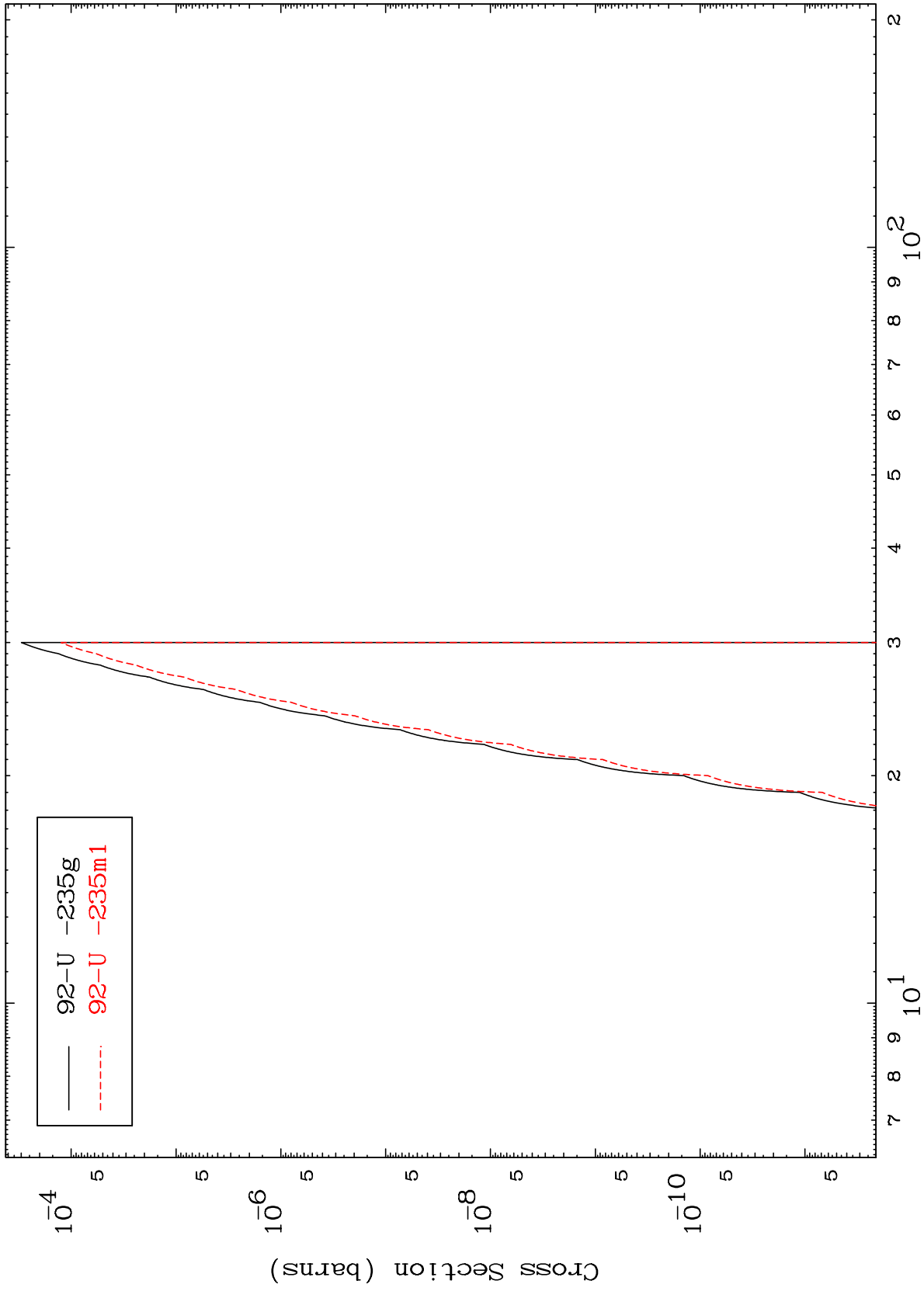
94-Pu-241

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(n,3n) α

