

Program EVALPLOT
(Version 2018-1)

by

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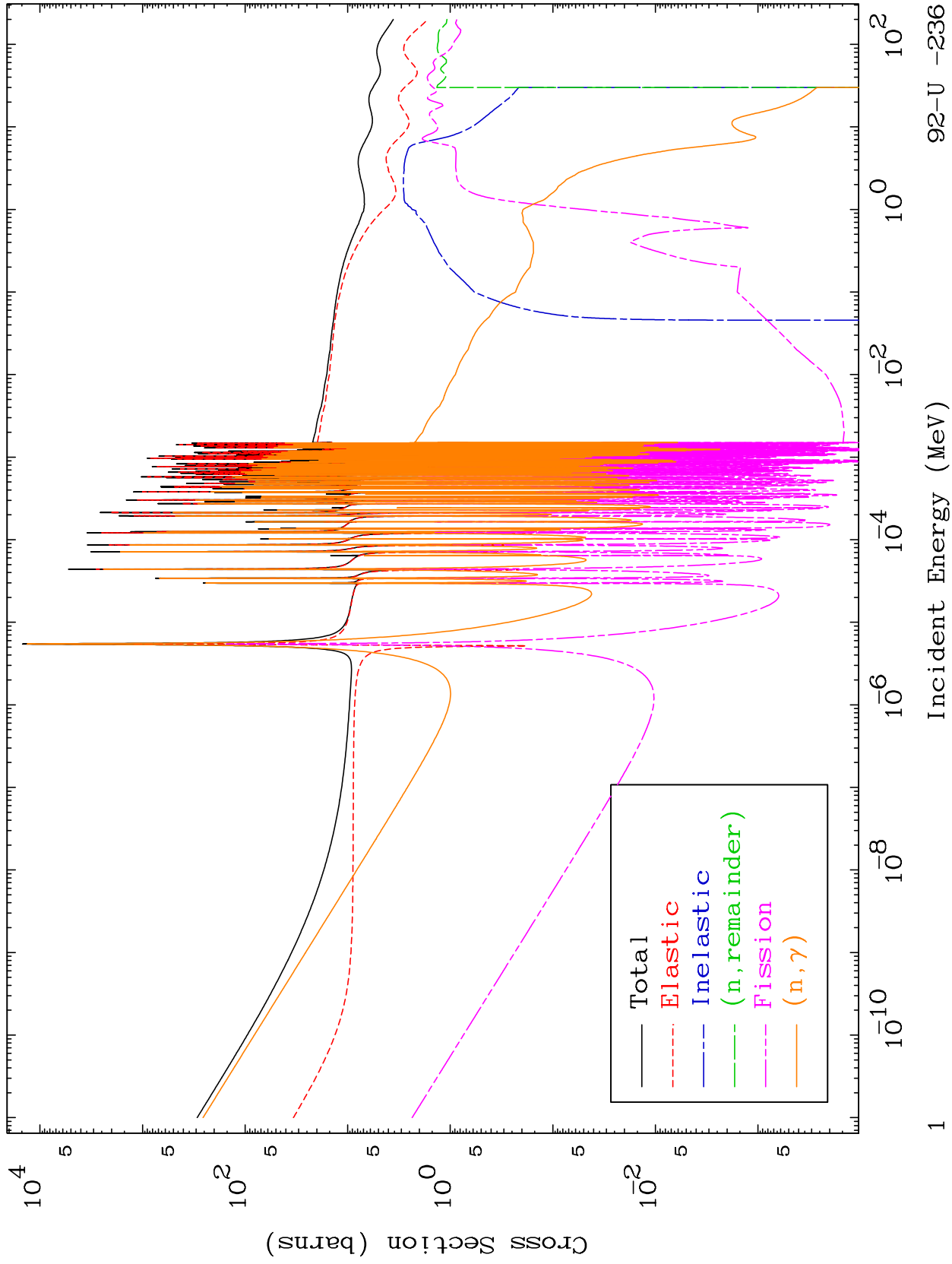
Press Mouse Button to Start

MAT 9231

Major

293 Kelvin Cross Sections

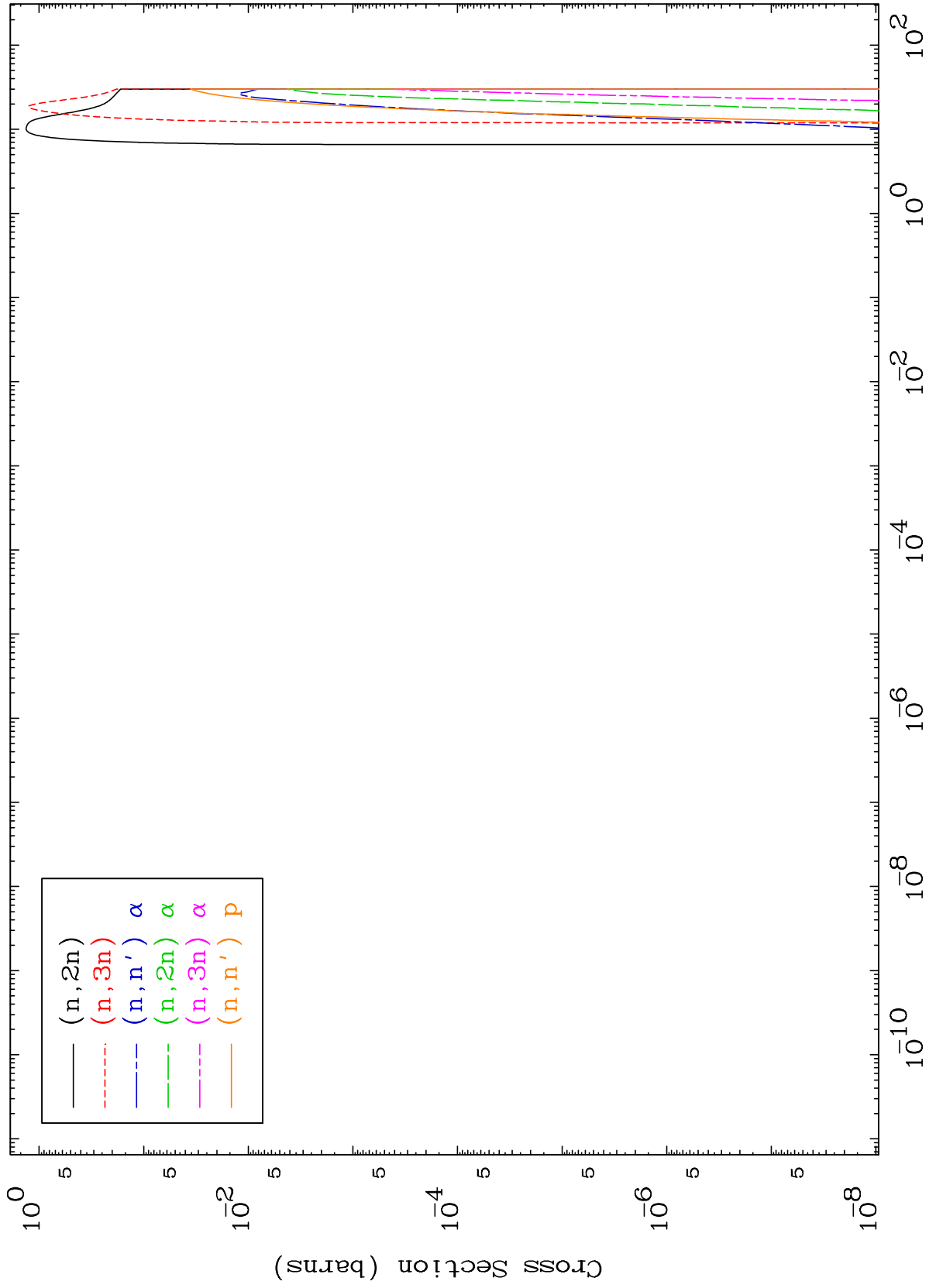
92-U -236



MAT 9231

Neutron Production
293 Kelvin Cross Sections

92-U -236



2

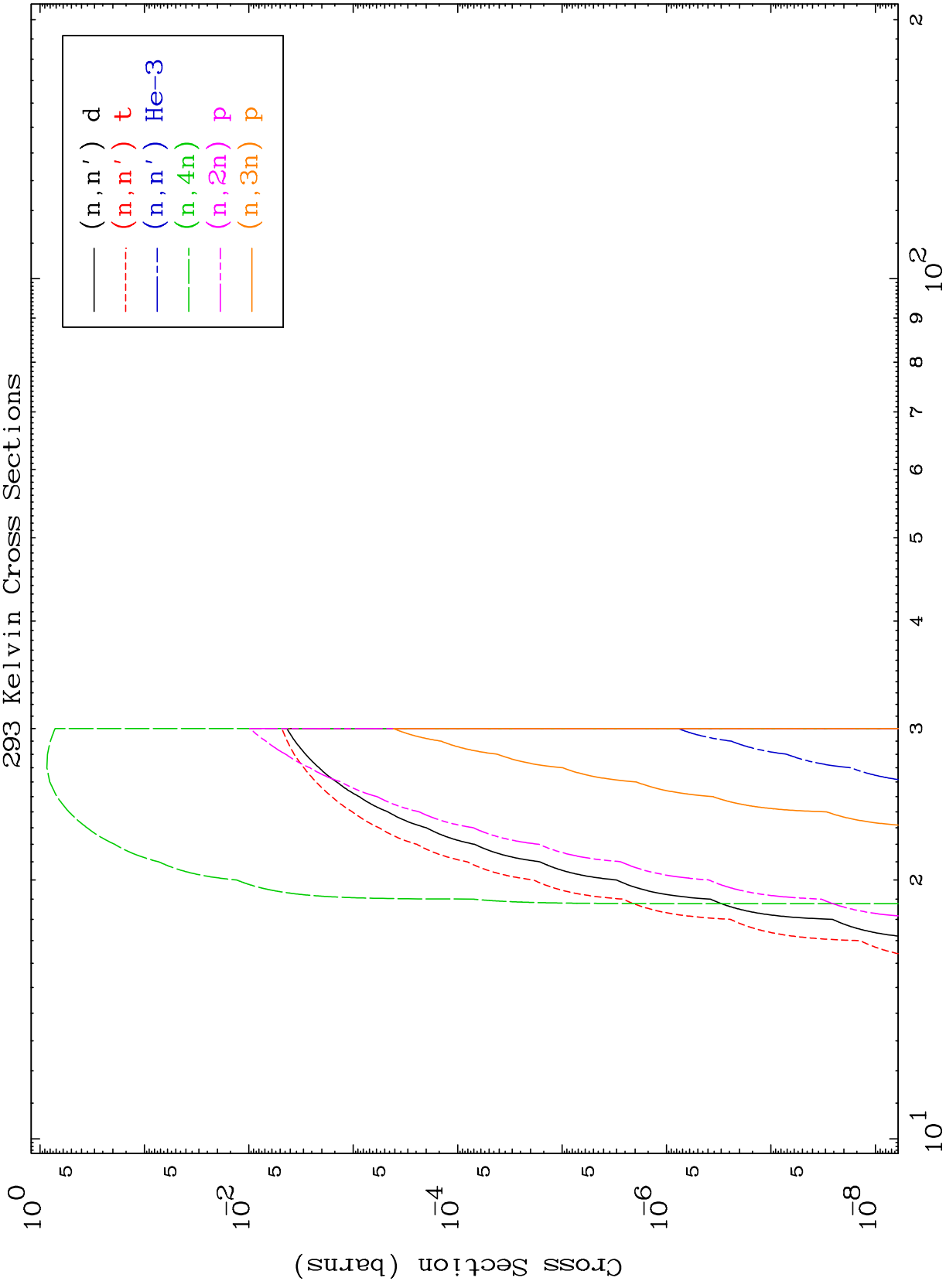
Incident Energy (MeV)

