

Program EVALPLOT  
(Version 2018-1)

by

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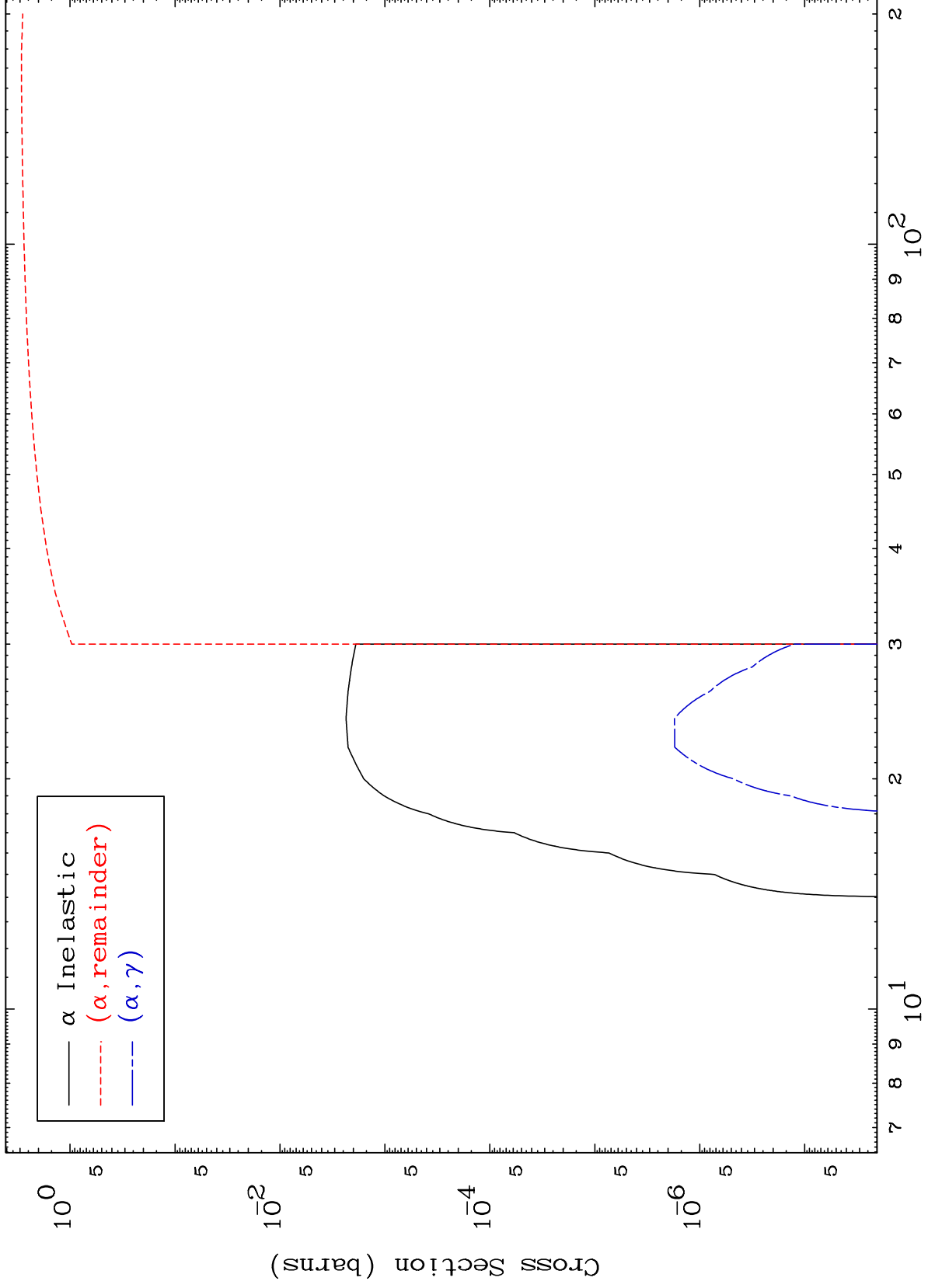
Press Mouse Button to Start

