

Program EVALPLOT
(Version 2018-1)

by

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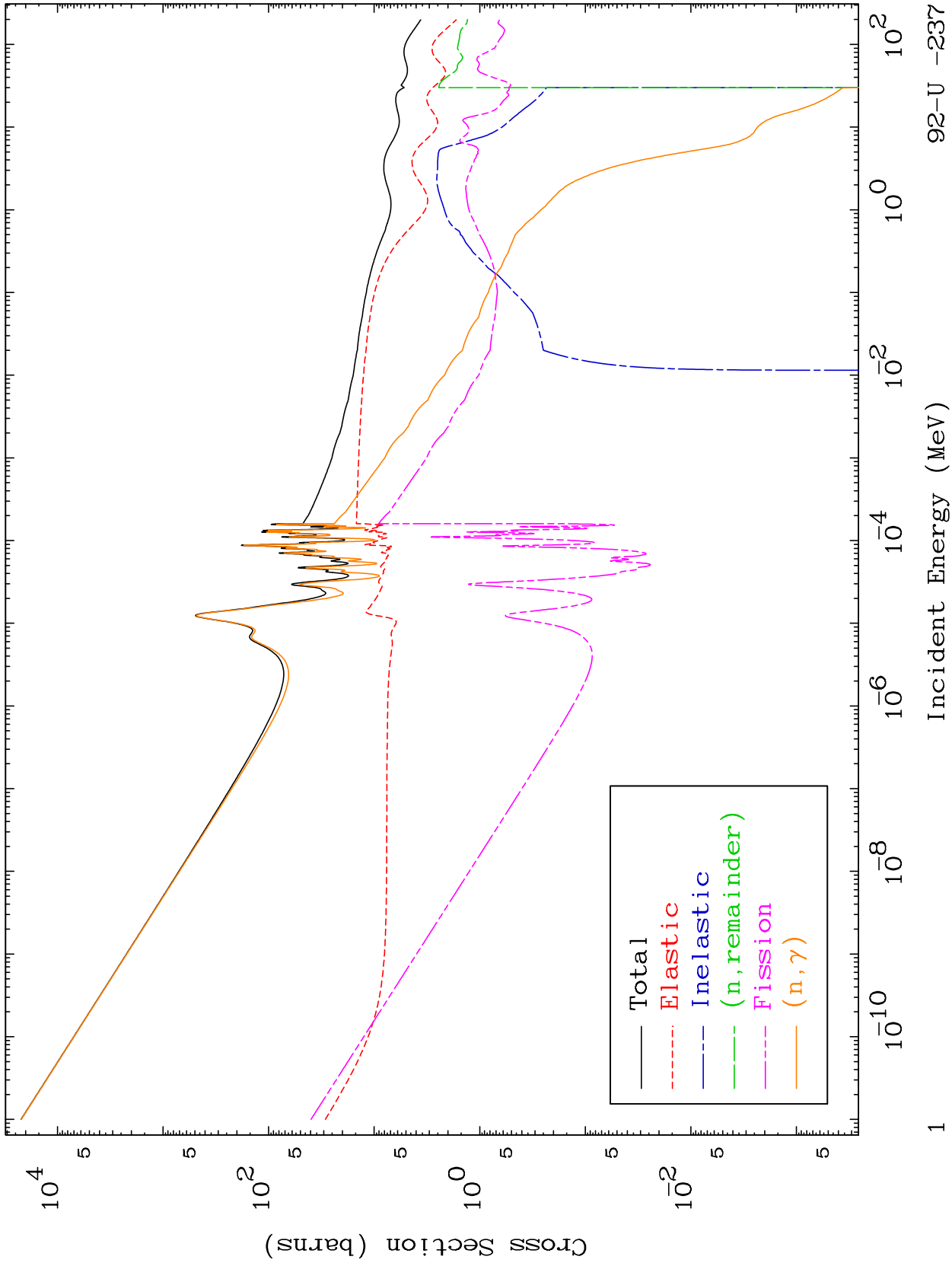
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9234

Major
293 Kelvin Cross Sections

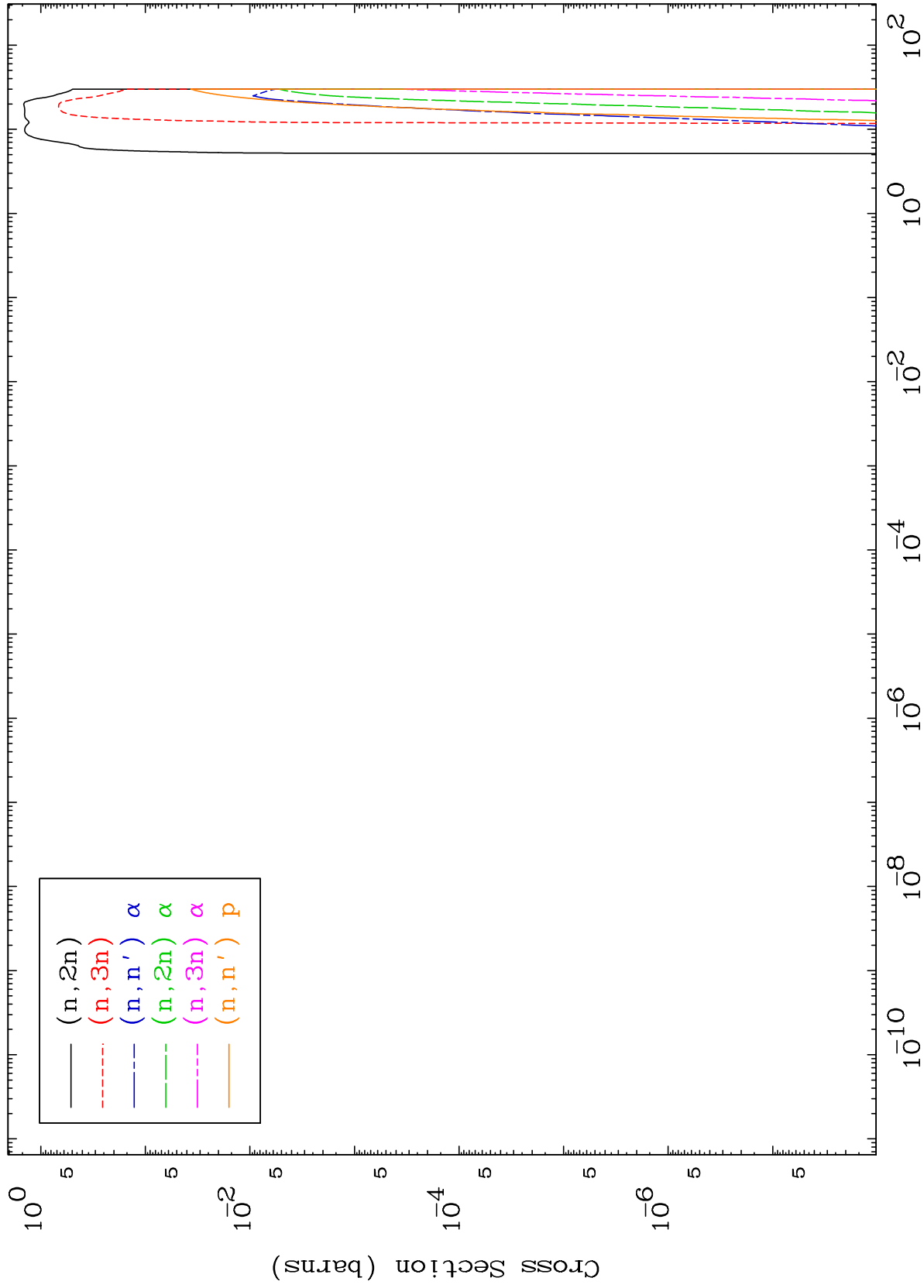
92-U -237



MAT 9234

Neutron Production
293 Kelvin Cross Sections

92-U -237



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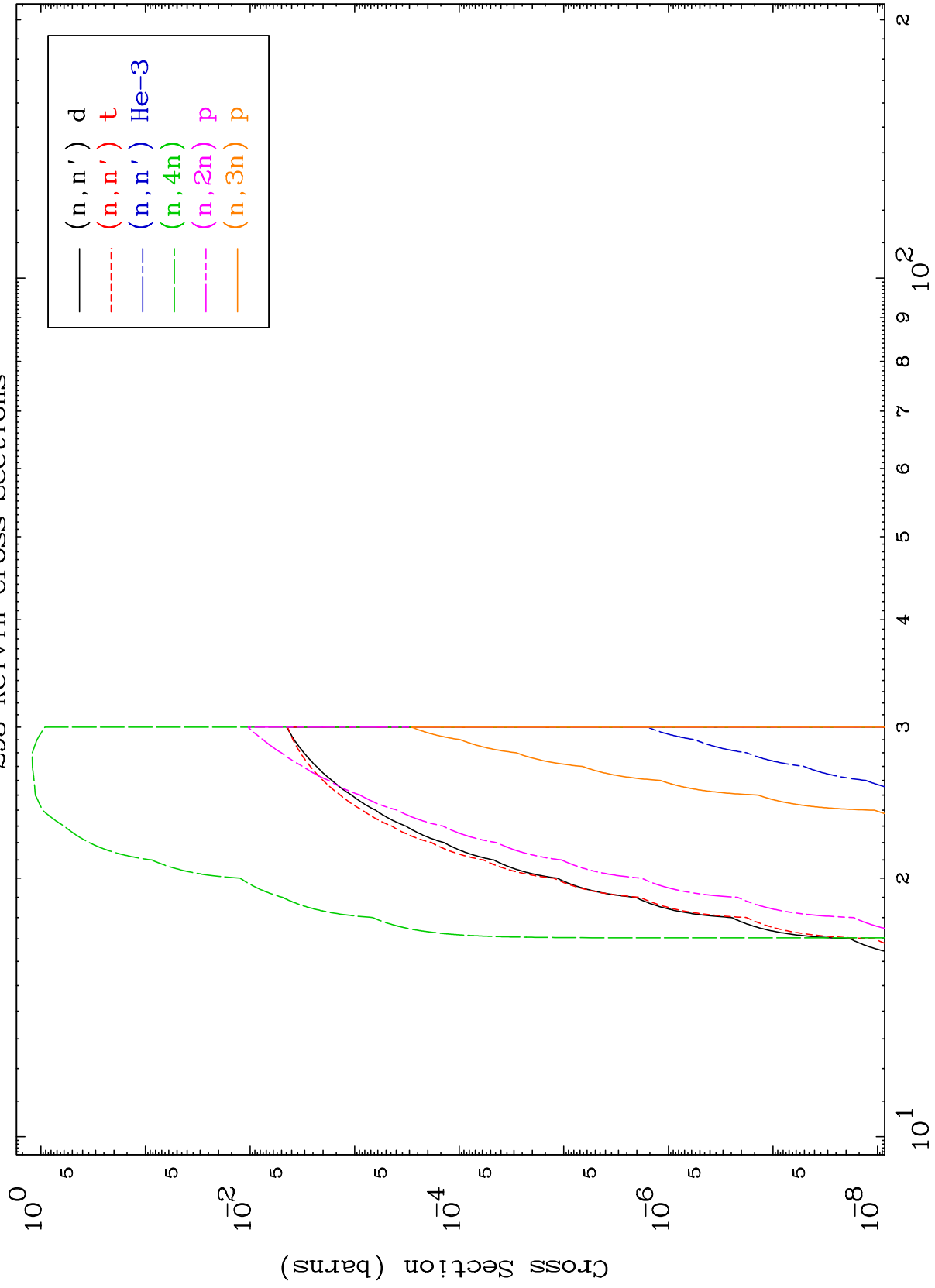
Incident Energy (MeV)