92-U -236

MAT 9231

Neutron Production
293 Kelvin Cross Sections

92-U -236



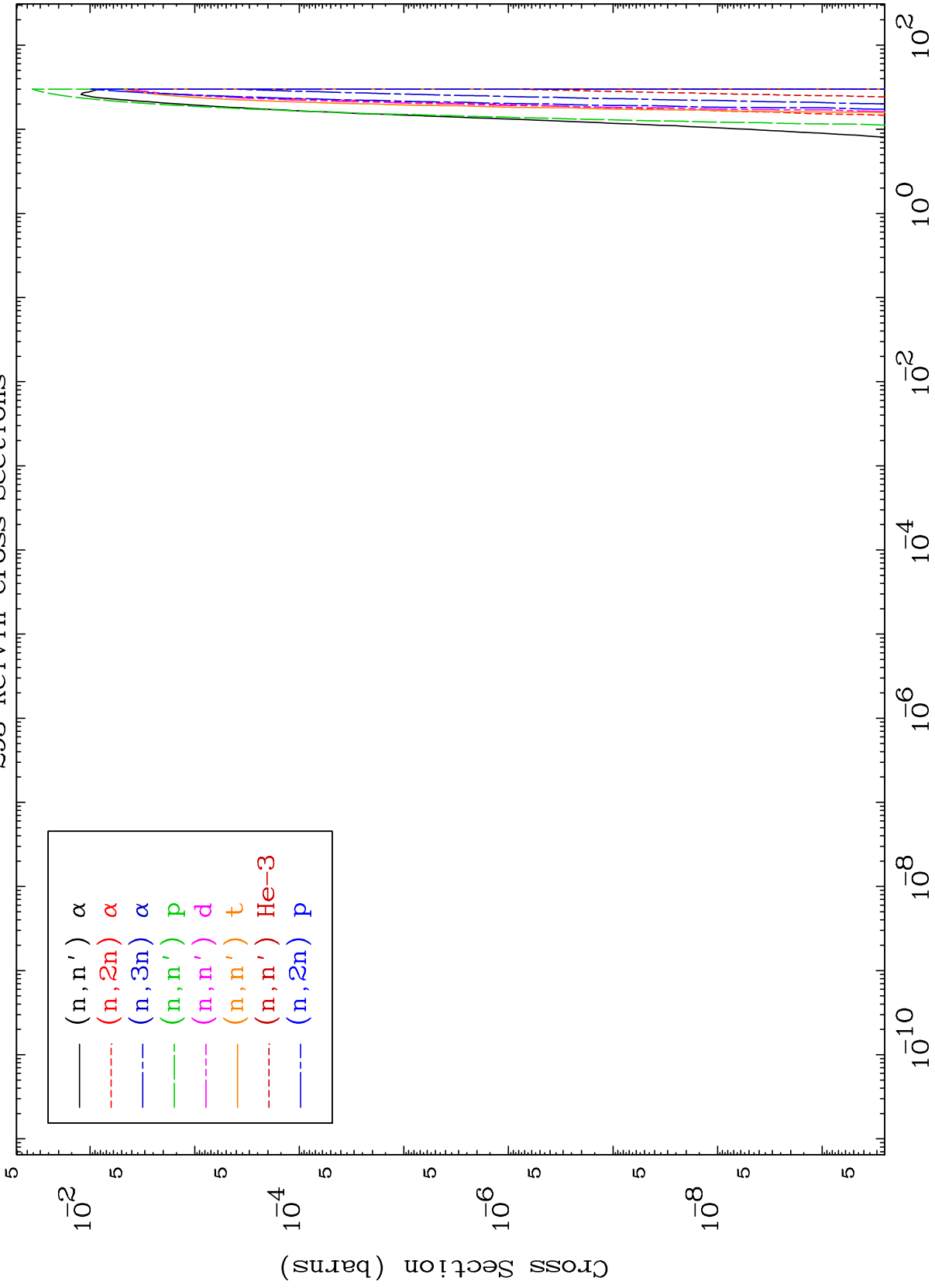
Incident Energy (MeV)

