

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

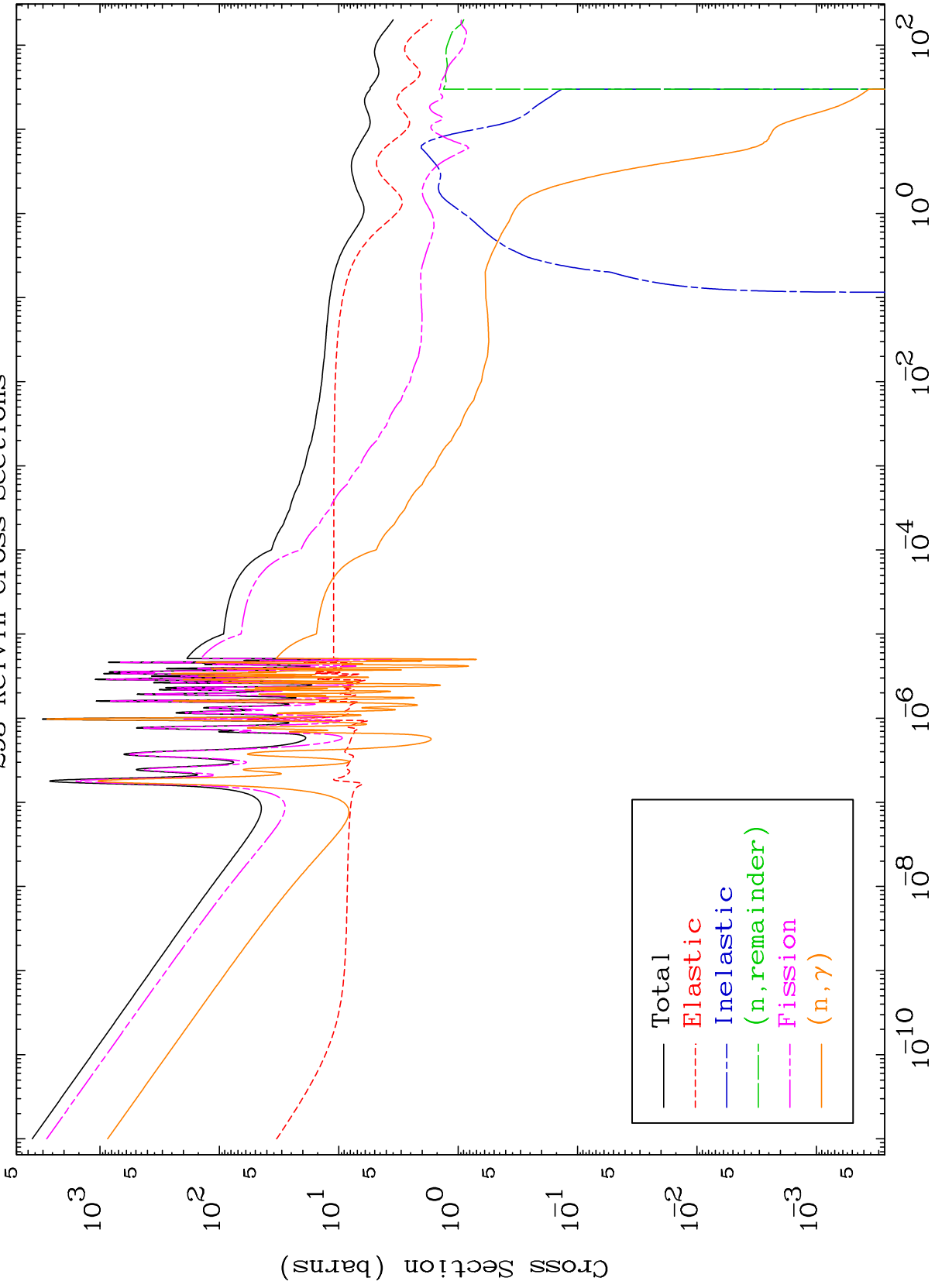
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9204