92-U -237

MAT 9234

Neutron Production
293 Kelvin Cross Sections

92-U -237



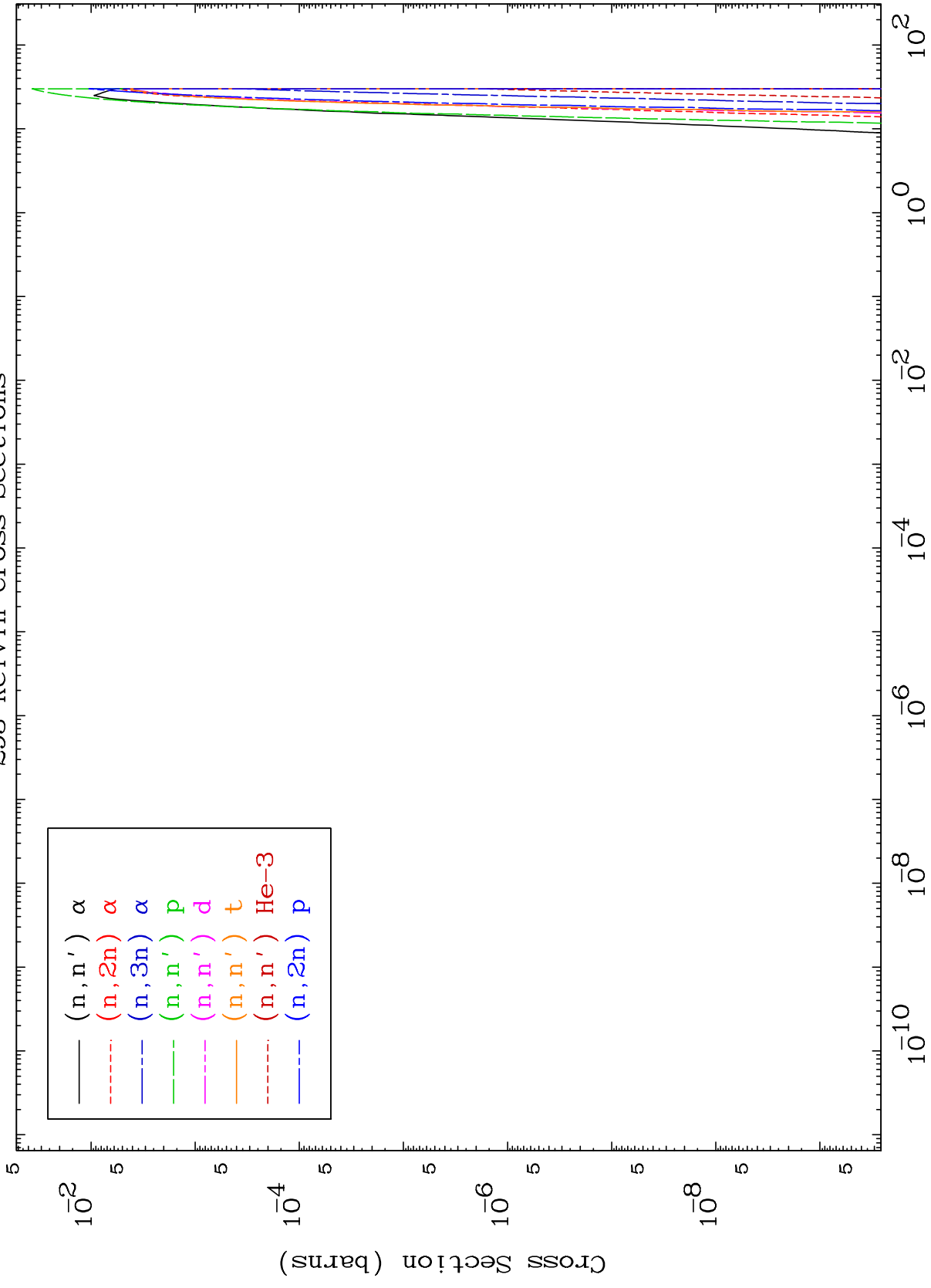
Incident Energy (MeV)