92-U -236

MAT 9231

293 Kelvin Cross Sections

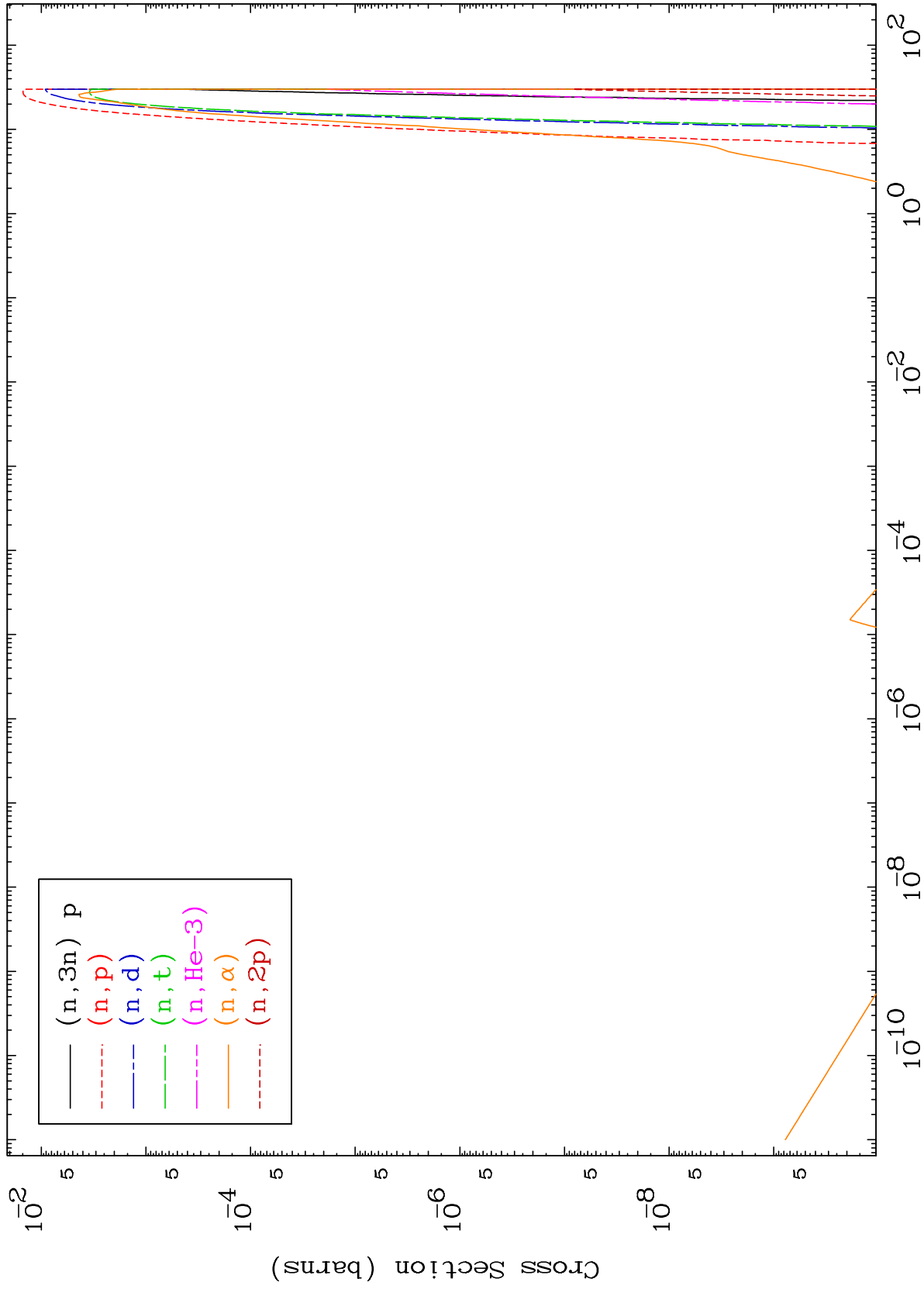
92-U -236



MAT 9231

293 Kelvin Cross Sections

92-U -236



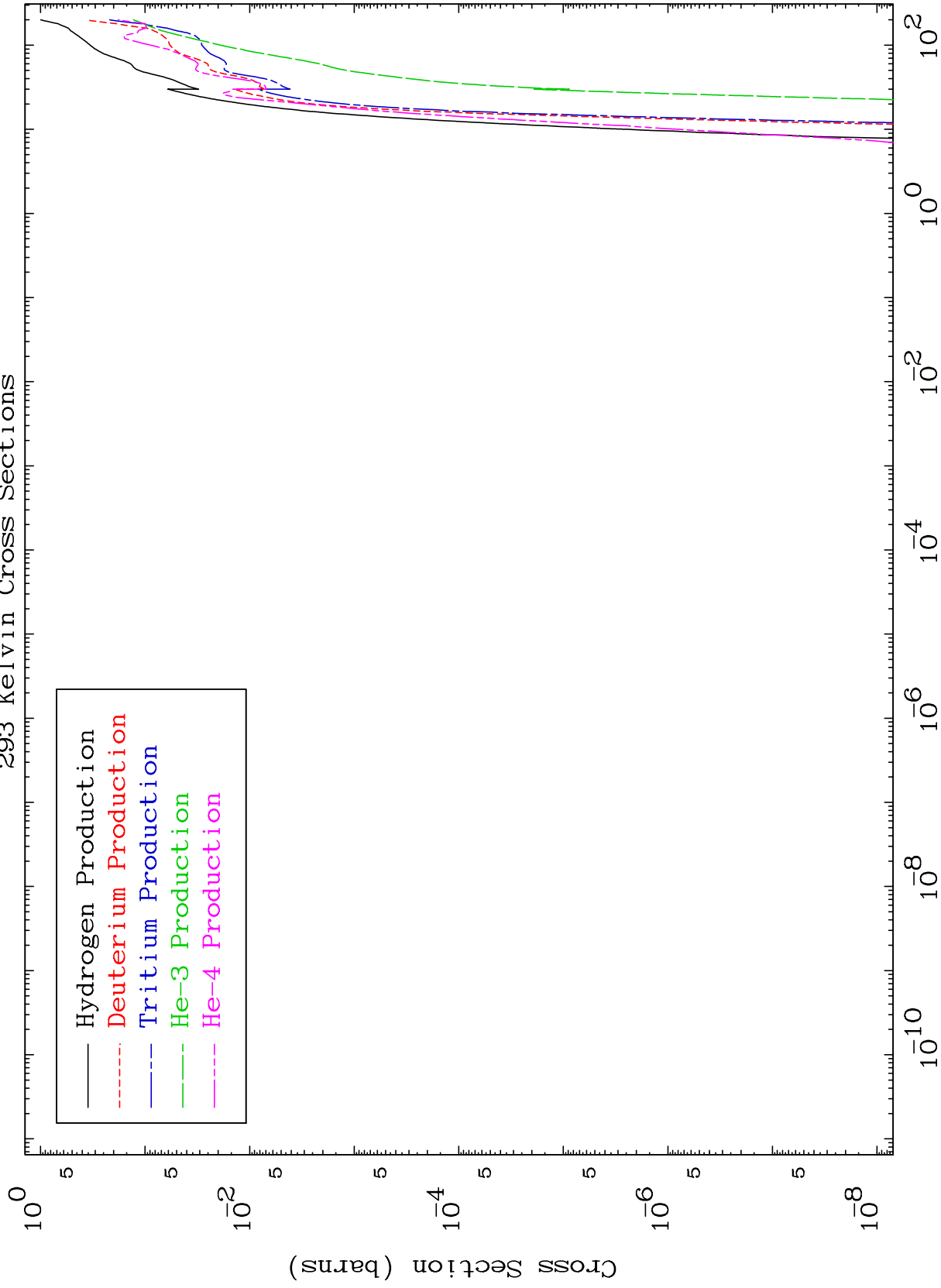
5

92-U -236

MAT 9231

Particle Production
293 Kelvin Cross Sections

92-U -236



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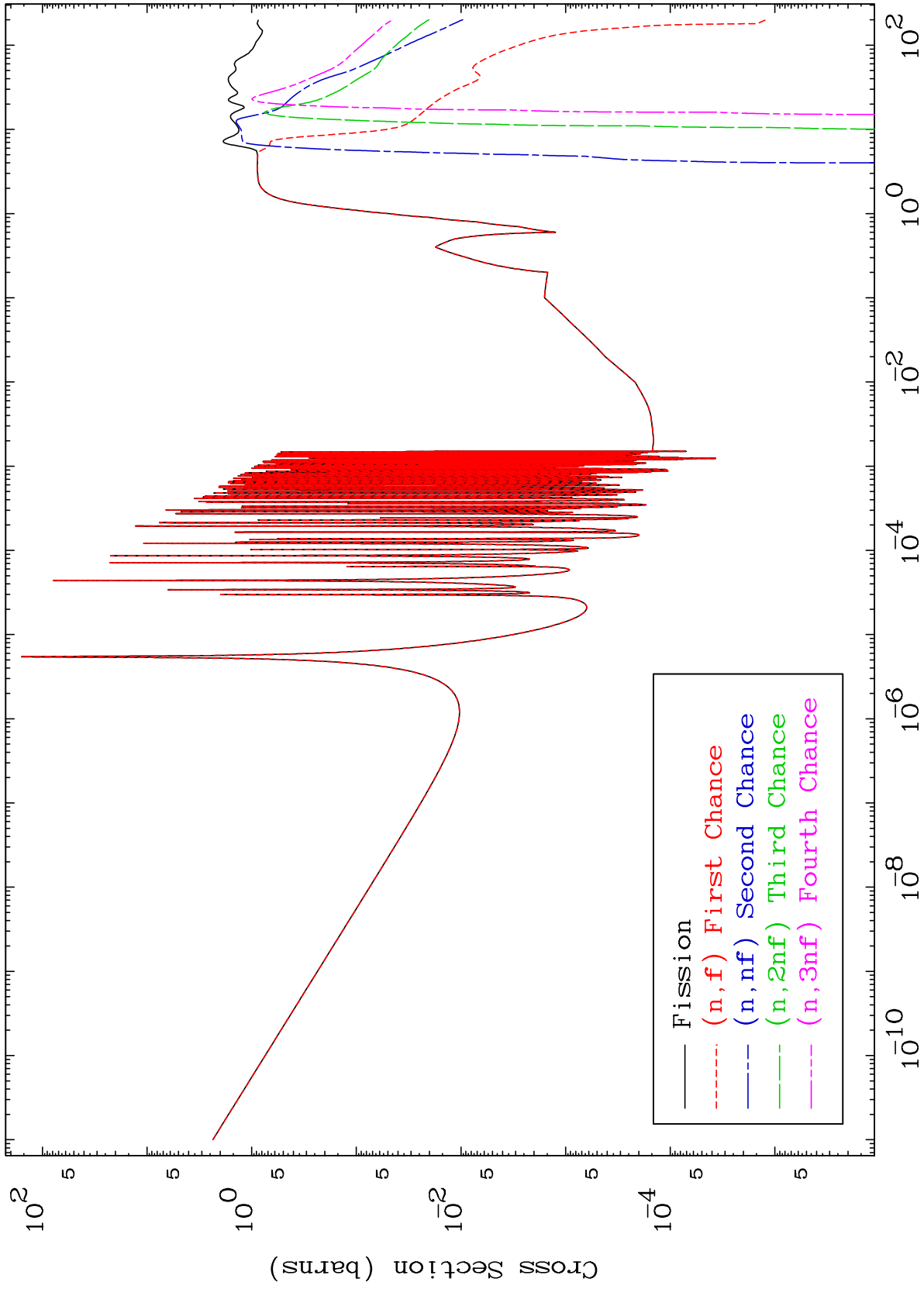
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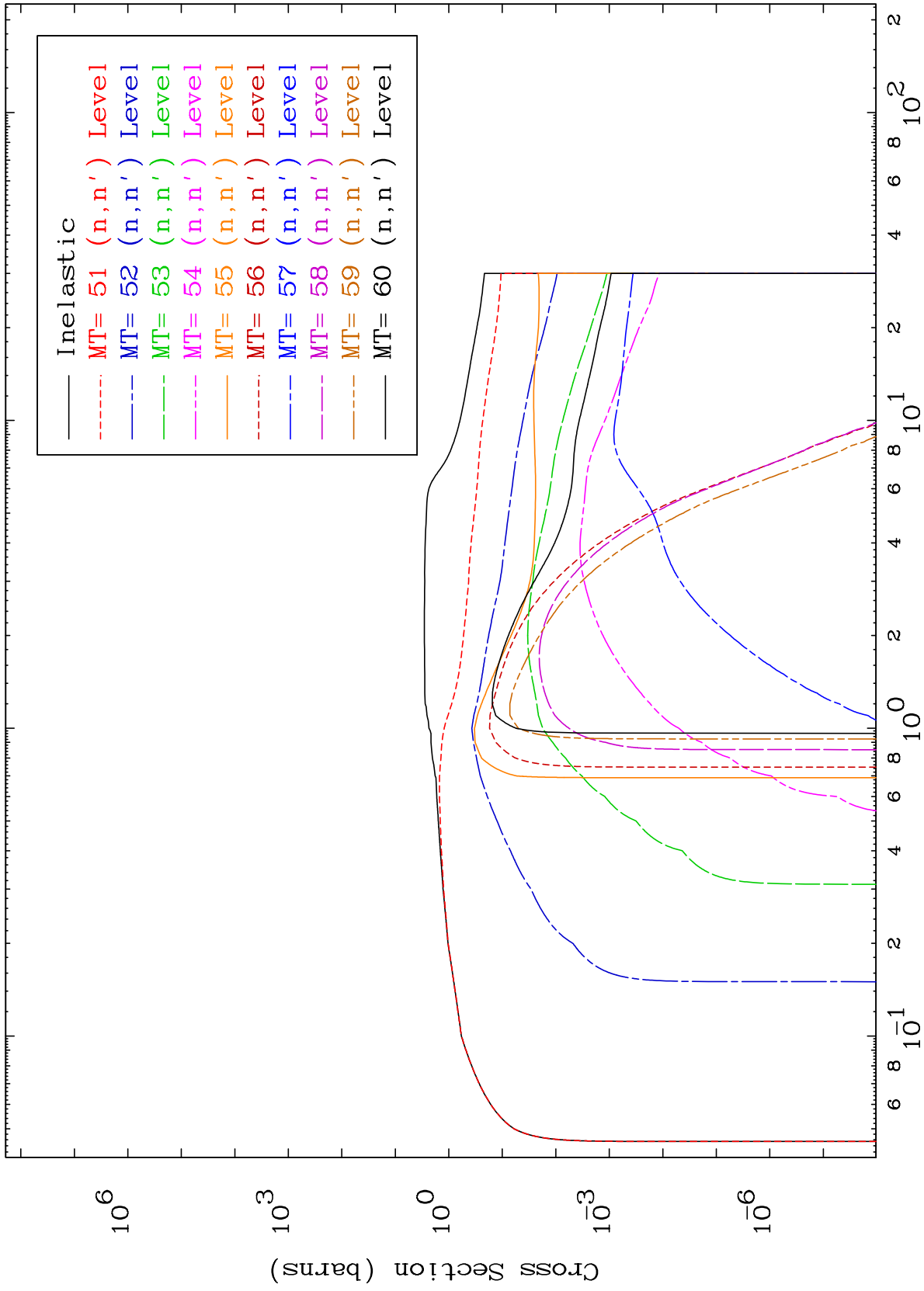
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MAT 9231

Fission
293 Kelvin Cross Sections

92-U -236

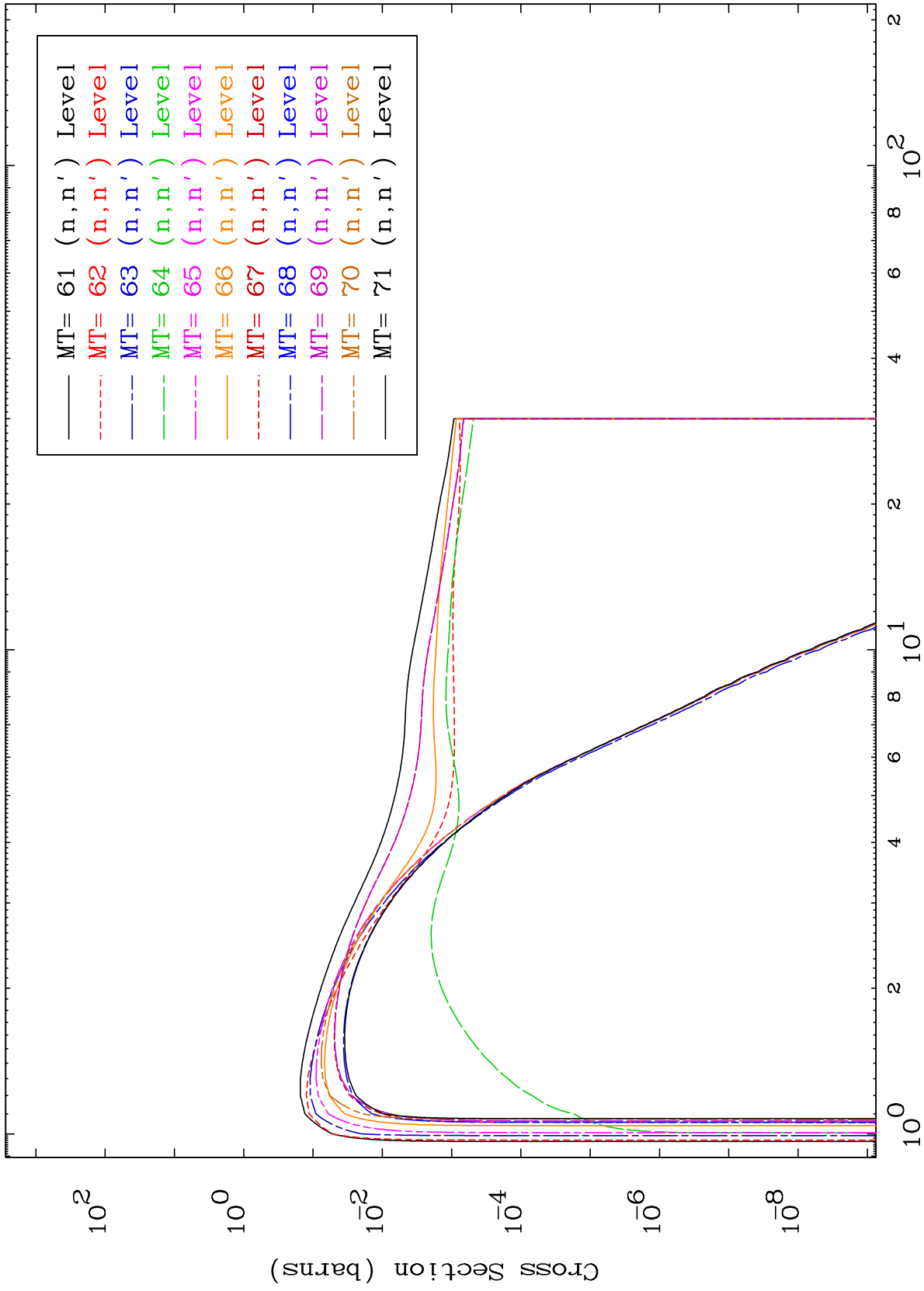




MAT 9231

(n,n') Level
293 Kelvin Cross Sections

92-U -236



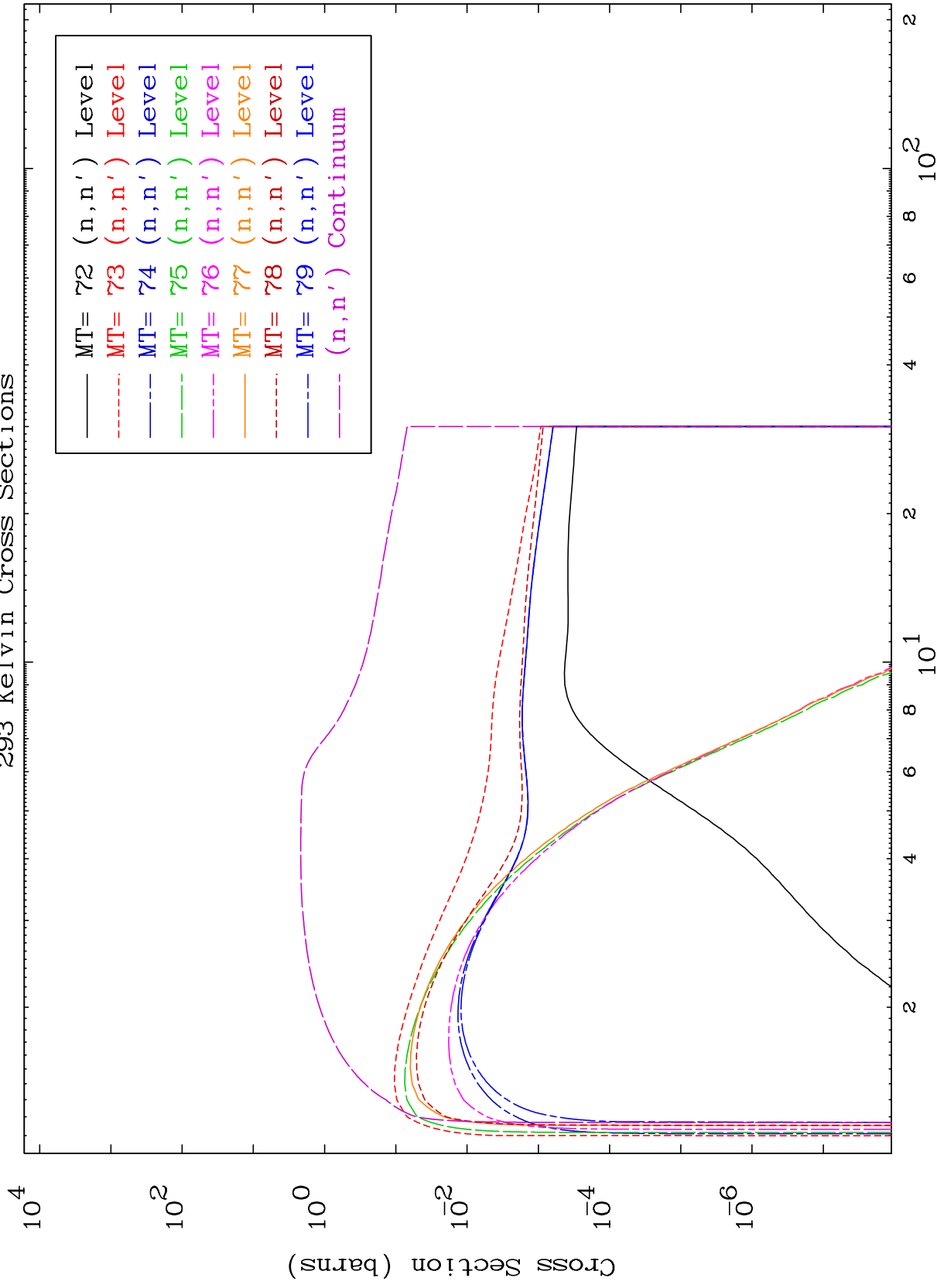
Incident Energy (MeV)