MAT 8917

0 Kelvin Cross Sections

$\alpha$  Major

89-Ac-222



1

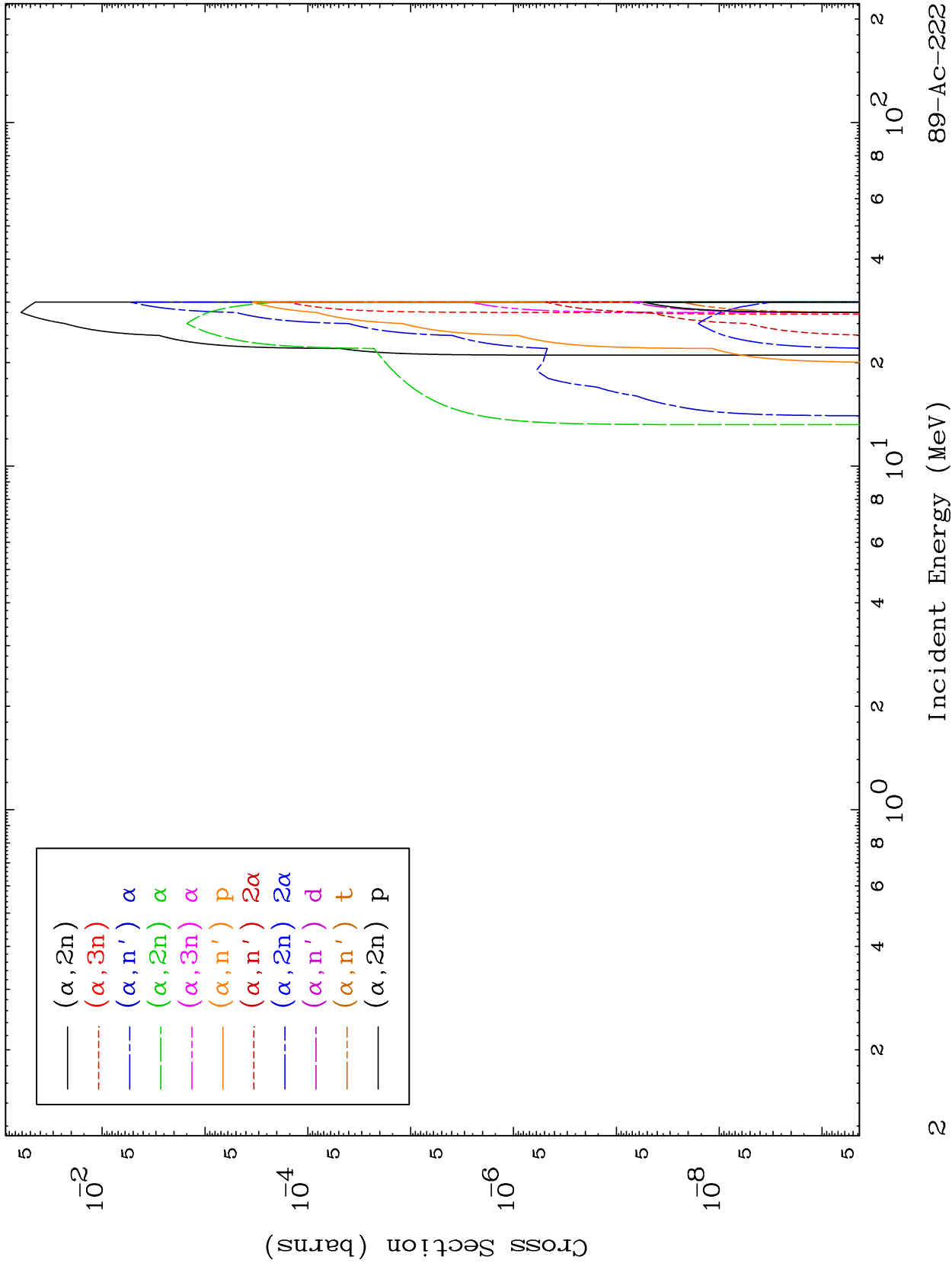
Incident Energy (MeV)