92-U -237

MAT 9234

293 Kelvin Cross Sections

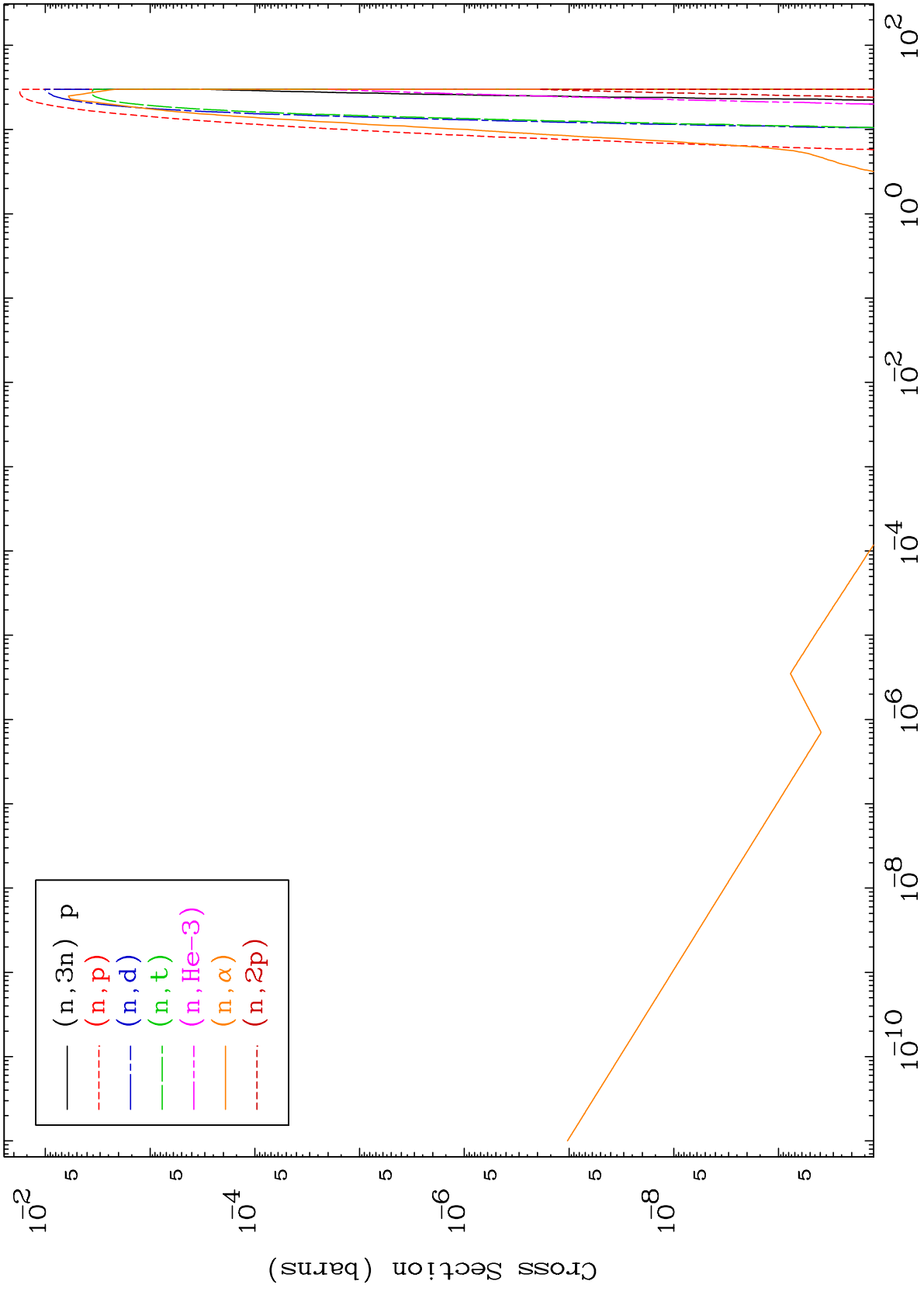
92-U -237



MAT 9234

293 Kelvin Cross Sections

92-U -237



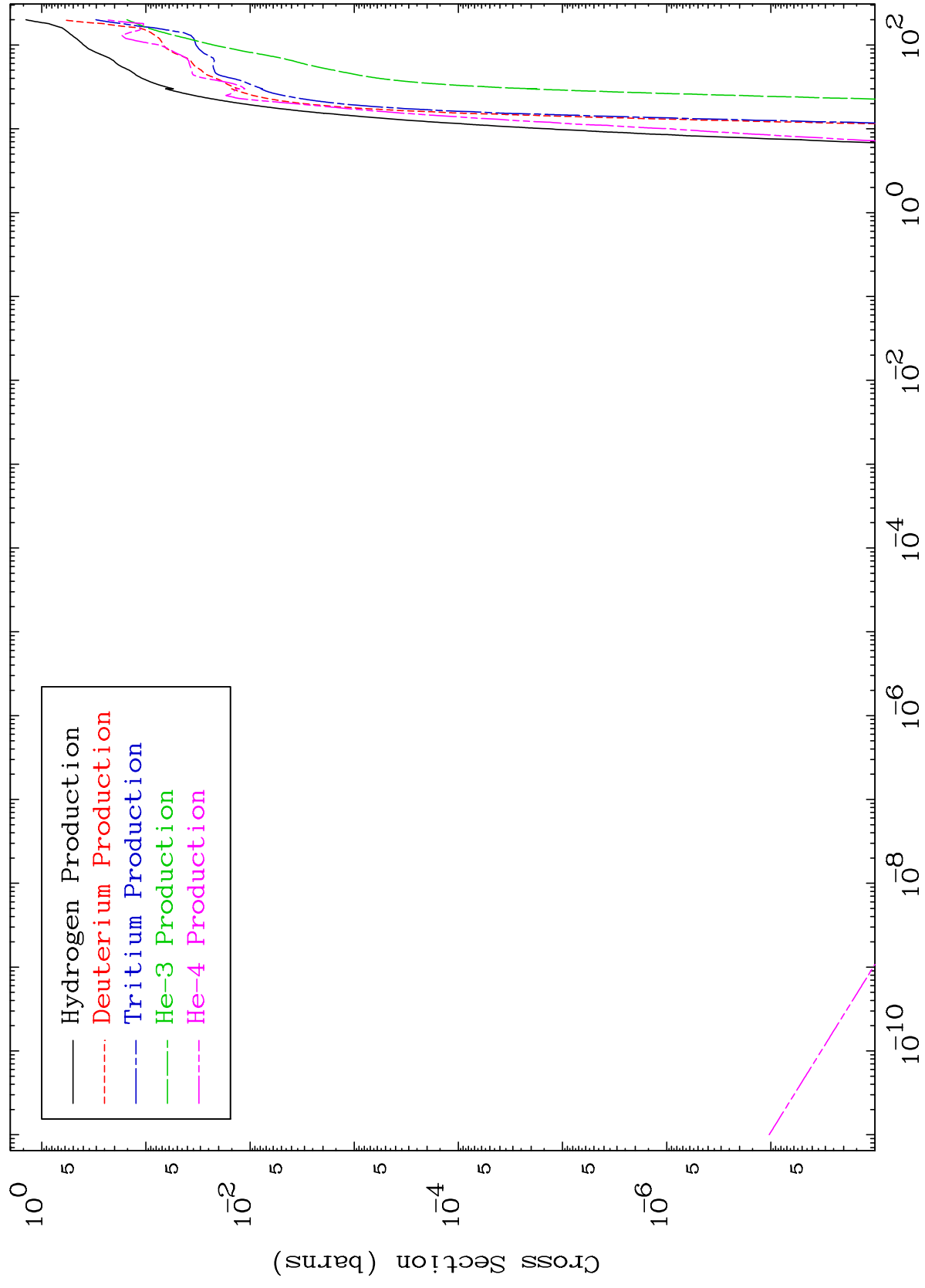
5

92-U -237

MAT 9234

Particle Production
293 Kelvin Cross Sections

92-U -237



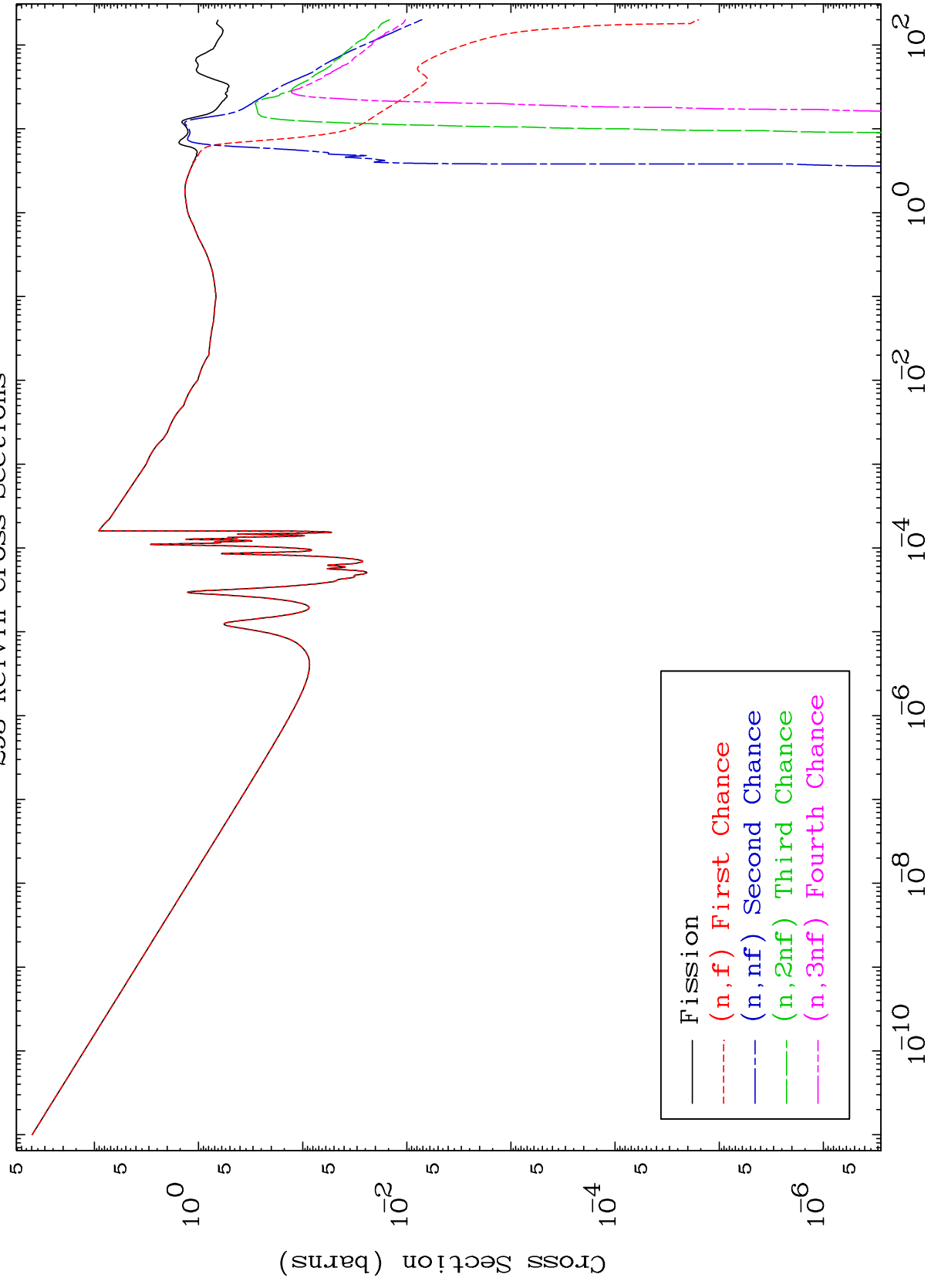
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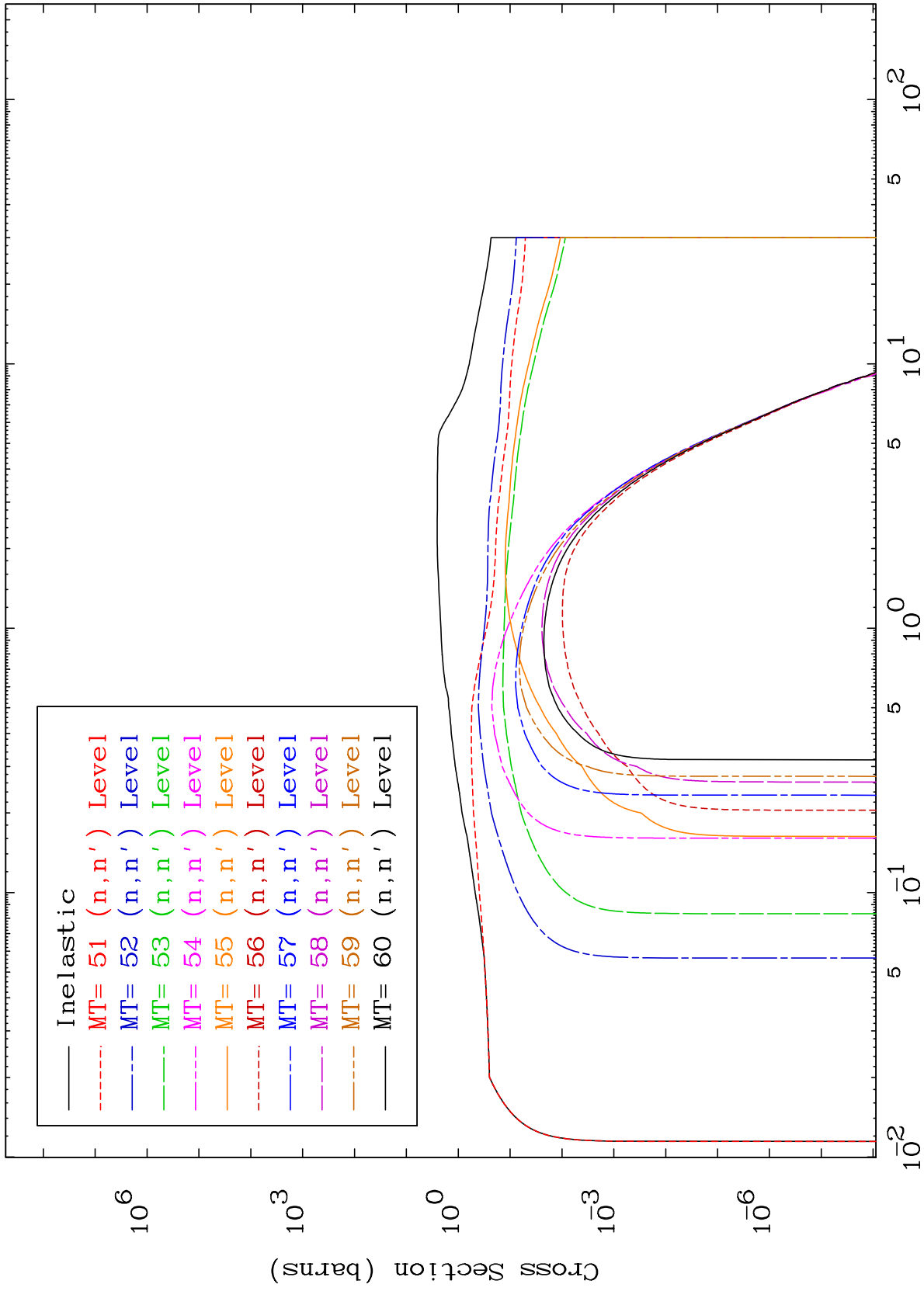
92-U -237

MAT 9234

Fission
293 Kelvin Cross Sections

92-U -237



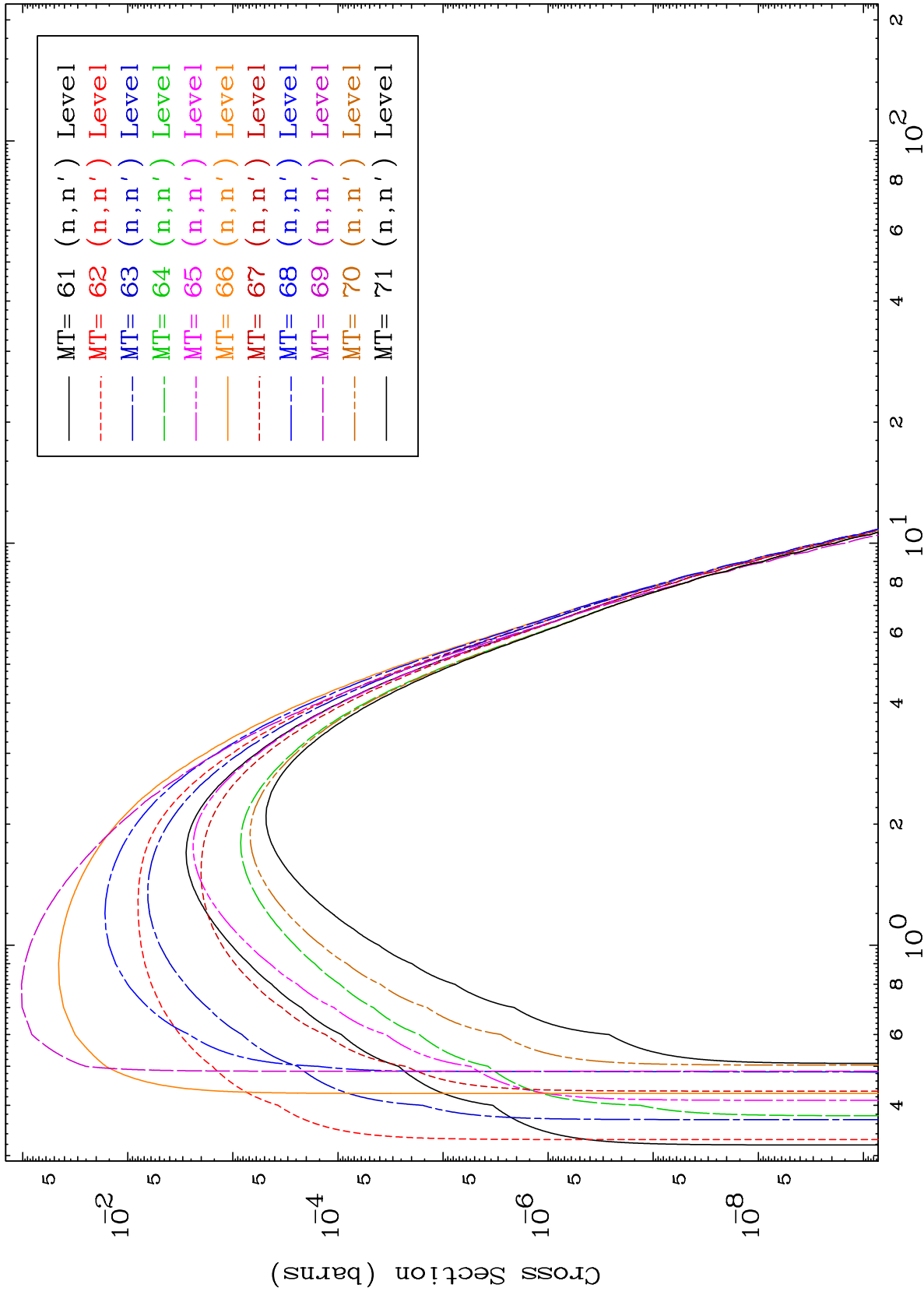


MAT 9234

(n,n') Level

92-U -237

293 Kelvin Cross Sections



9

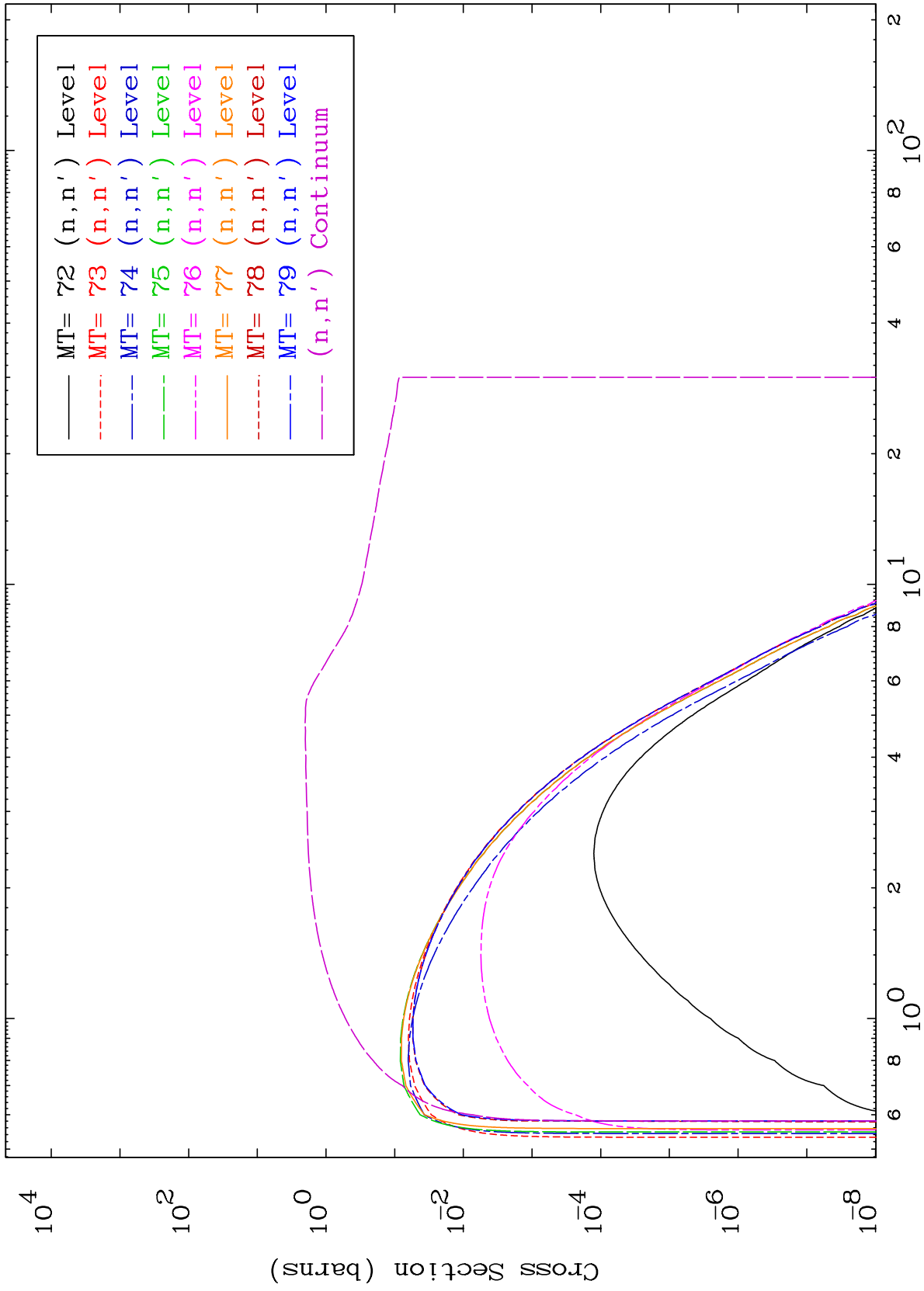
Incident Energy (MeV)