92-U -236

MAT 9231

(n,n') Level
293 Kelvin Cross Sections

92-U -236



10

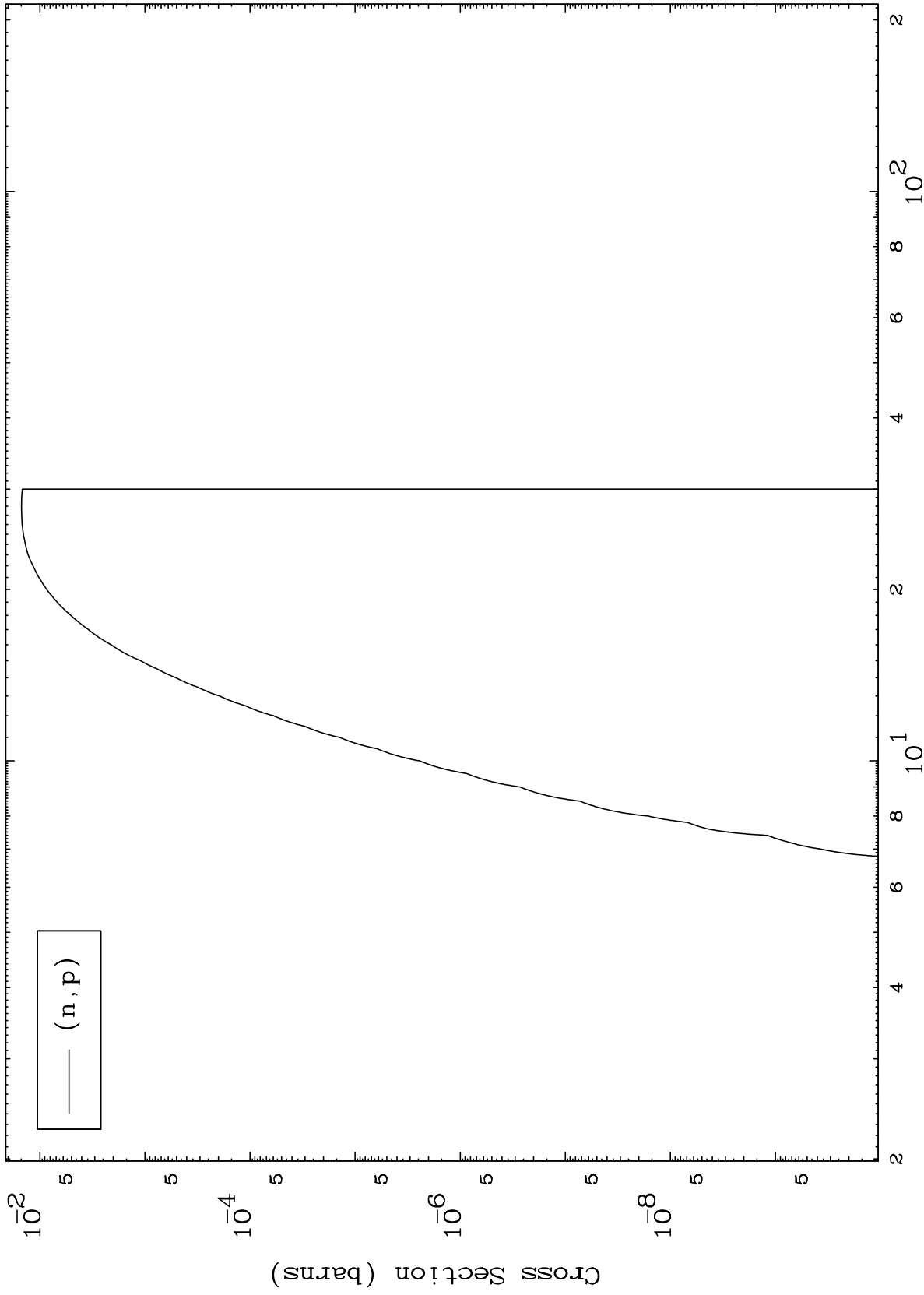
Incident Energy (MeV)

92-U -236

MAT 9231

(n,p) Levels
293 Kelvin Cross Sections

92-U -236



11

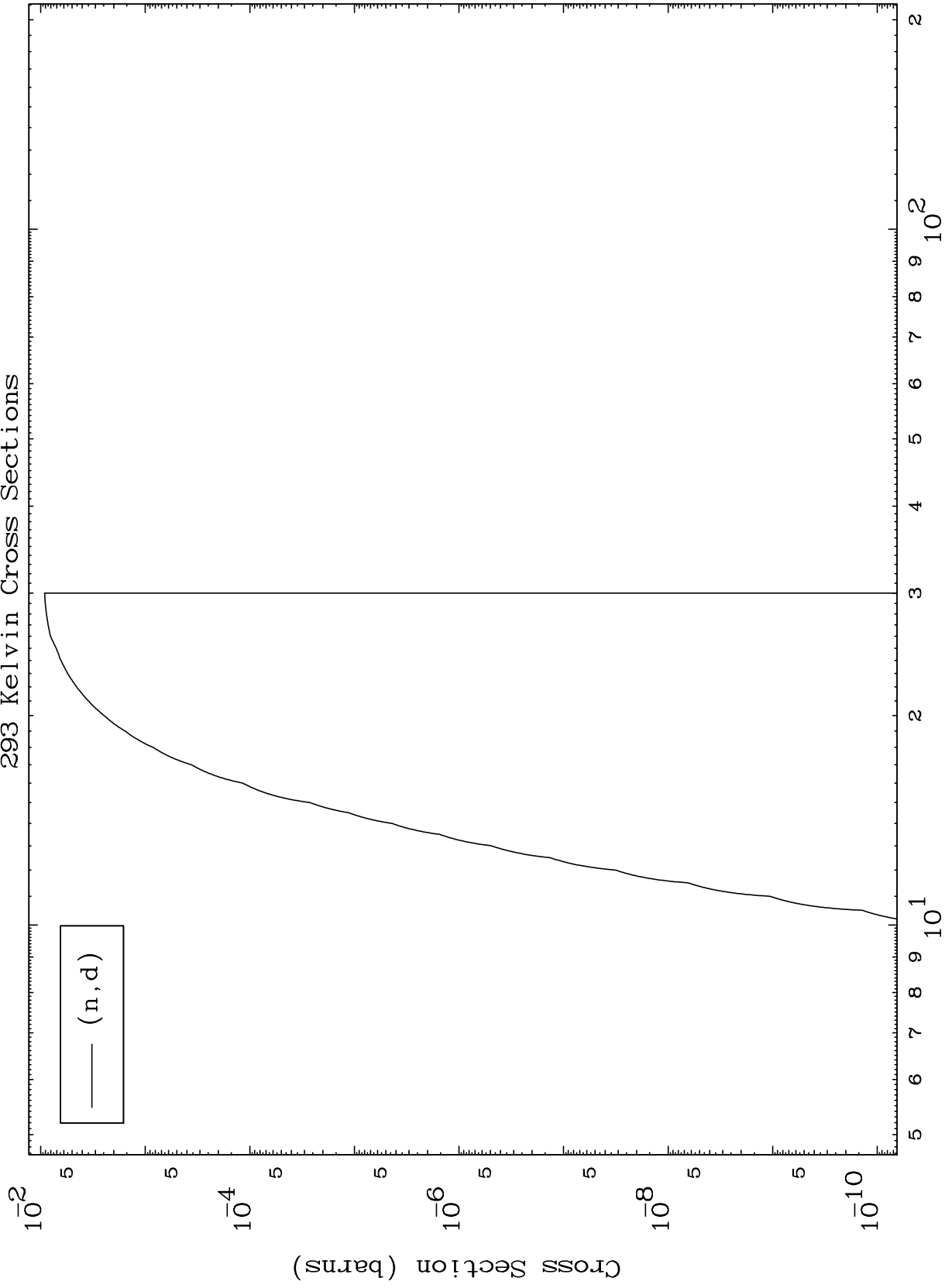
Incident Energy (MeV)

92-U -236

MAT 9231

(n,d) Levels
293 Kelvin Cross Sections

92-U -236



12

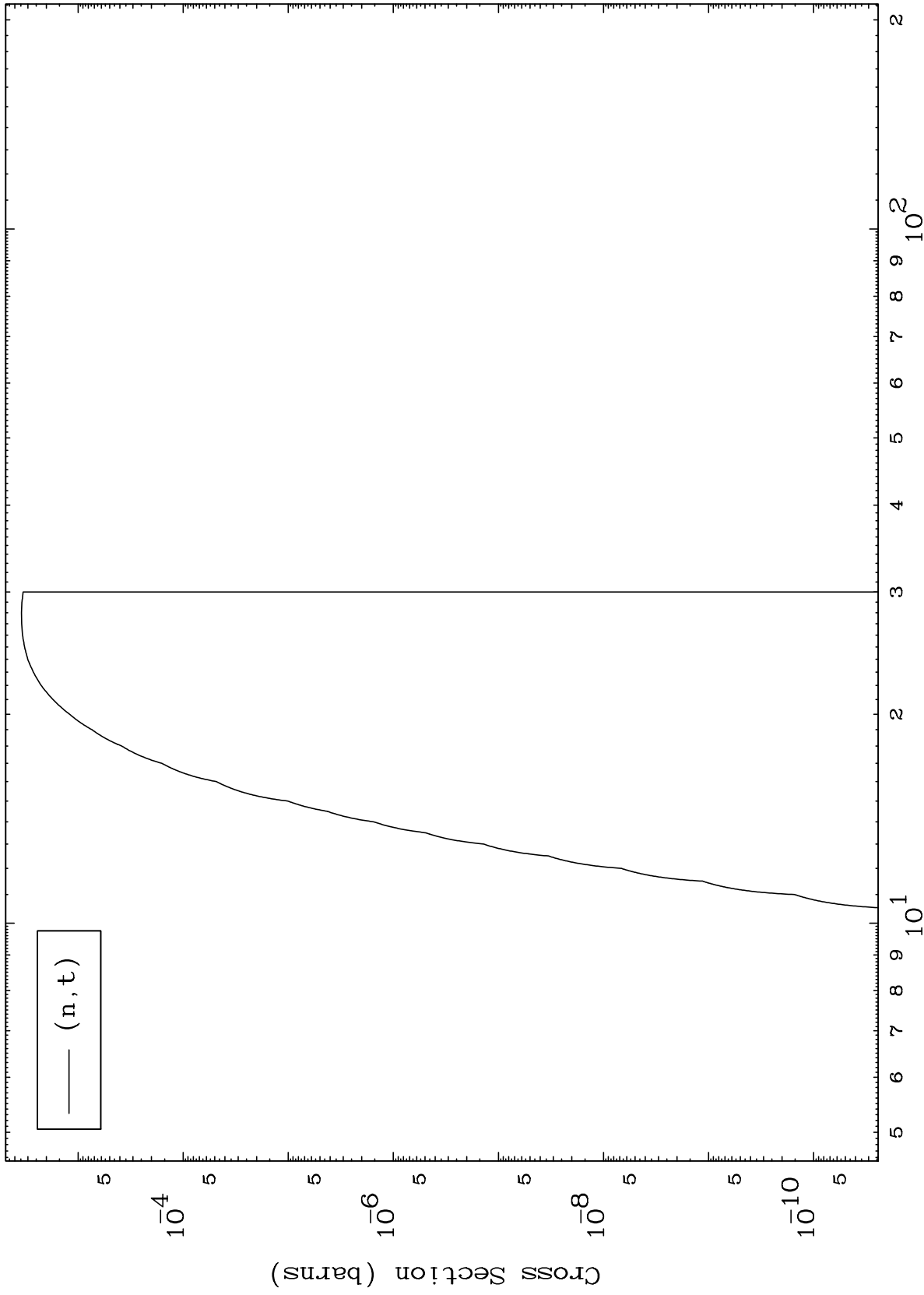
Incident Energy (MeV)

92-U -236

MAT 9231

(n,t) Levels
293 Kelvin Cross Sections

92-U -236



13

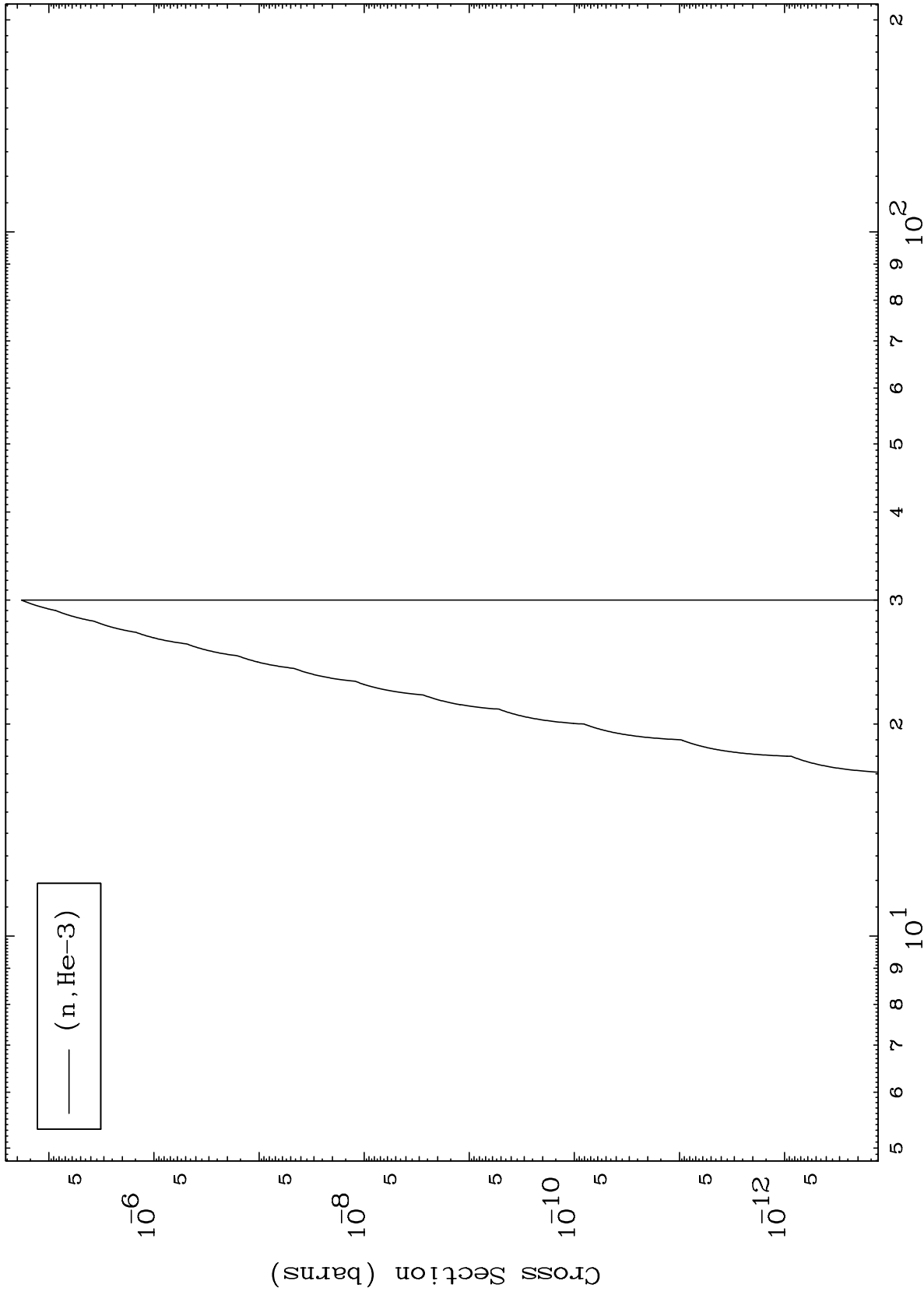
Incident Energy (MeV)