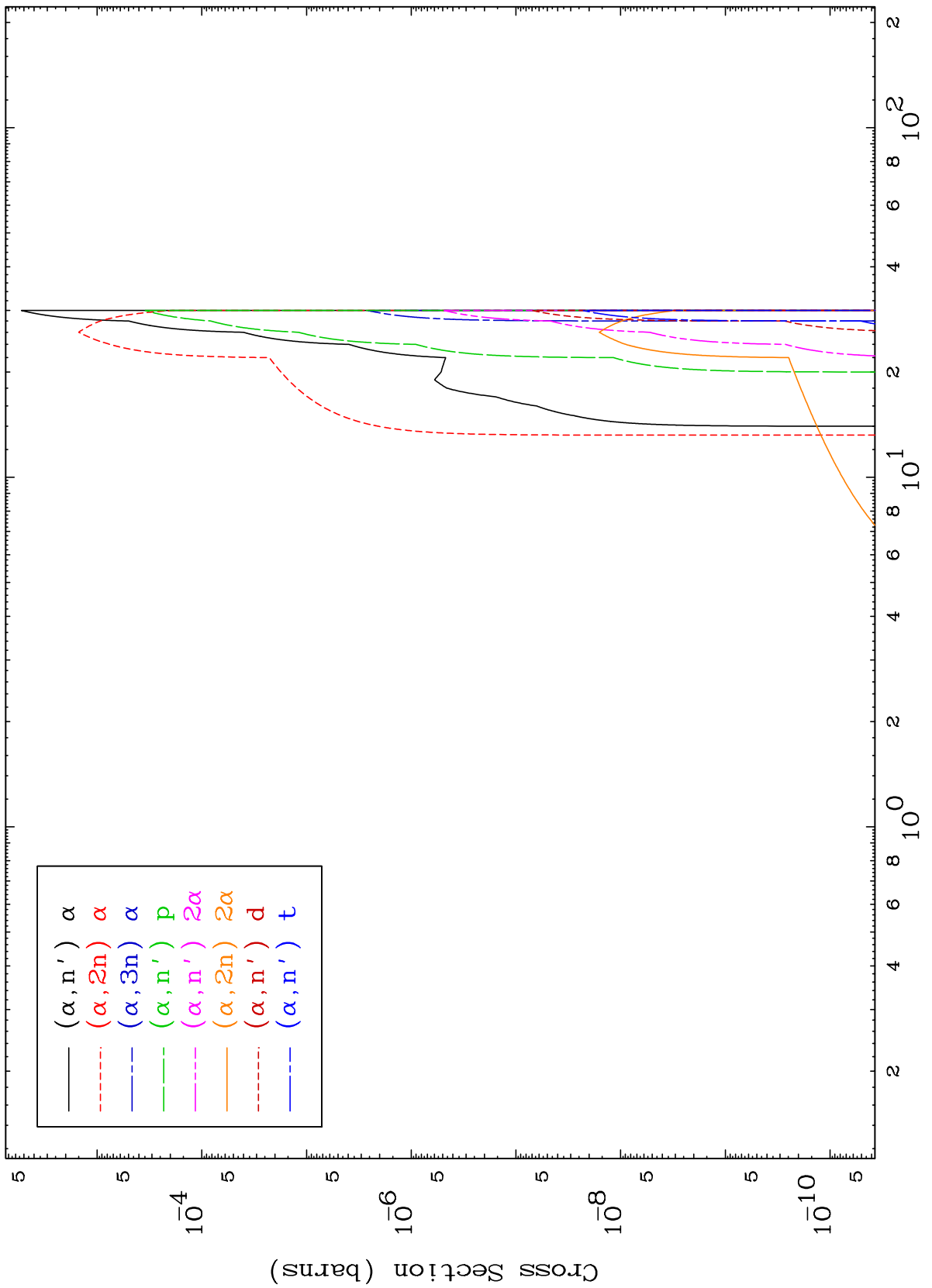
89-Ac-222

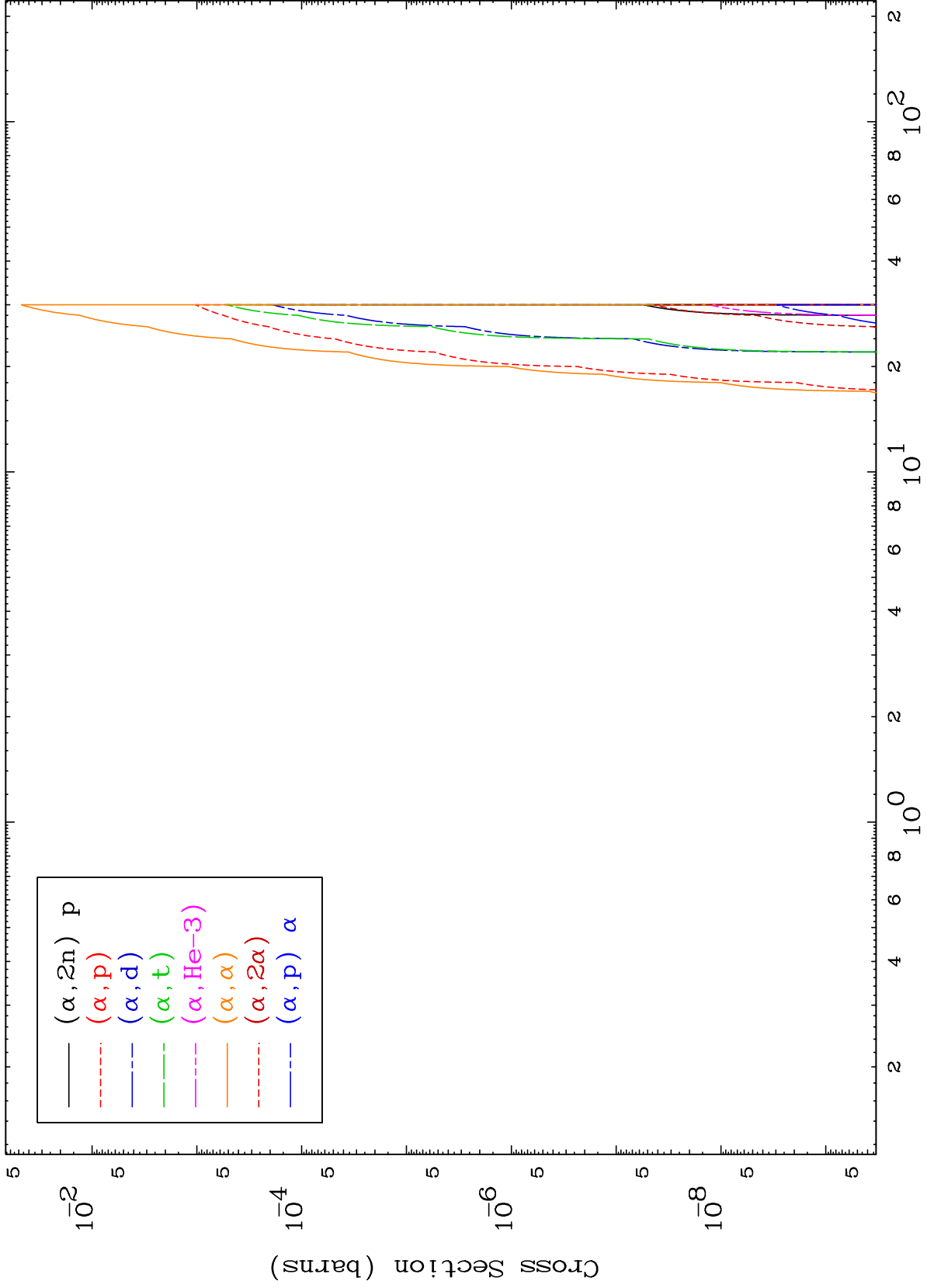
MAT 8917

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

