

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

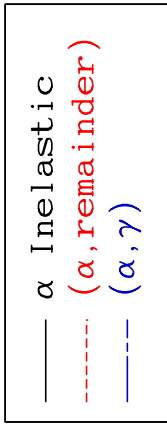
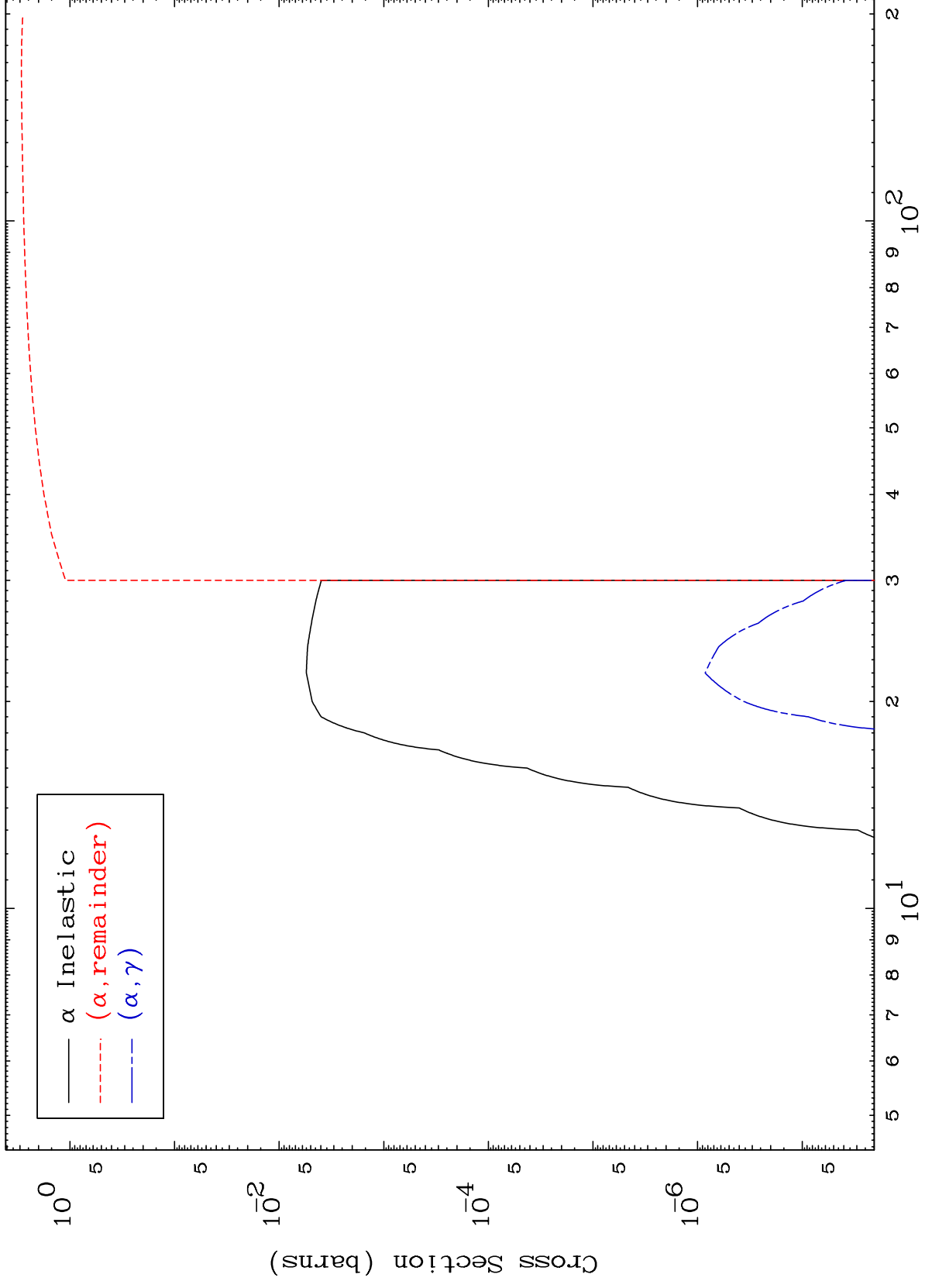
Press Mouse Button to Start