92-U -236

MAT 9231

(n,He3) Levels
293 Kelvin Cross Sections

92-U -236



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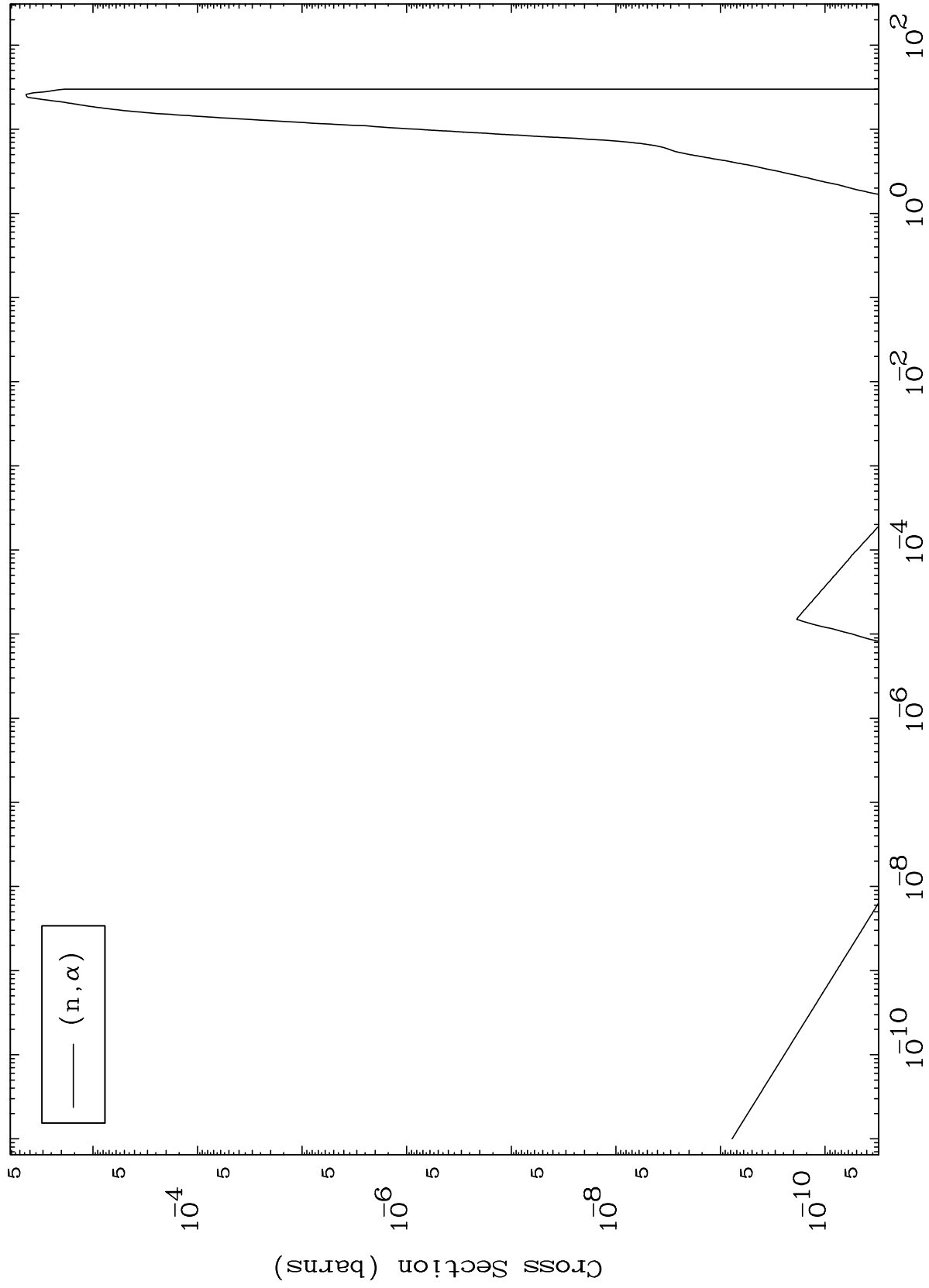
Incident Energy (MeV)

92-U -236

MAT 9231

(n,α) Levels
293 Kelvin Cross Sections

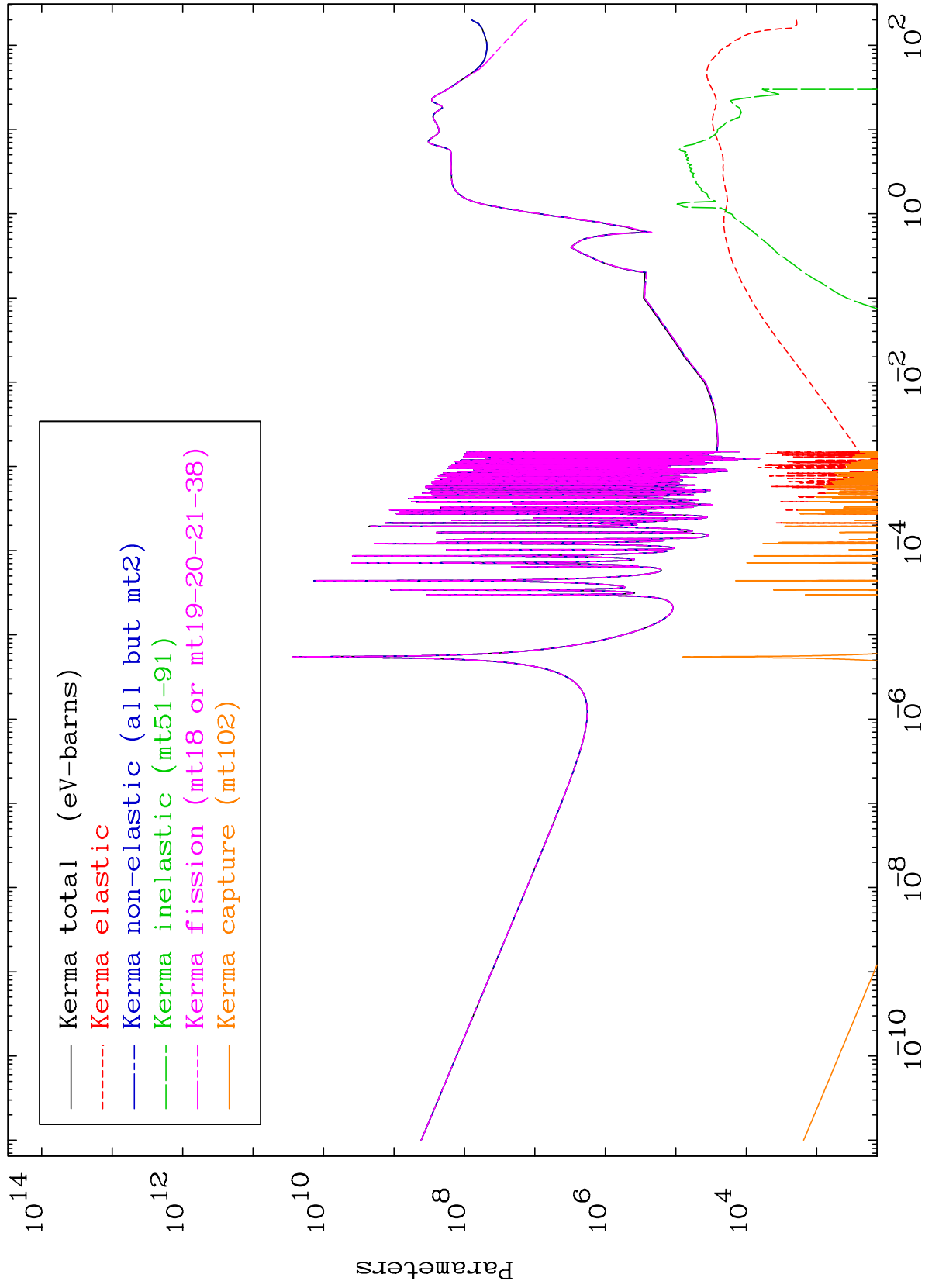
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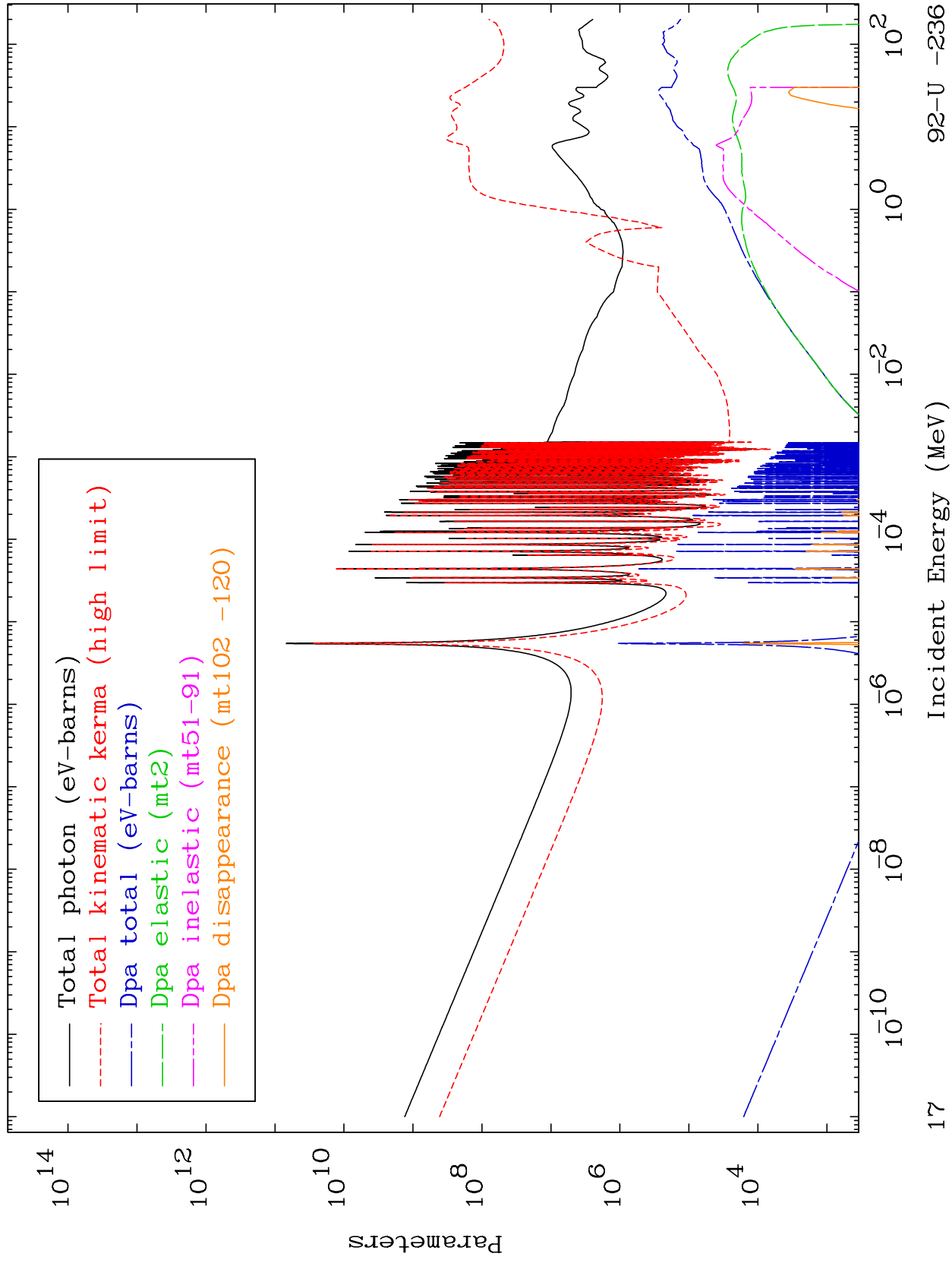