92-U -237

MAT 9234

(n,n') Level
293 Kelvin Cross Sections

92-U -237



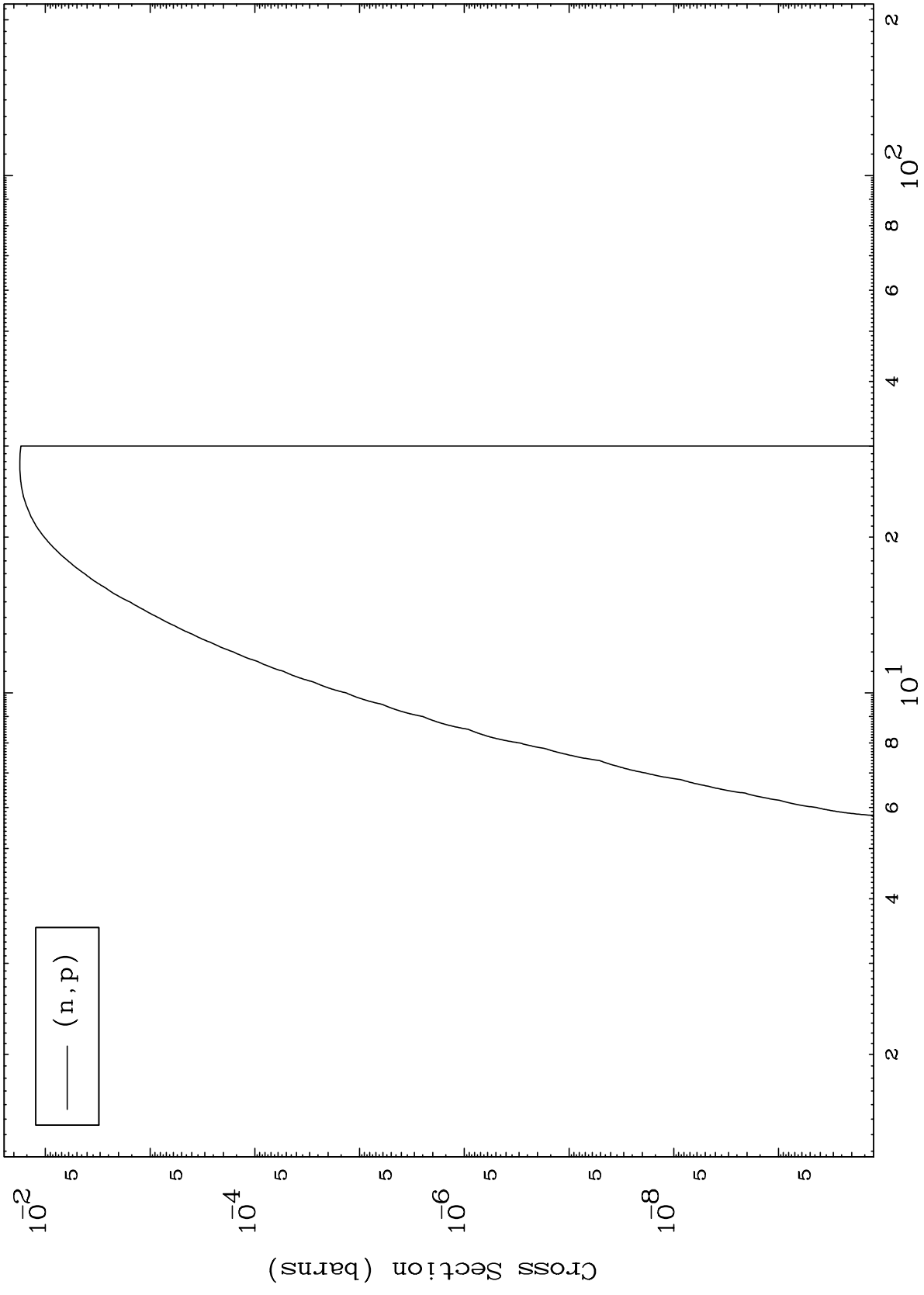
Incident Energy (MeV)

92-U -237

MAT 9234

(n,p) Levels
293 Kelvin Cross Sections

92-U -237



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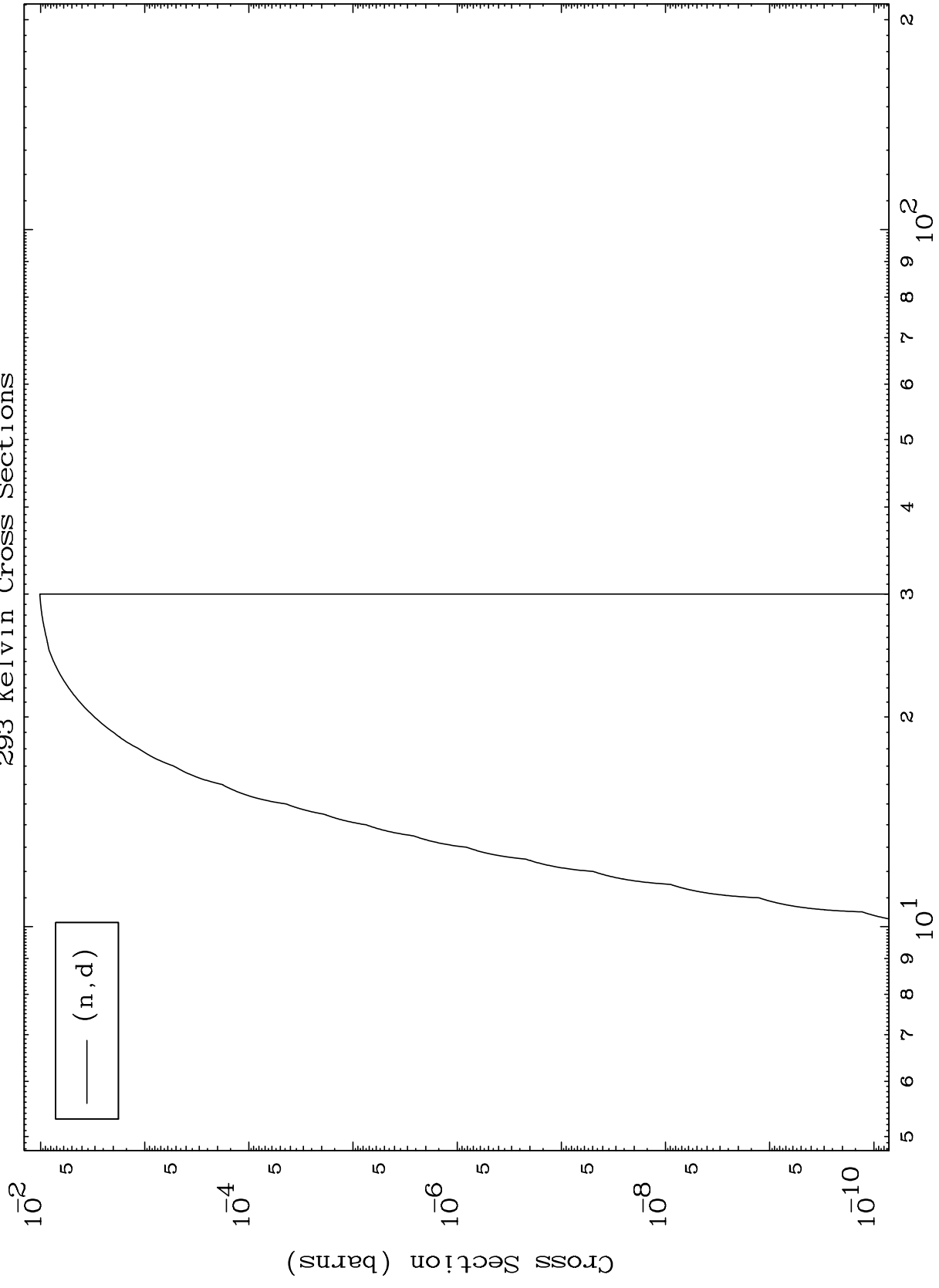
Incident Energy (MeV)

92-U -237

MAT 9234

(n,d) Levels
293 Kelvin Cross Sections

92-U -237



12

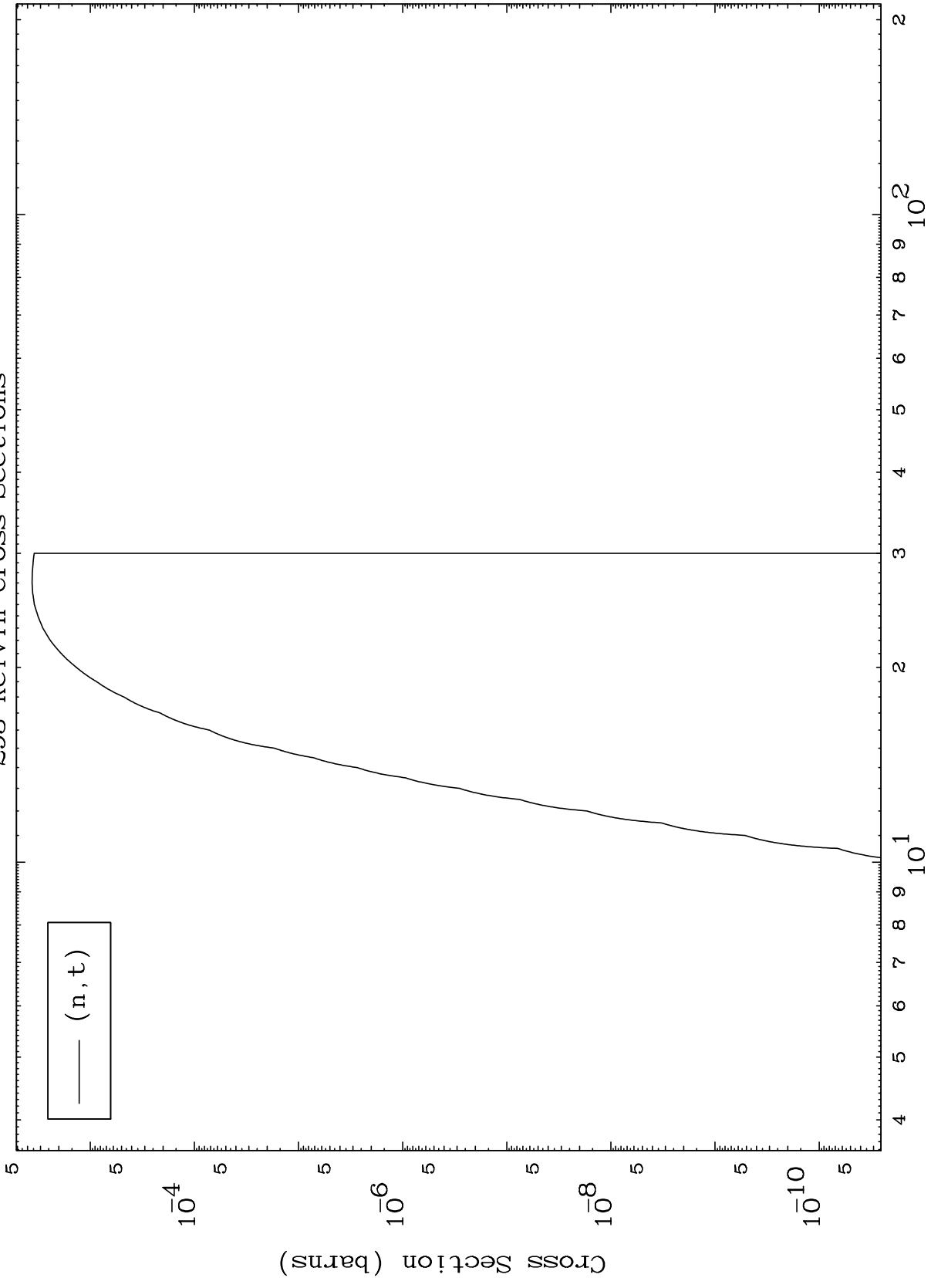
Incident Energy (MeV)

92-U -237

MAT 9234

(n,t) Levels
293 Kelvin Cross Sections

92-U -237



(n,t)

13

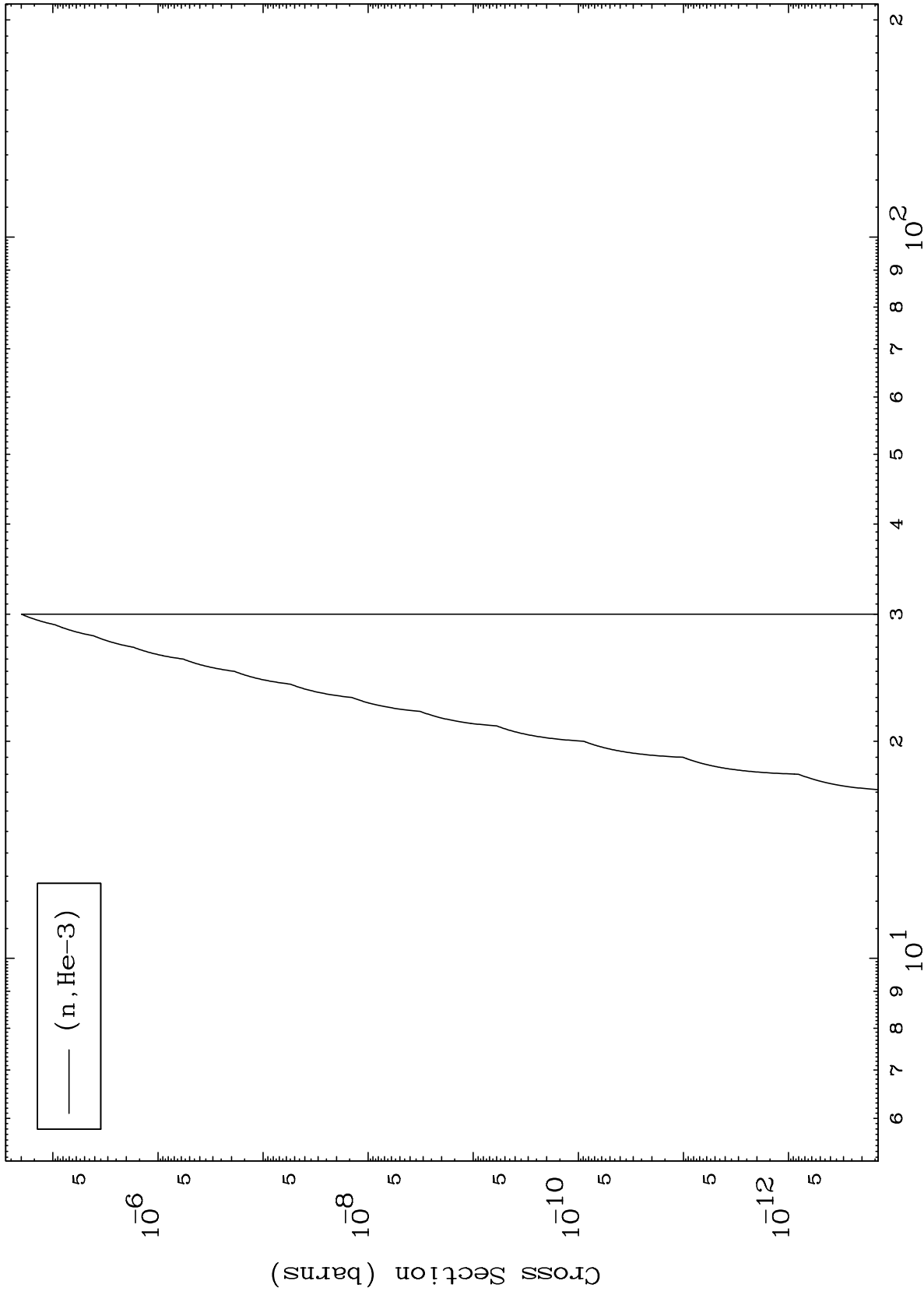
Incident Energy (MeV)

