

Program EVALPLOT
(Version 2018-1)

by

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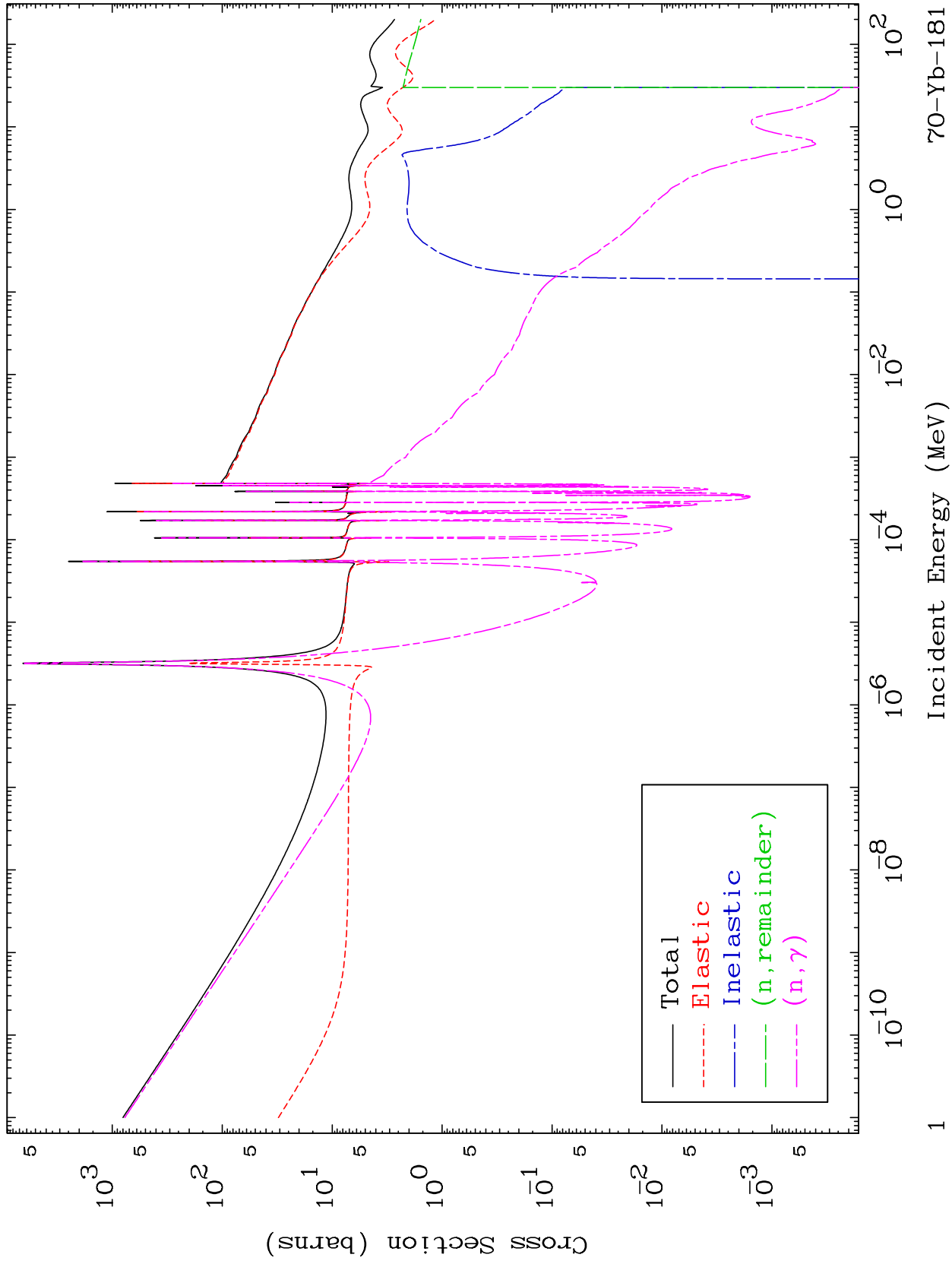
Press Mouse Button to Start

MAT 7064

Major

293 Kelvin Cross Sections

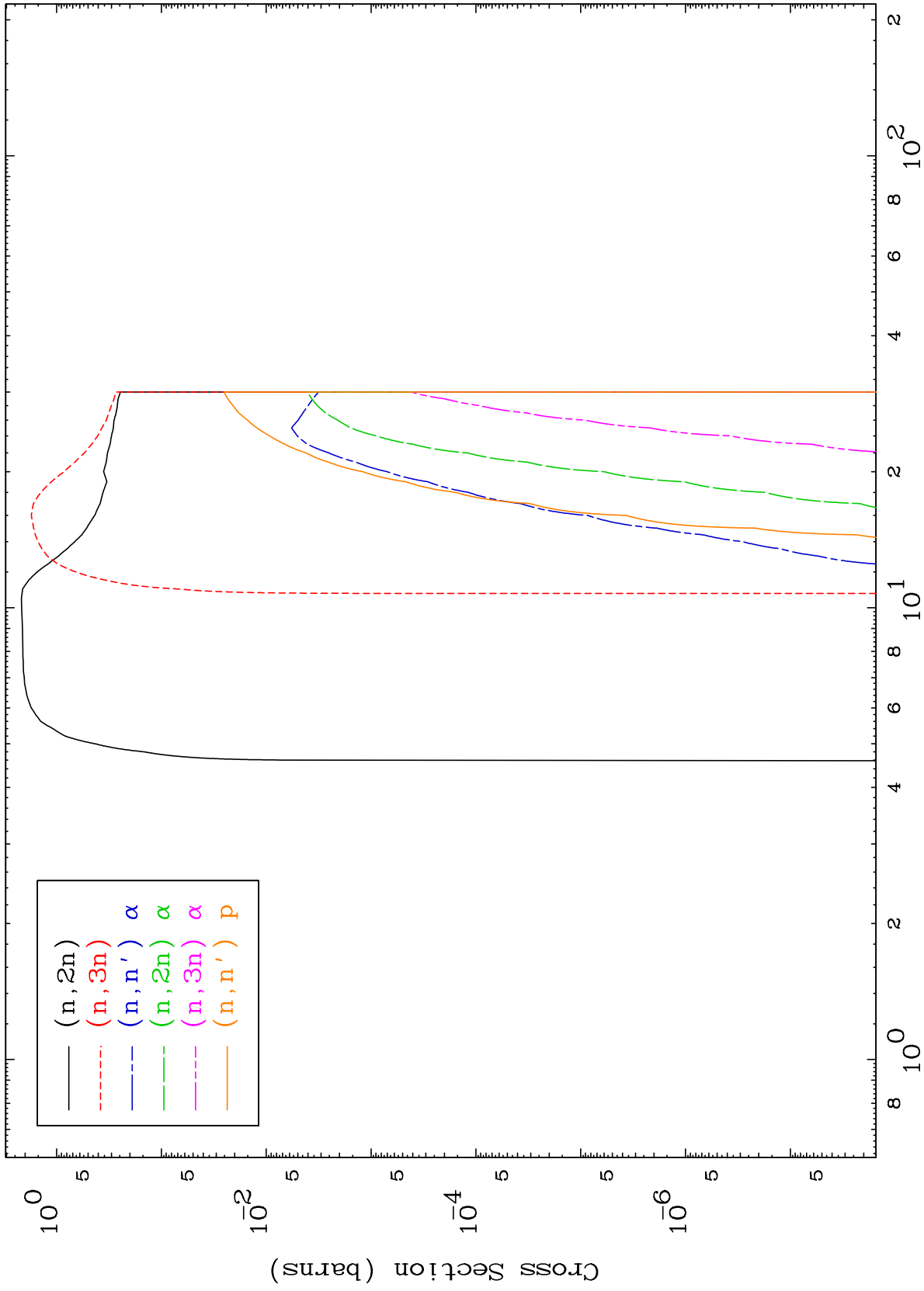
70-Yb-181



MAT 7064

Neutron Production
293 Kelvin Cross Sections

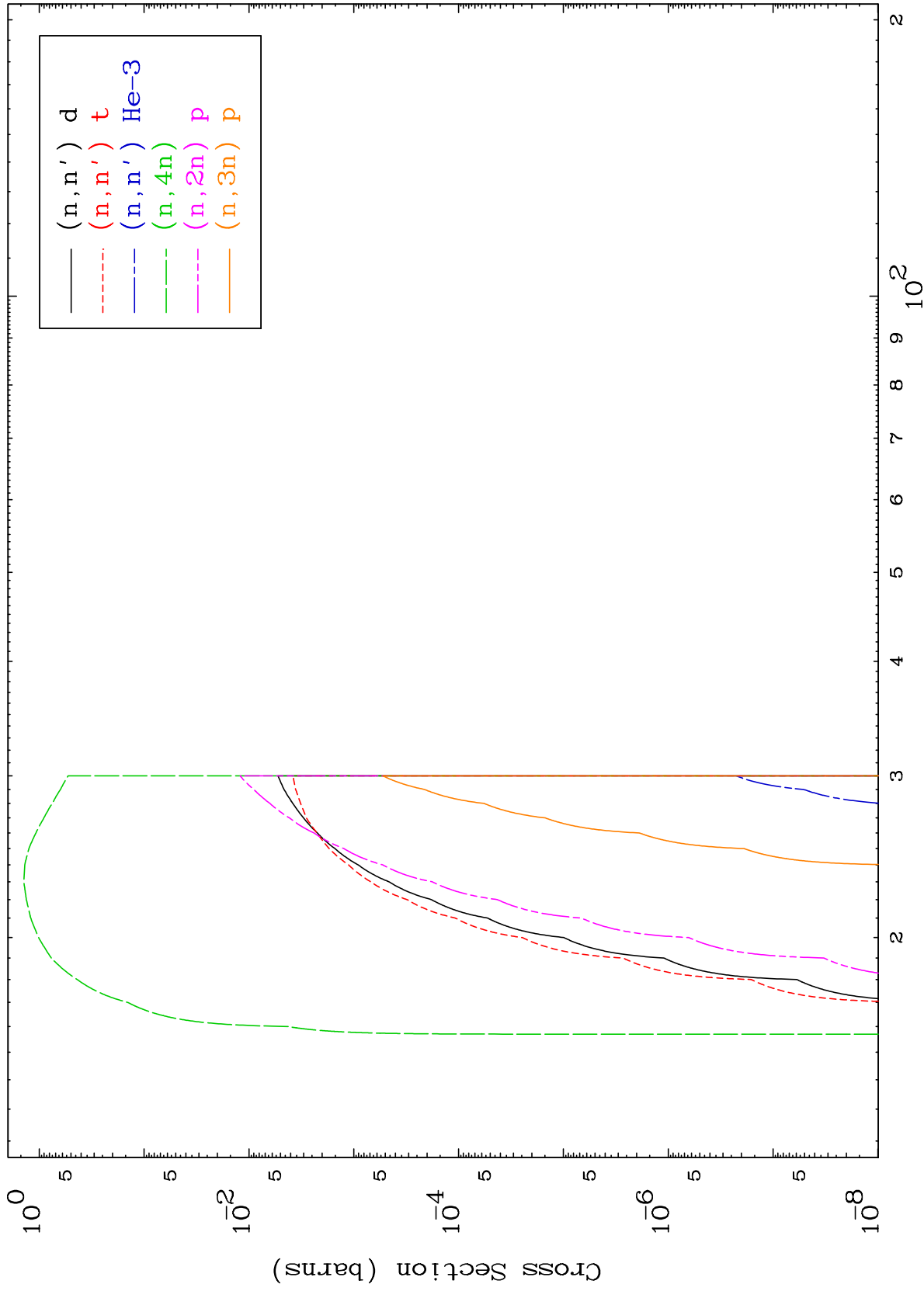
70-Yb-181



2

Incident Energy (MeV)