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Incident Energy (MeV)

92-U -236

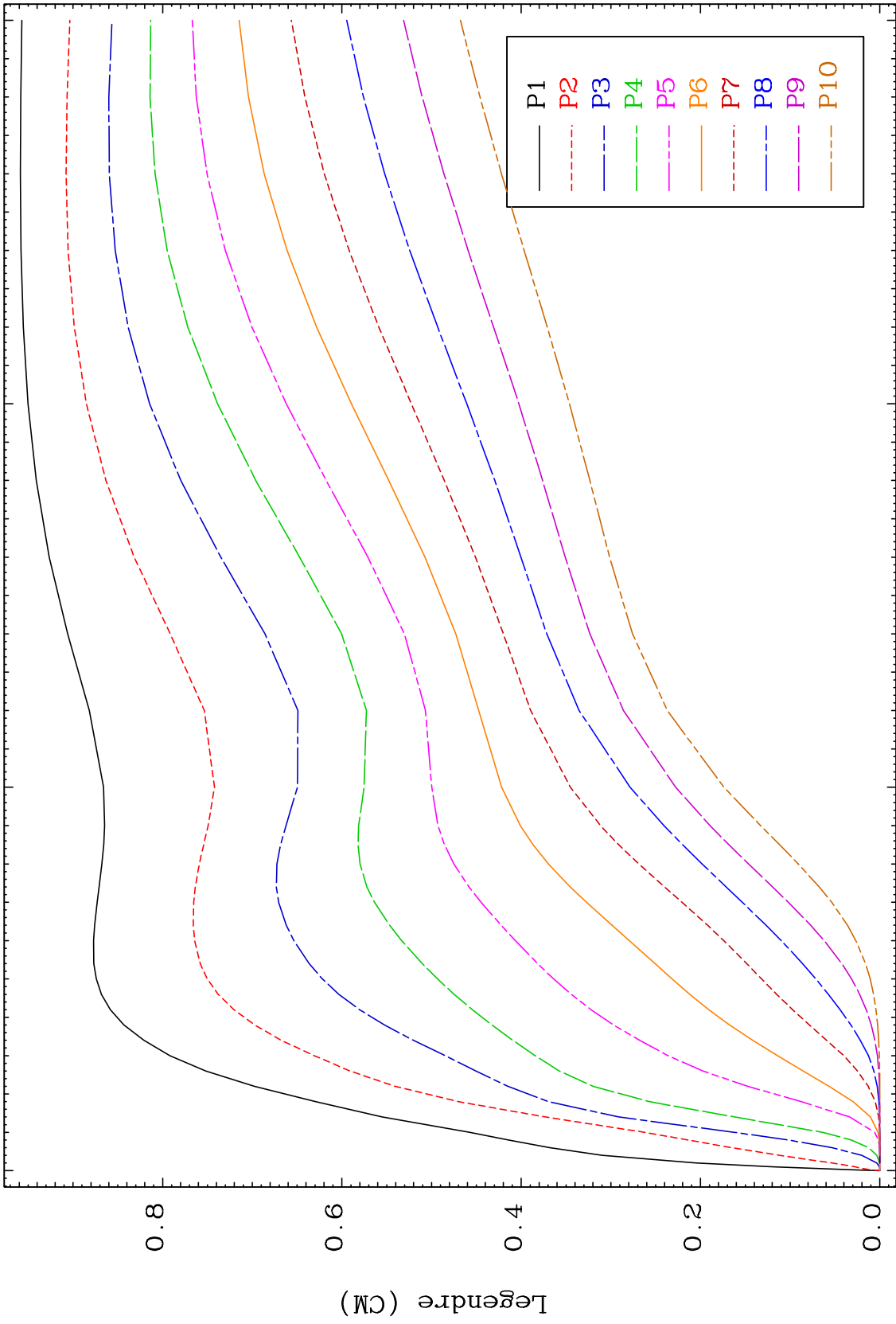




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Elastic Legendre Coefficients

92-U -236



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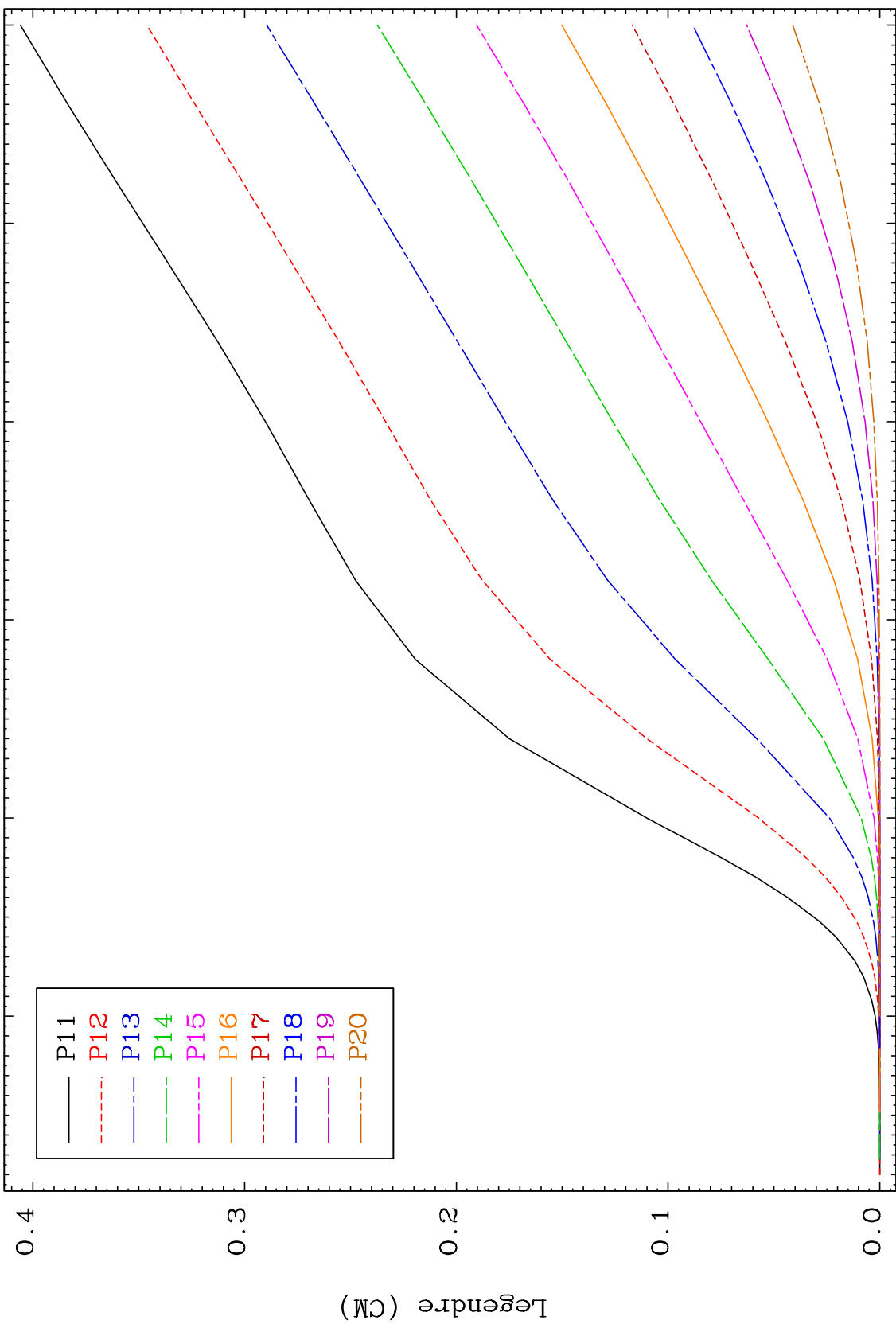
Incident Energy (MeV)

92-U -236

MAT 9231

Elastic Legendre Coefficients

92-U -236



19

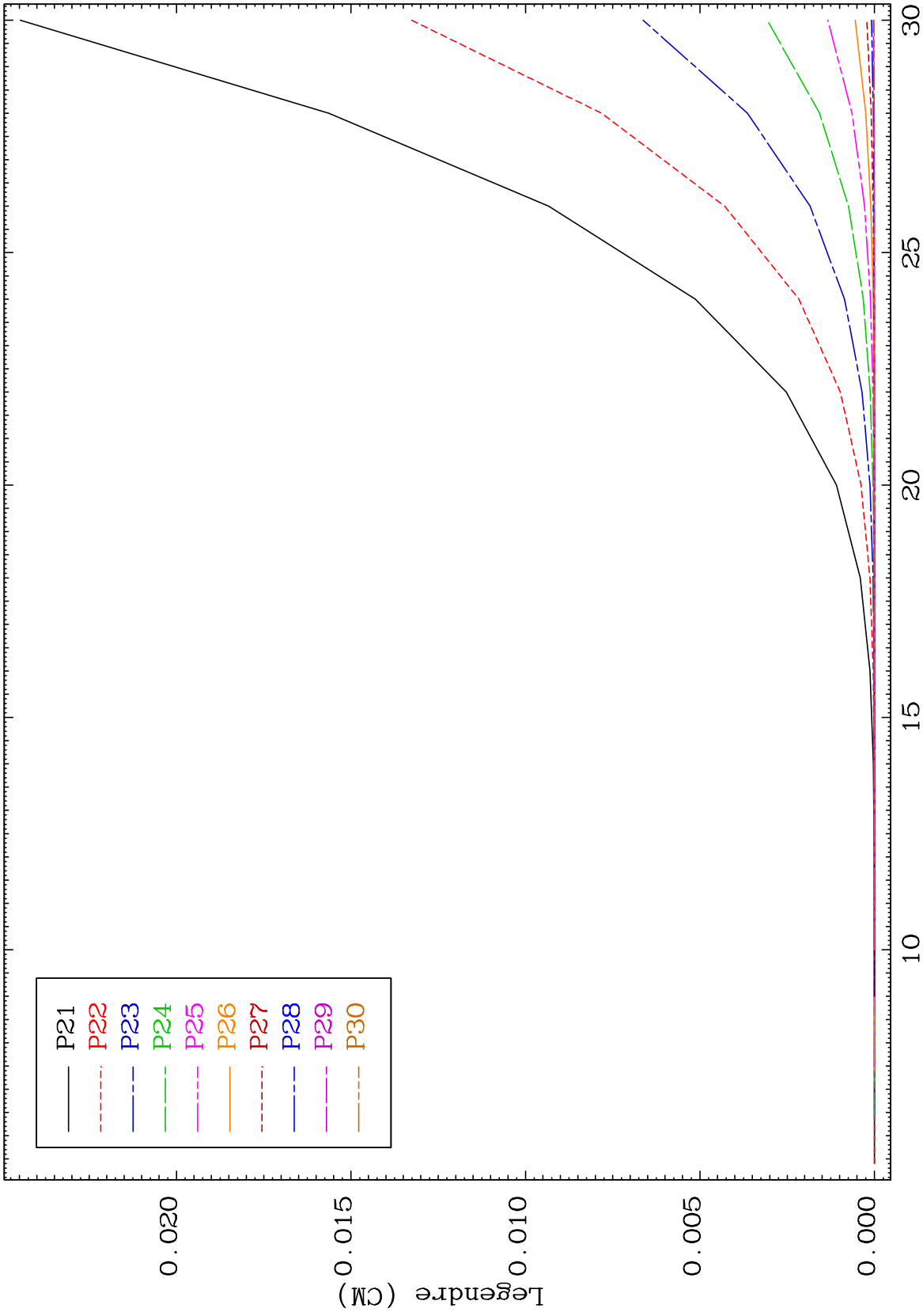
Incident Energy (MeV)