92-U -237

MAT 9234

(n,He3) Levels
293 Kelvin Cross Sections

92-U -237



14

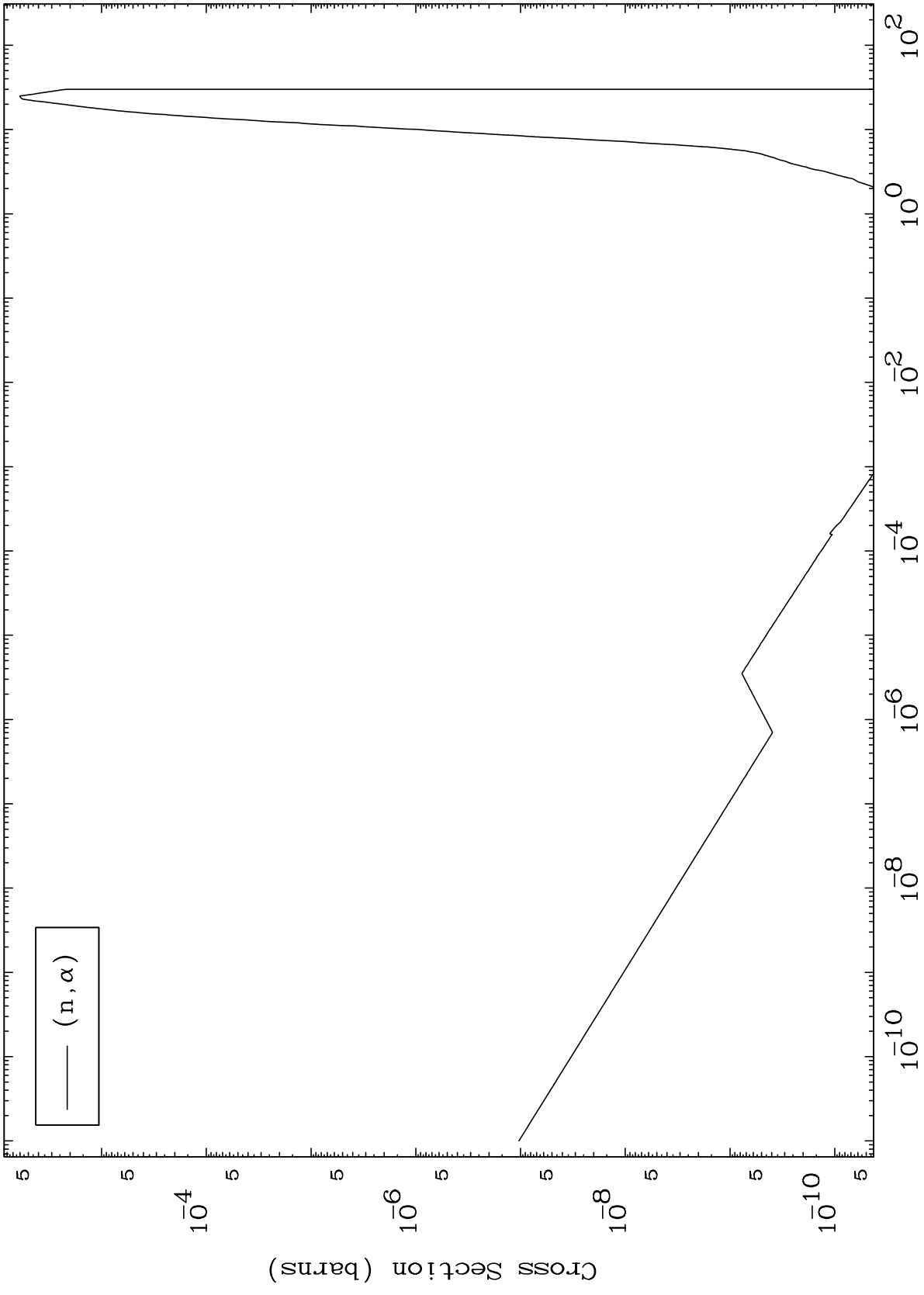
Incident Energy (MeV)

92-U -237

MAT 9234

(n,α) Levels
293 Kelvin Cross Sections

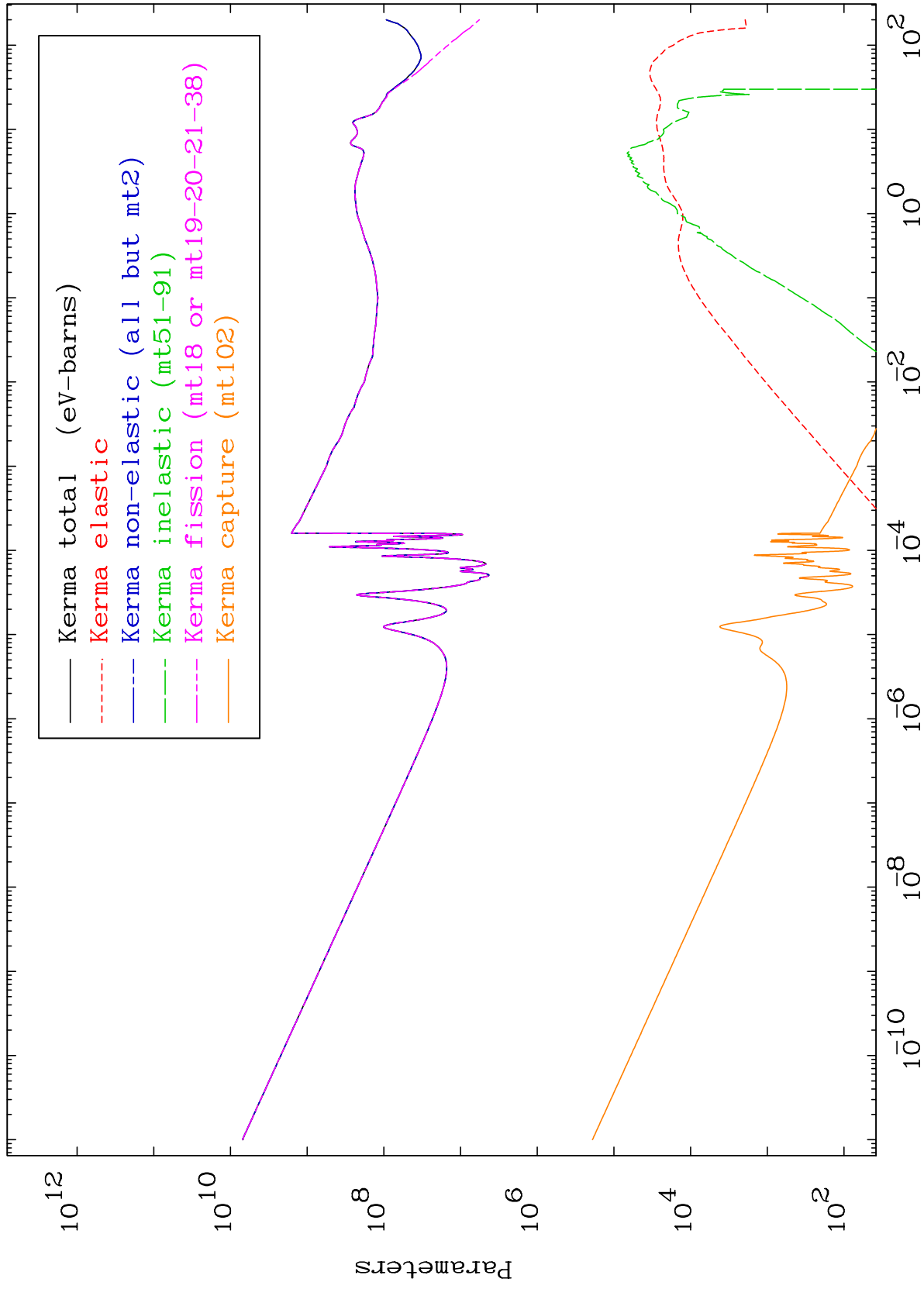
92-U -237

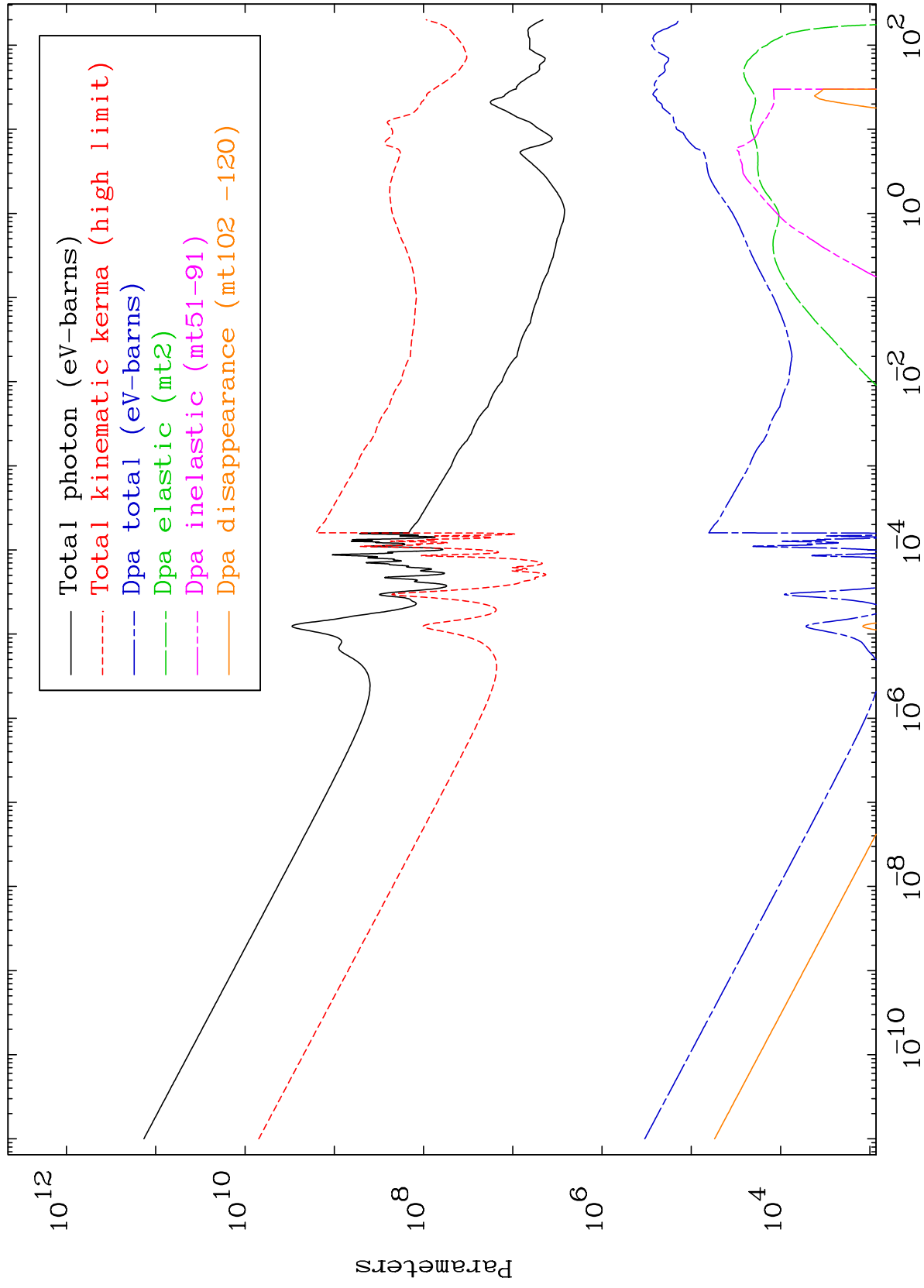


(n,α)