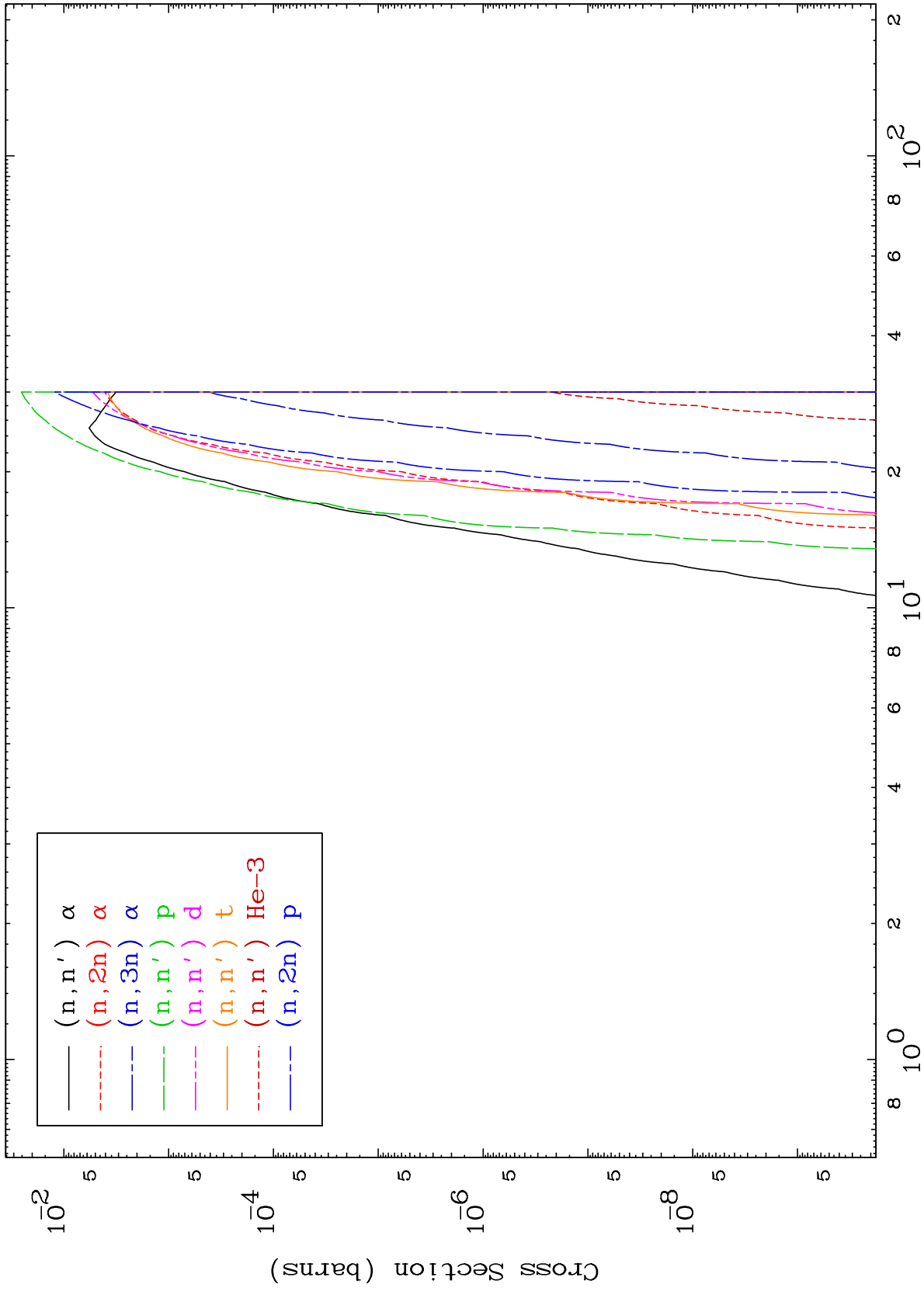
70-Yb-181



MAT 7064

293 Kelvin Cross Sections

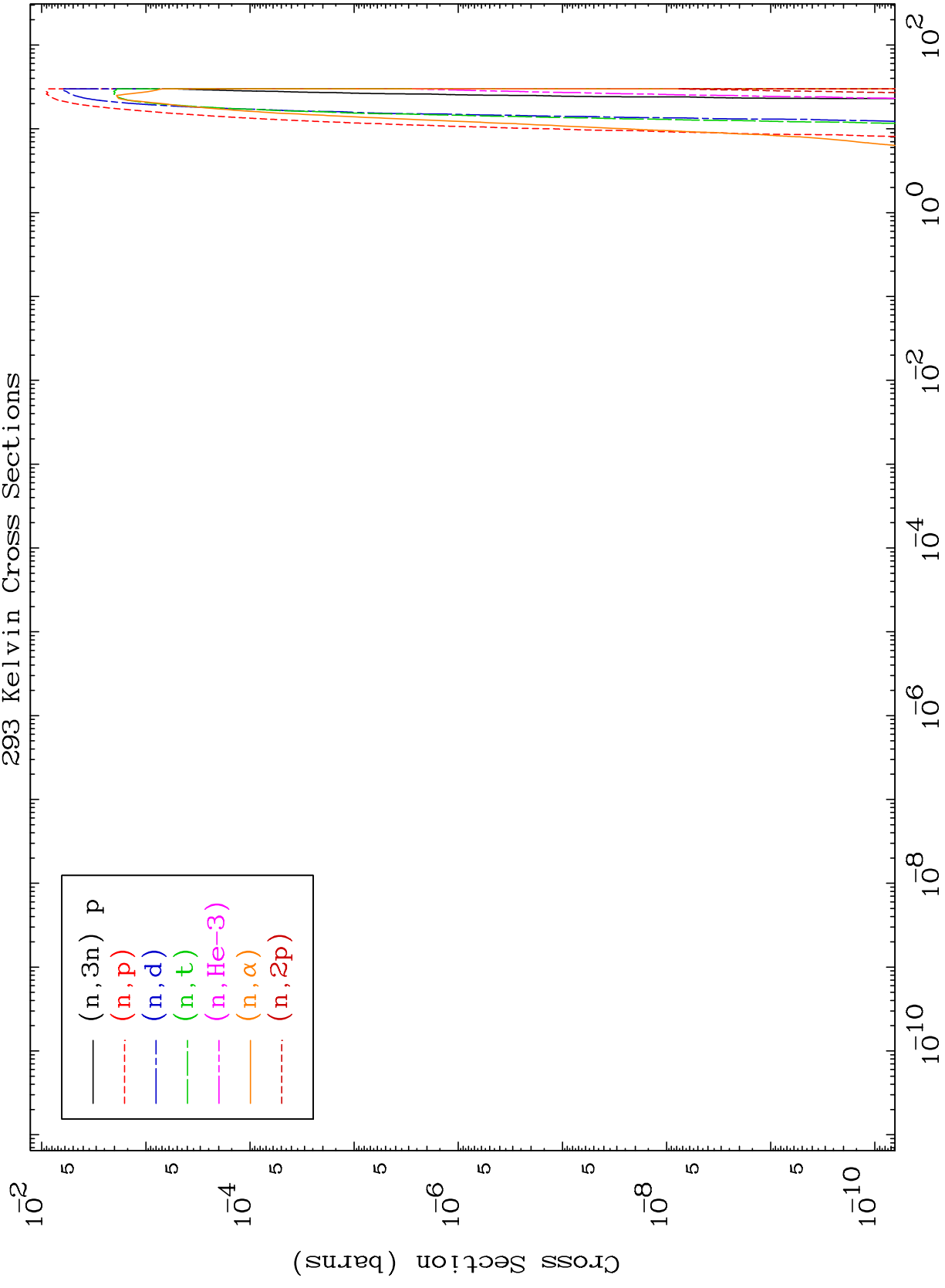
70-Yb-181



MAT 7064

293 Kelvin Cross Sections

70-Yb-181



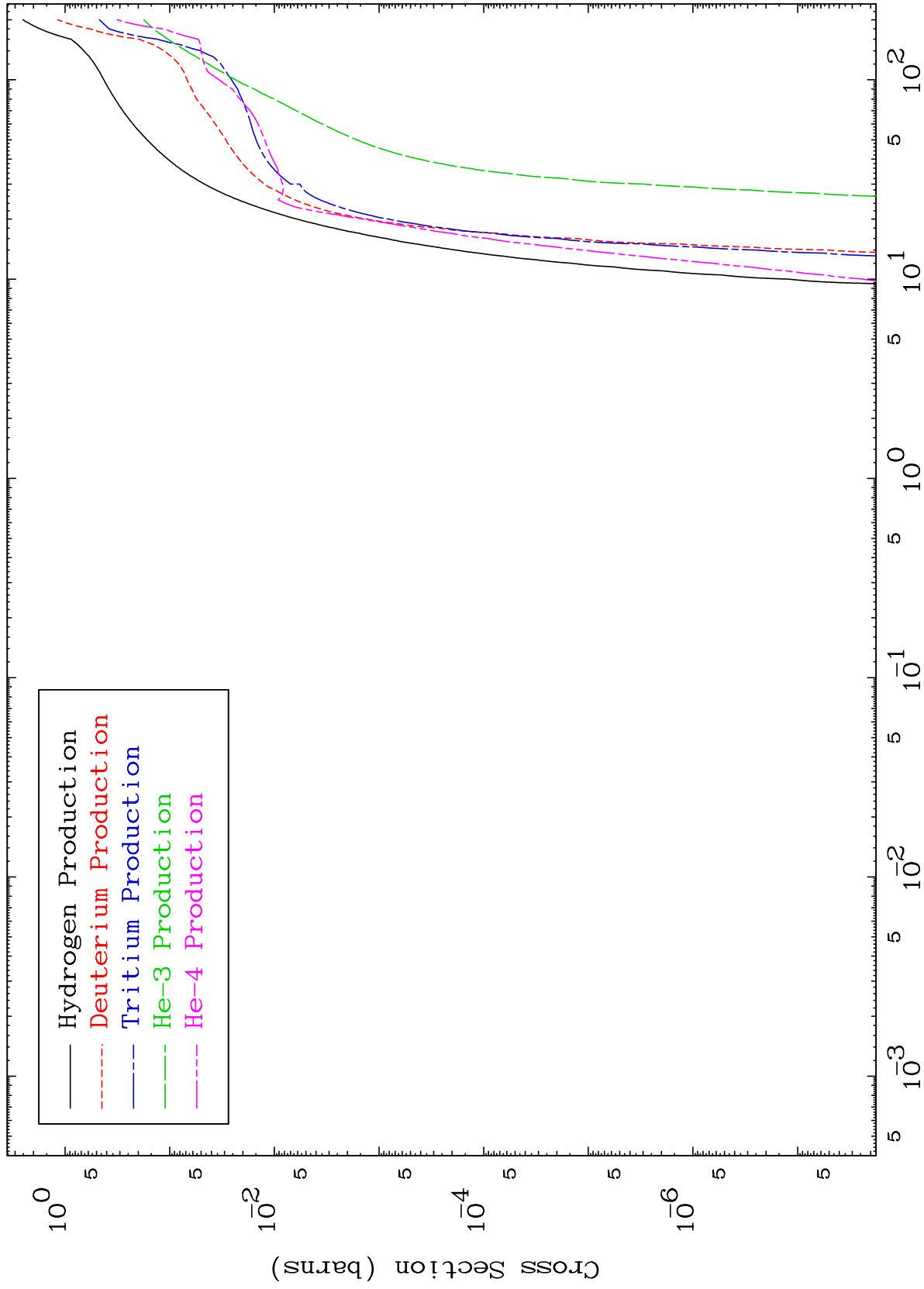
5

70-Yb-181

MAT 7064

Particle Production
293 Kelvin Cross Sections

70-Yb-181



6

Incident Energy (MeV)