89-Ac-222





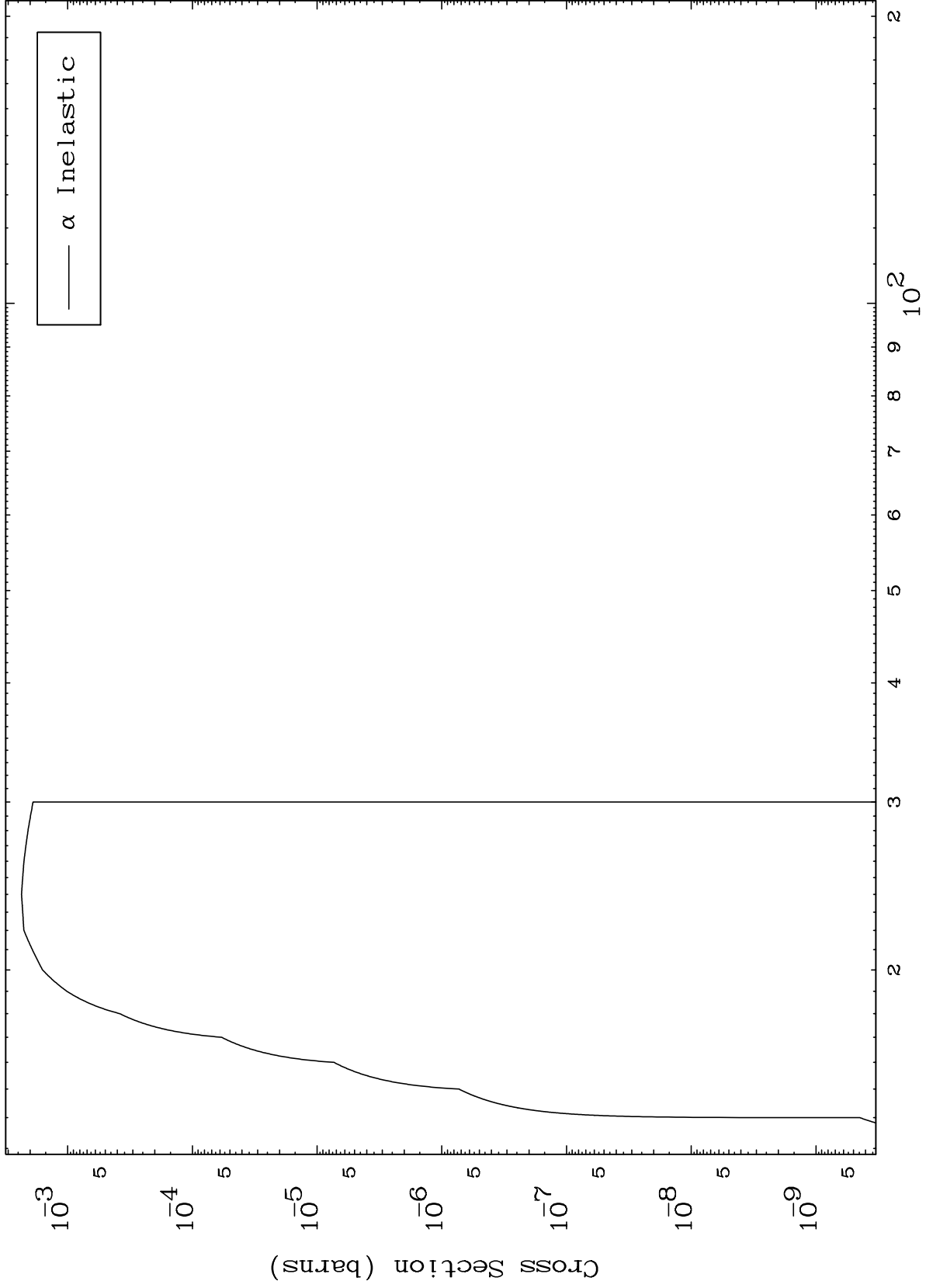


MAT 8917

( $\alpha, n'$ ) Level

89-Ac-222