92-U -237

Incident Energy (MeV)

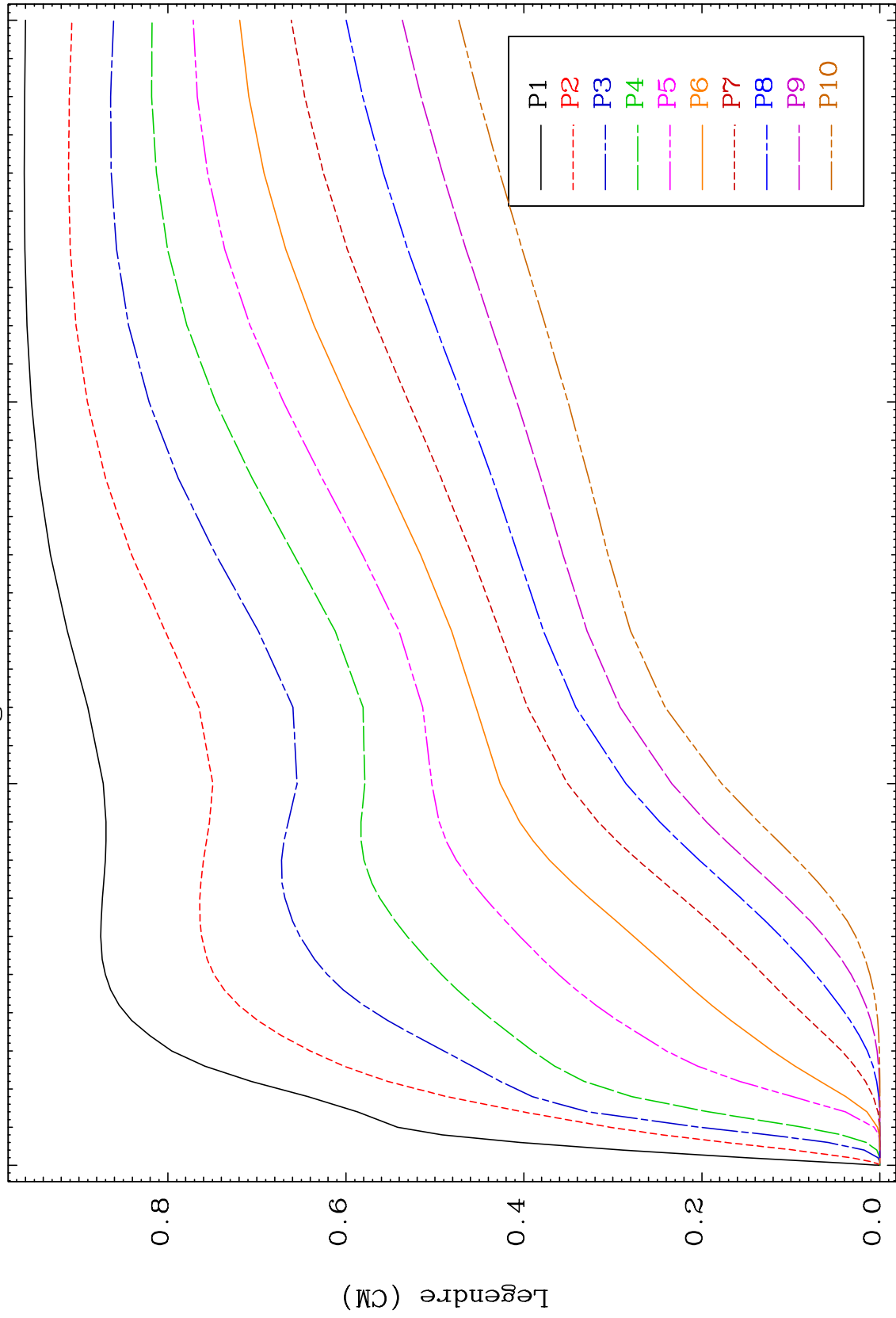




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Elastic Legendre Coefficients

92-U -237



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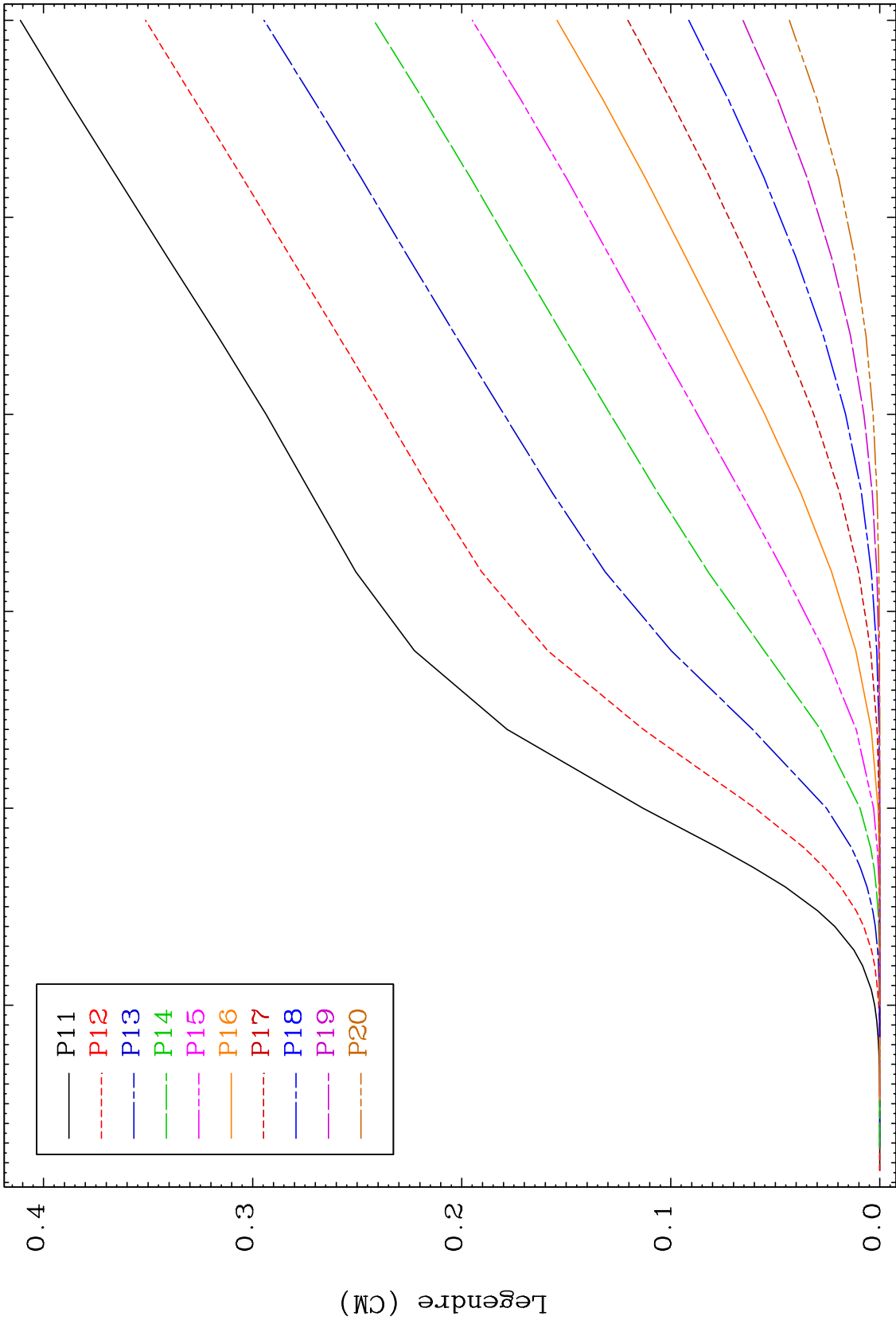
Incident Energy (MeV)

92-U -237

MAT 9234

Elastic Legendre Coefficients

92-U -237



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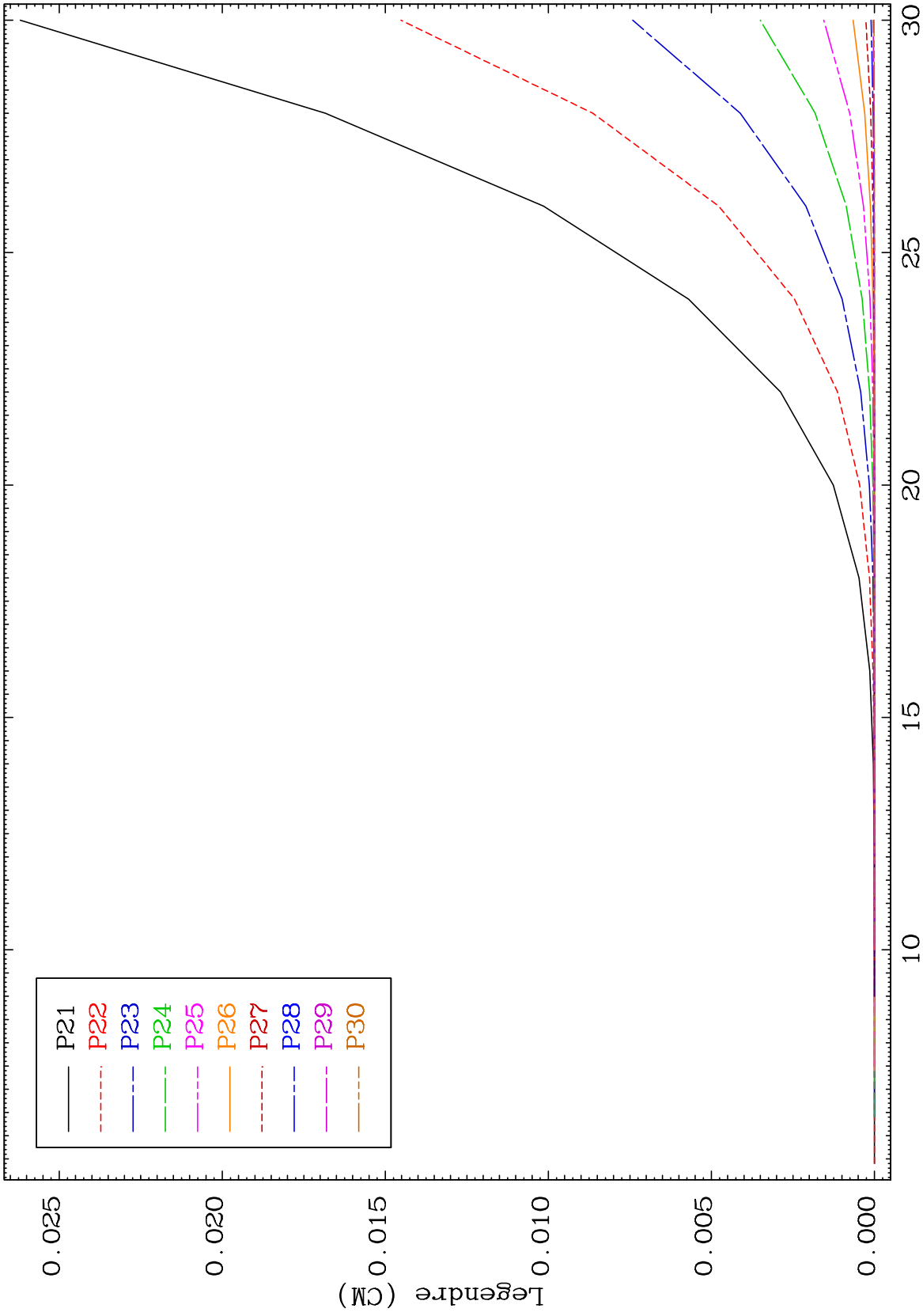
Incident Energy (MeV)