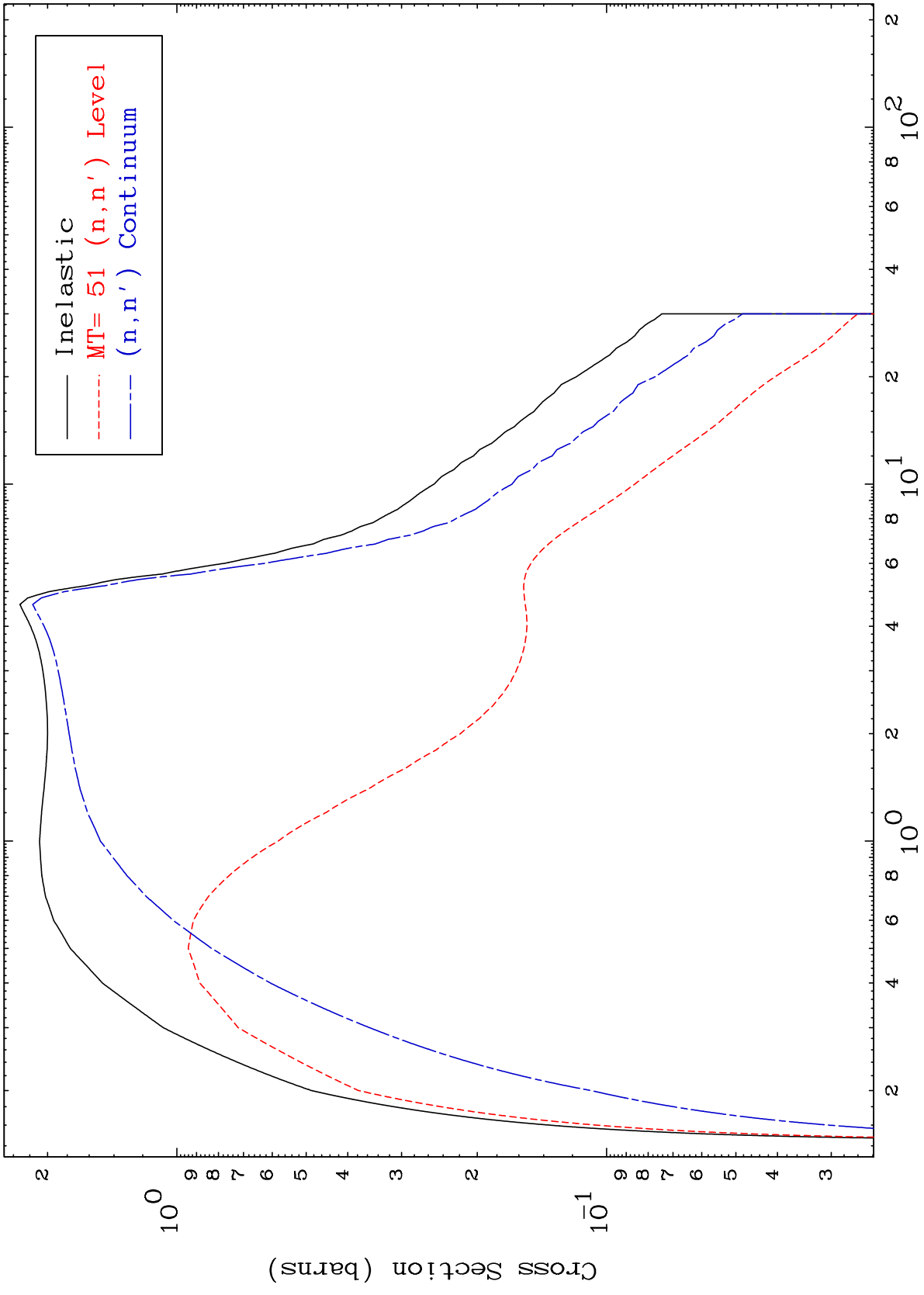
70-Yb-181

MAT 7064

(n,n') Level

70-Yb-181

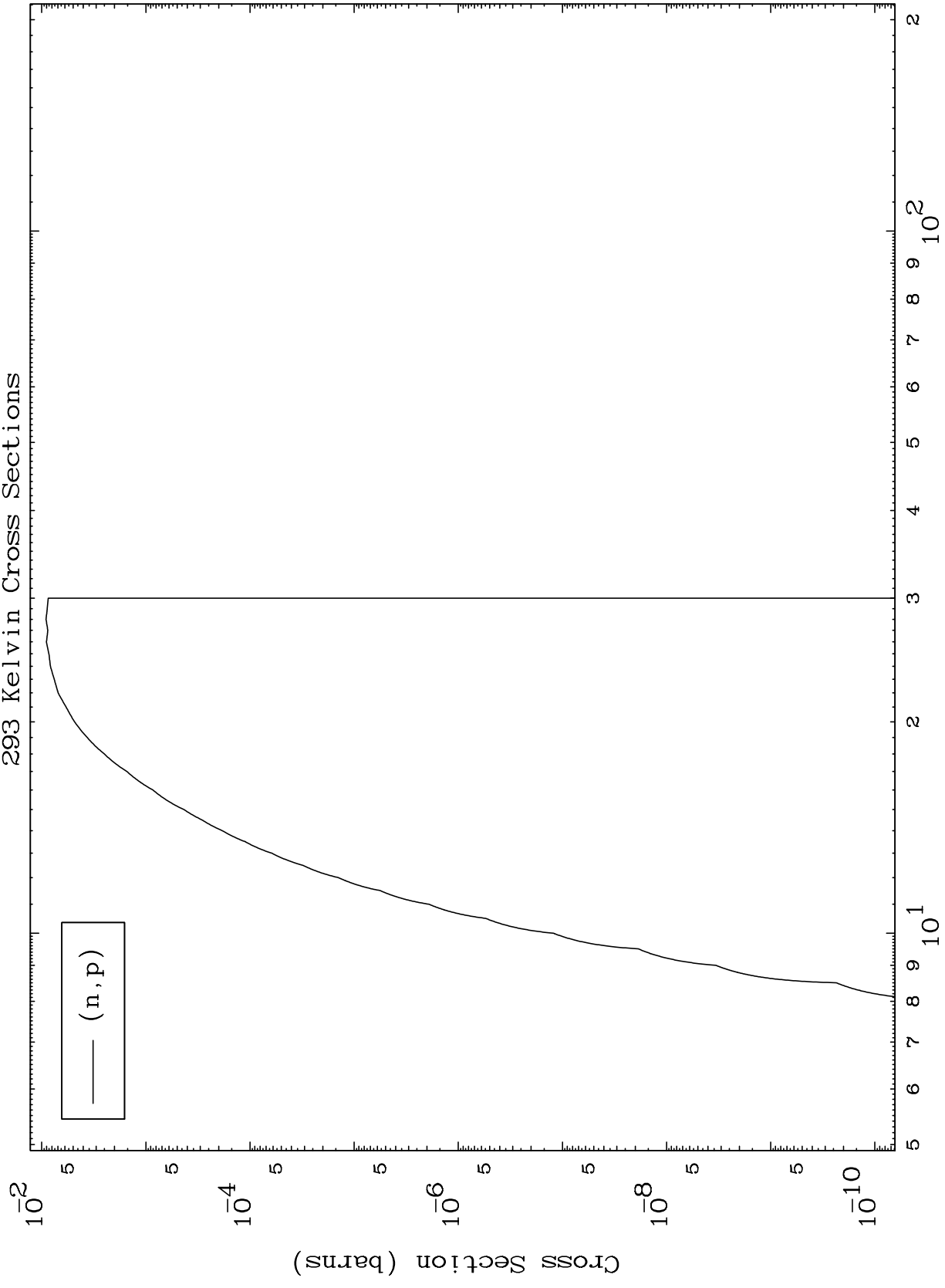
293 Kelvin Cross Sections



MAT 7064

(n,p) Levels
293 Kelvin Cross Sections

70-Yb-181



8

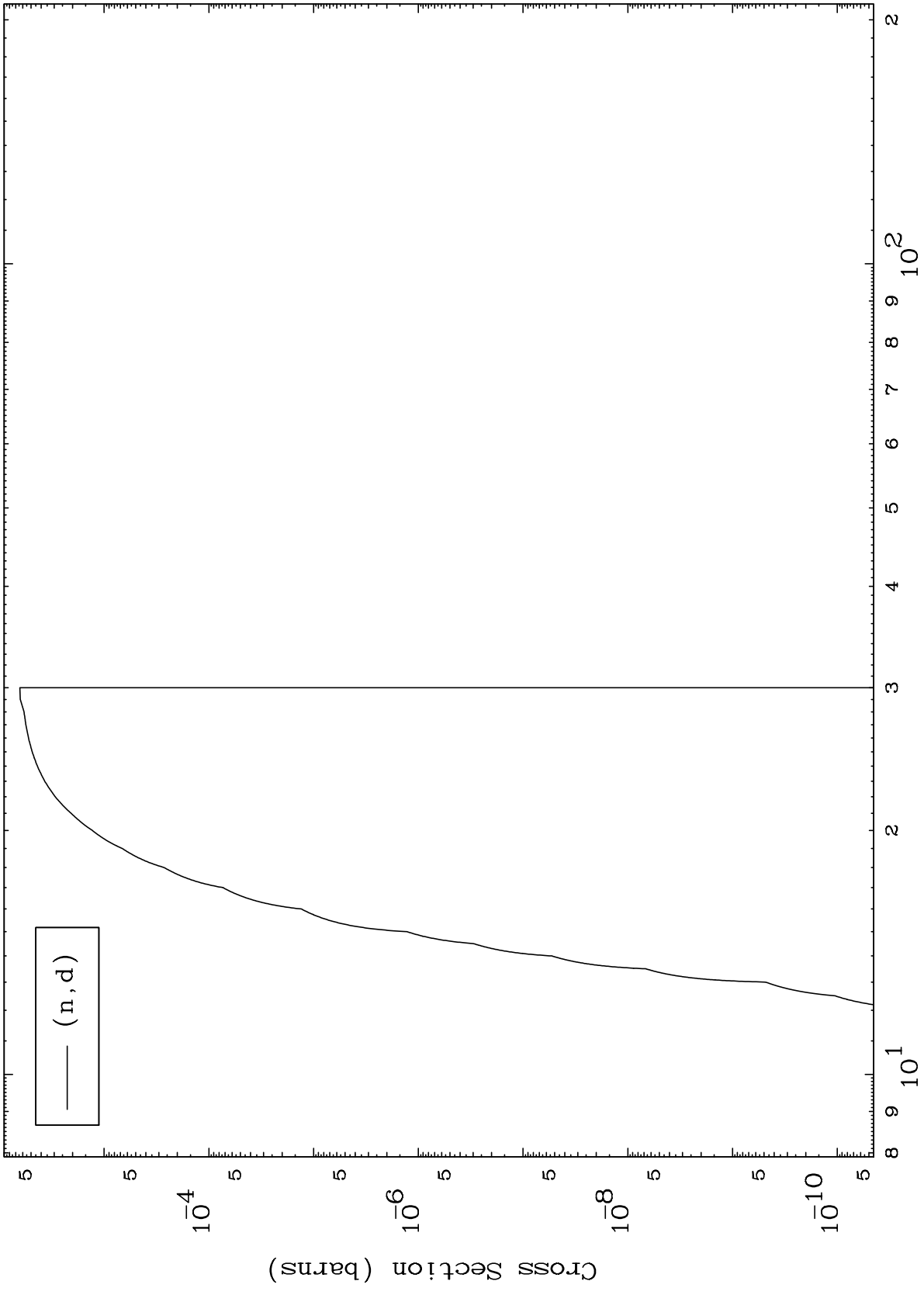
Incident Energy (MeV)