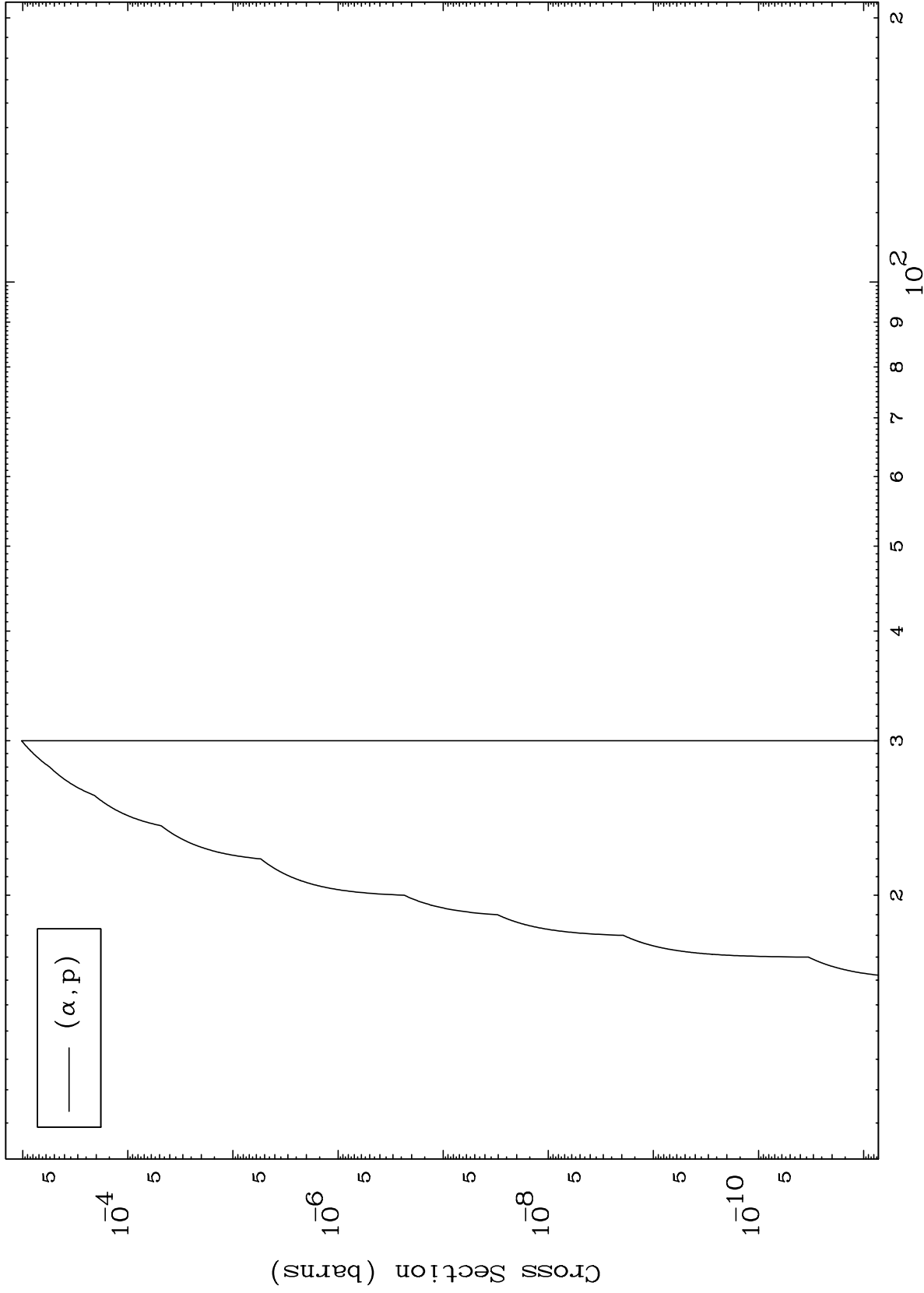
0 Kelvin Cross Sections

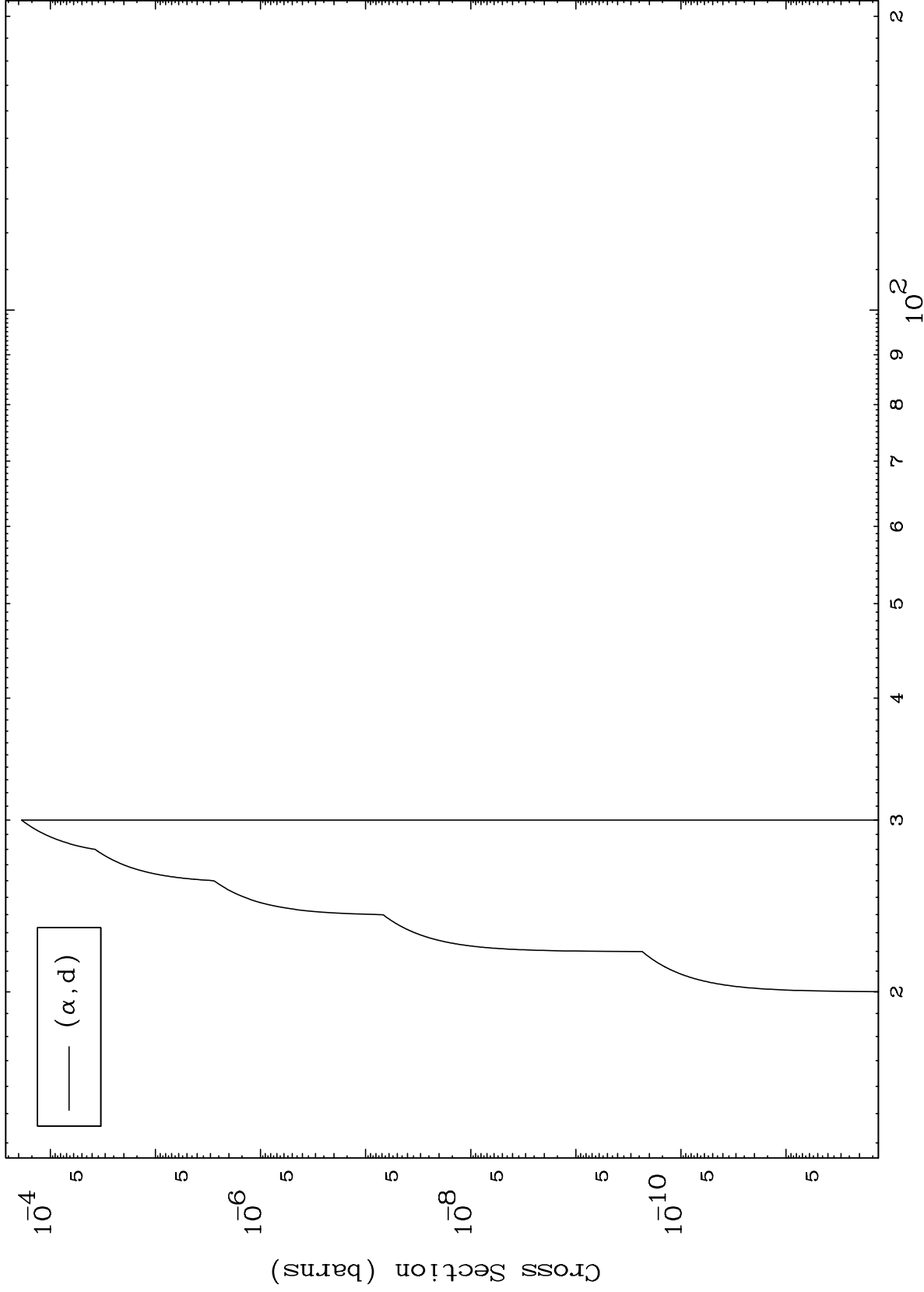


Incident Energy (MeV)