94-Pu-241

Radionuclide Production Cross Section



25

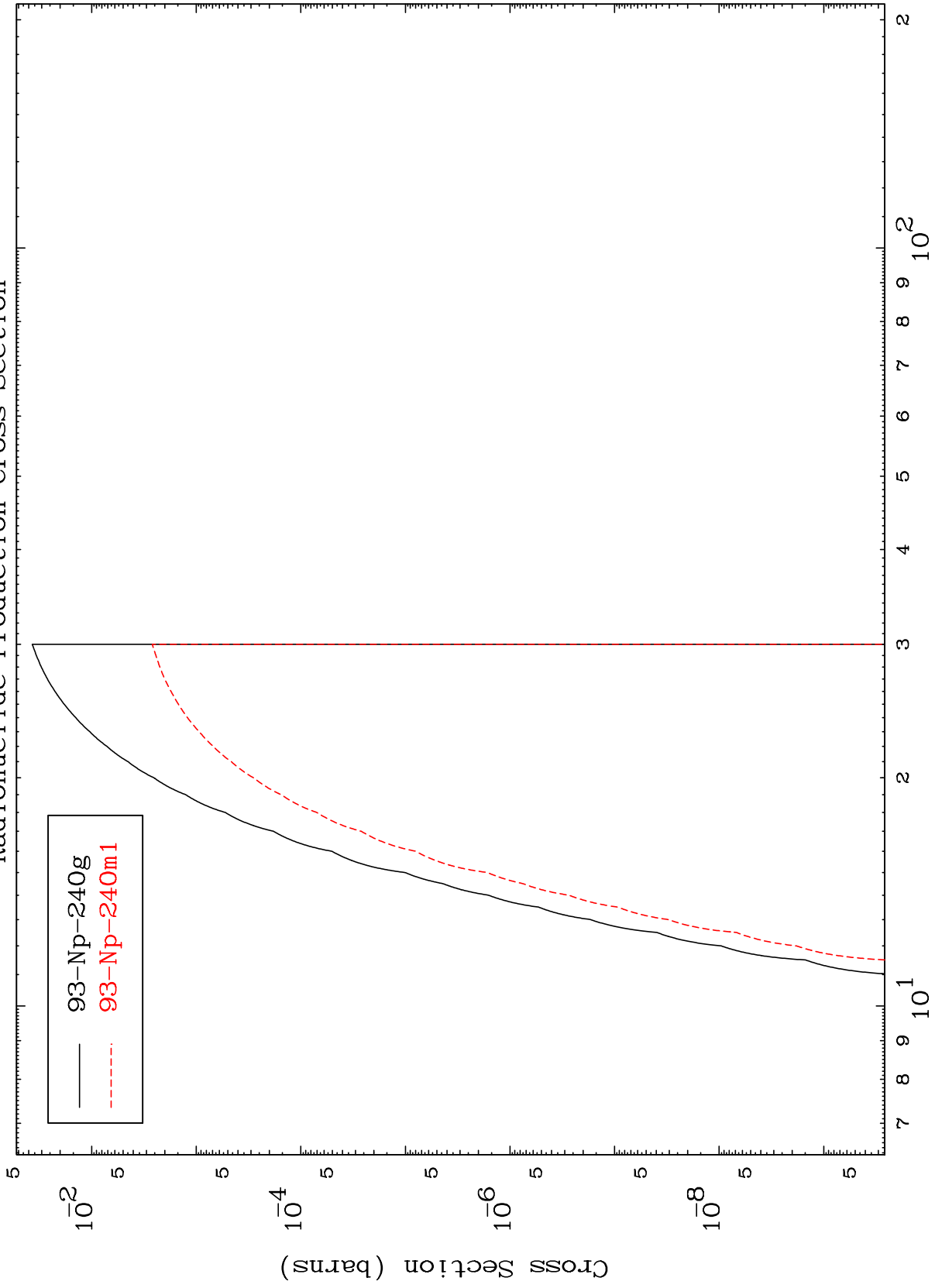
Incident Energy (MeV)