70-Yb-181

MAT 7064

(n,d) Levels
293 Kelvin Cross Sections

70-Yb-181



9

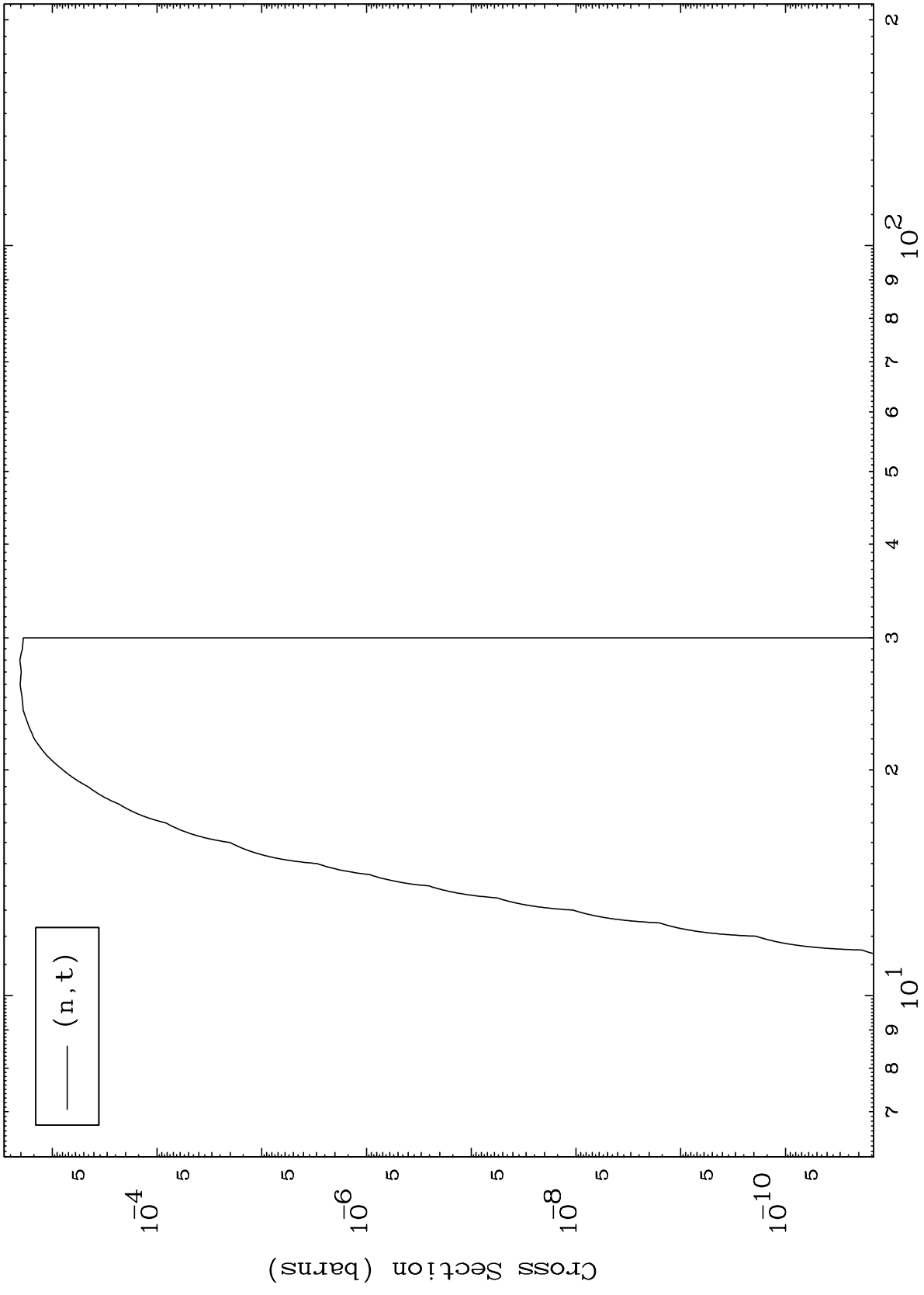
Incident Energy (MeV)

70-Yb-181

MAT 7064

(n,t) Levels
293 Kelvin Cross Sections

70-Yb-181



10

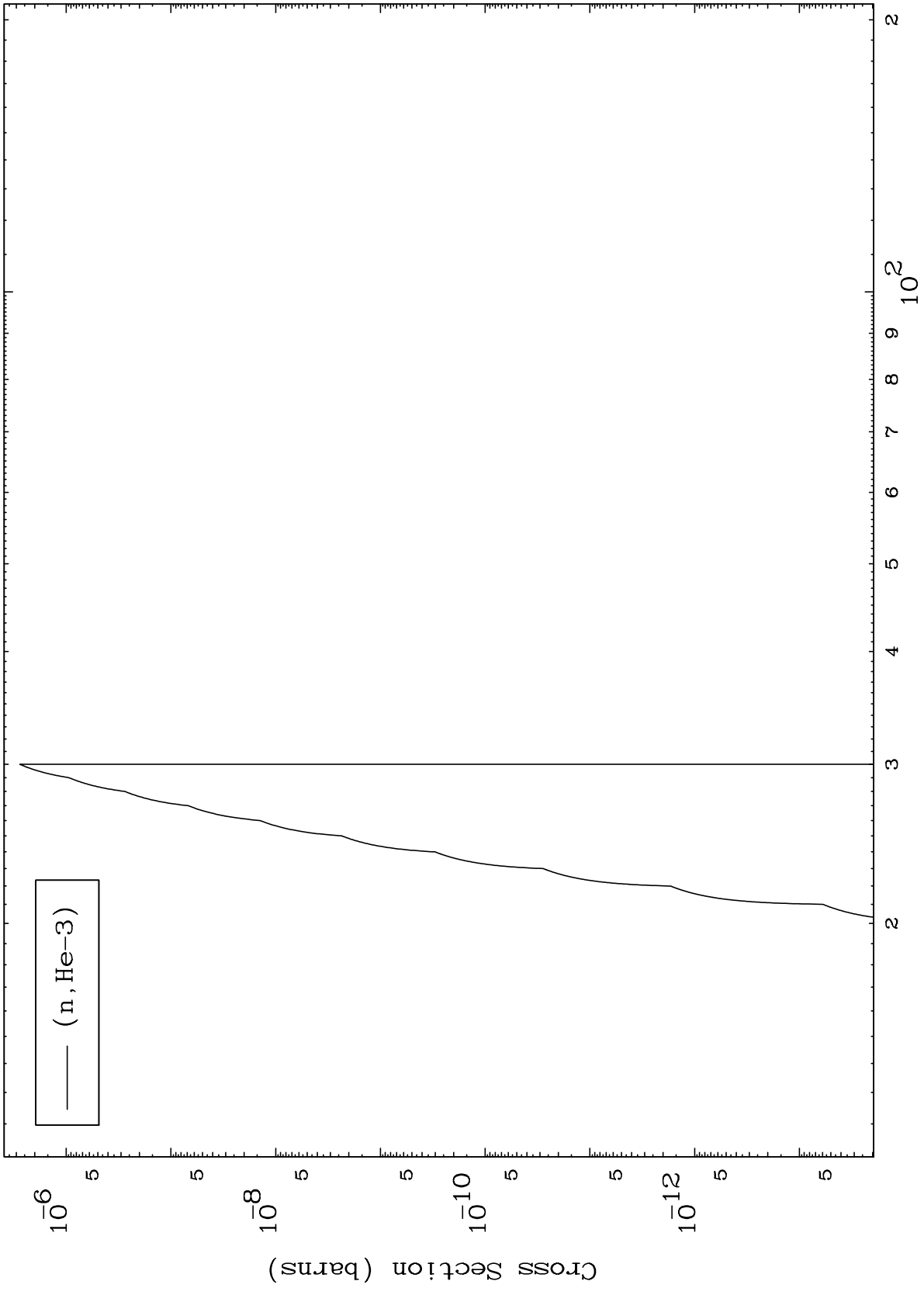
Incident Energy (MeV)