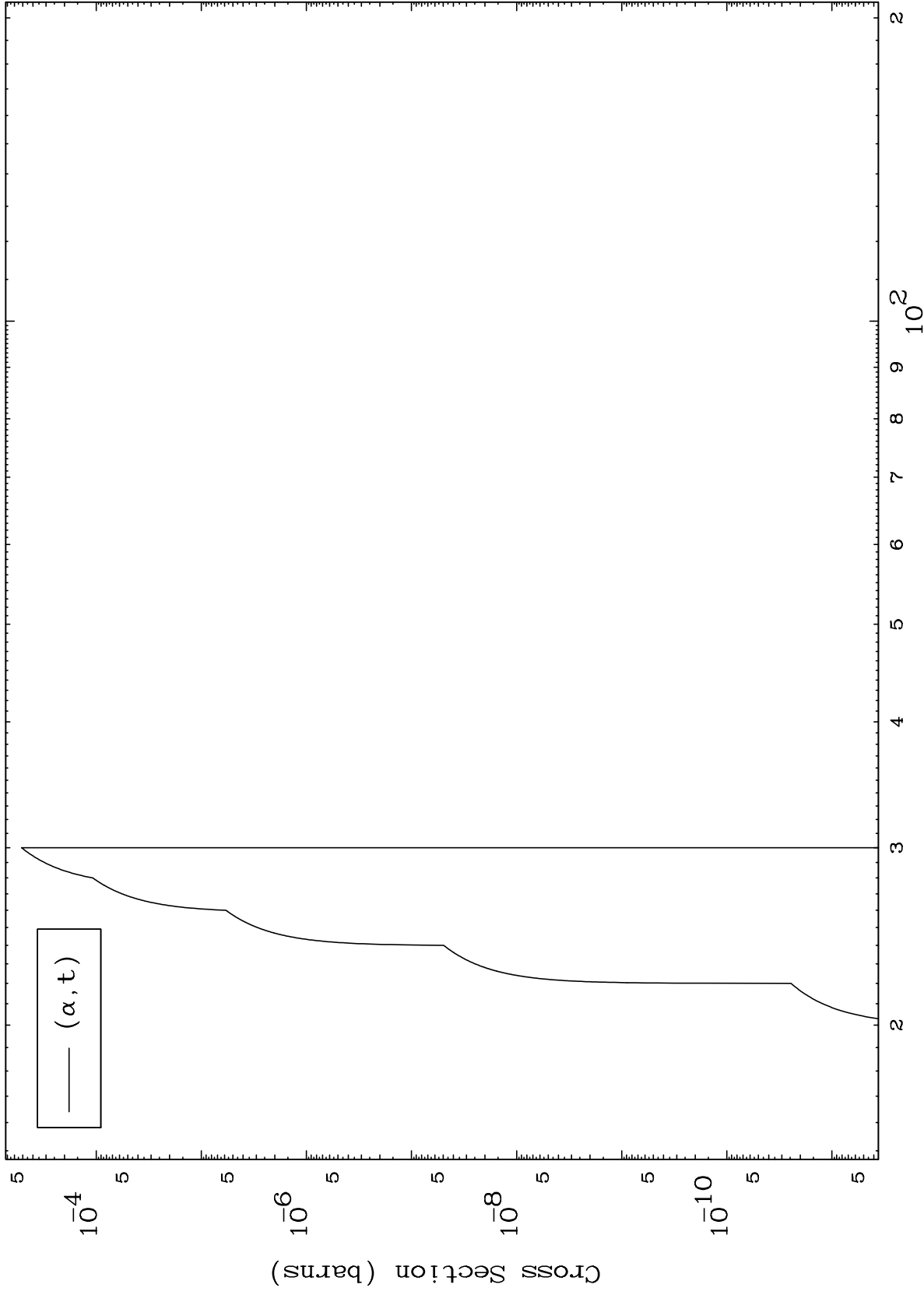
89-Ac-222

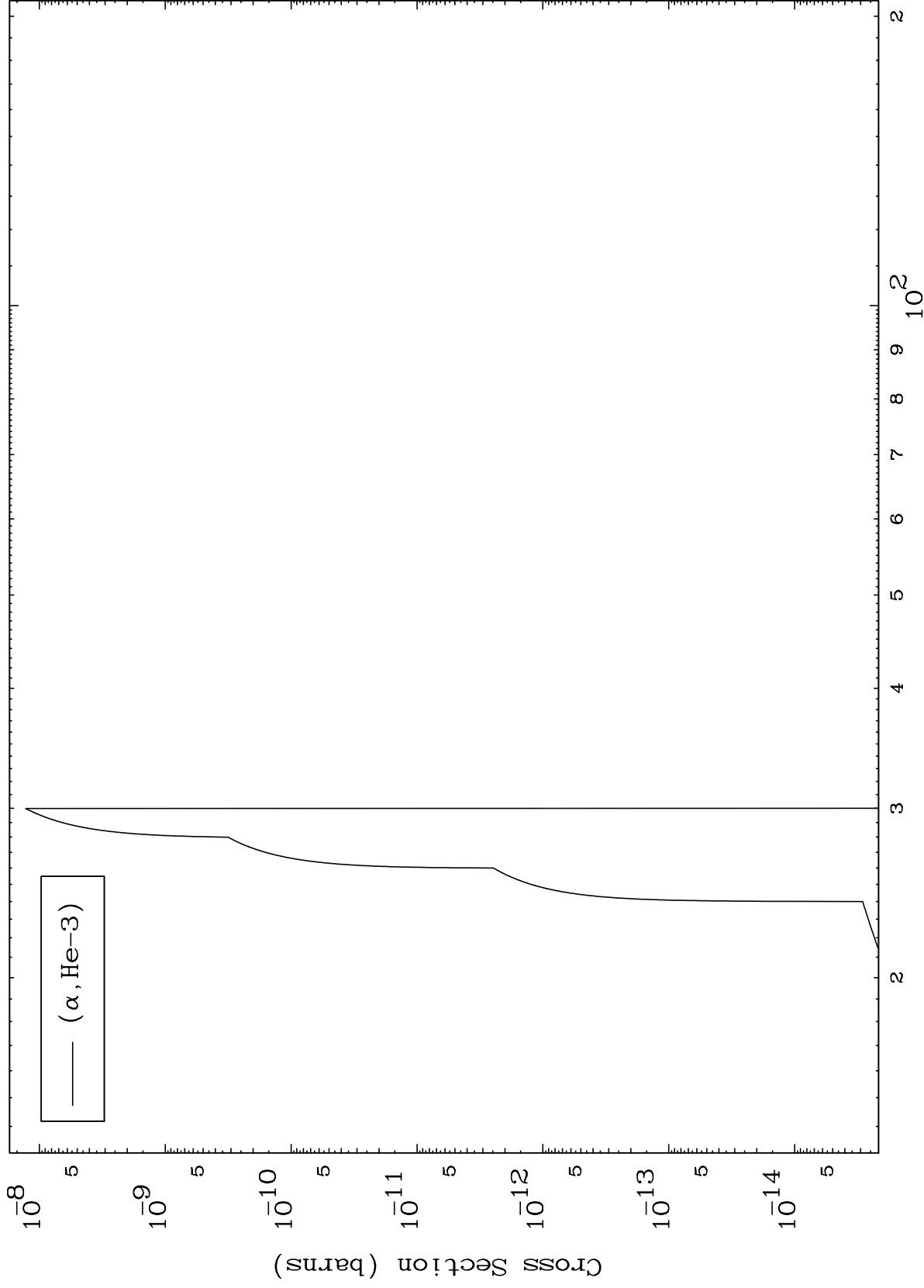
5









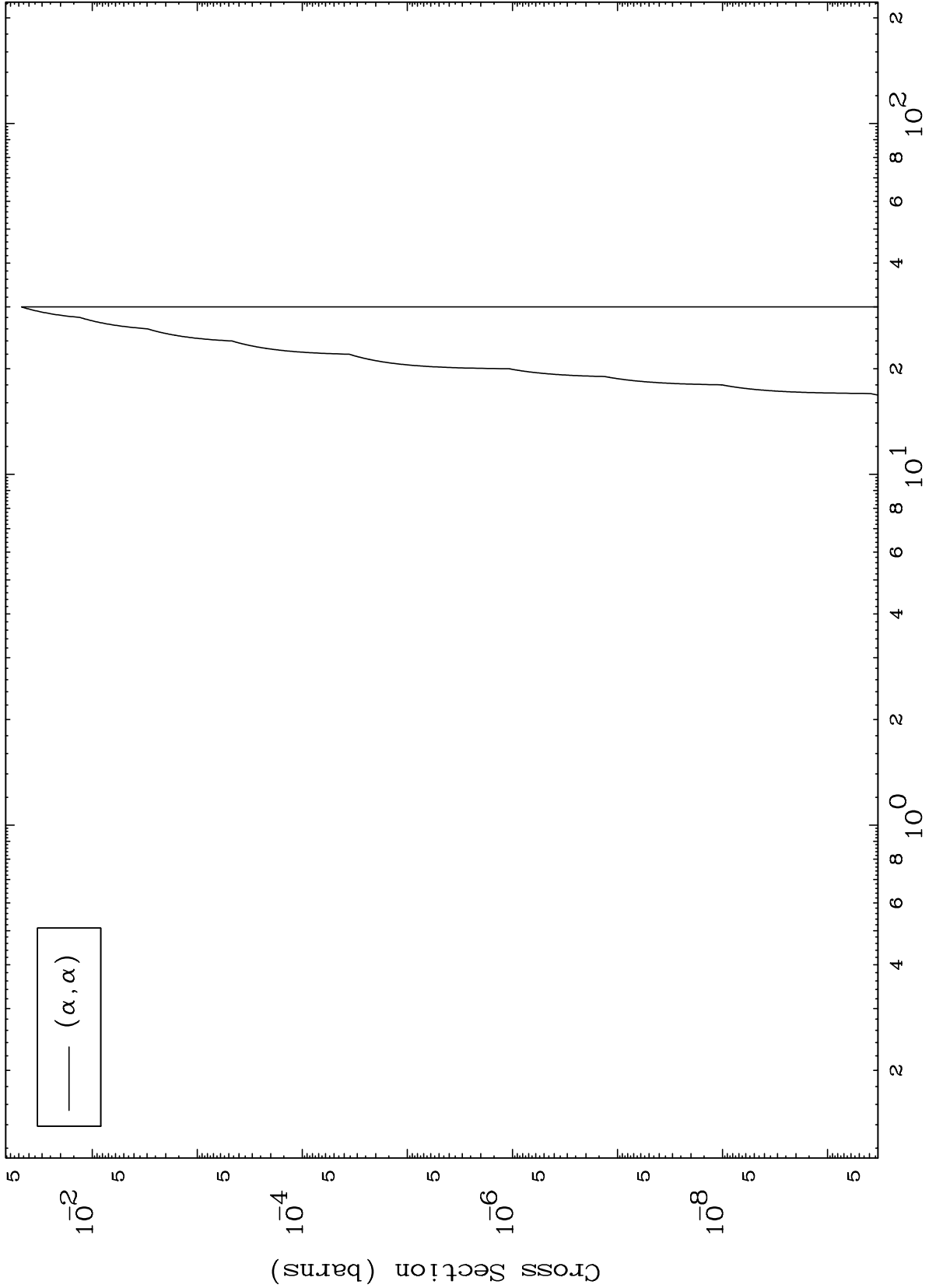