MAT 8579

$\alpha$  Major

85-At-221

0 Kelvin Cross Sections



1

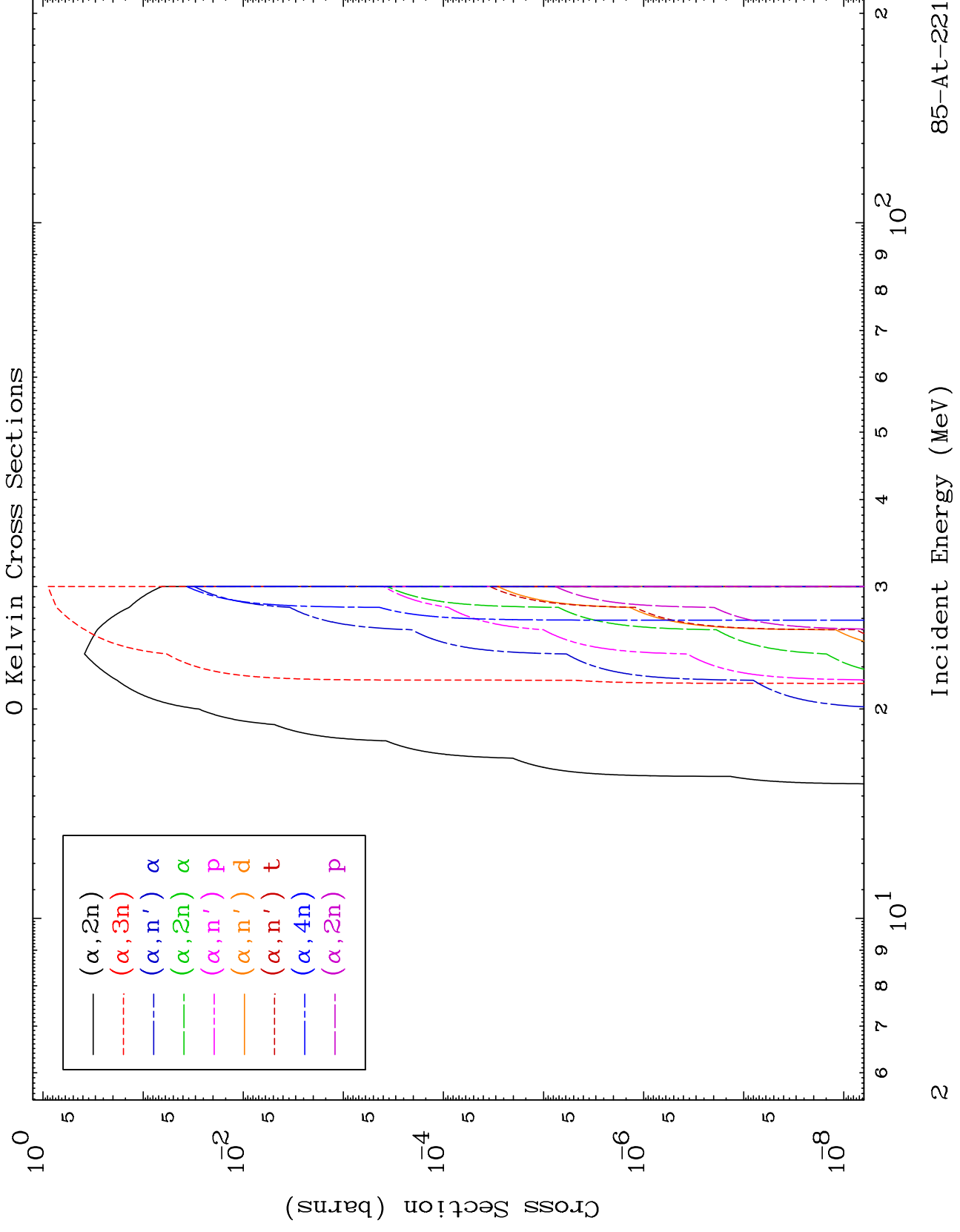
Incident Energy (MeV)