70-Yb-181

MAT 7064

(n,He3) Levels
293 Kelvin Cross Sections

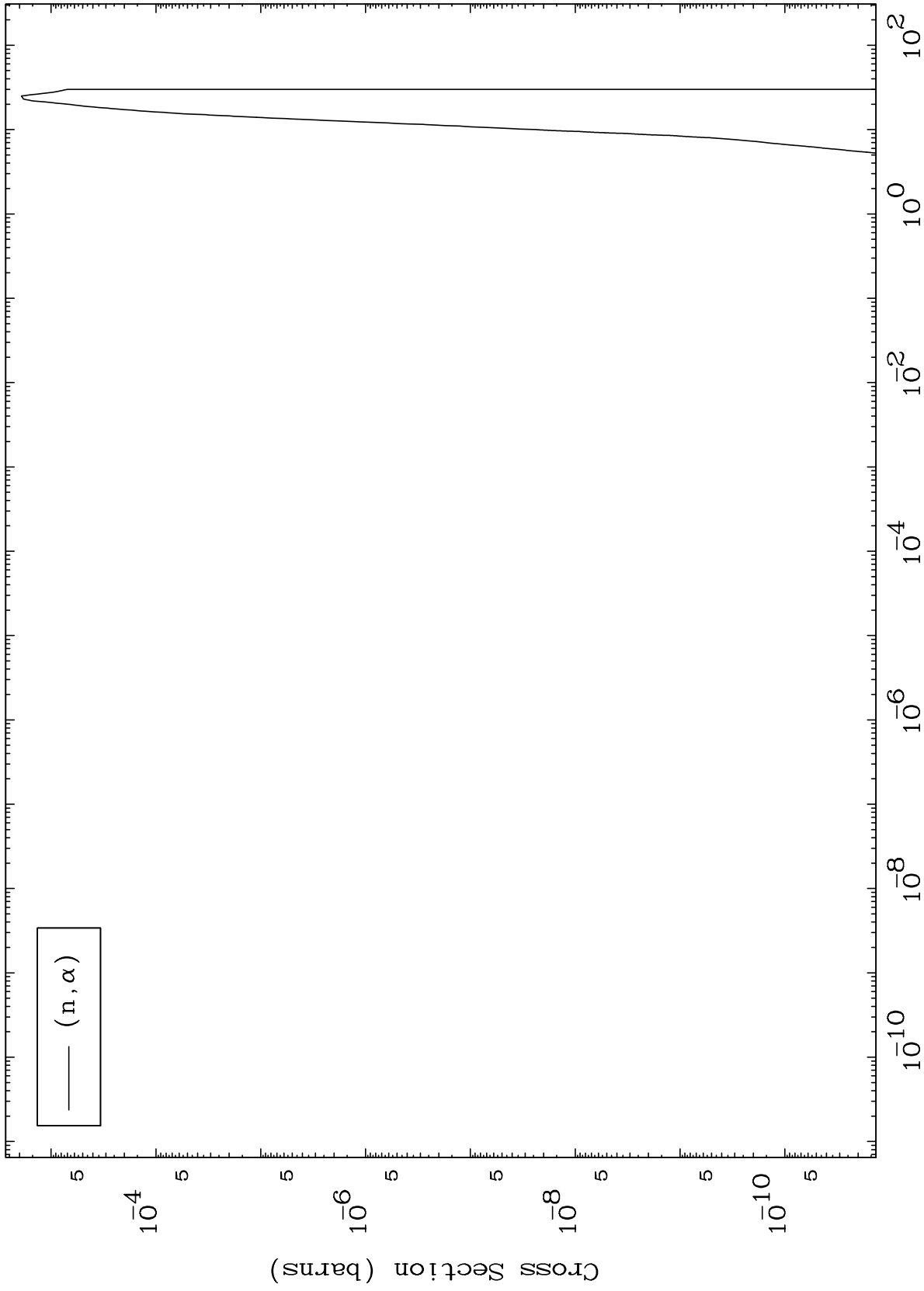
70-Yb-181



MAT 7064

(n,α) Levels
293 Kelvin Cross Sections

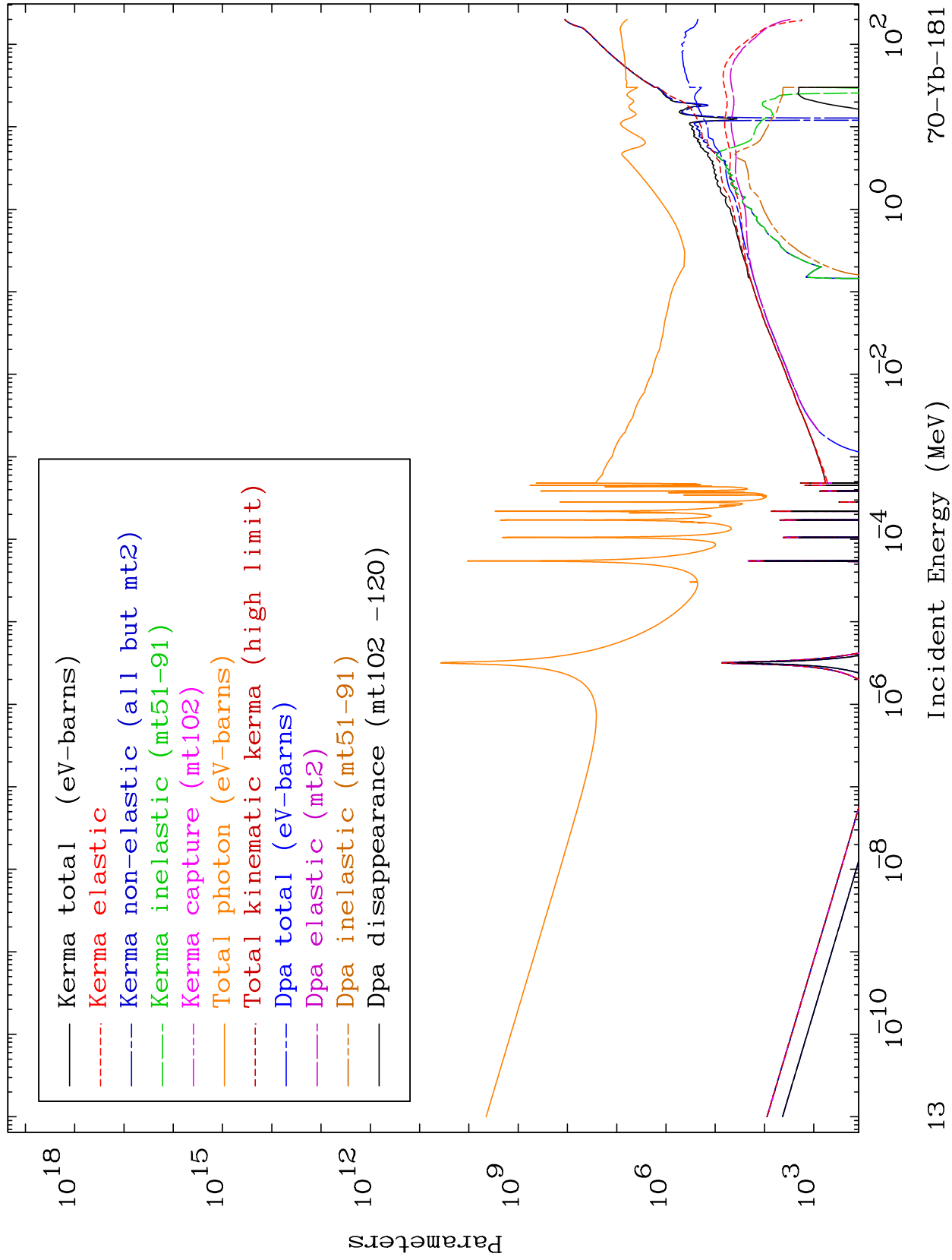
70-Yb-181

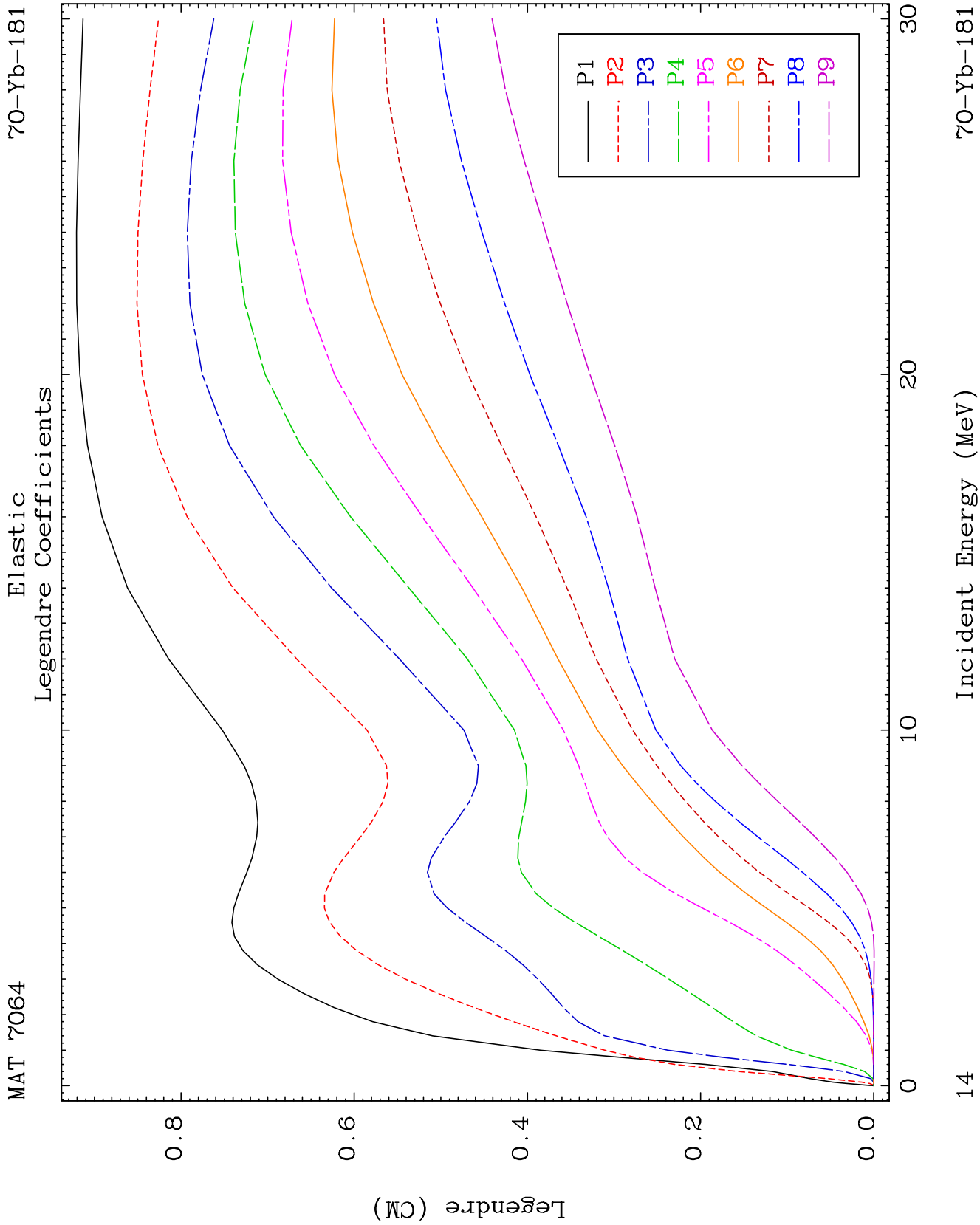


12

Incident Energy (MeV)