MAT 8917

( $\alpha, \alpha$ ) Levels

89-Ac-222

0 Kelvin Cross Sections



10

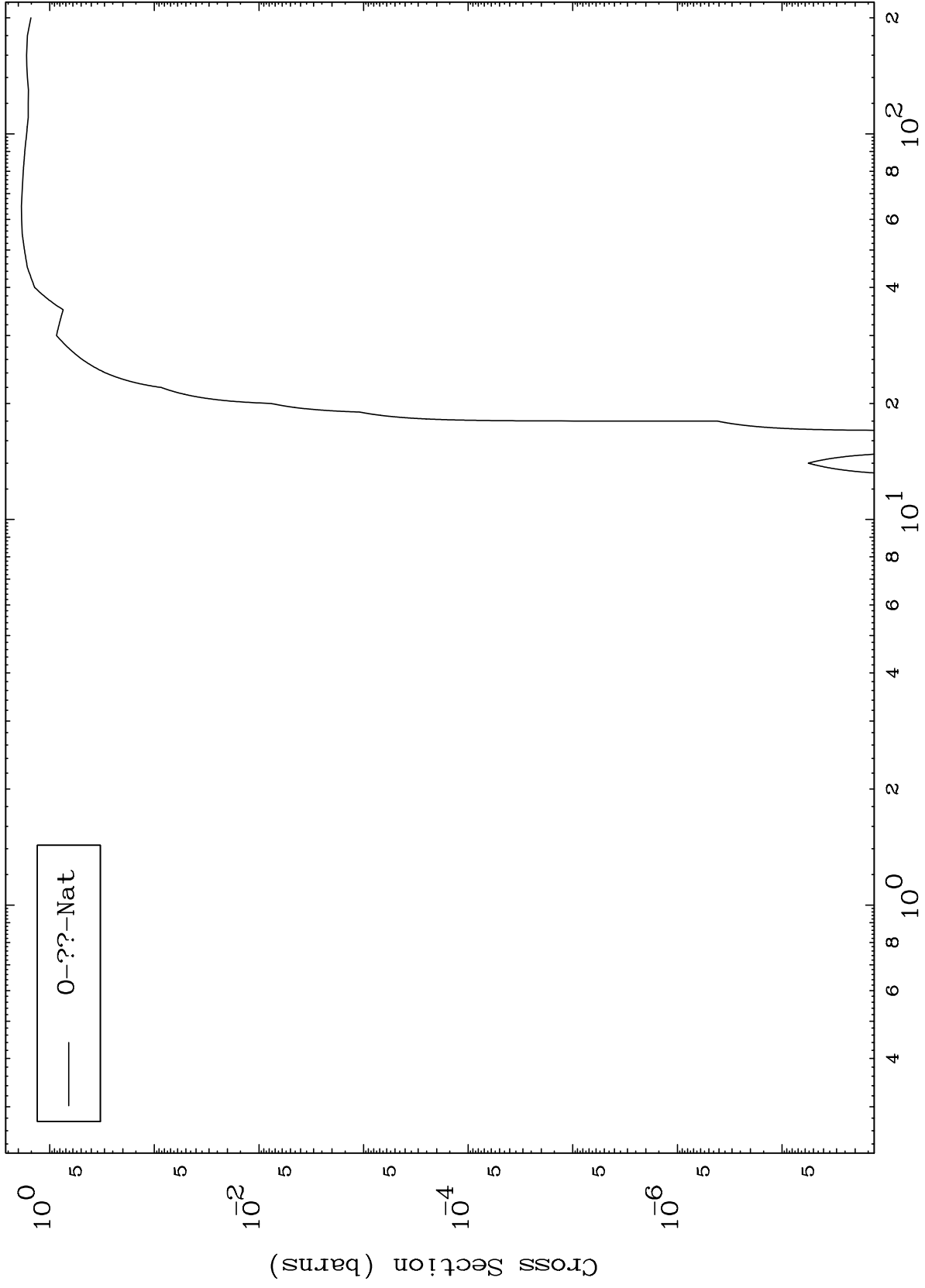
Incident Energy (MeV)