92-U -236

MAT 9231

Elastic Legendre Coefficients

92-U -236



20

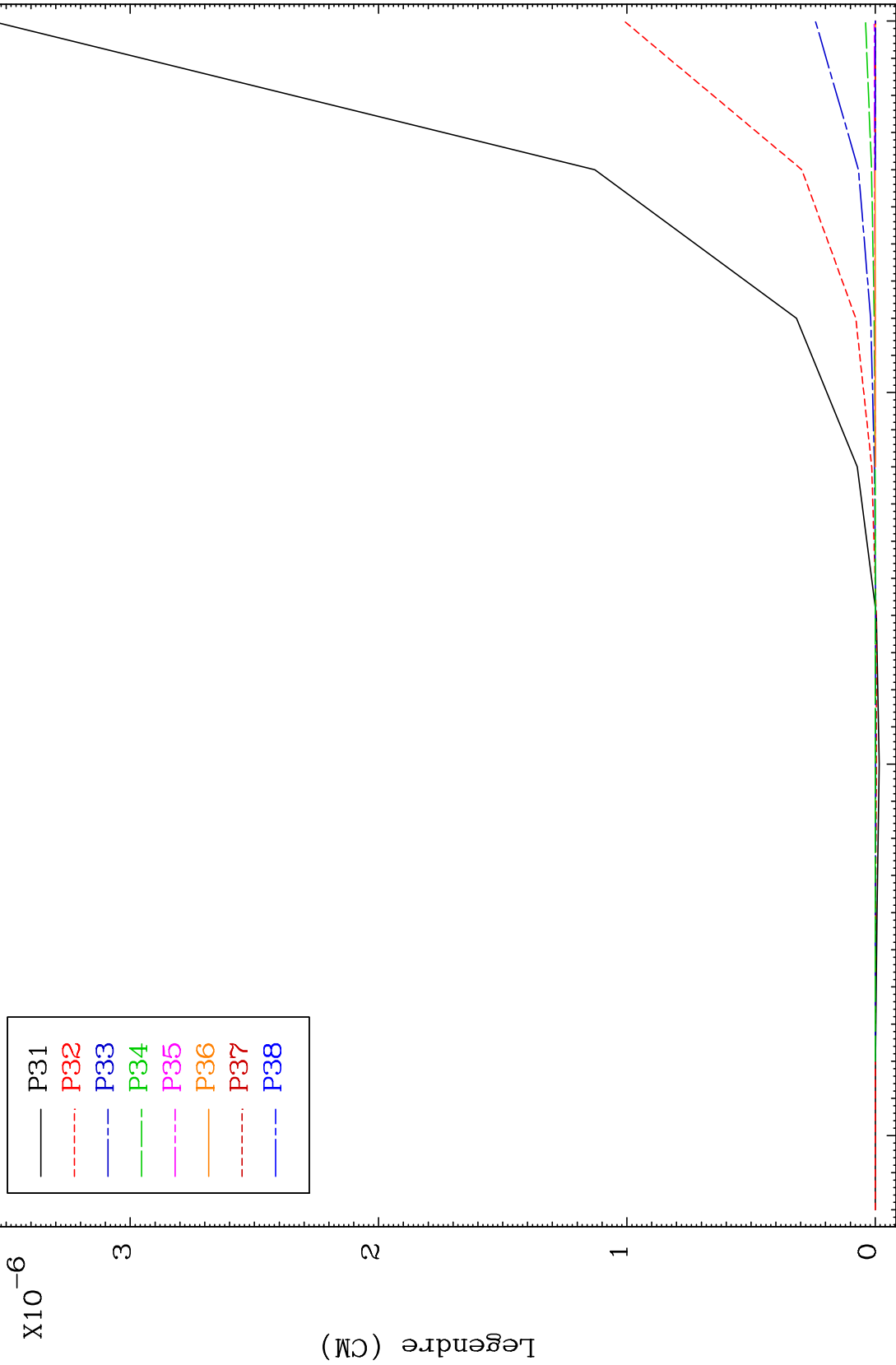
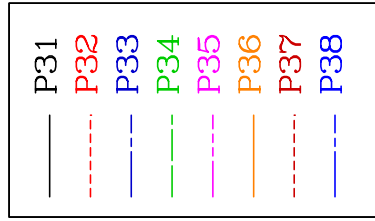
Incident Energy (MeV)

92-U -236

MAT 9231

Elastic Legendre Coefficients

92-U -236



21

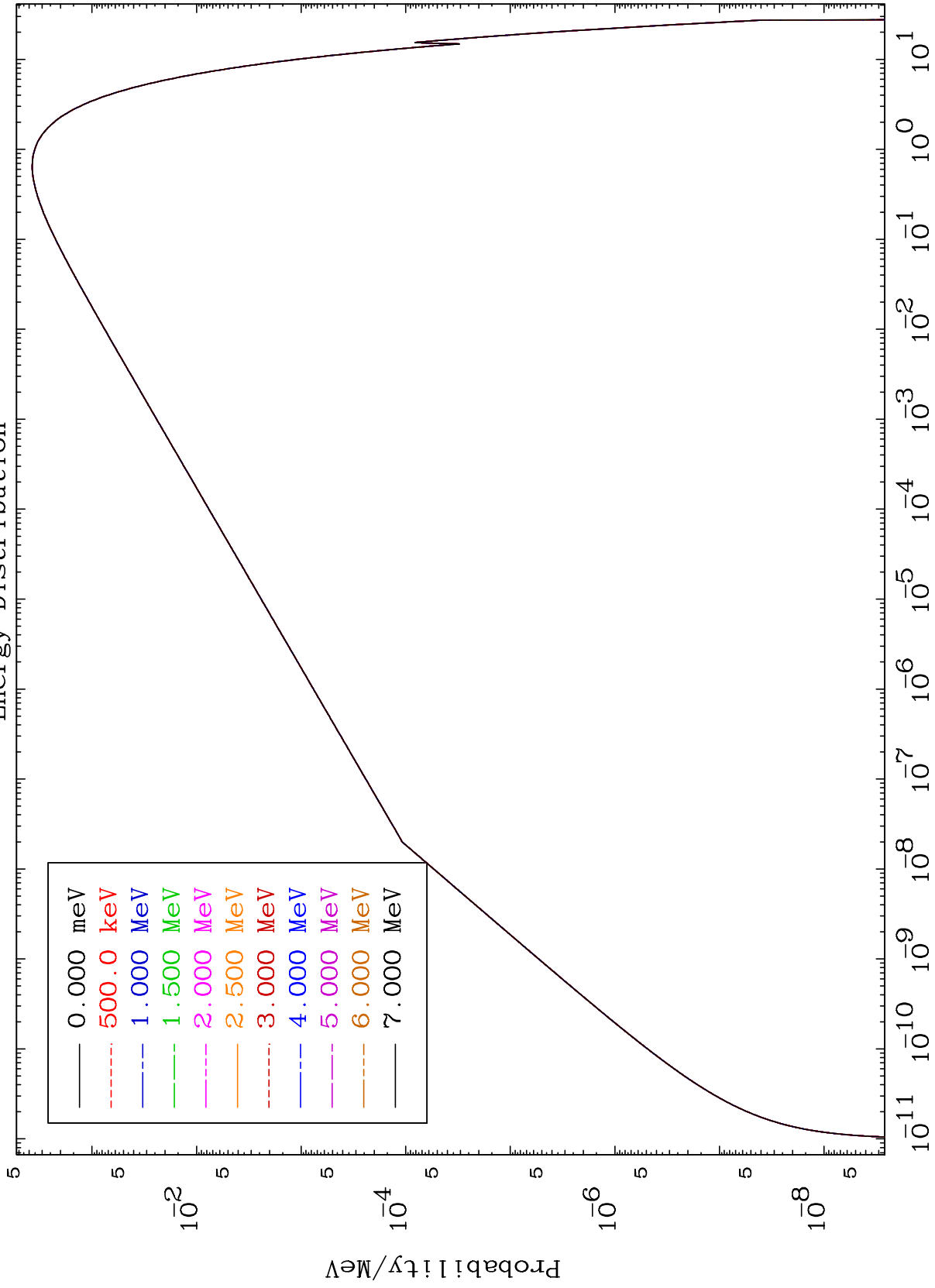
Incident Energy (MeV)