94-Pu-241

MAT 9443

94-Pu-241

(n,n') p
Radionuclide Production Cross Section

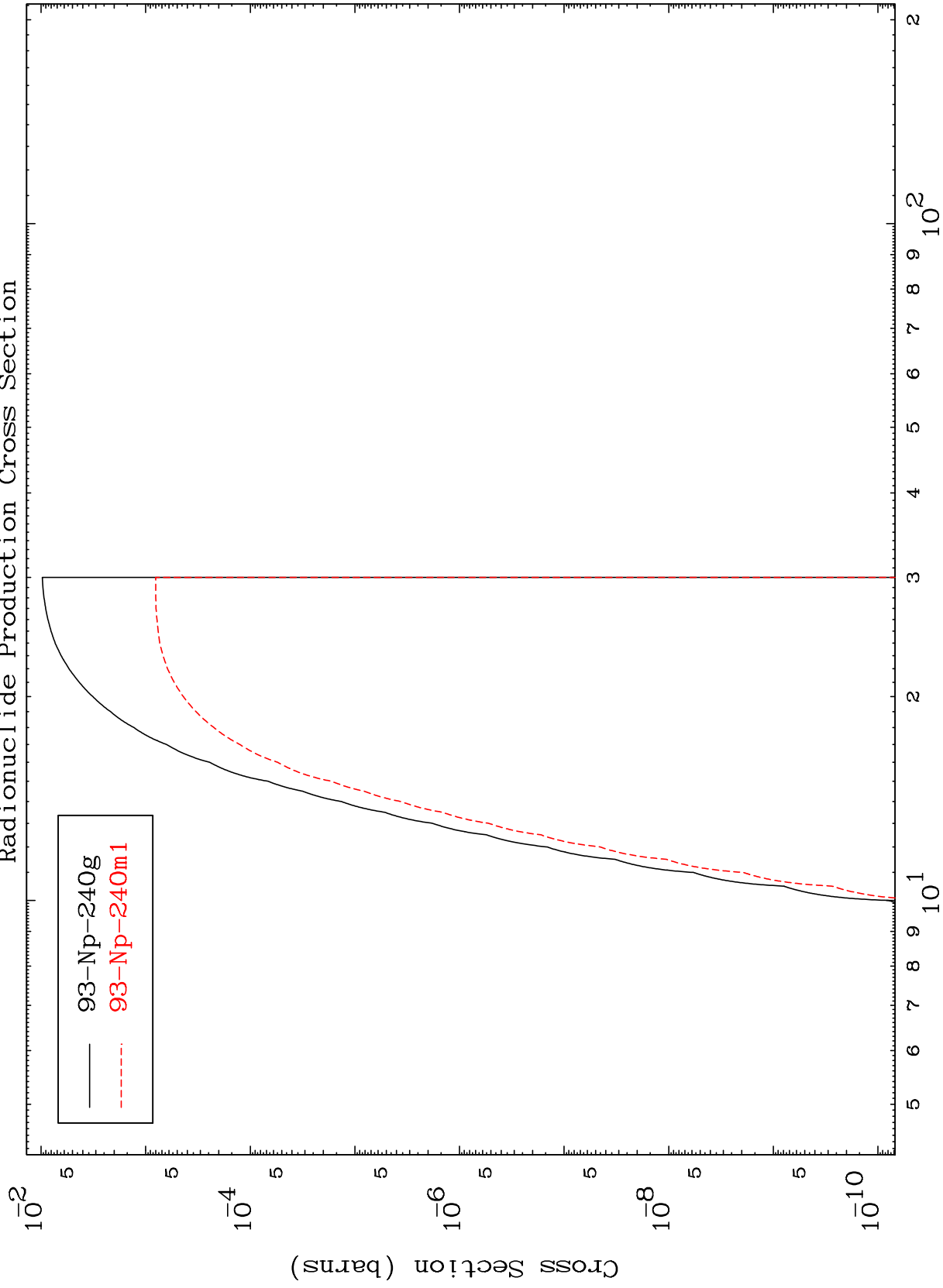


93-Np-240g
93-Np-240m1

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94-Pu-241

(n,d)
Radionuclide Production Cross Section



27

Incident Energy (MeV)

94-Pu-241