Major  
293 Kelvin Cross Sections

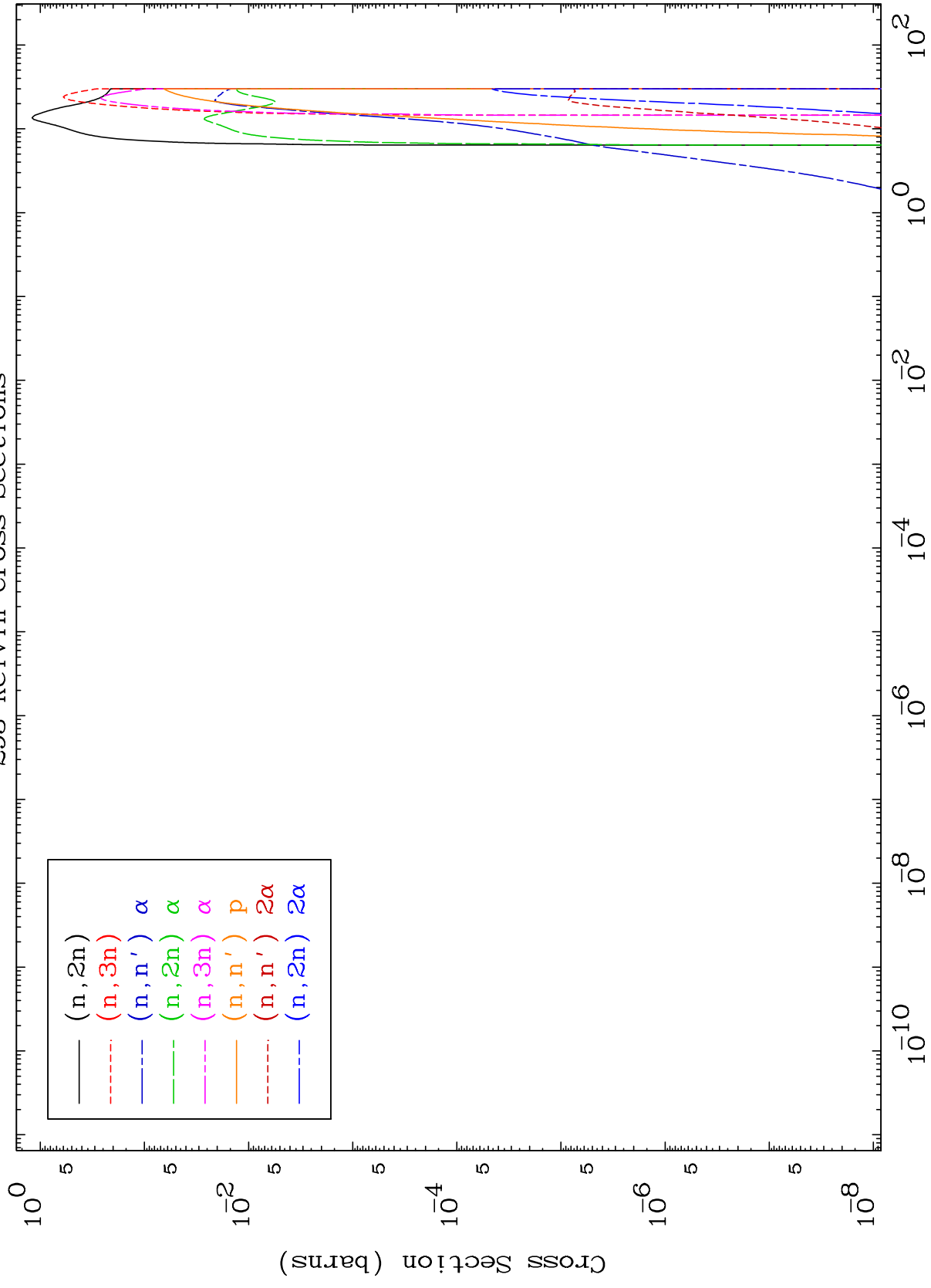
92-U -227



MAT 9204

Neutron Production  
293 Kelvin Cross Sections

92-U -227



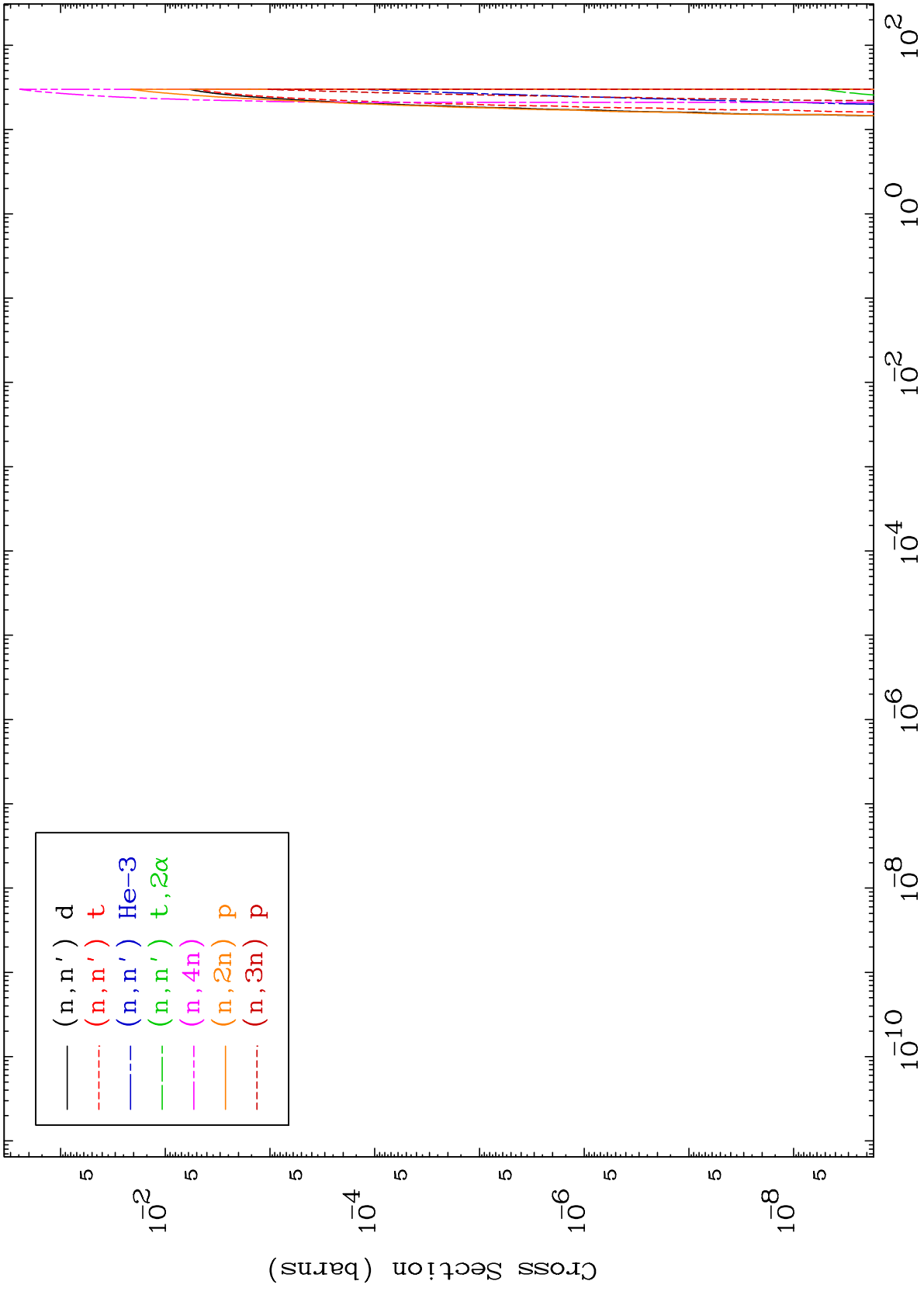
2

92-U -227

MAT 9204

### Neutron Production 293 Kelvin Cross Sections

92-U -227



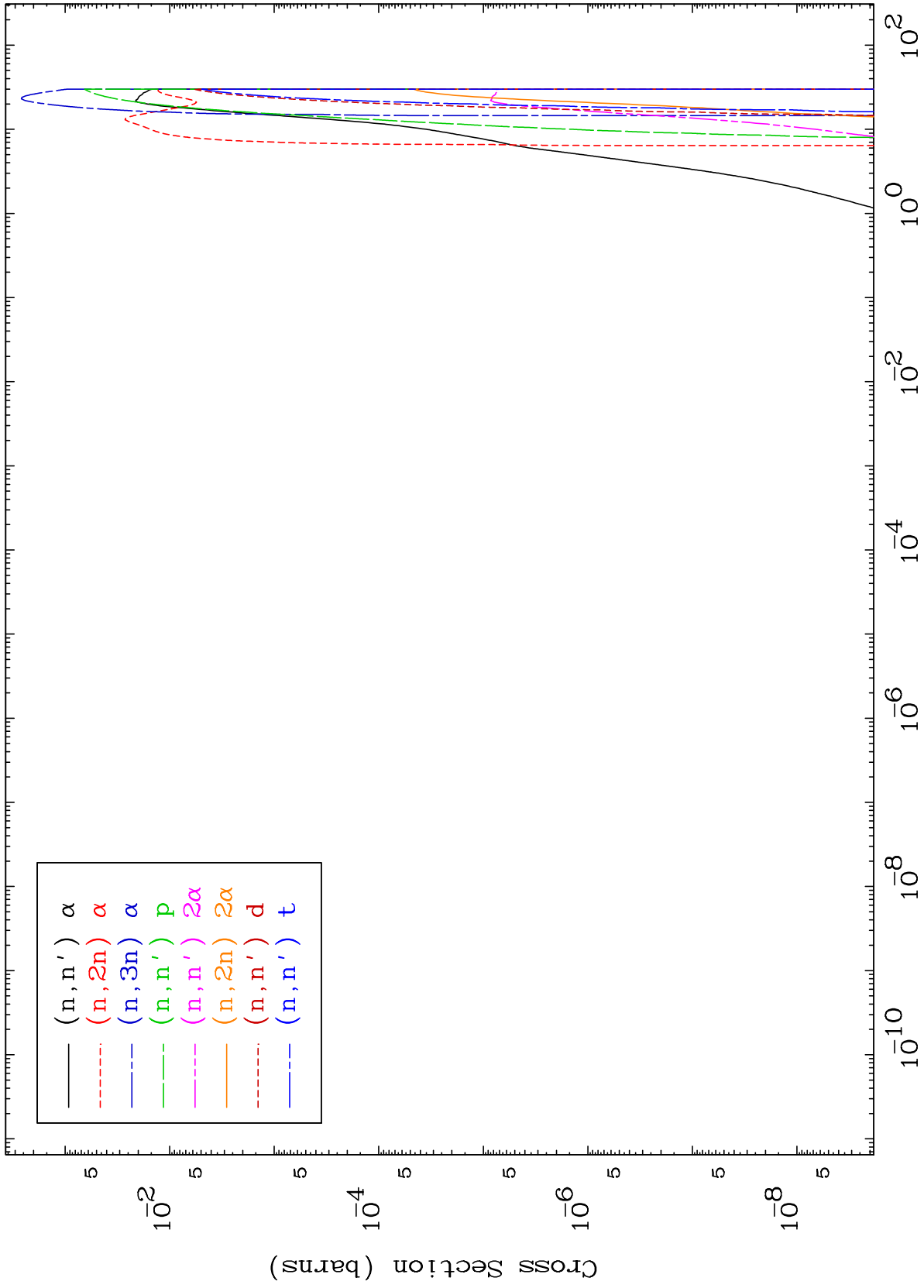
92-U -227

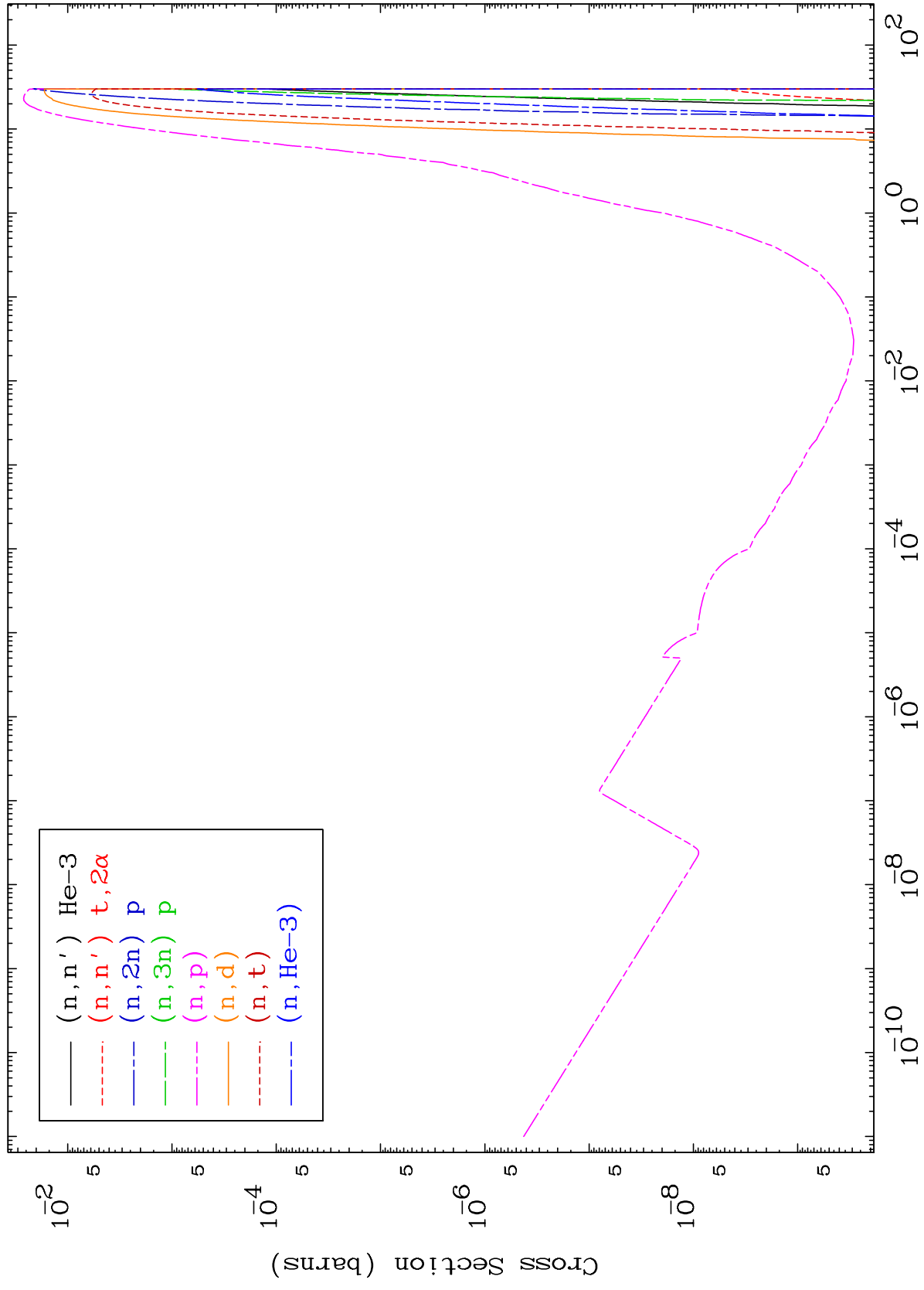
Incident Energy (MeV)