92-U -236

MAT 9231

Fission Energy Distribution

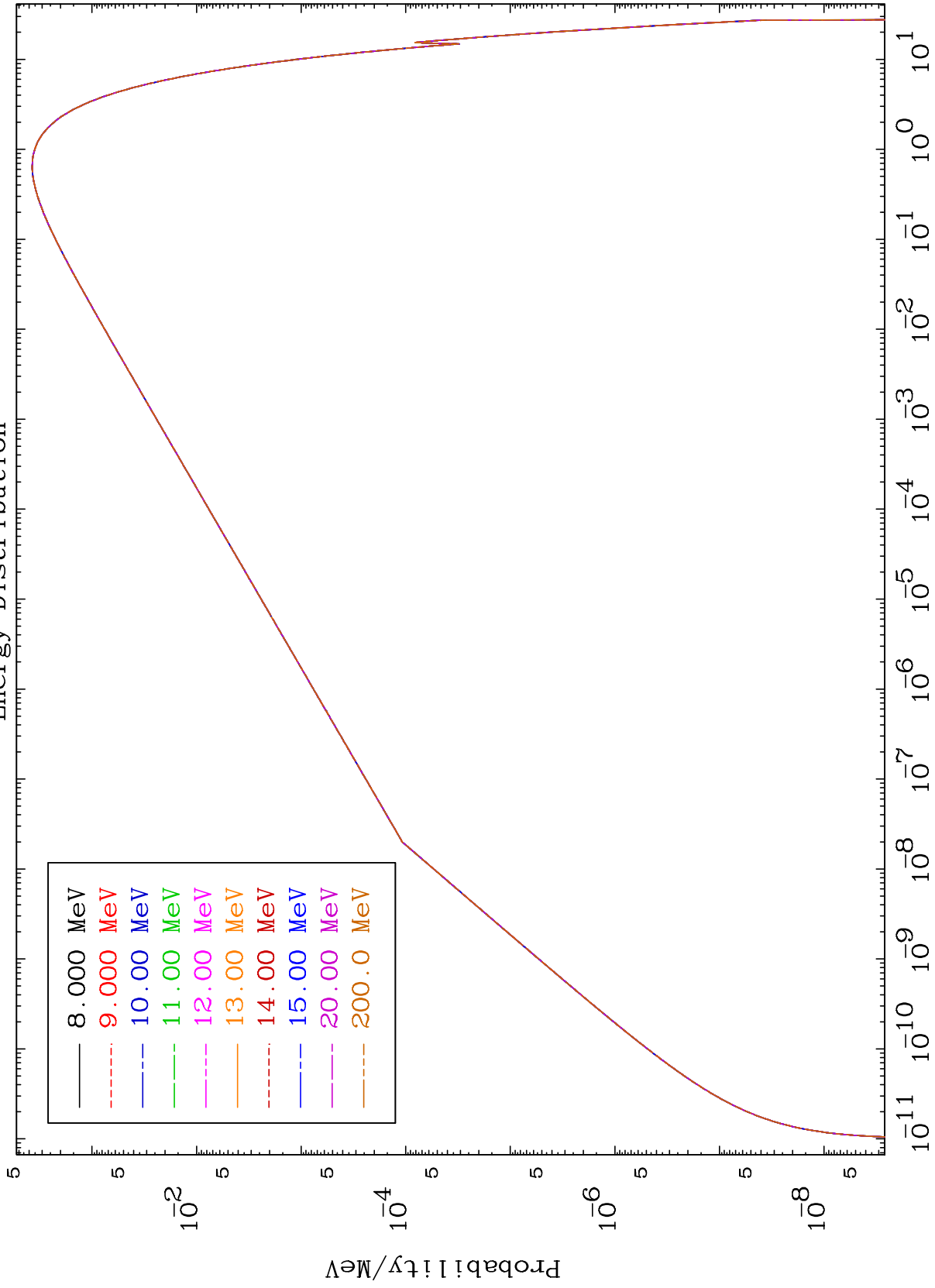
92-U -236



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Fission Energy Distribution

92-U -236



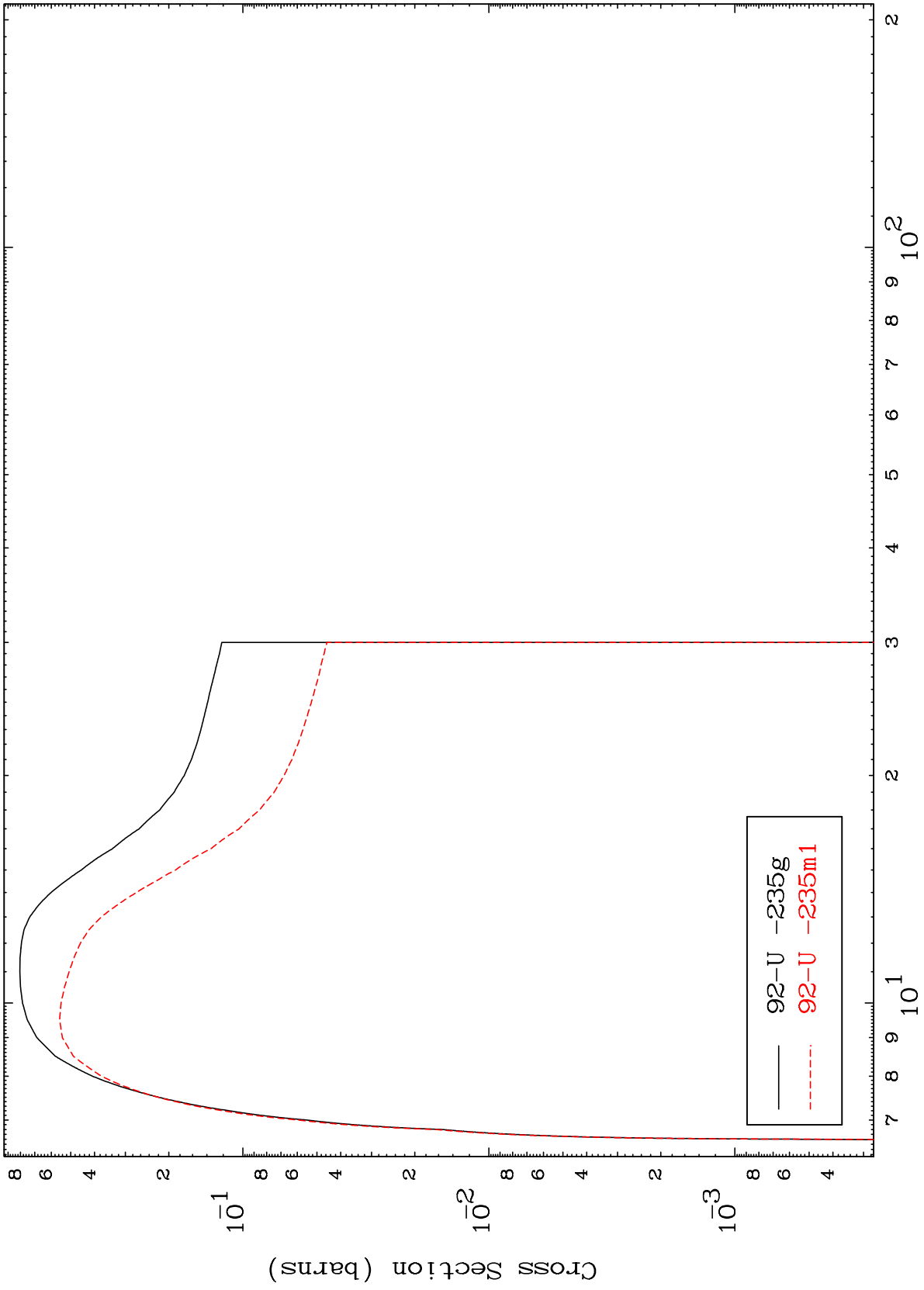
23

92-U -236

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92-U -236

(n,2n)
Radionuclide Production Cross Section



24

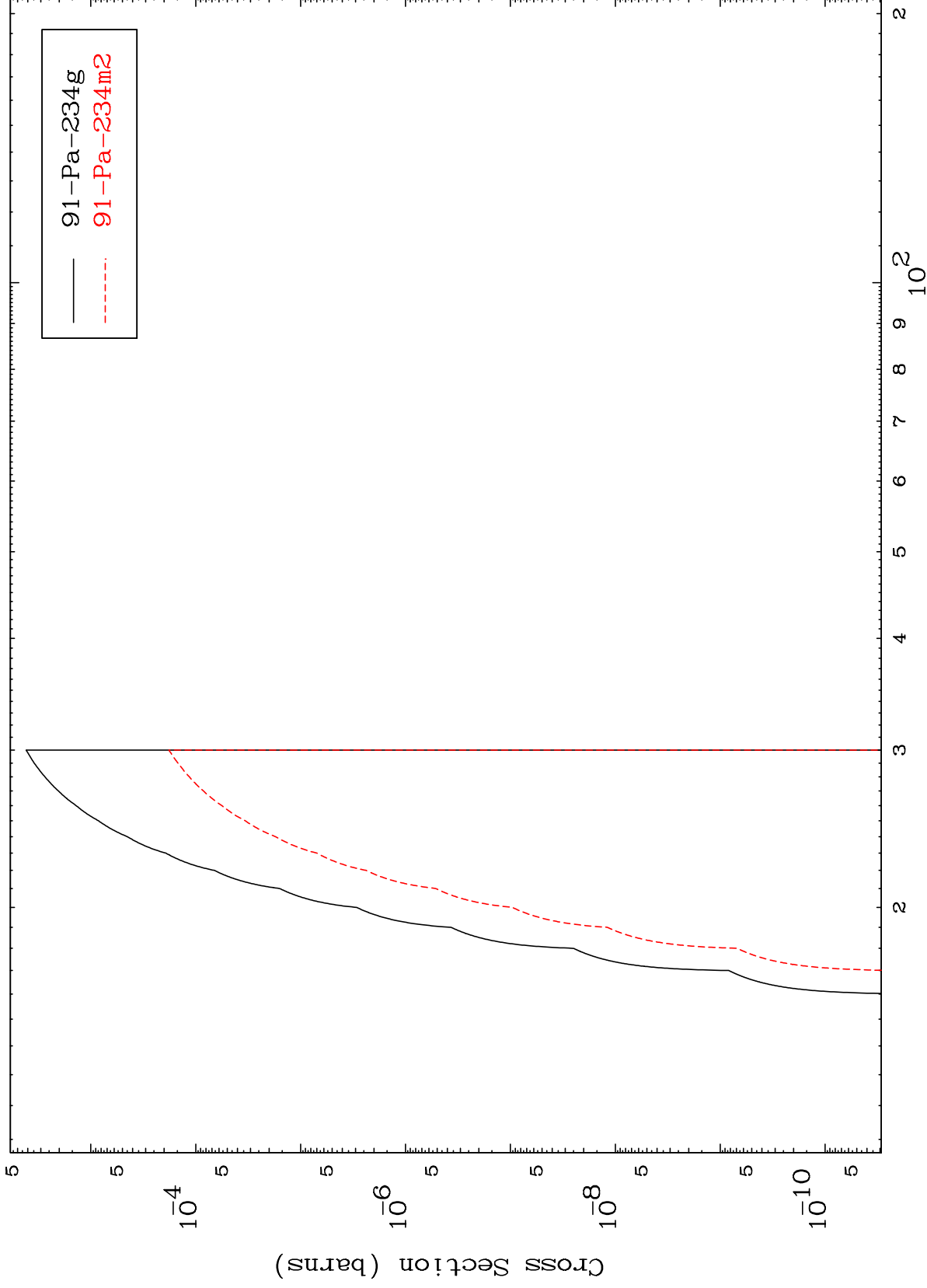
92-U -236

MAT 9231

(n,n') d

92-U -236

Radionuclide Production Cross Section



25

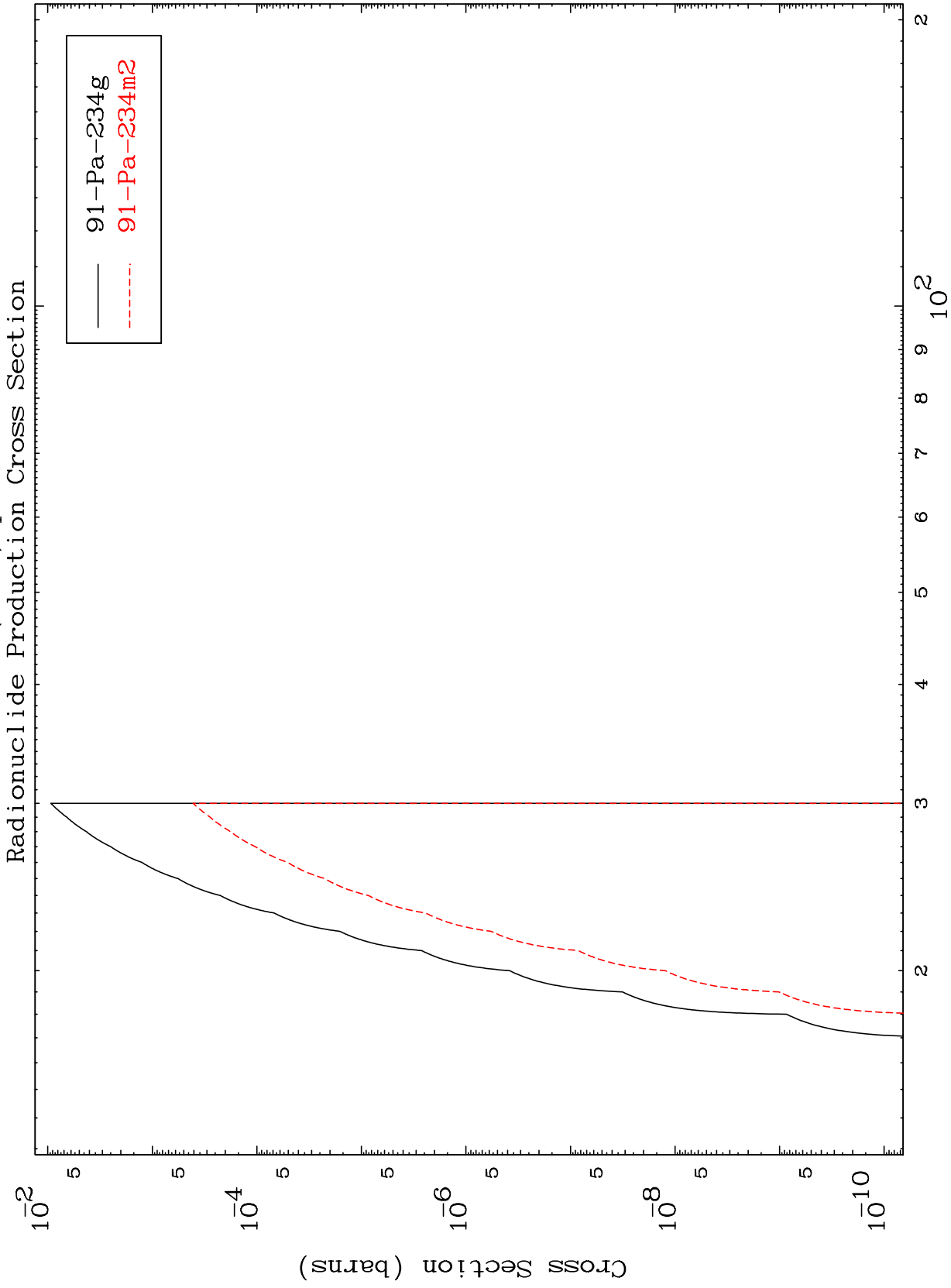
Incident Energy (MeV)

92-U -236

MAT 9231

(n,2n) p

92-U -236



26

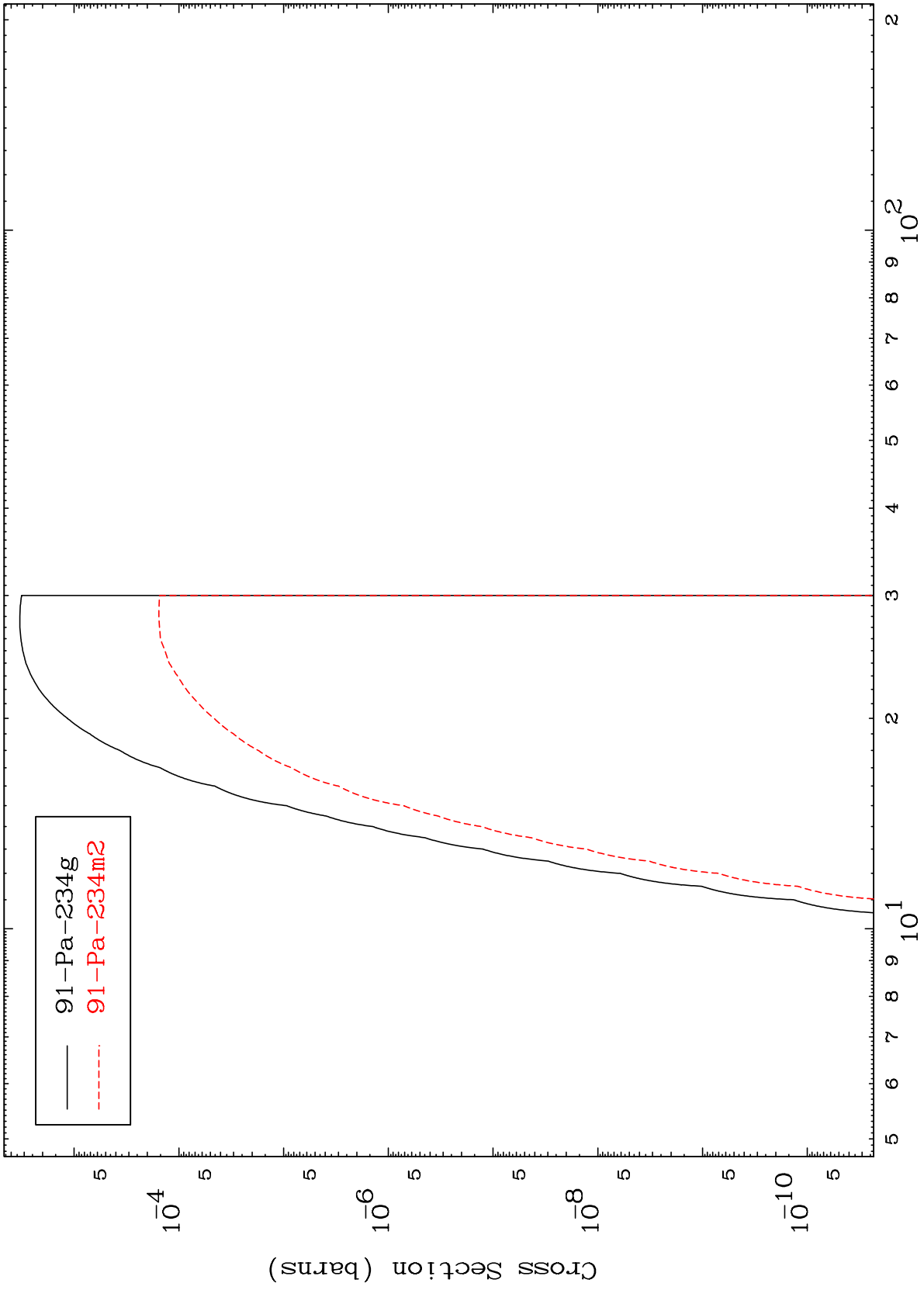
Incident Energy (MeV)

92-U -236

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92-U -236

(n, t)
Radionuclide Production Cross Section



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92-U -236