70-Yb-181

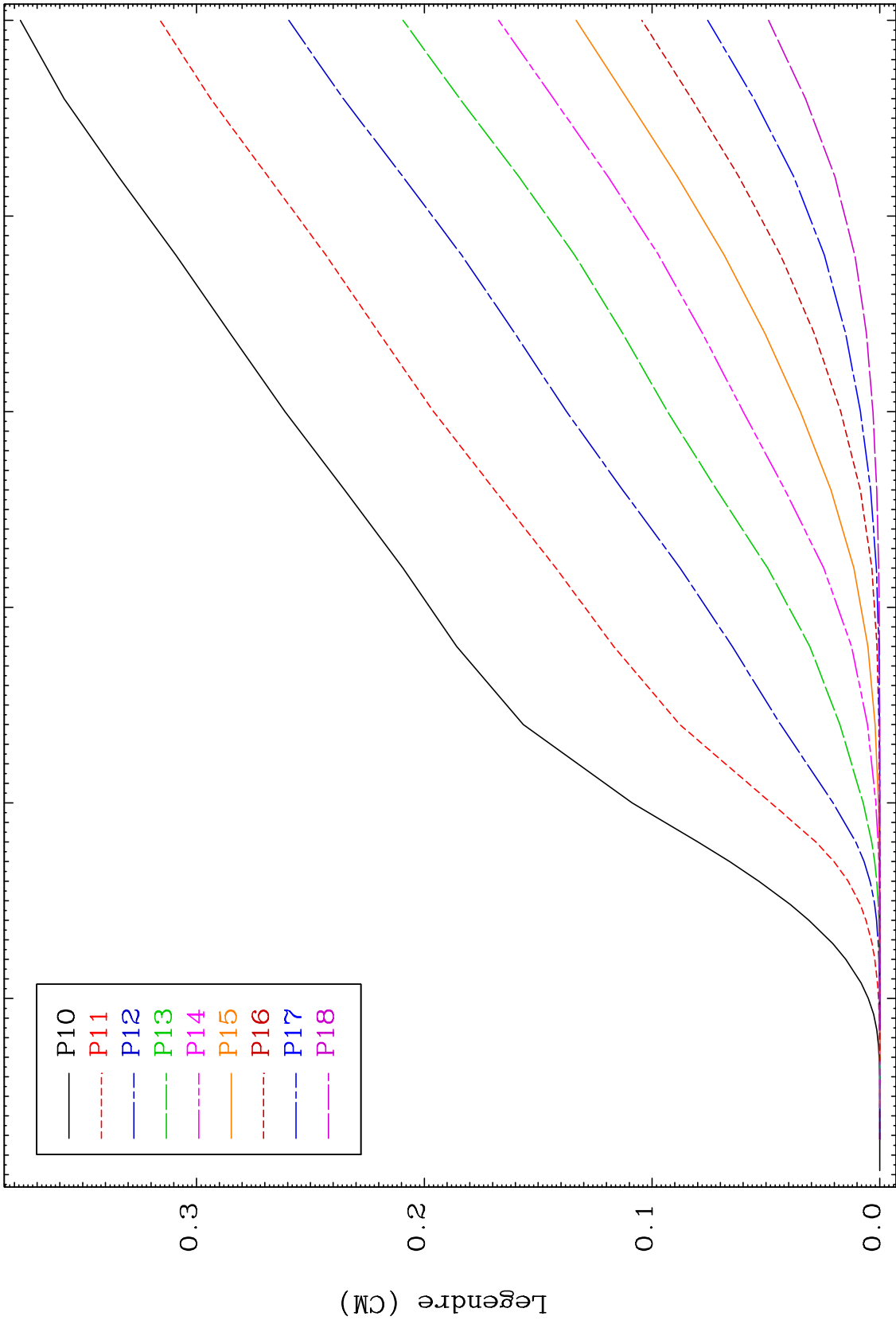




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Elastic Legendre Coefficients

70-Yb-181



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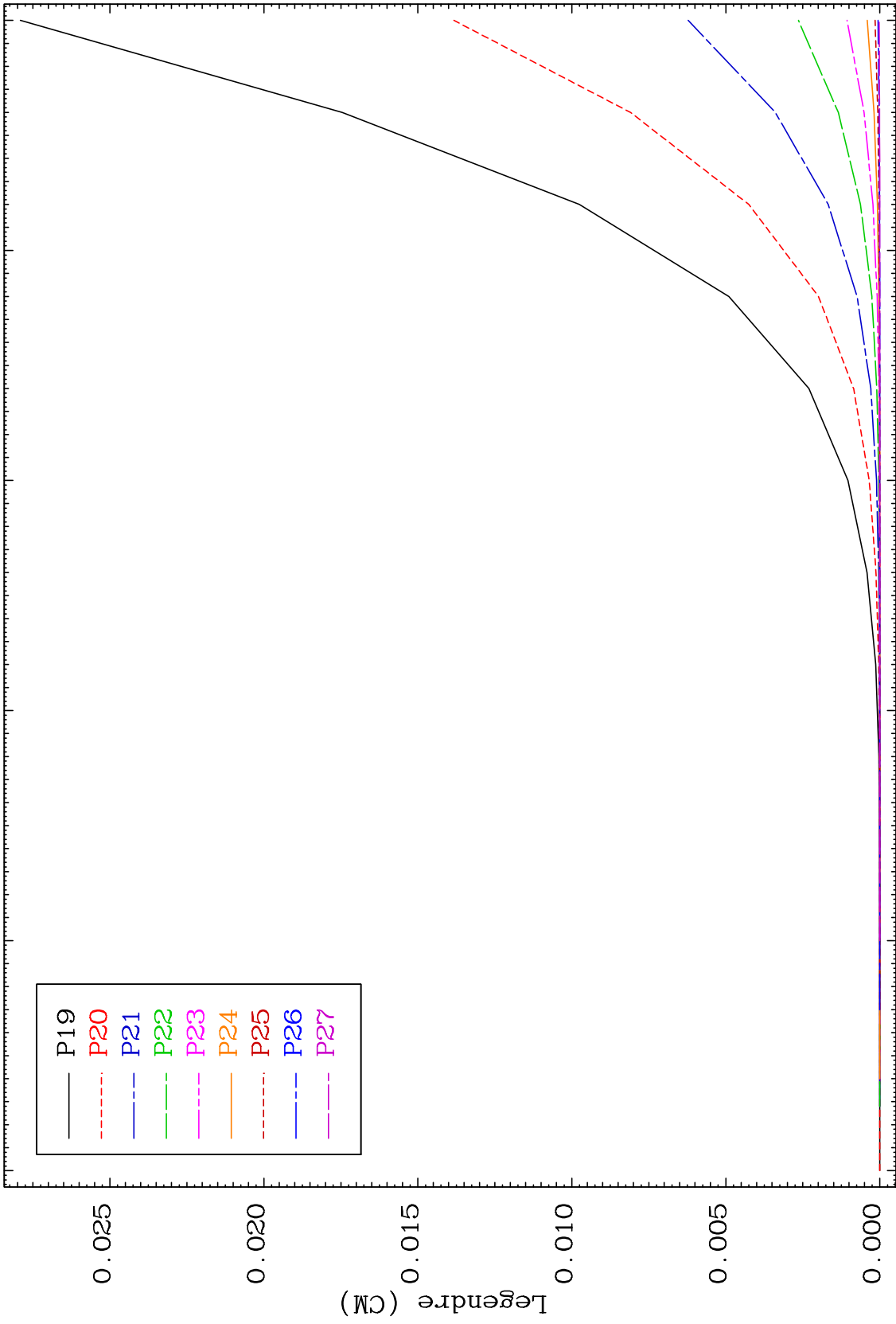
Incident Energy (MeV)

70-Yb-181

MAT 7064

Elastic Legendre Coefficients

70-Yb-181



70-Yb-181

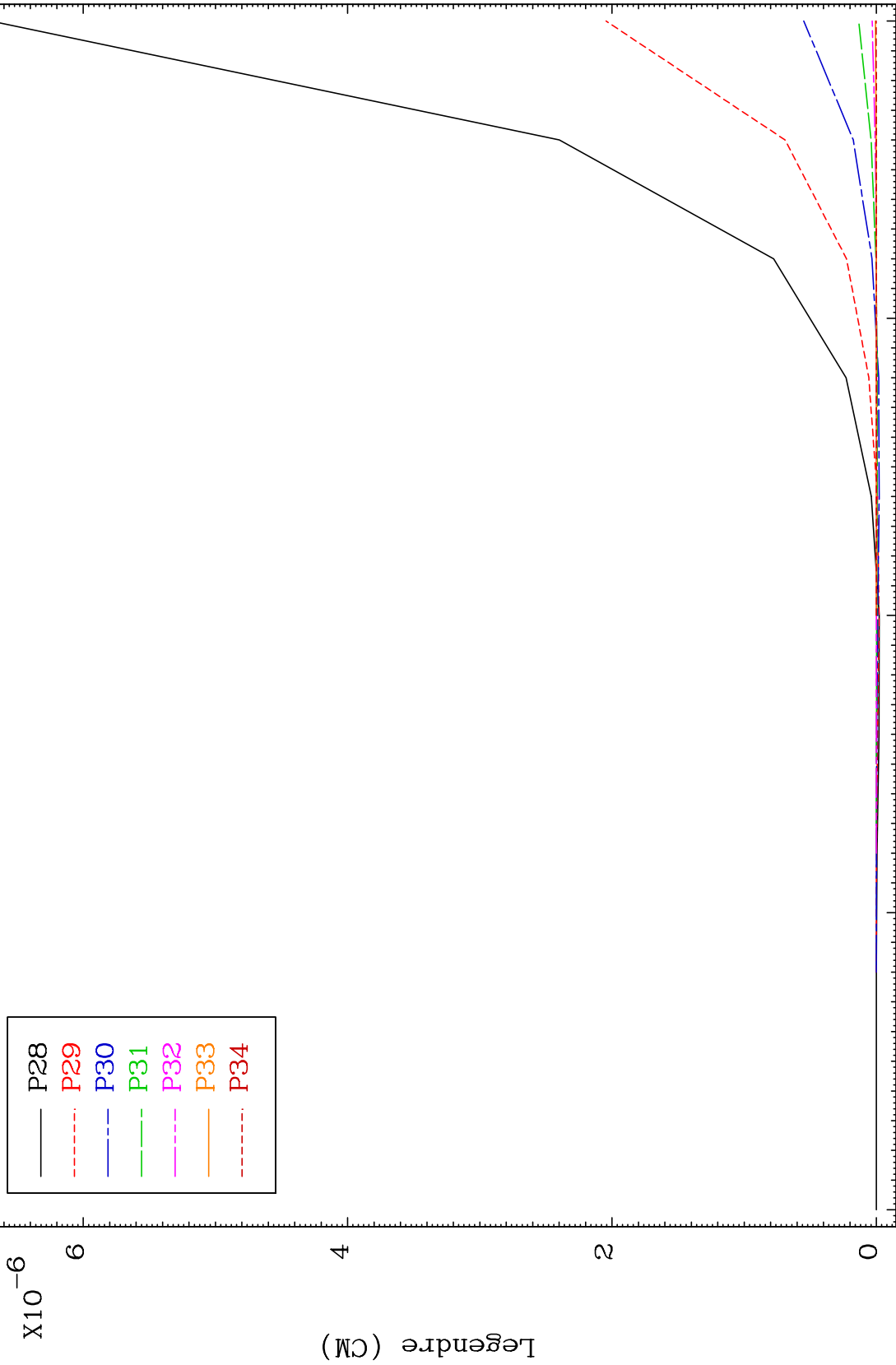
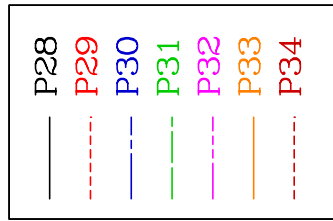
Incident Energy (MeV)