MAT 9204

293 Kelvin Cross Sections

92-U -227

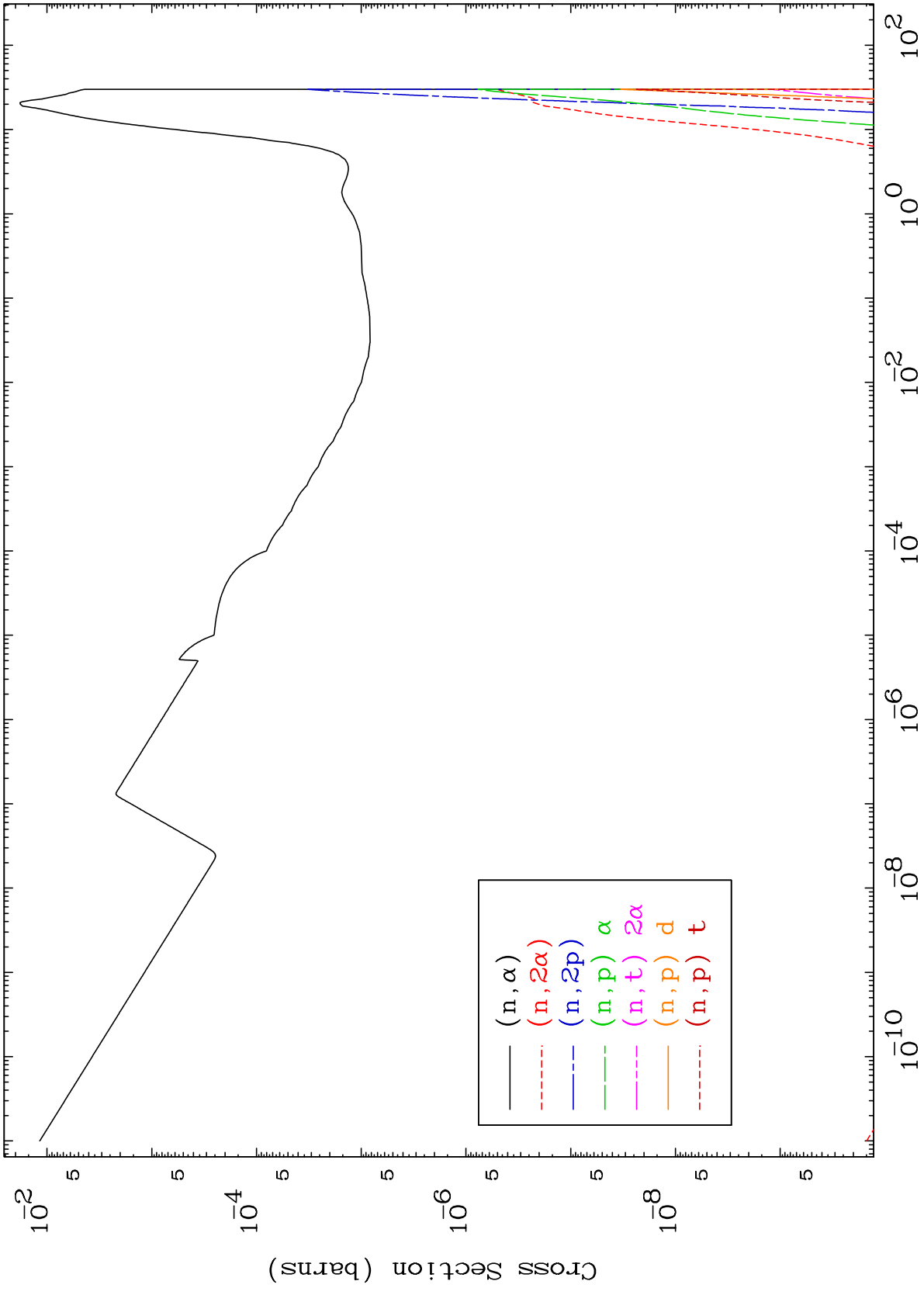




MAT 9204

293 Kelvin Cross Sections

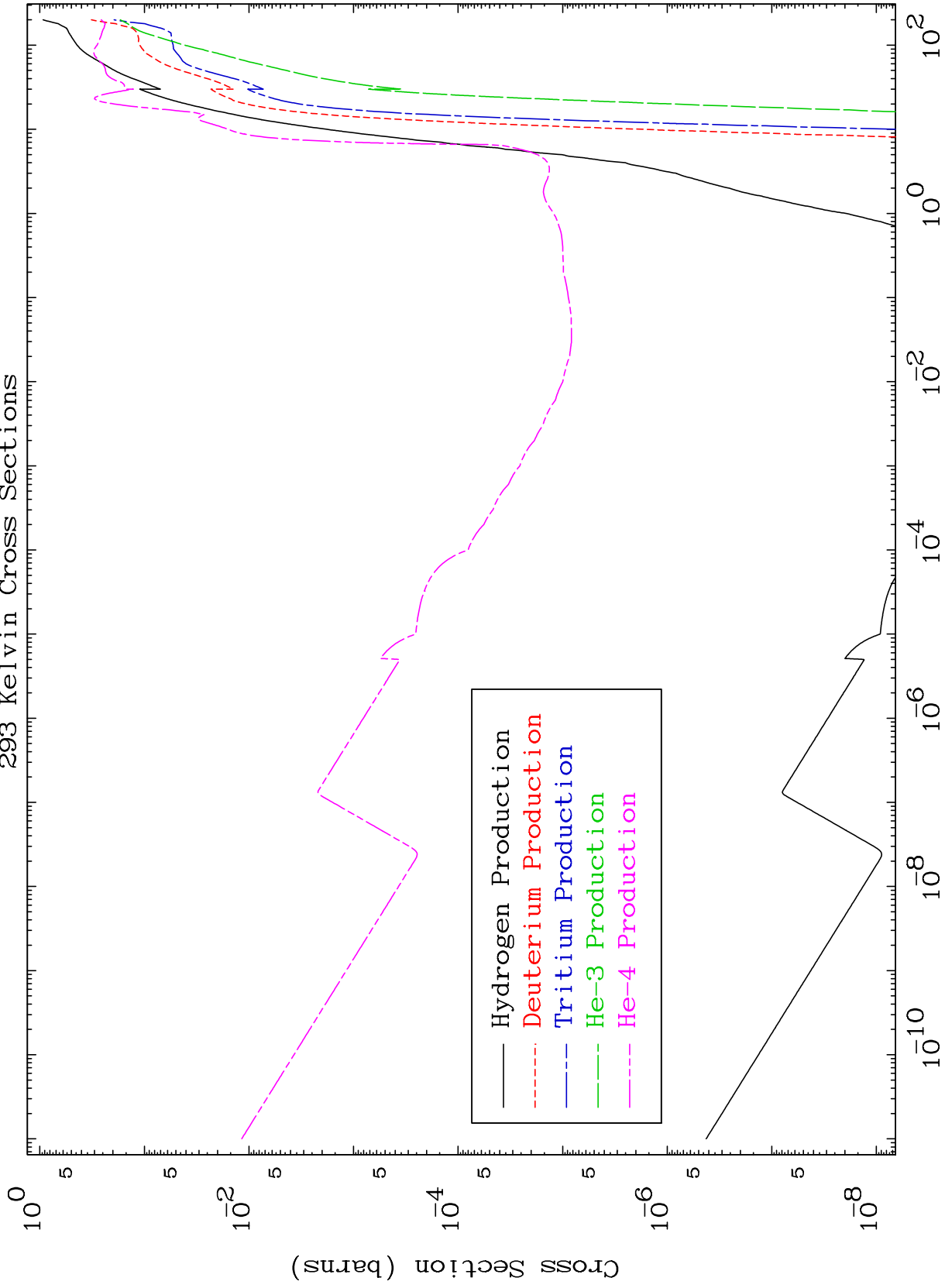
92-U -227



MAT 9204

Particle Production  
293 Kelvin Cross Sections

92-U -227



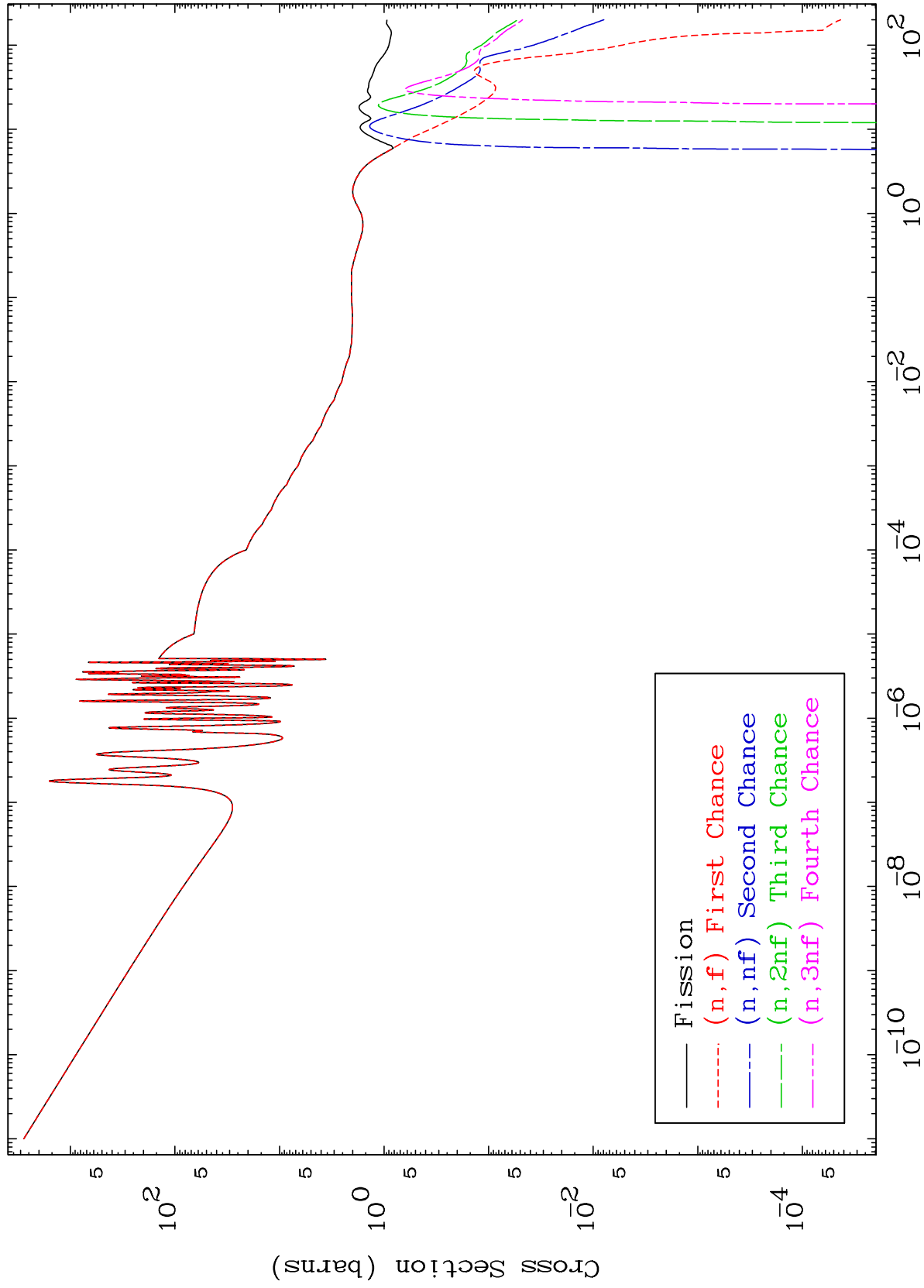
7



MAT 9204

Fission  
293 Kelvin Cross Sections

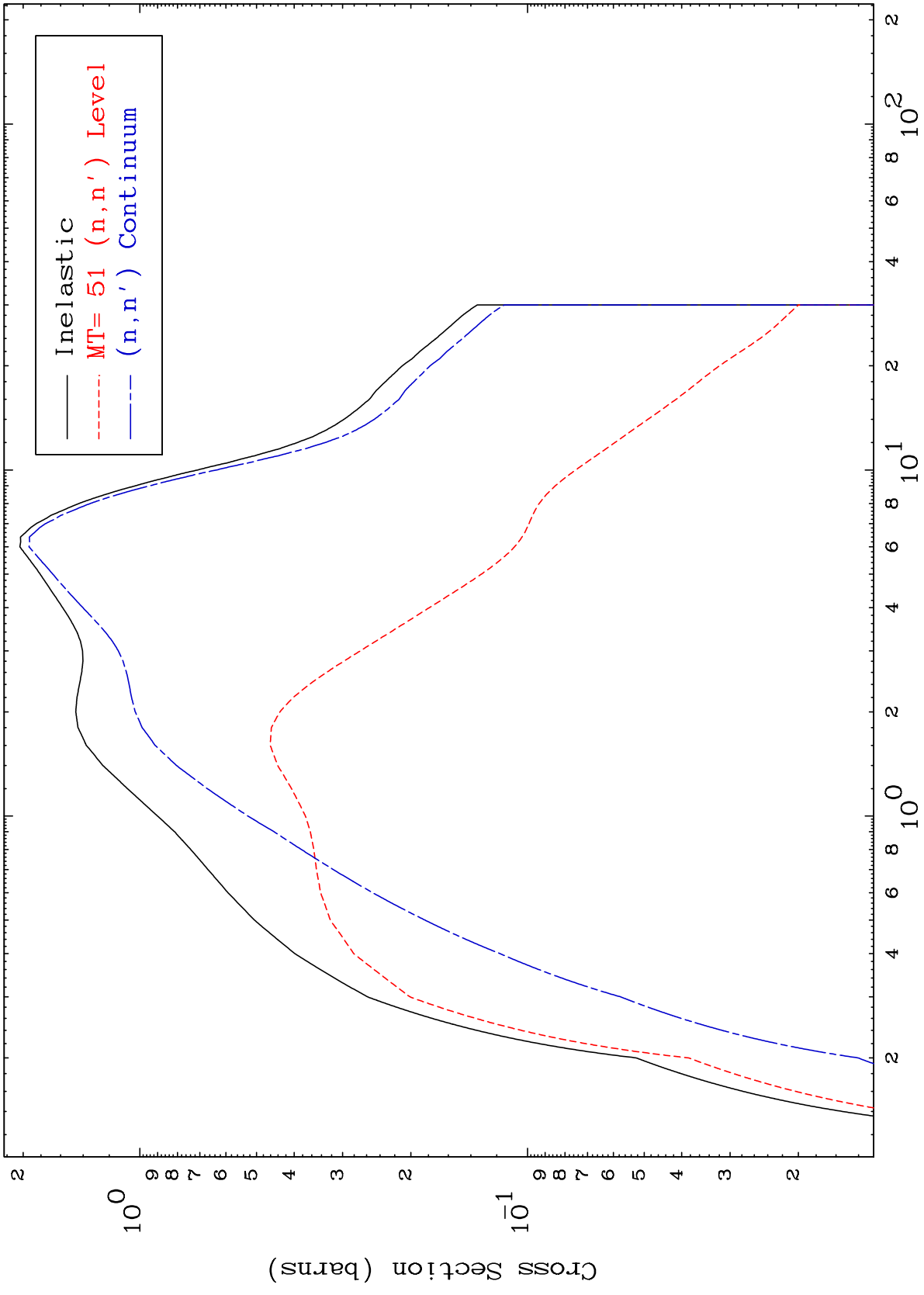
92-U -227



MAT 9204

(n,n') Level  
293 Kelvin Cross Sections

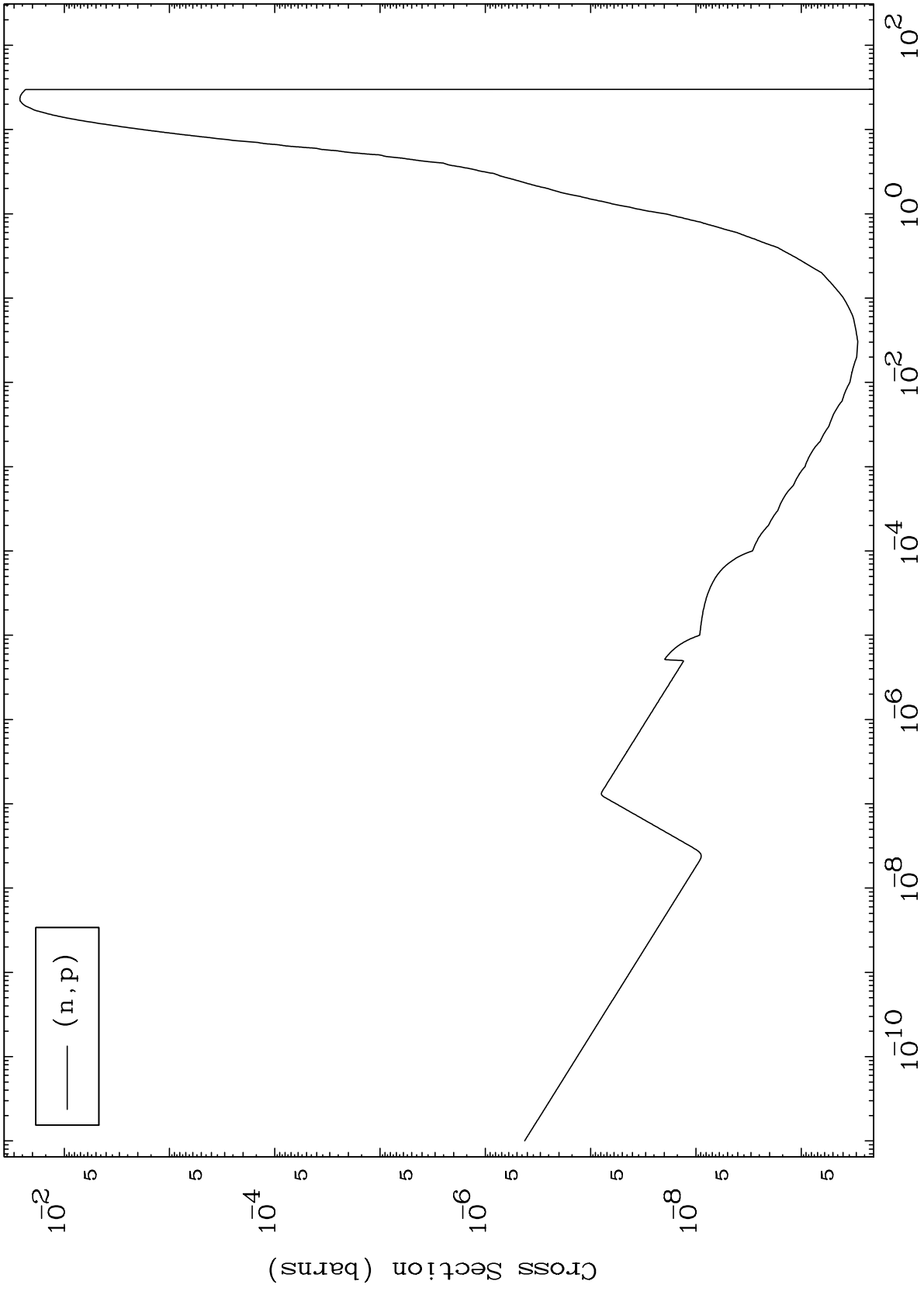
92-U -227



MAT 9204

(n,p) Levels  