92-U -237

MAT 9234

Elastic Legendre Coefficients

92-U -237



20

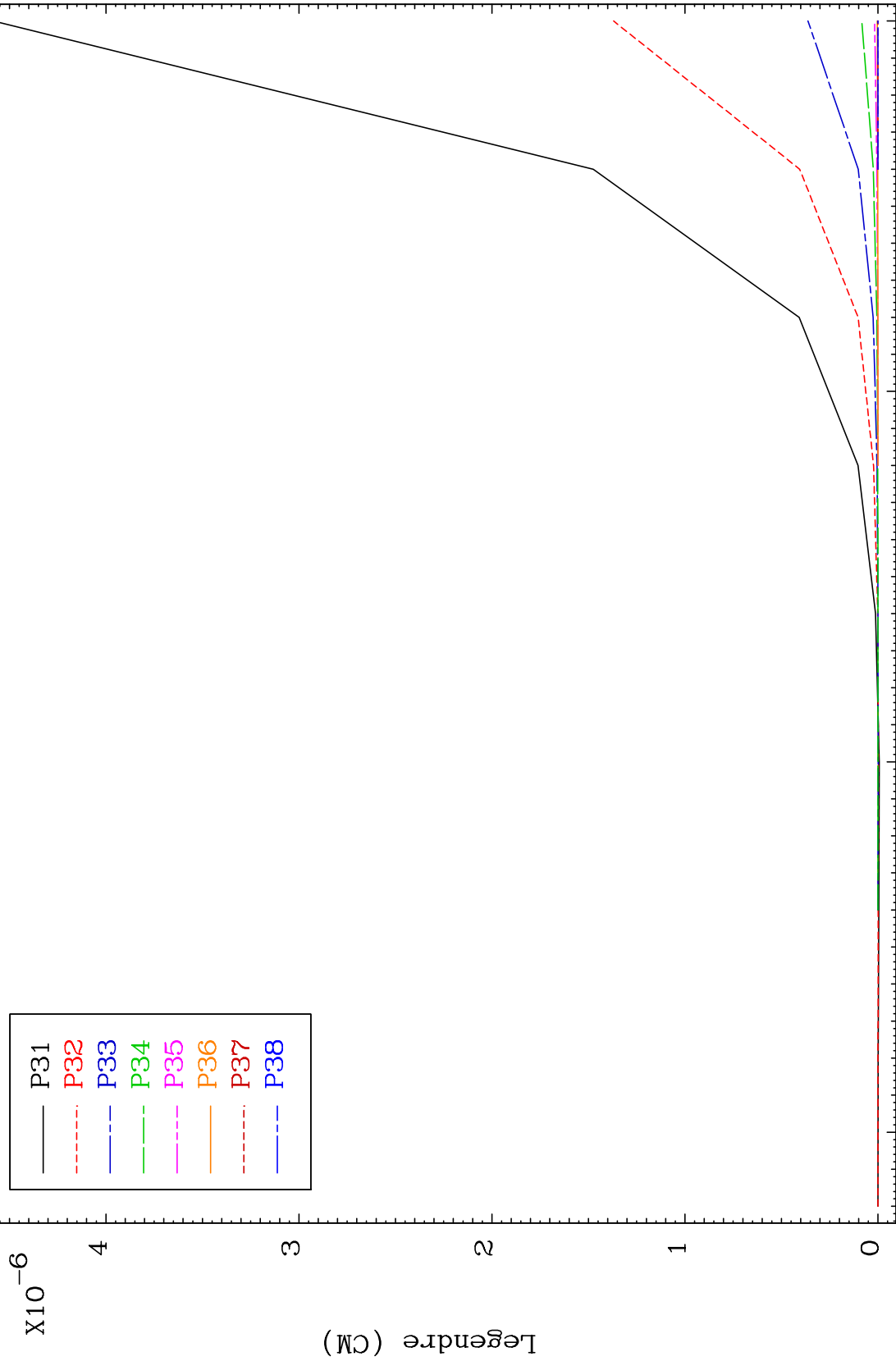
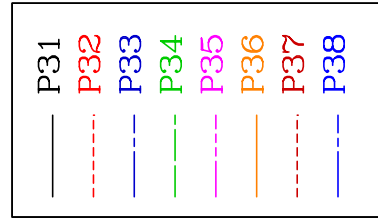
Incident Energy (MeV)

92-U -237

MAT 9234

Elastic Legendre Coefficients

92-U -237



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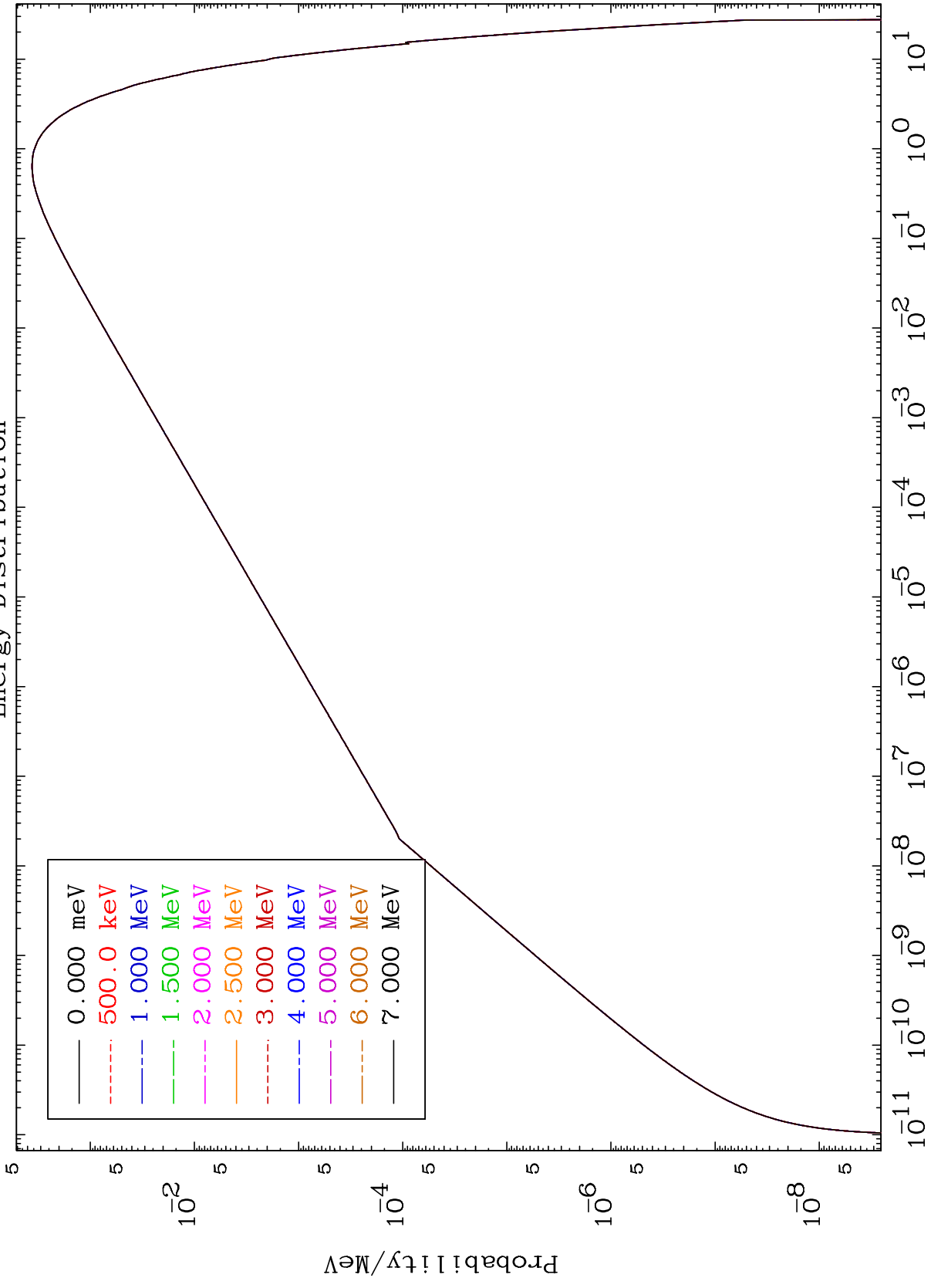
Incident Energy (MeV)

92-U -237

MAT 9234

Fission Energy Distribution

92-U -237

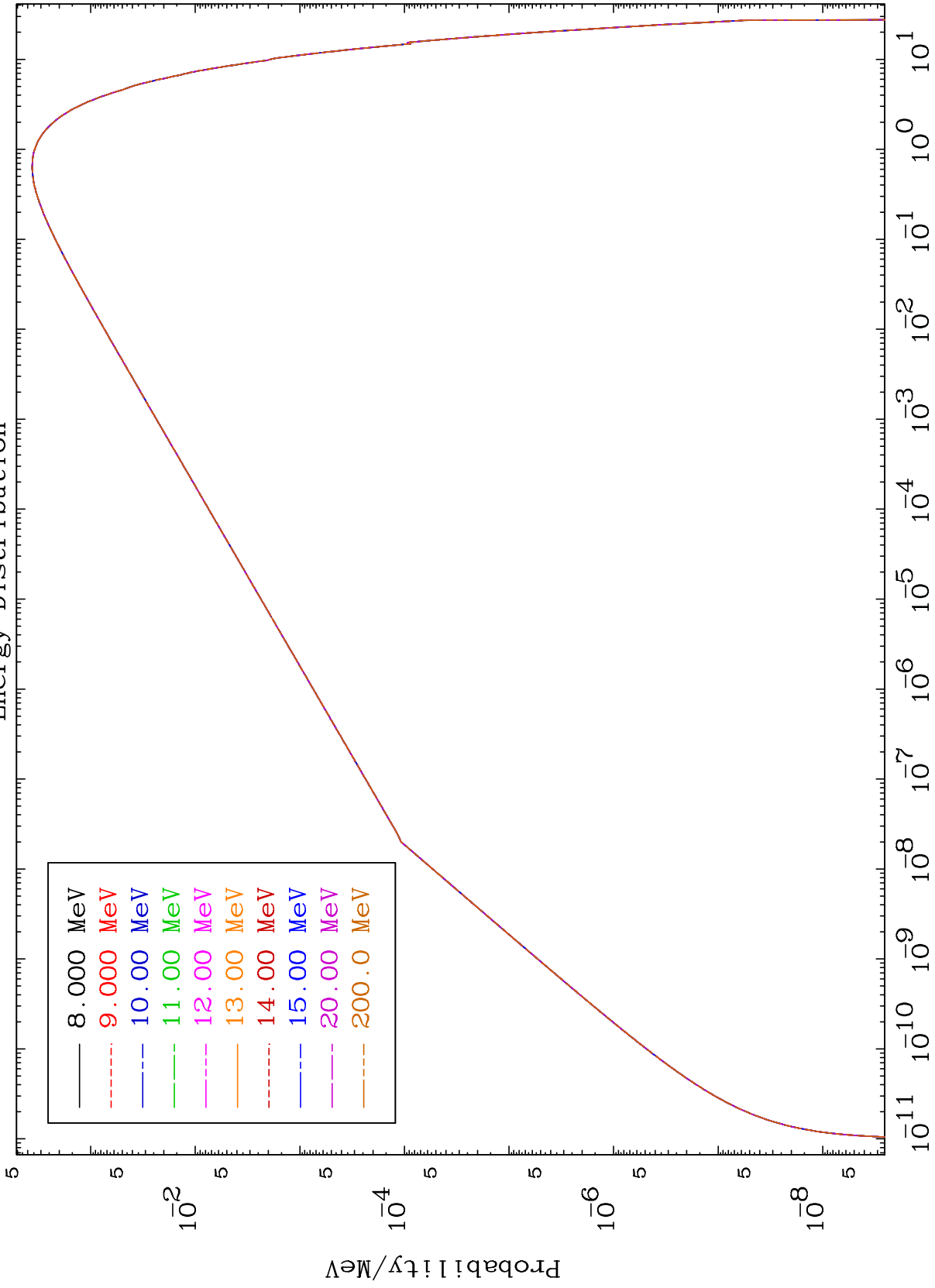


92-U -237

MAT 9234

Fission Energy Distribution

92-U -237



23

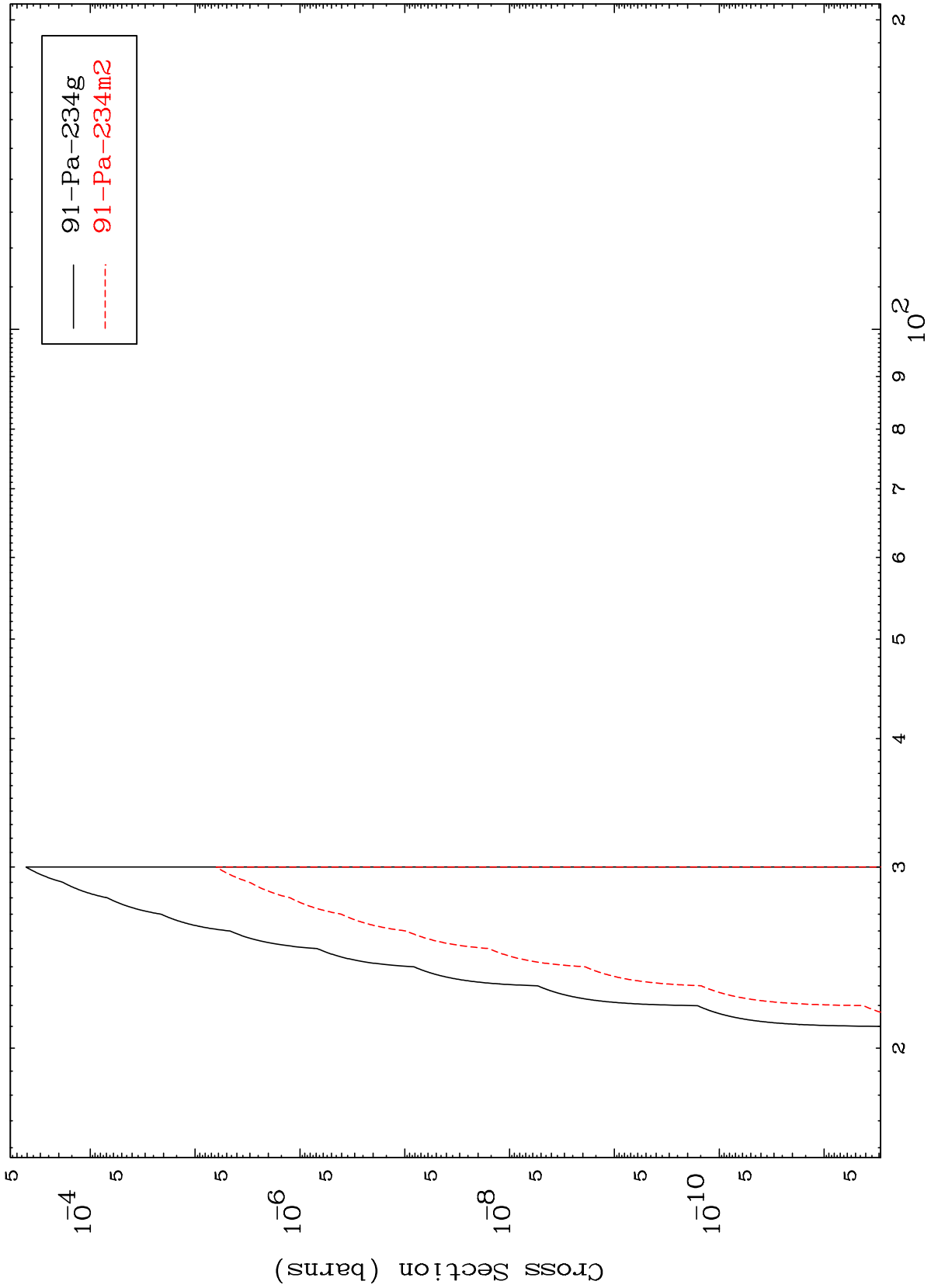
92-U -237

MAT 9234

(n,2n) d

92-U -237

Radionuclide Production Cross Section



24

Incident Energy (MeV)