16

MAT 7064

Elastic Legendre Coefficients

70-Yb-181



17

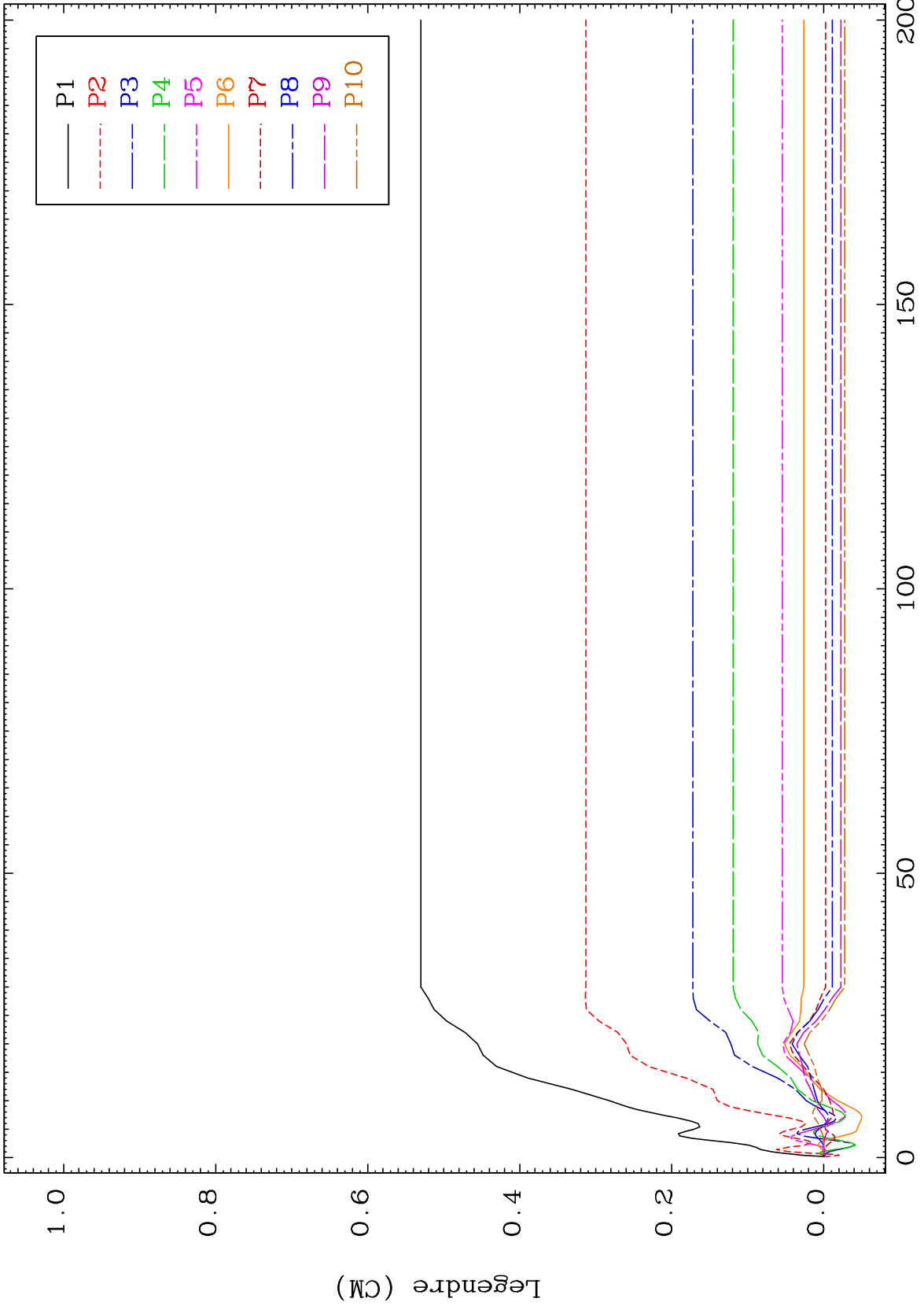
Incident Energy (MeV)

70-Yb-181

MAT 7064

MT= 51 (n,n') Level
Legendre Coefficients

70-Yb-181



18

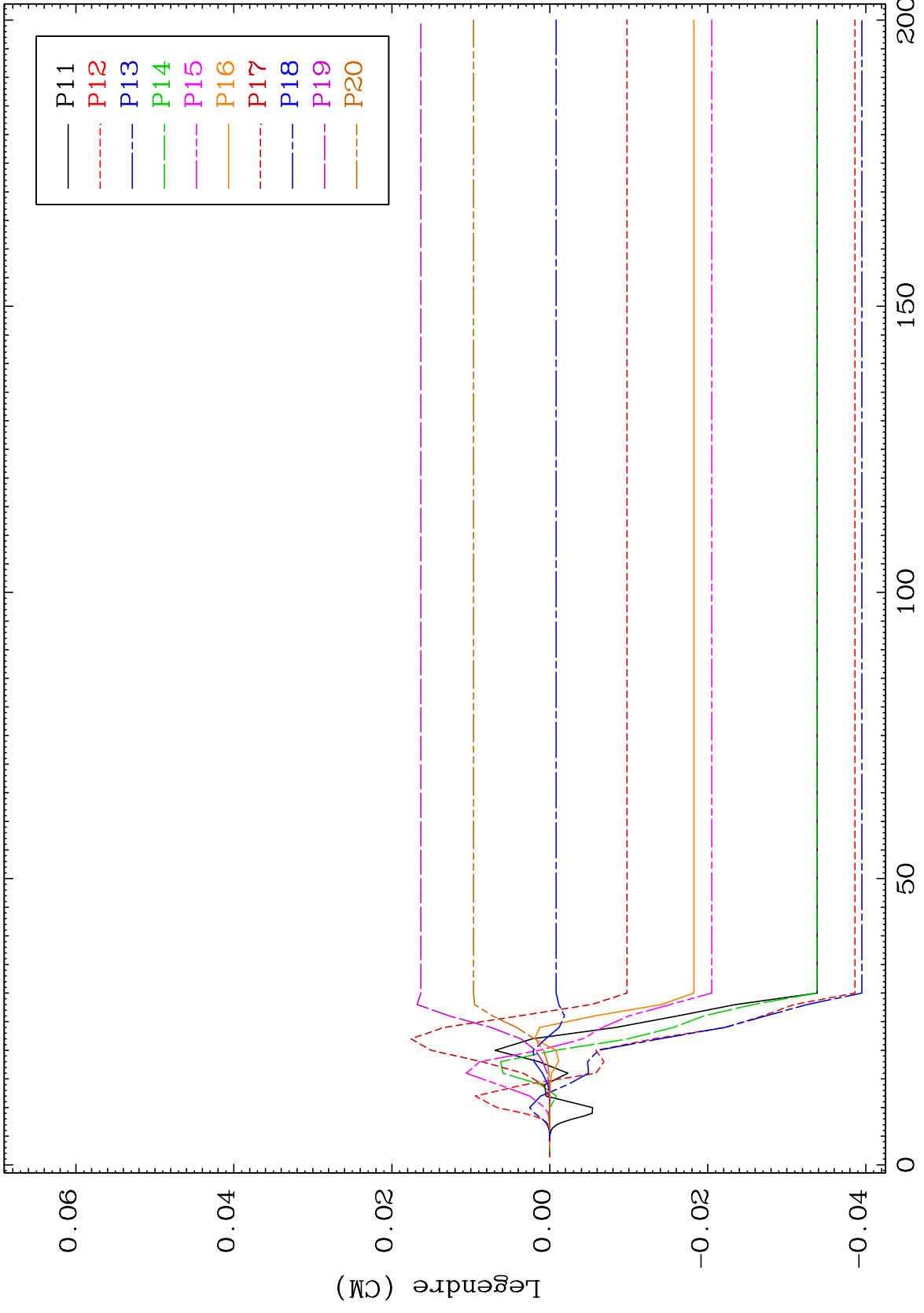
Incident Energy (MeV)