89-Ac-222

MAT 8917

89-Ac-222

$\alpha$  Fission  
Radionuclide Production Cross Section

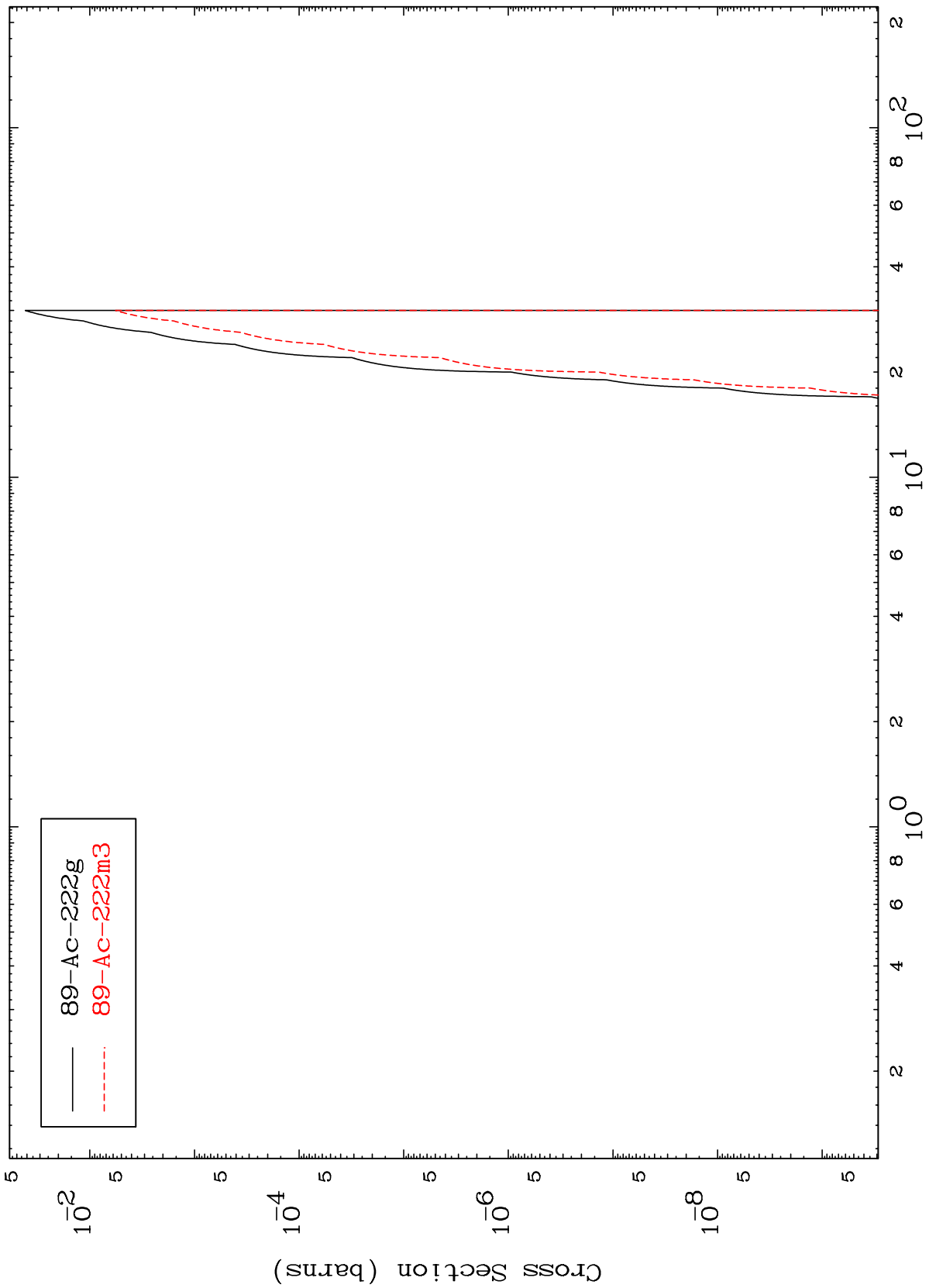


MAT 8917

( $\alpha, \alpha$ )

89-Ac-222

Radionuclide Production Cross Section



12

Incident Energy (MeV)

89-Ac-222