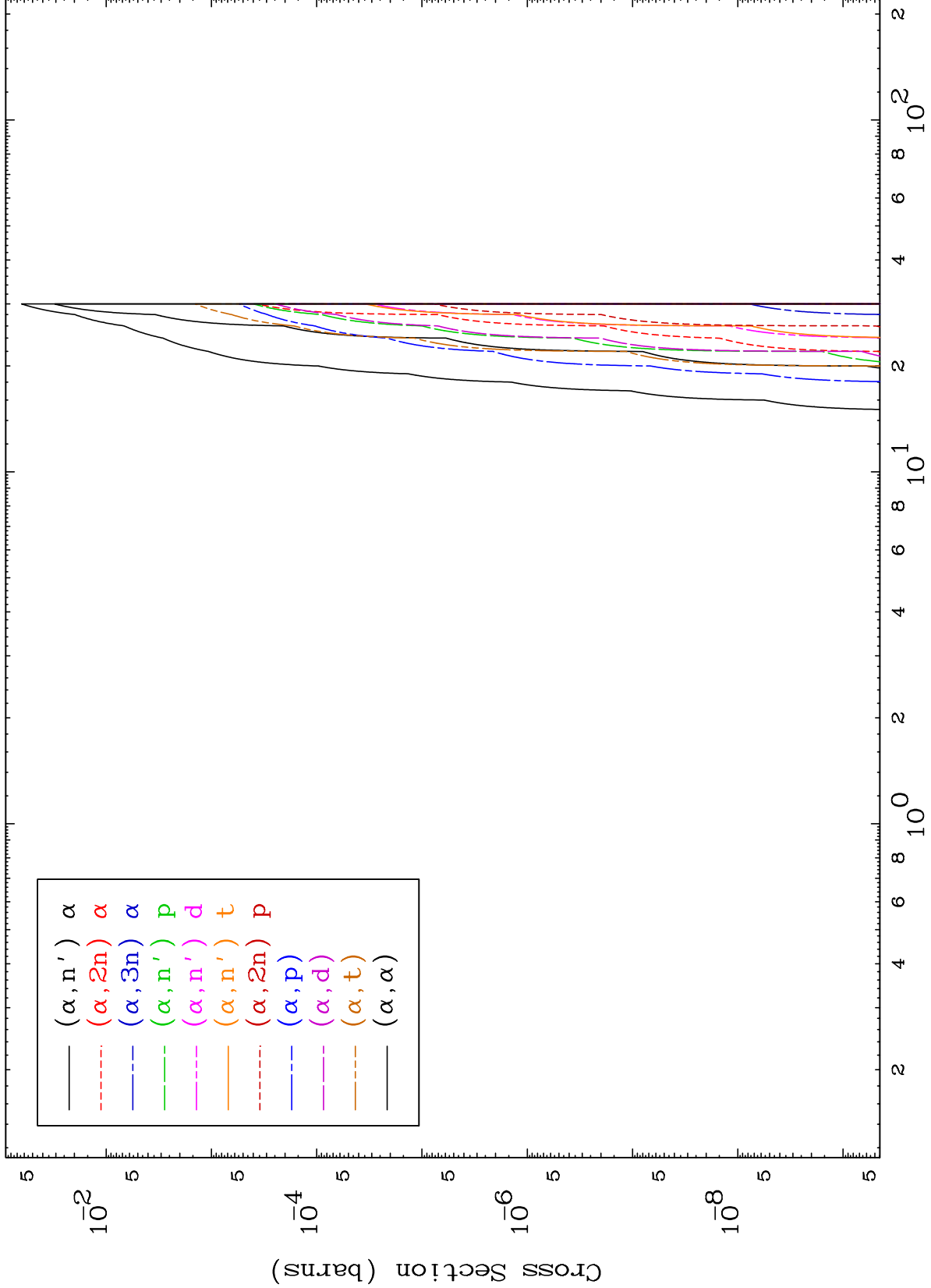
85-At-221

MAT 8579