70-Yb-181

MAT 7064

MT= 51 (n,n') Level
Legendre Coefficients

70-Yb-181



19

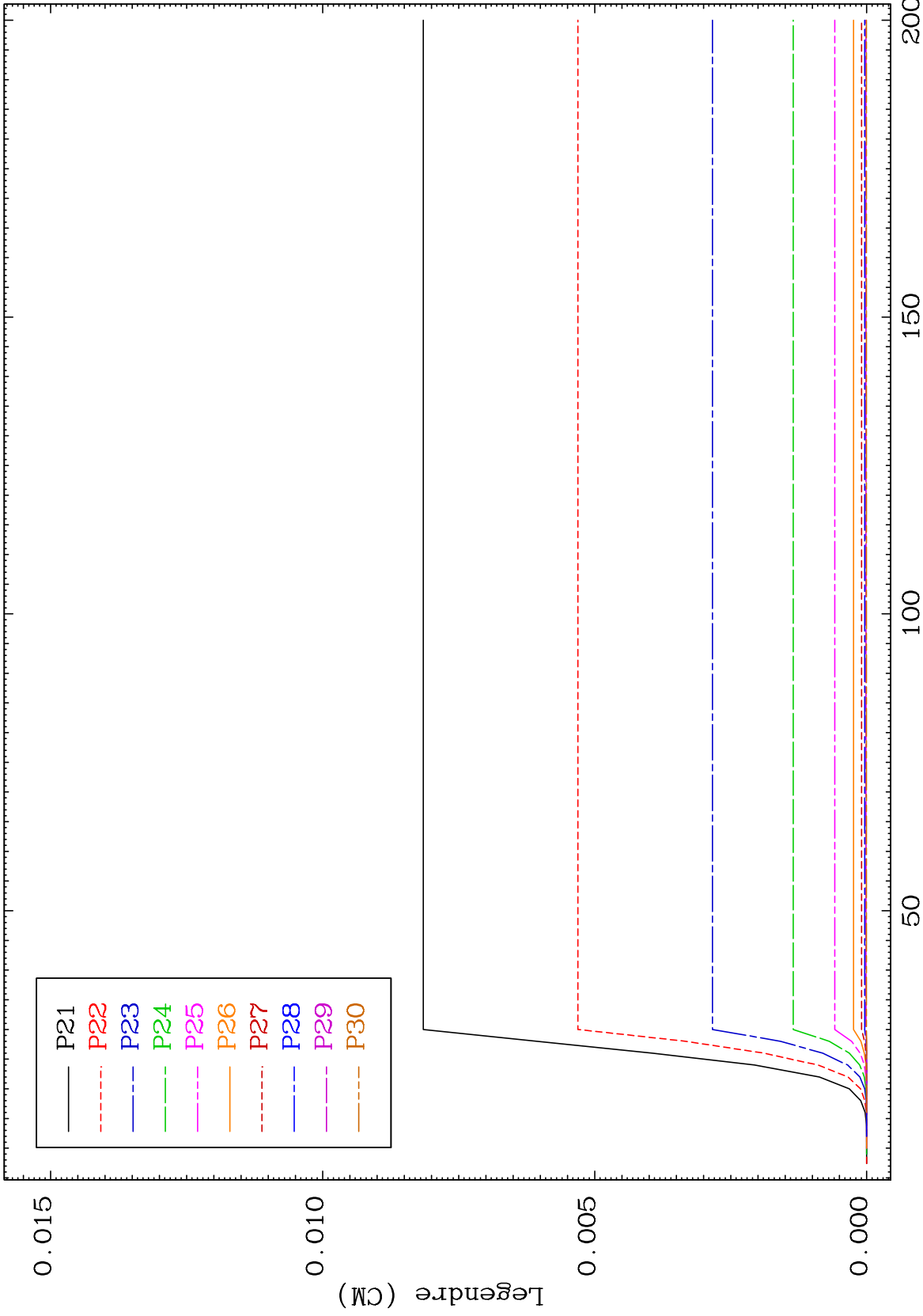
Incident Energy (MeV)

70-Yb-181

MAT 7064

MT= 51 (n,n') Level
Legendre Coefficients

70-Yb-181



20

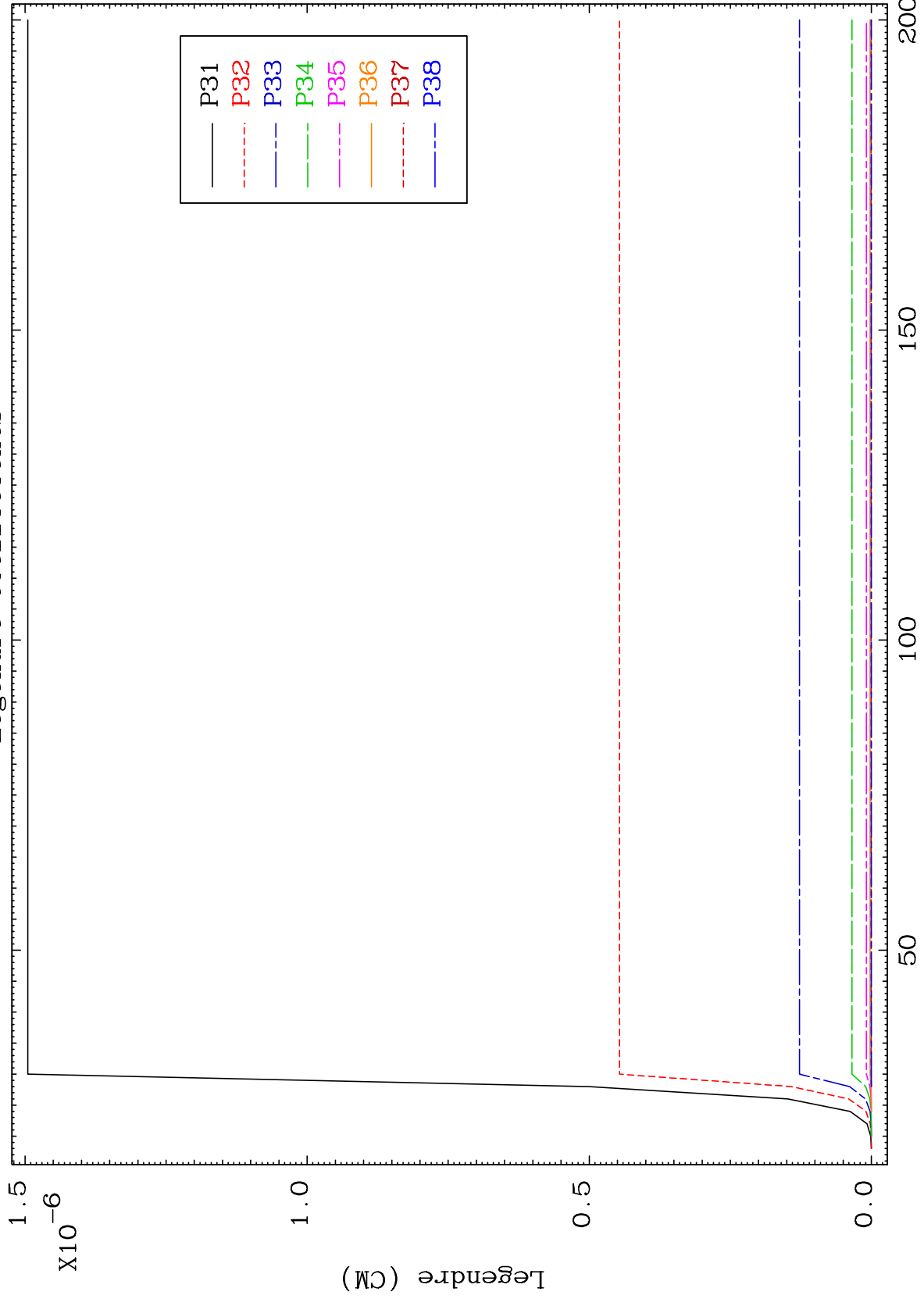
Incident Energy (MeV)

70-Yb-181

MAT 7064

MT= 51 (n,n') Level
Legendre Coefficients

70-Yb-181



21

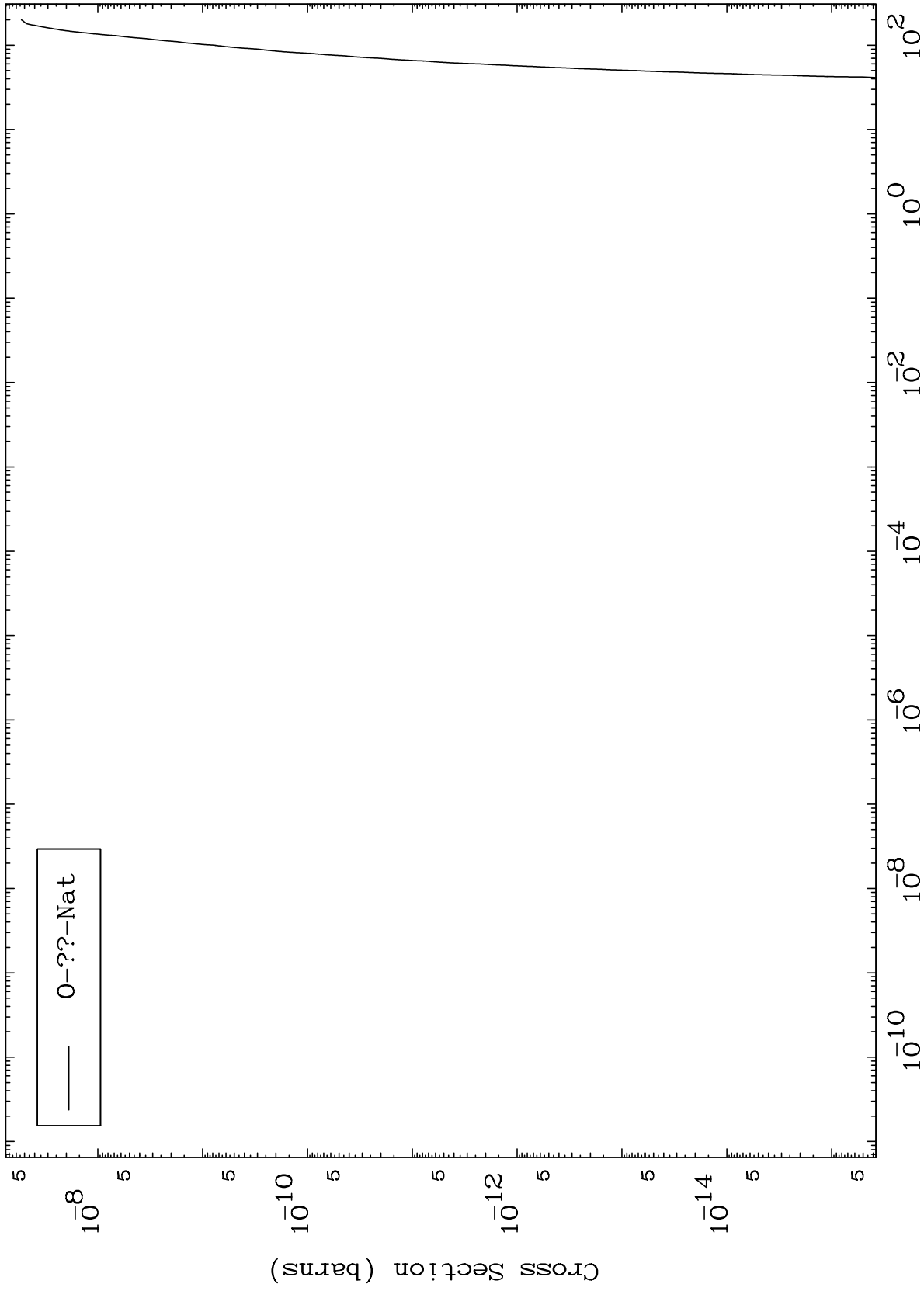
Incident Energy (MeV)

70-Yb-181

MAT 7064

Fission
Radionuclide Production Cross Section

70-Yb-181



22

Incident Energy (MeV)

70-Yb-181