293 Kelvin Cross Sections

92-U -227



10

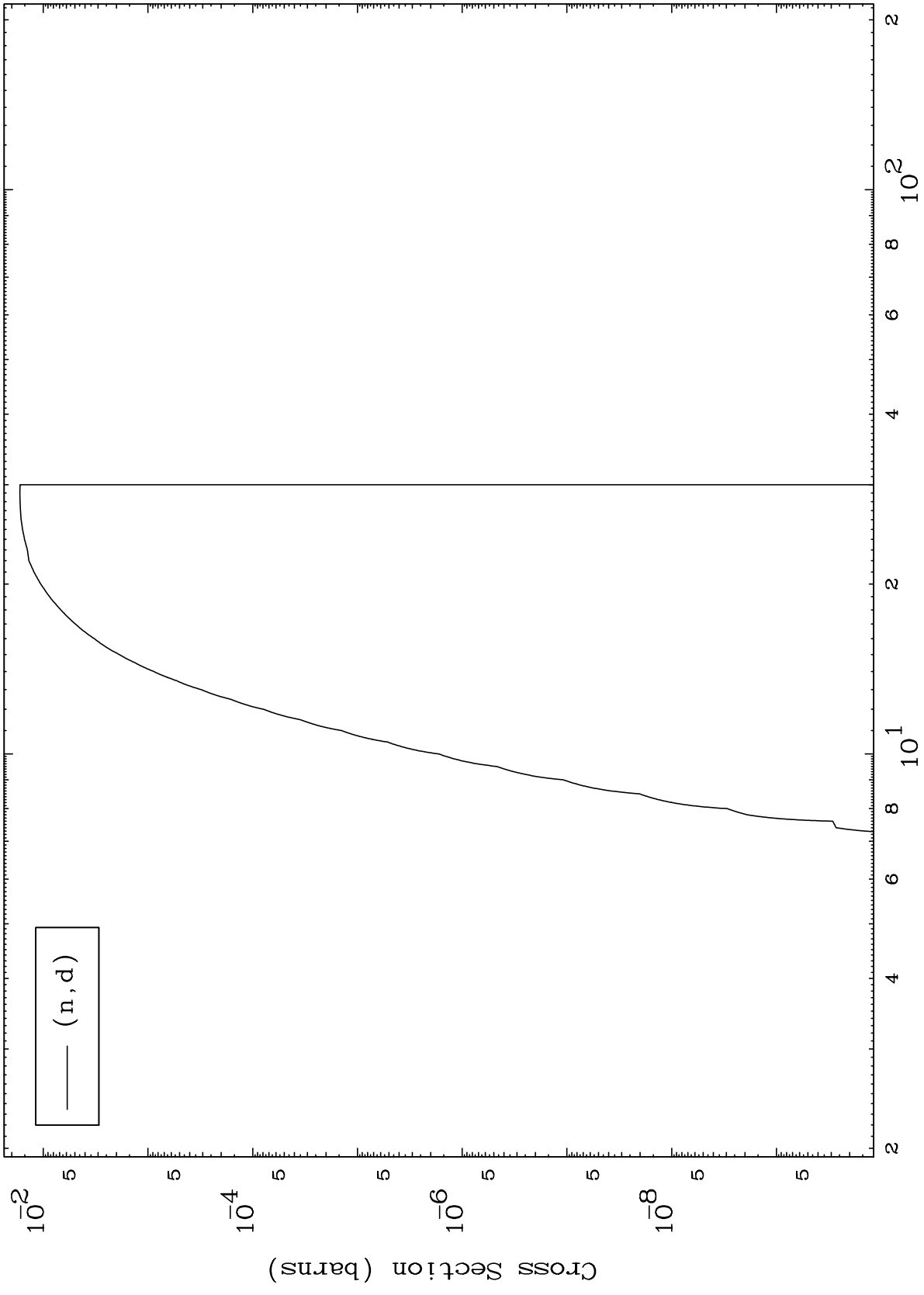
Incident Energy (MeV)

92-U -227

MAT 9204

(n,d) Levels  
293 Kelvin Cross Sections

92-U -227



11

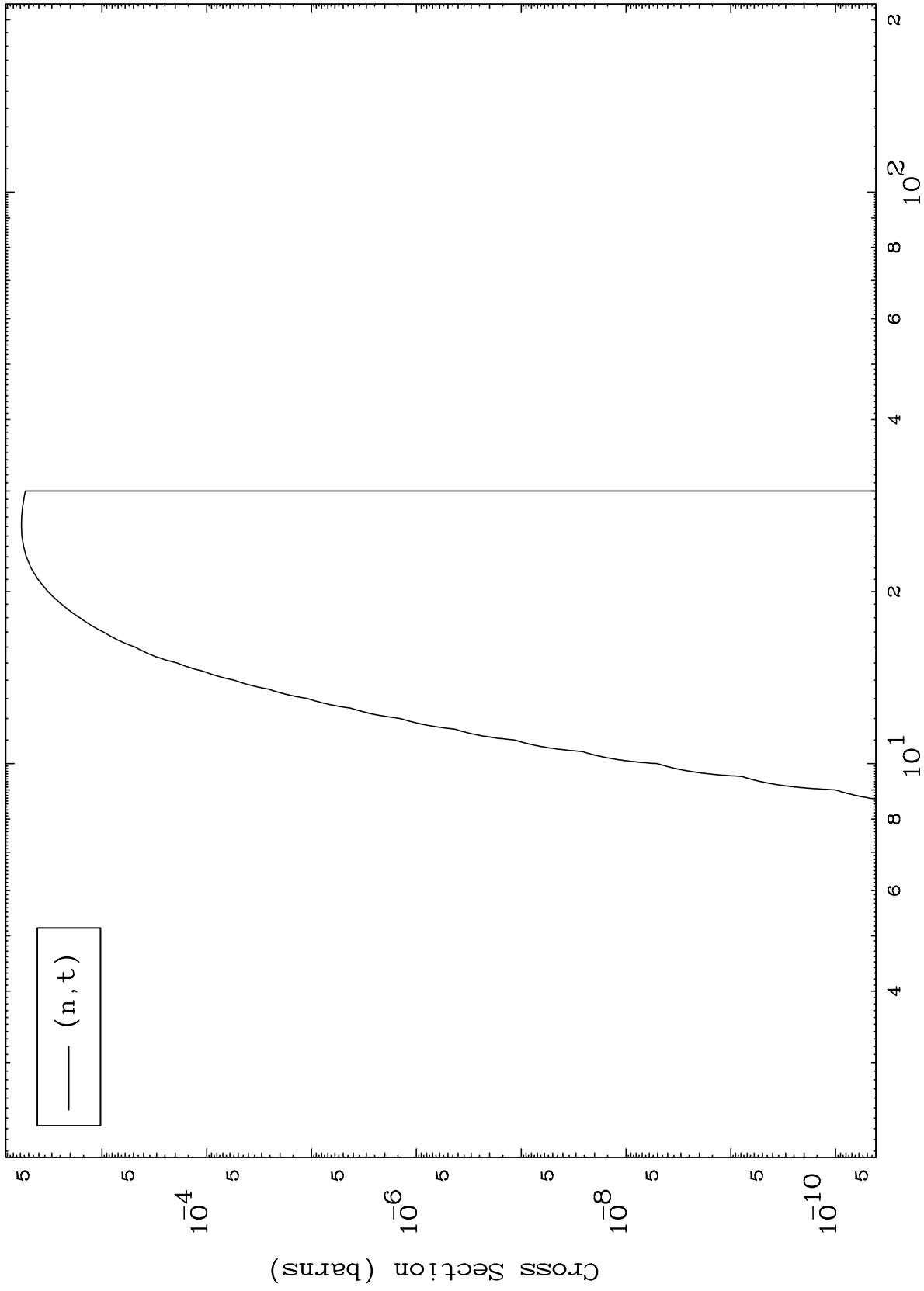
Incident Energy (MeV)

92-U -227

MAT 9204

(n,t) Levels  
293 Kelvin Cross Sections

92-U -227



12

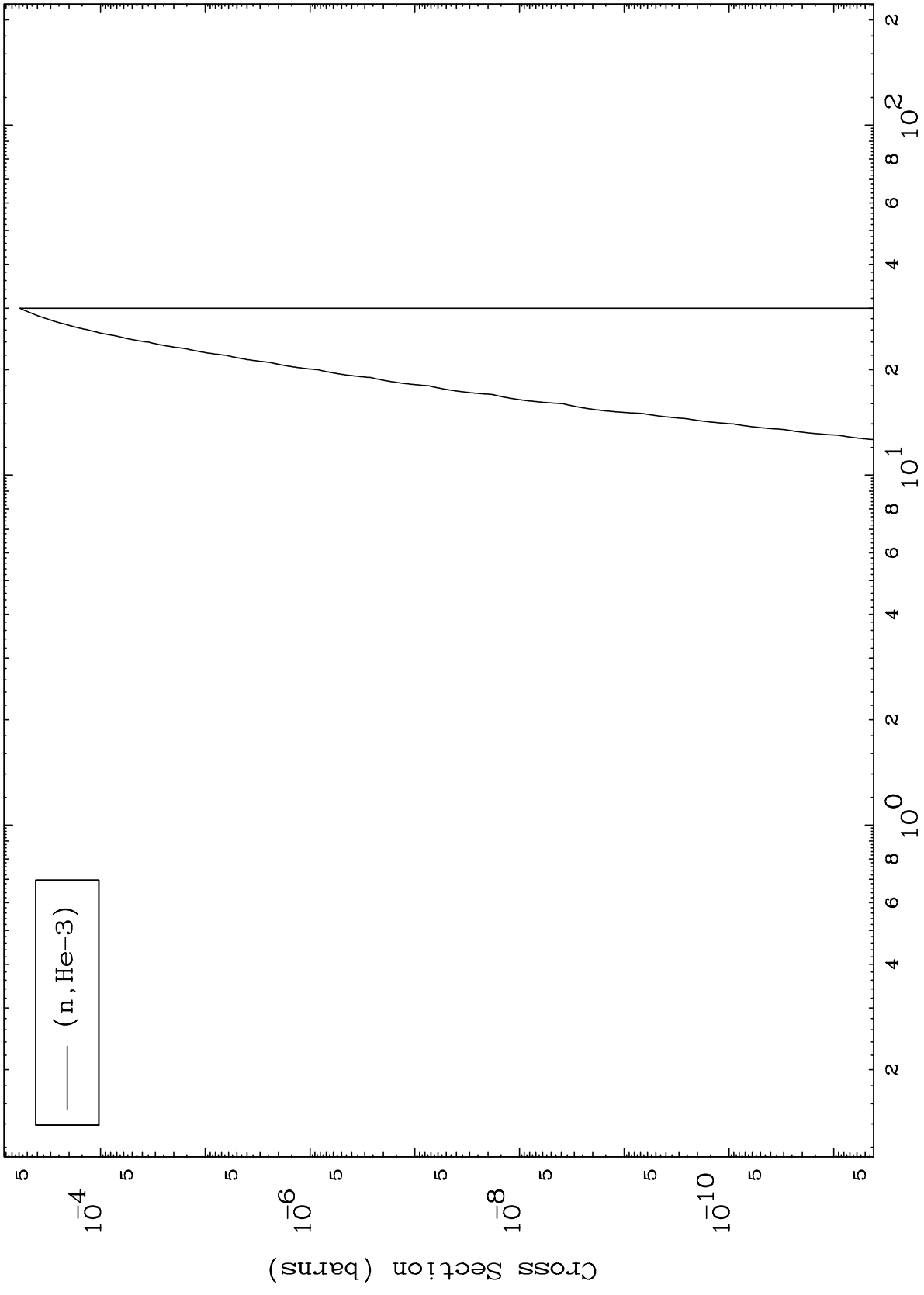
Incident Energy (MeV)

92-U -227

MAT 9204

(n,He3) Levels  
293 Kelvin Cross Sections

92-U -227



13

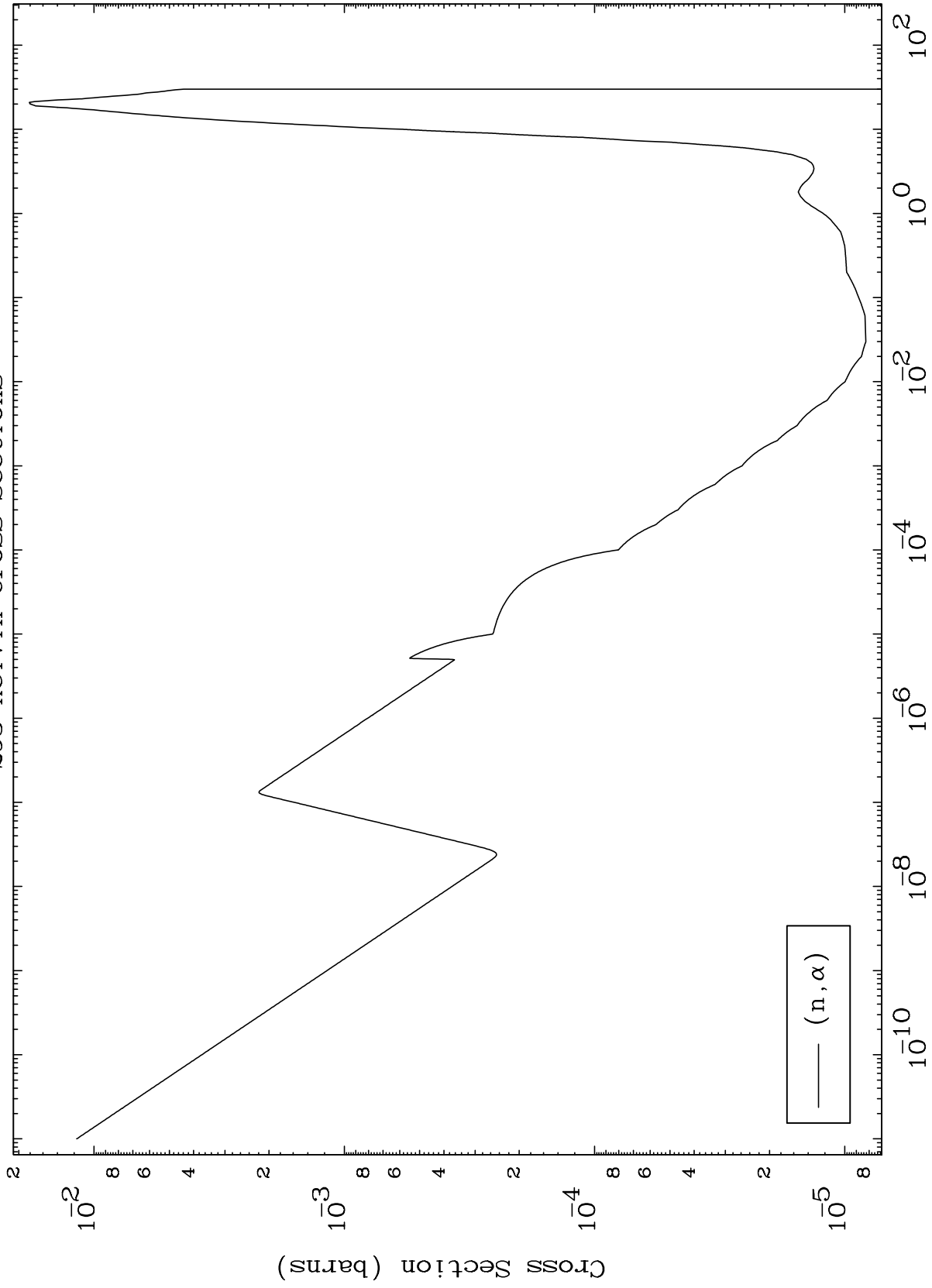
Incident Energy (MeV)