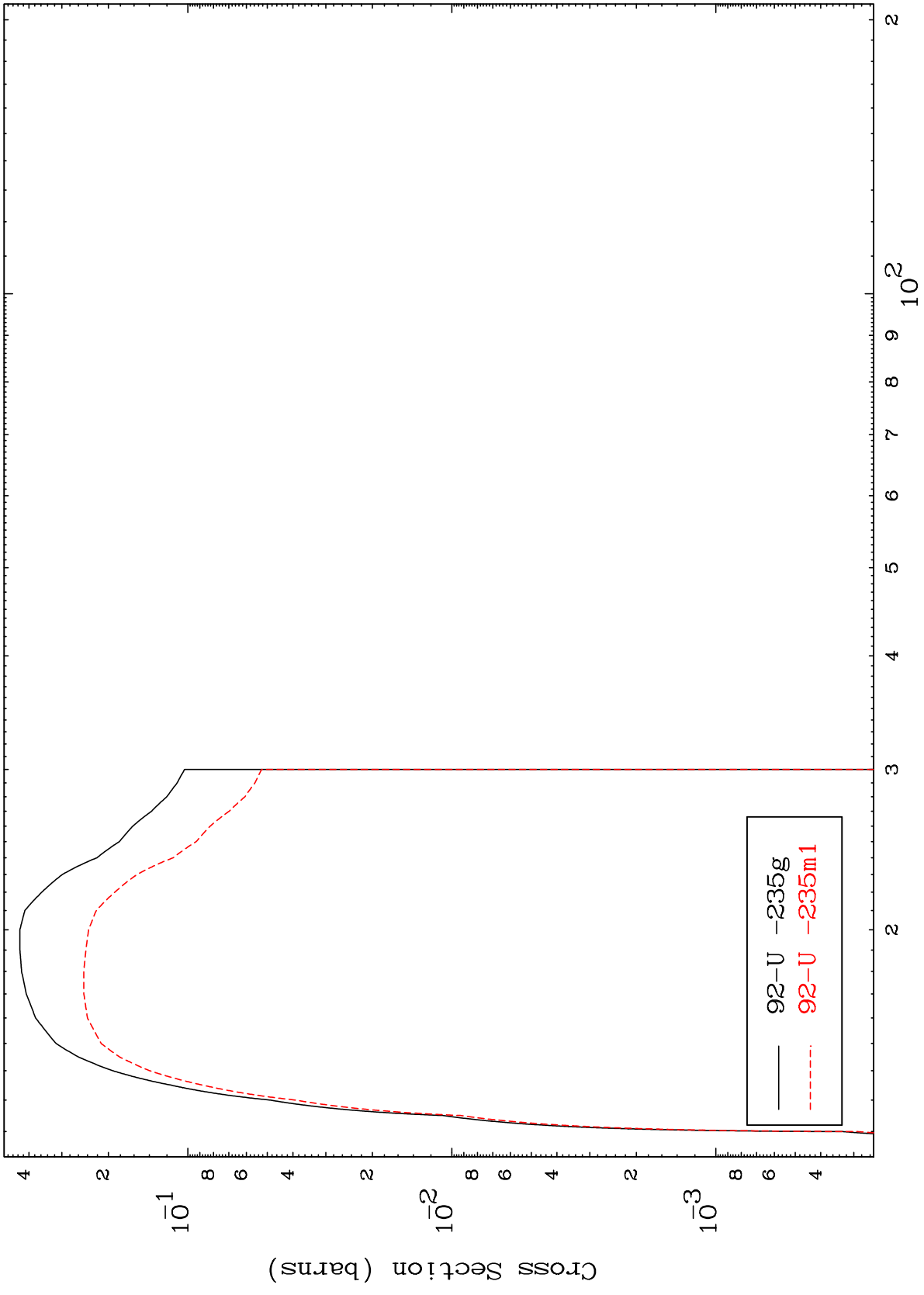
92-U -237

MAT 9234

(n,3n)

92-U -237

Radionuclide Production Cross Section



25

Incident Energy (MeV)

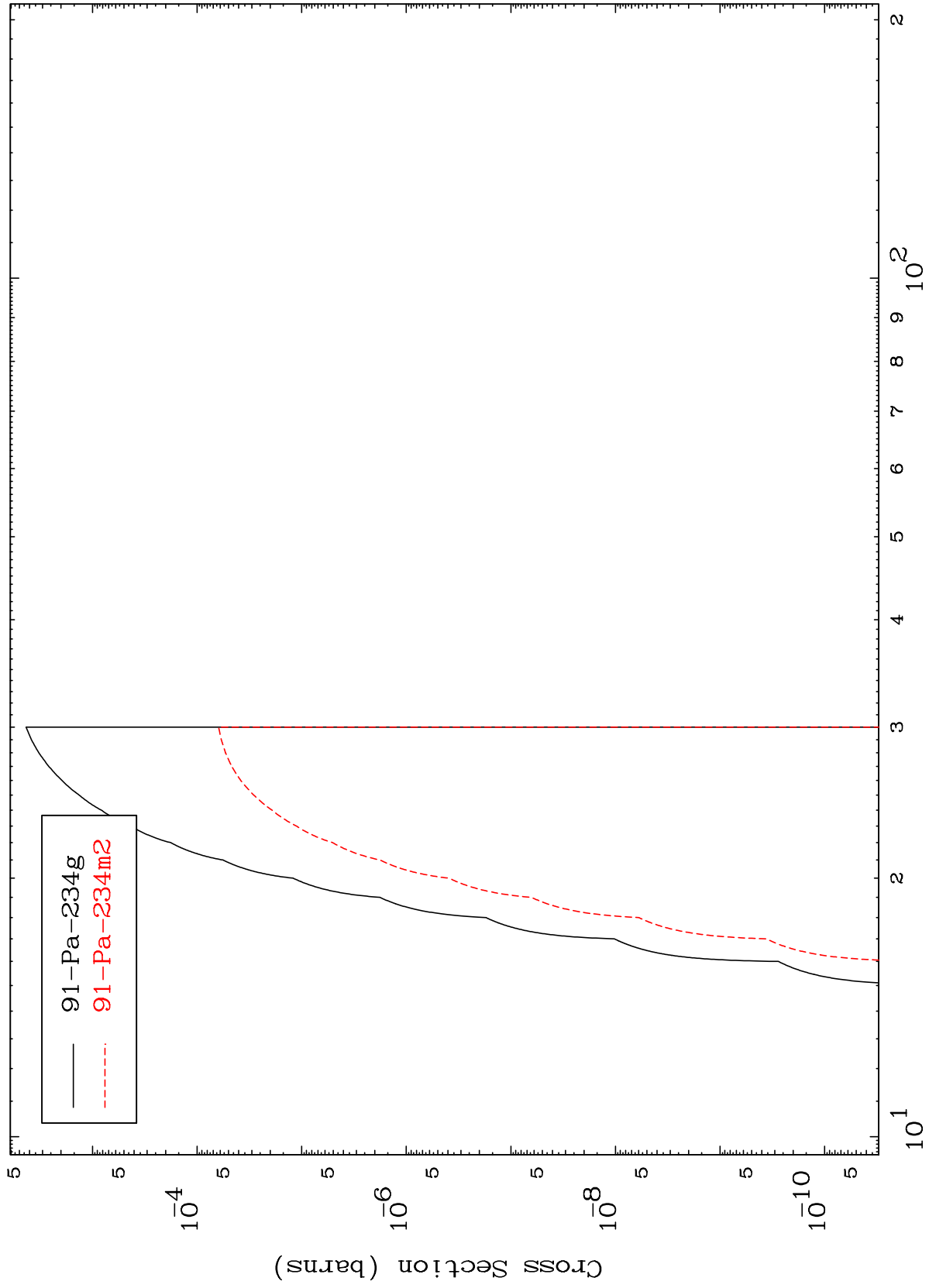
92-U -237

MAT 9234

(n,n') t

92-U -237

Radionuclide Production Cross Section



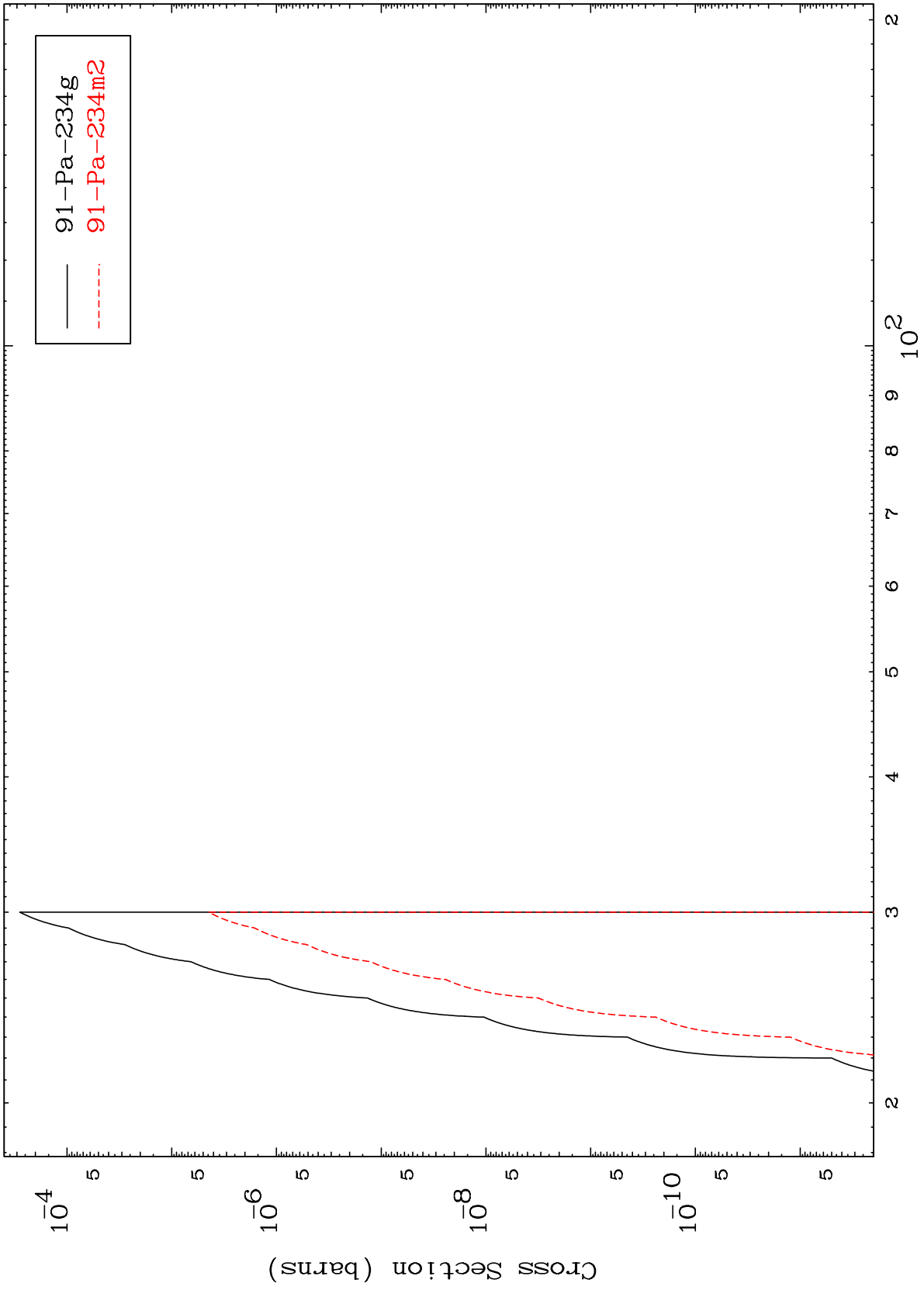
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(n,3n) p

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Radionuclide Production Cross Section



27

Incident Energy (MeV)

92-U -237