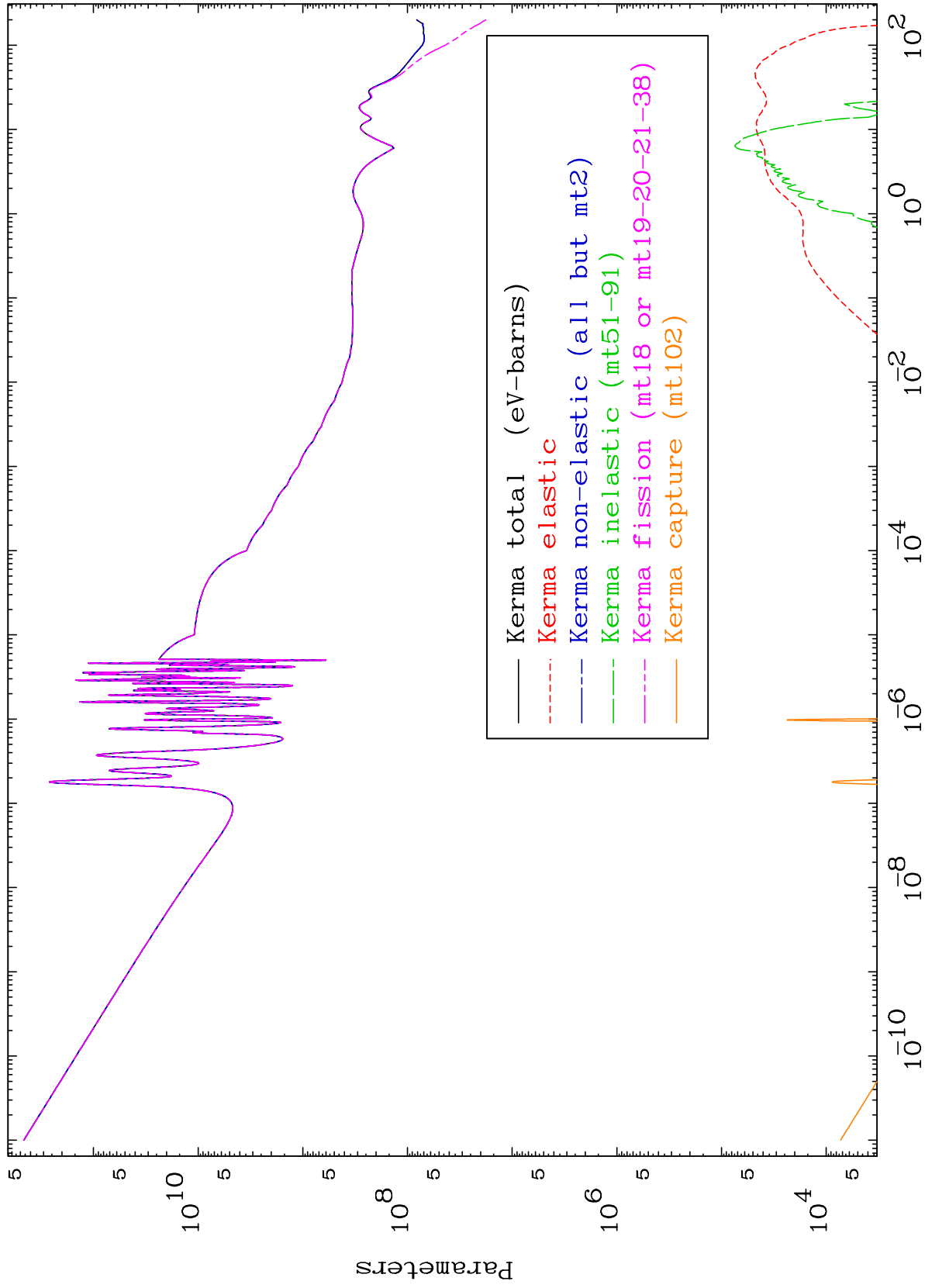
92-U -227

MAT 9204

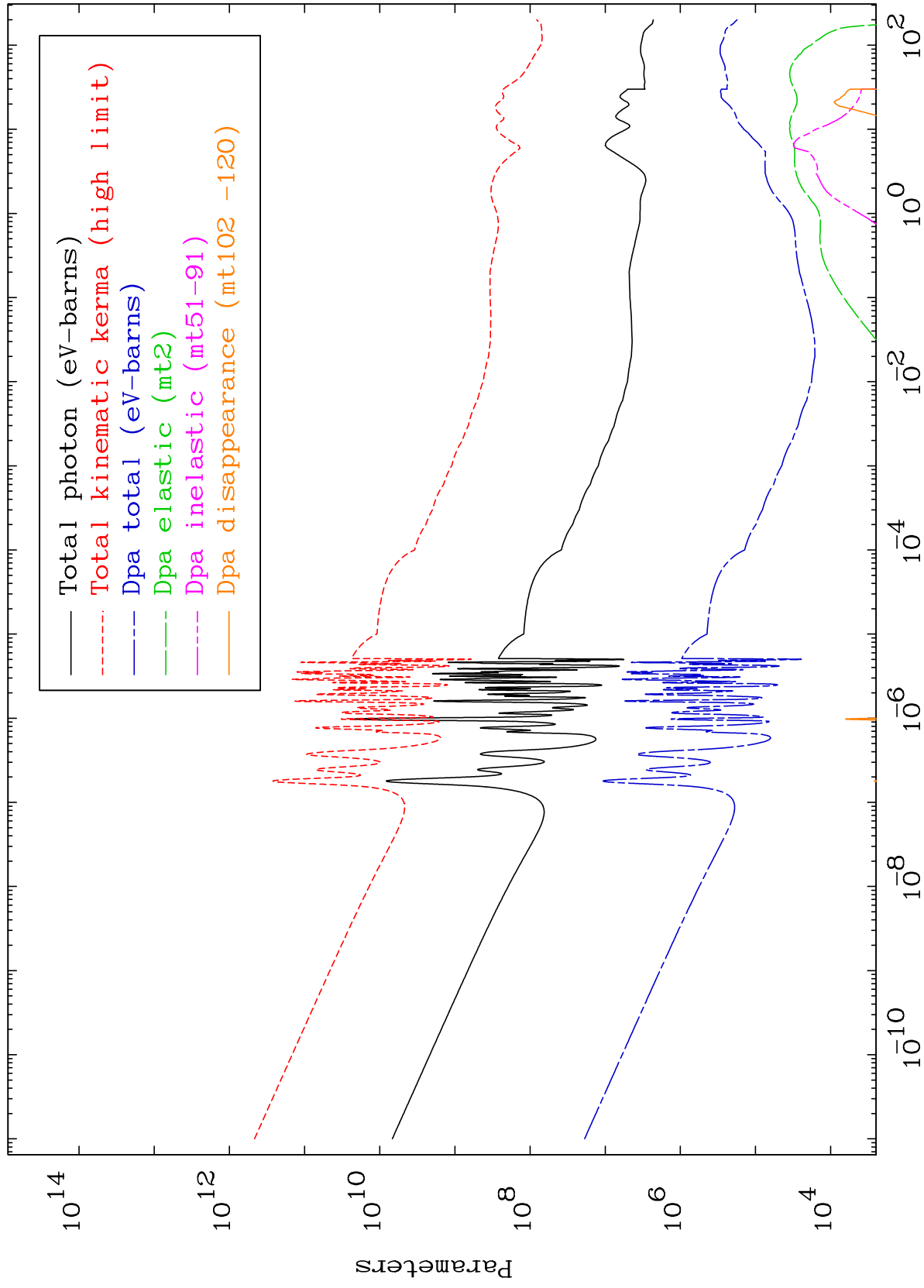
(n,α) Levels  
293 Kelvin Cross Sections

92-U -227





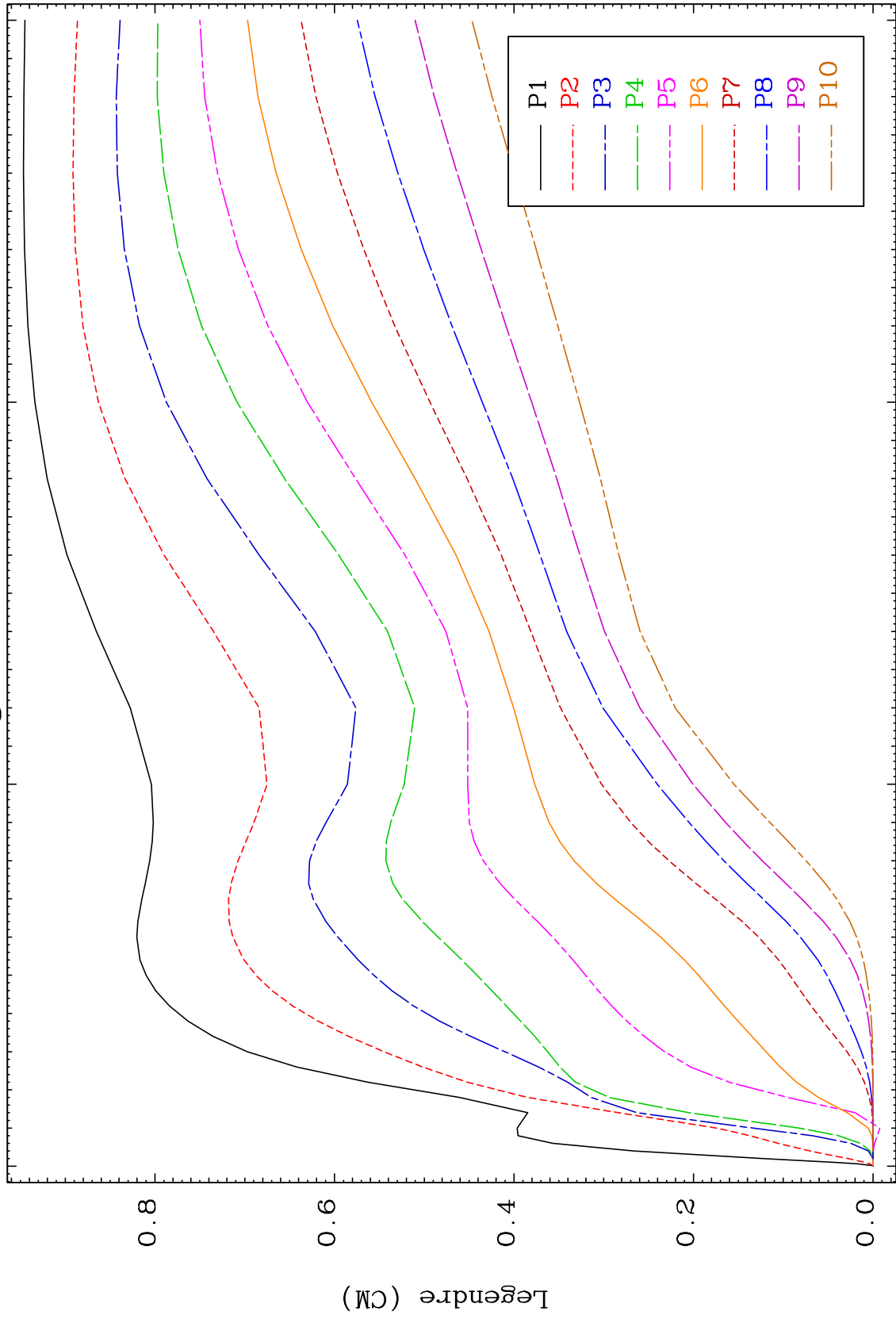




MAT 9204

Elastic Legendre Coefficients

92-U -227



17

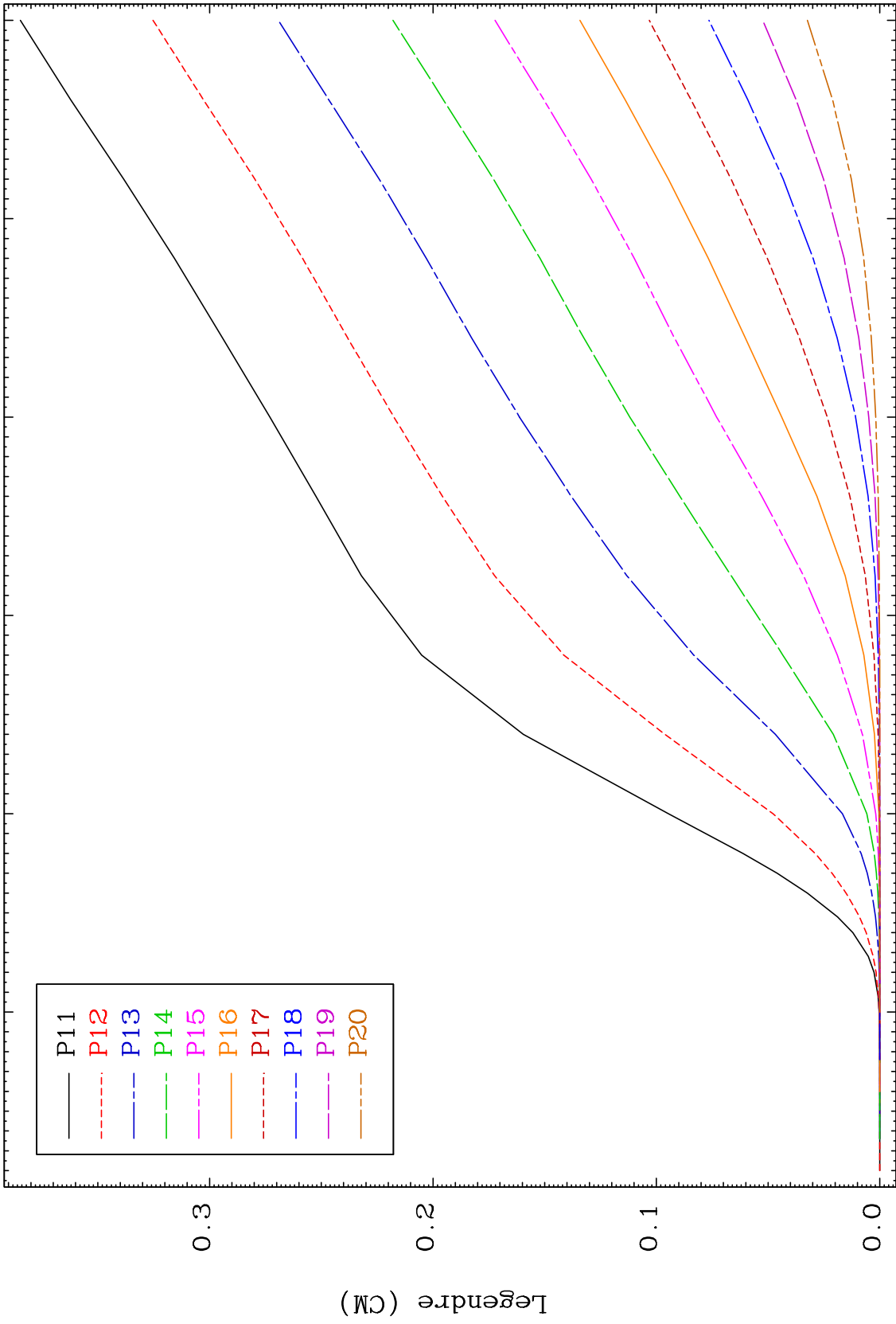
Incident Energy (MeV)

92-U -227

MAT 9204

Elastic Legendre Coefficients

92-U -227



18

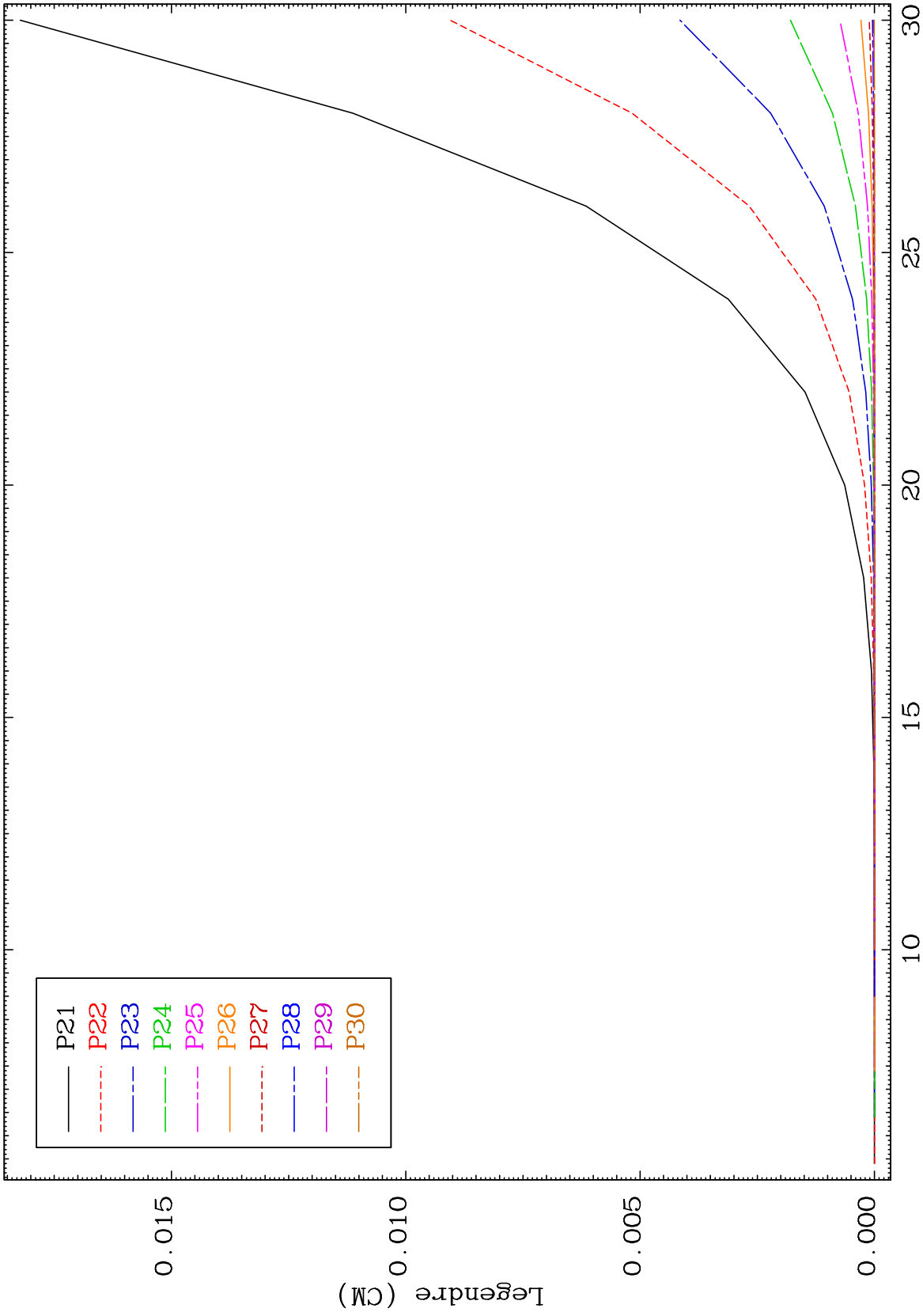
Incident Energy (MeV)

92-U -227

MAT 9204

### Elastic Legendre Coefficients

92-U -227



19

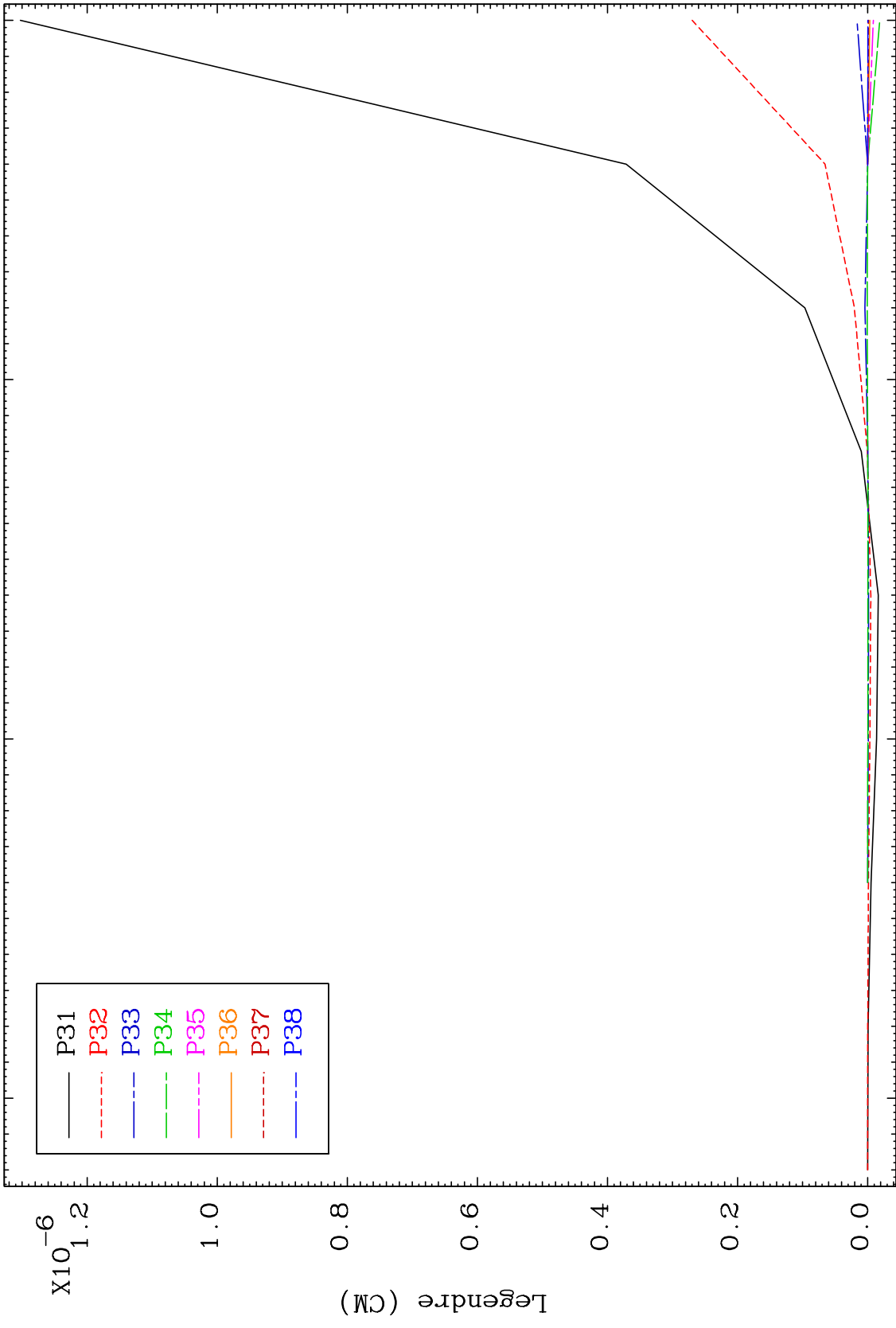
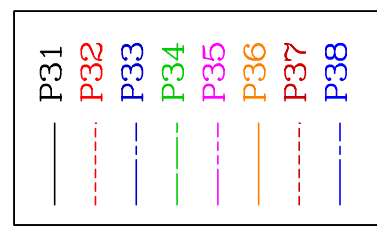
Incident Energy (MeV)

92-U -227

MAT 9204

Elastic Legendre Coefficients

92-U -227



20

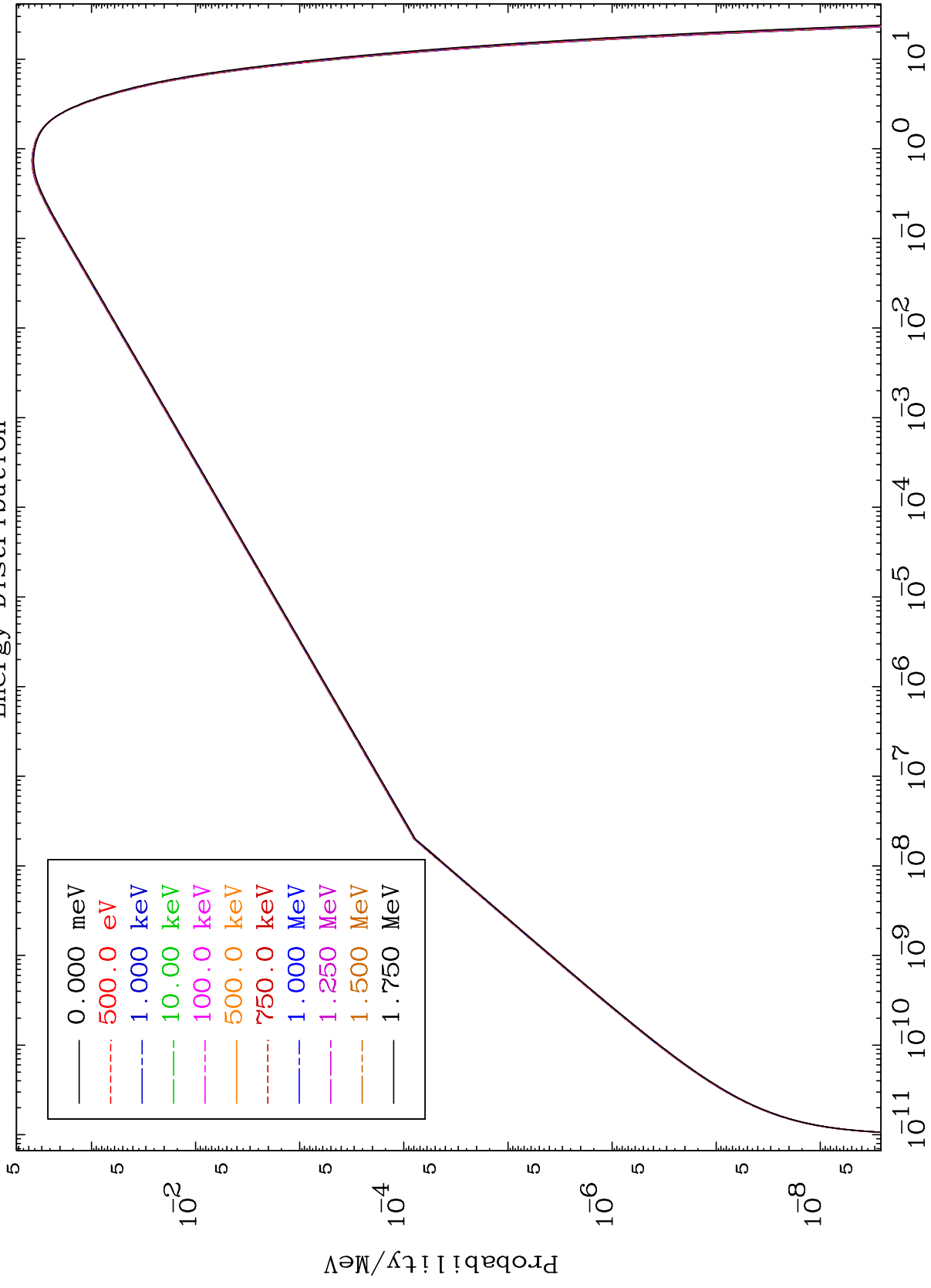
Incident Energy (MeV)

92-U -227

MAT 9204

### Fission Energy Distribution

92-U -227

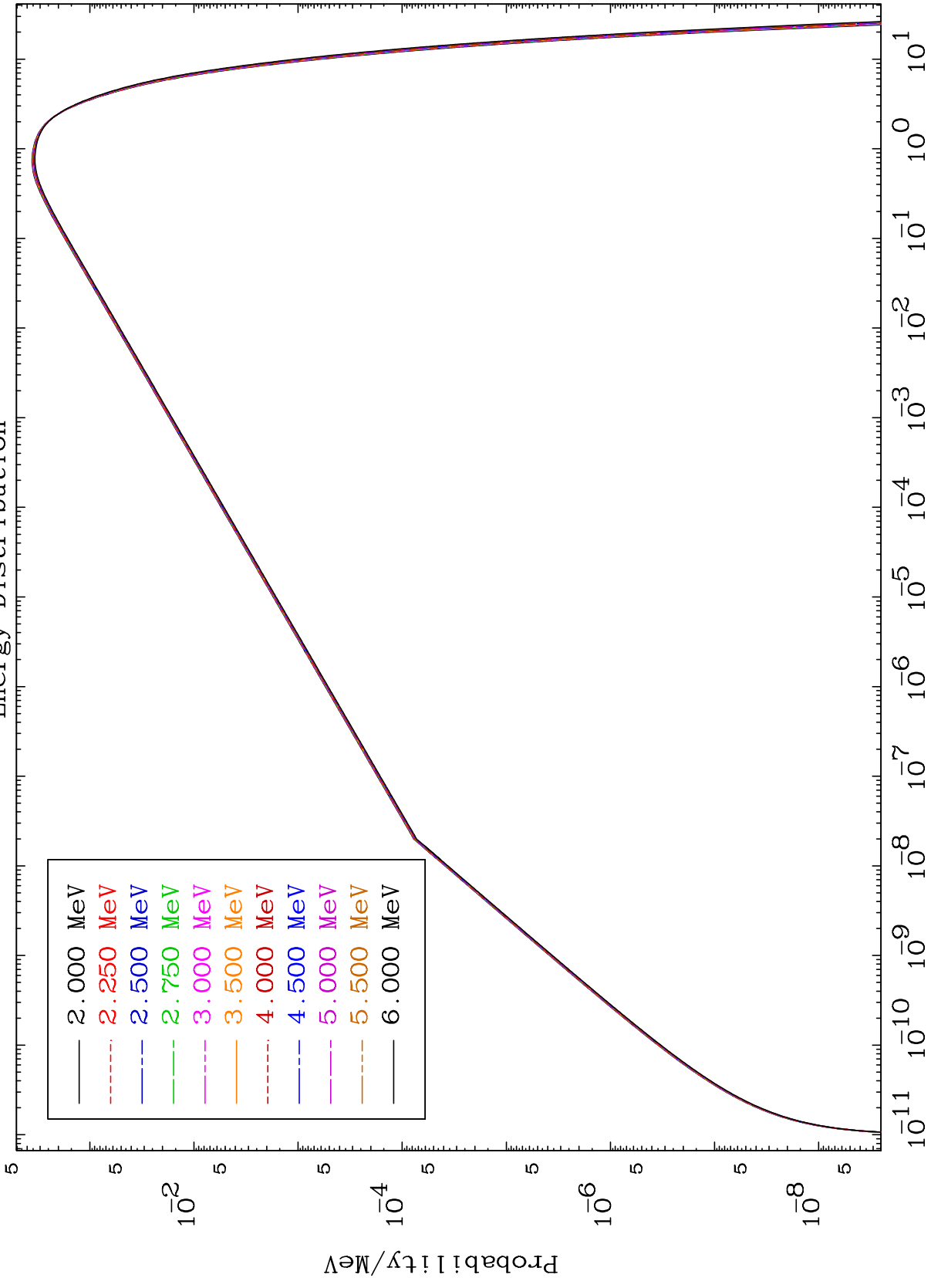


92-U -227

MAT 9204

### Fission Energy Distribution

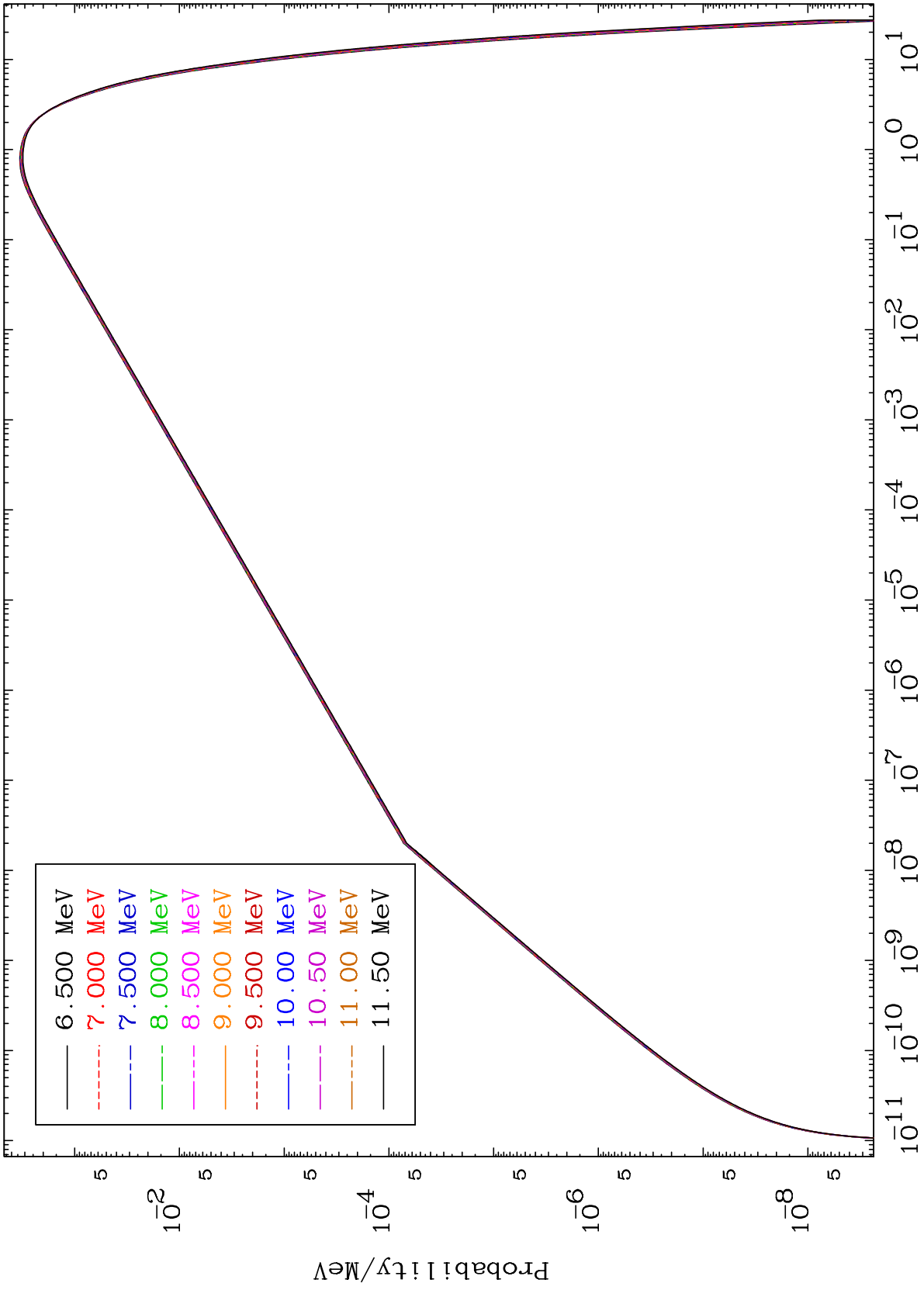
92-U -227



MAT 9204

# Fission Energy Distribution

92-U -227



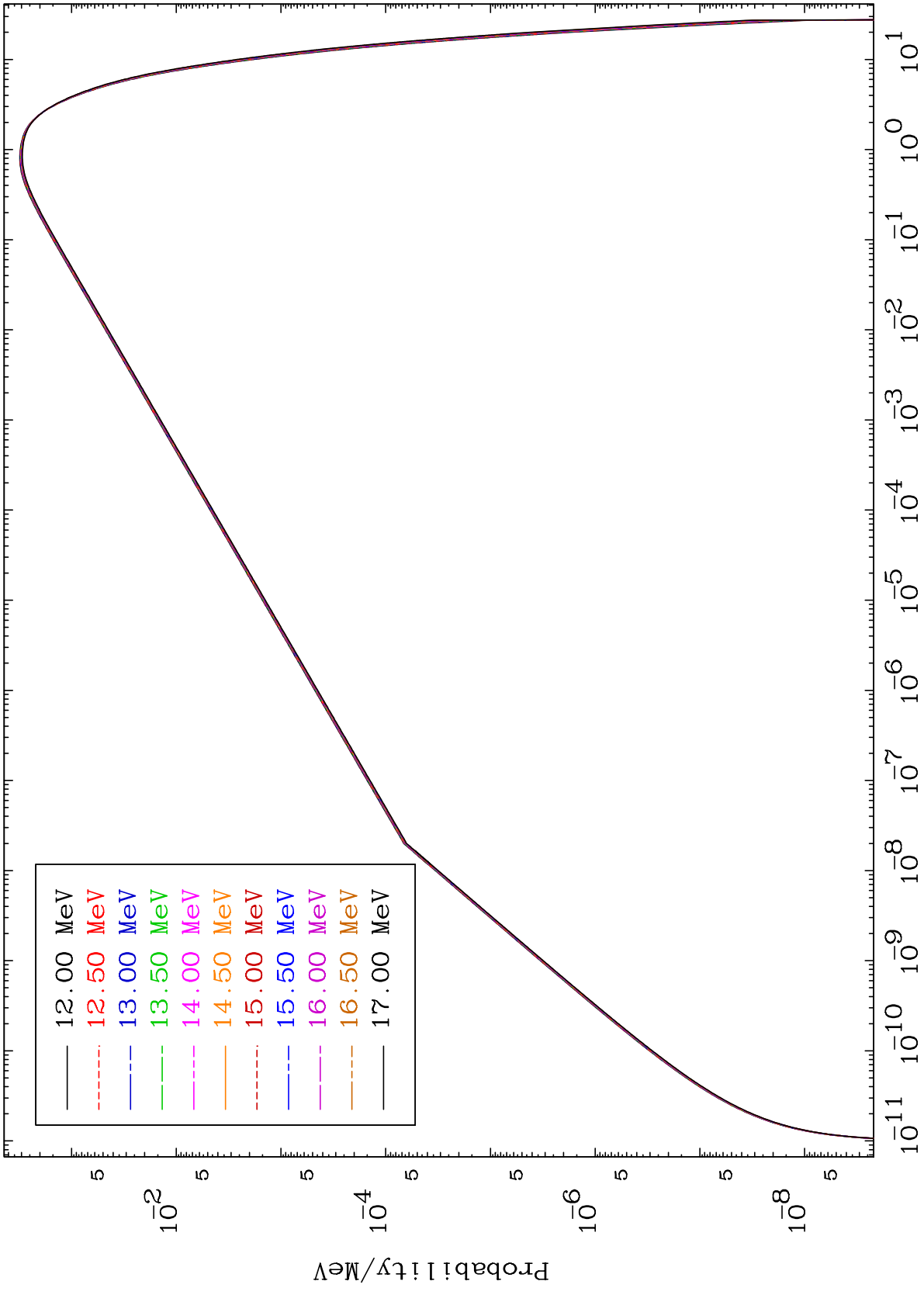
92-U -227



MAT 9204

### Fission Energy Distribution

92-U -227

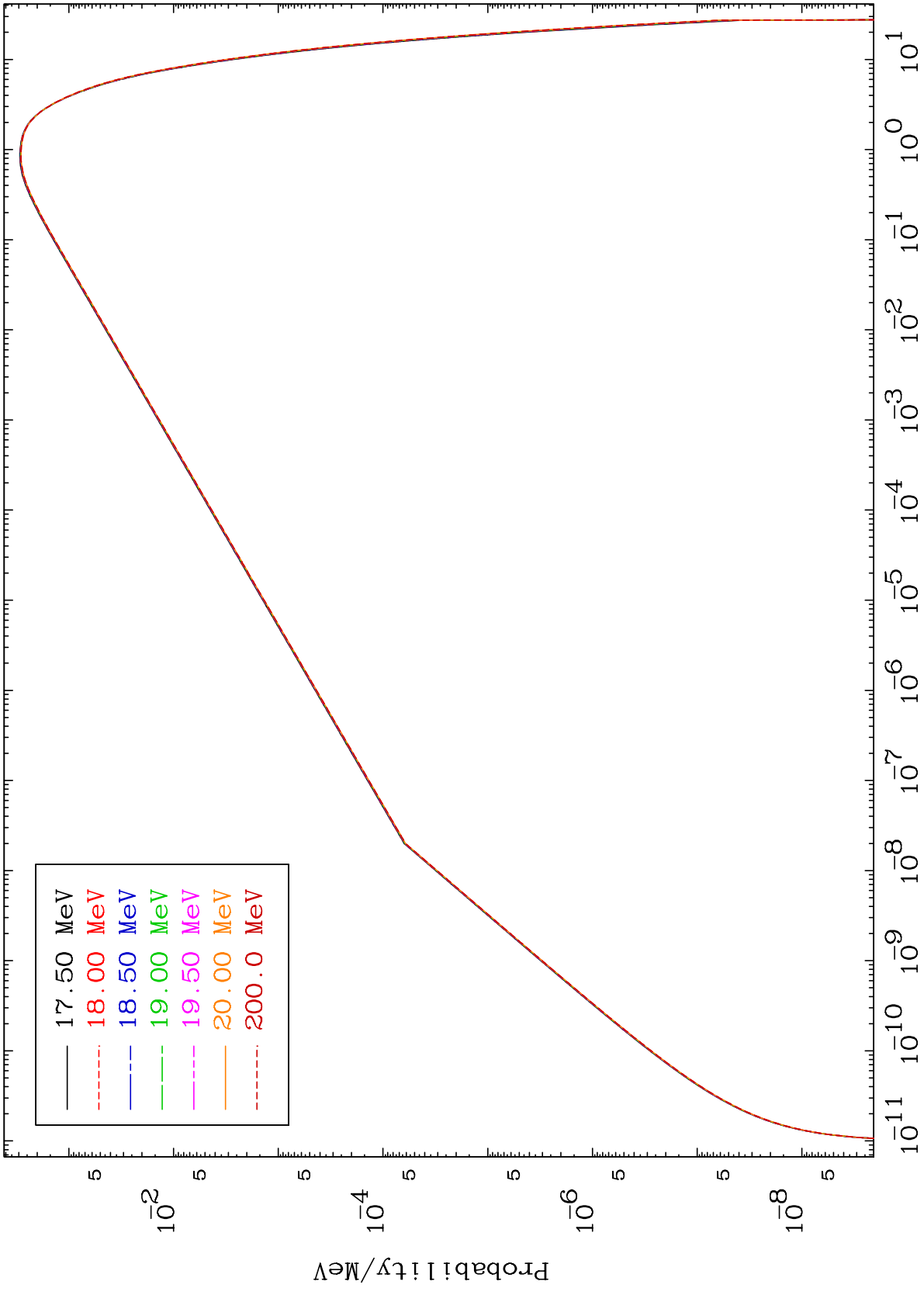


92-U -227

MAT 9204

### Fission Energy Distribution

92-U -227



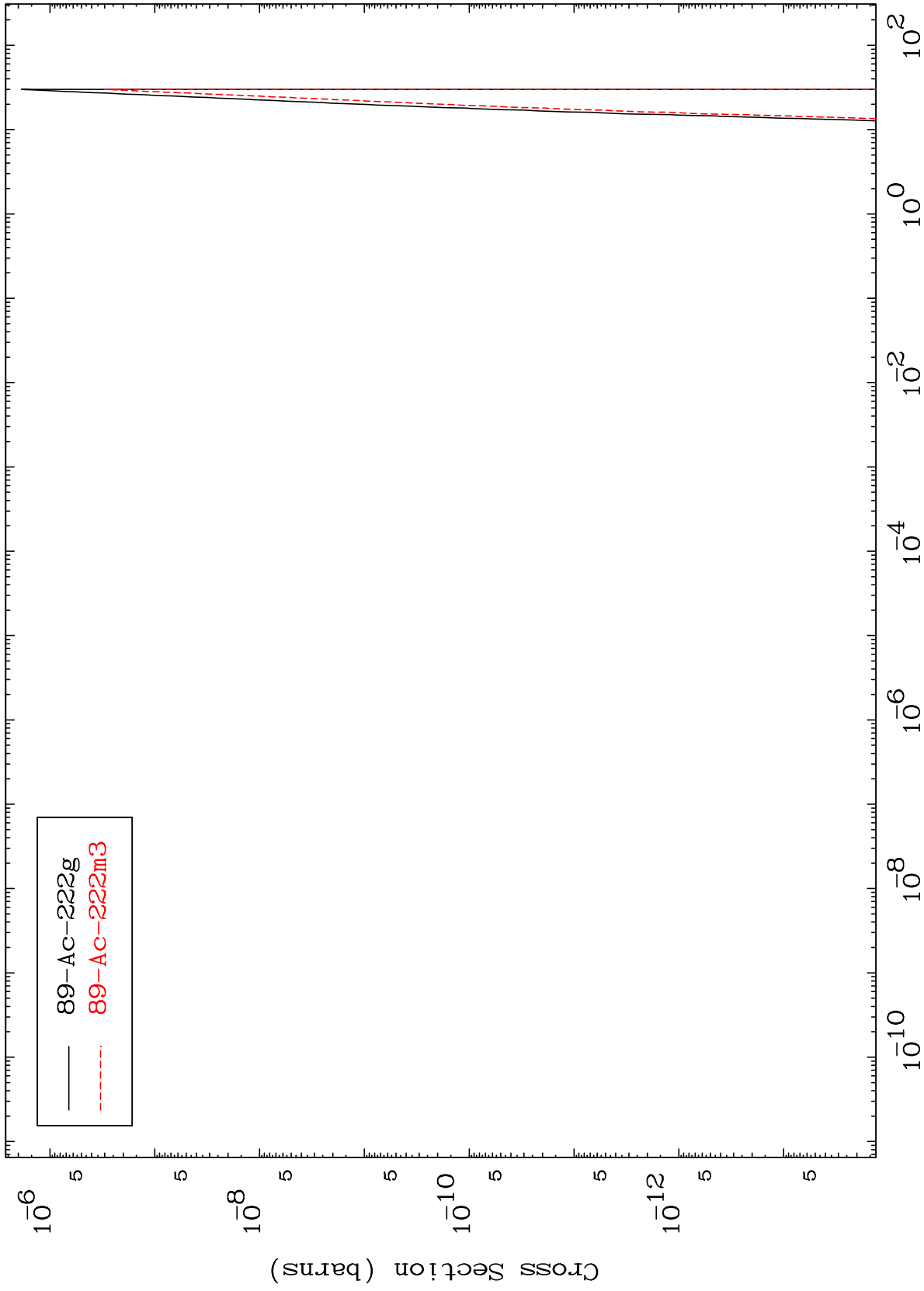
92-U -227

MAT 9204

(n,n') p  $\alpha$

92-U -227

Radionuclide Production Cross Section



26

Incident Energy (MeV)

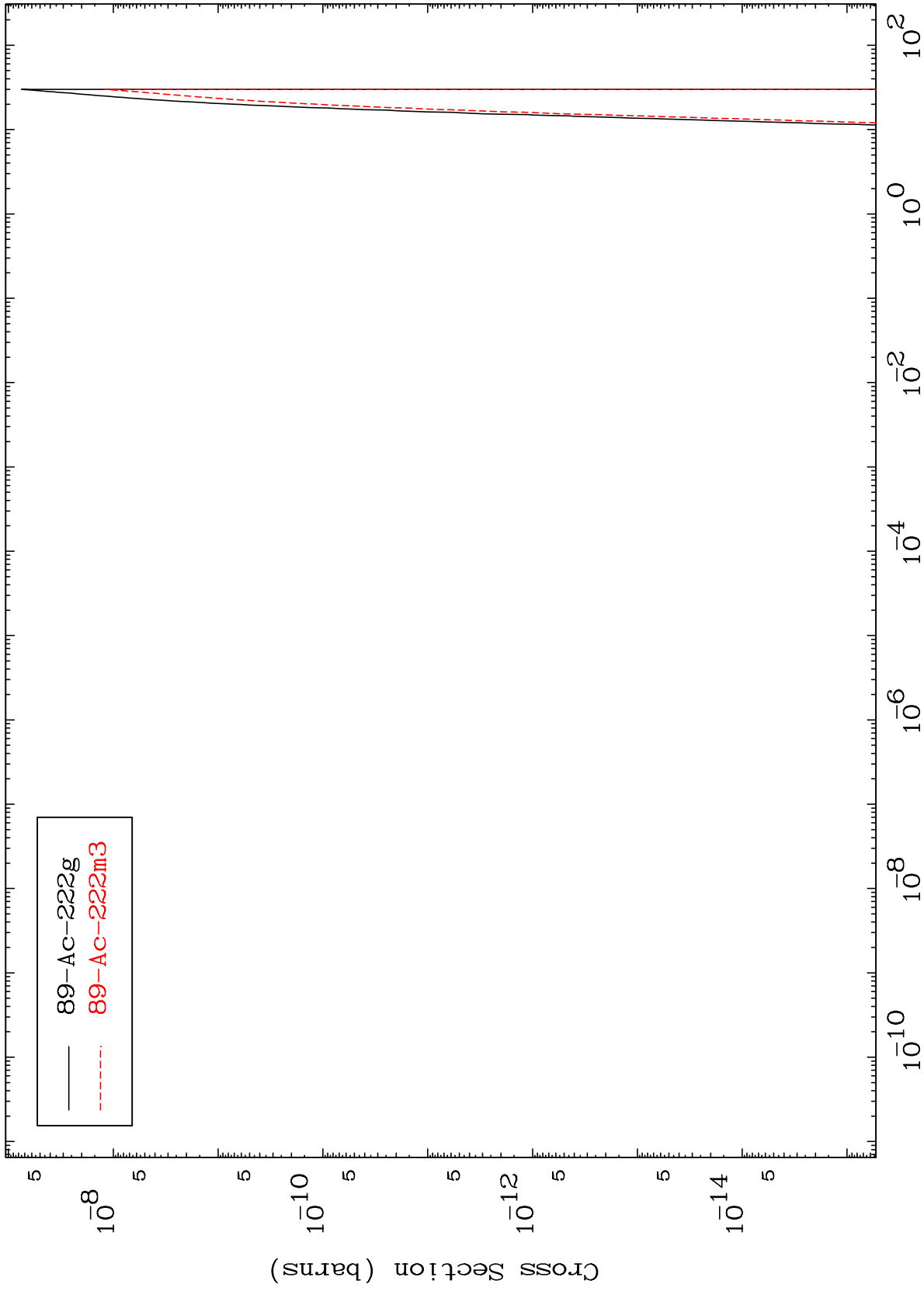
92-U -227

MAT 9204

(n,d)  $\alpha$

92-U -227

### Radionuclide Production Cross Section



27

Incident Energy (MeV)

92-U -227