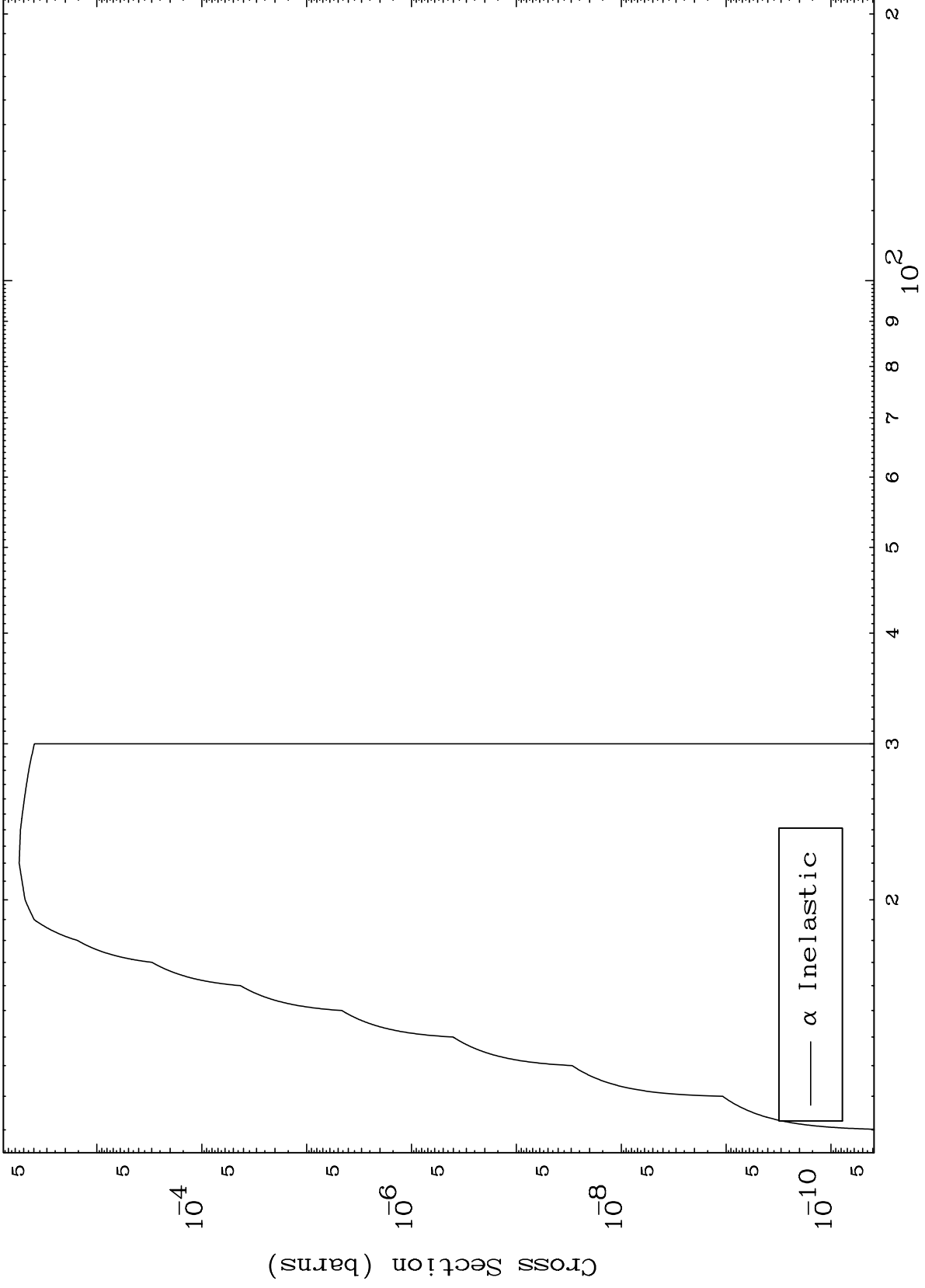
$\alpha$  Neutron Production

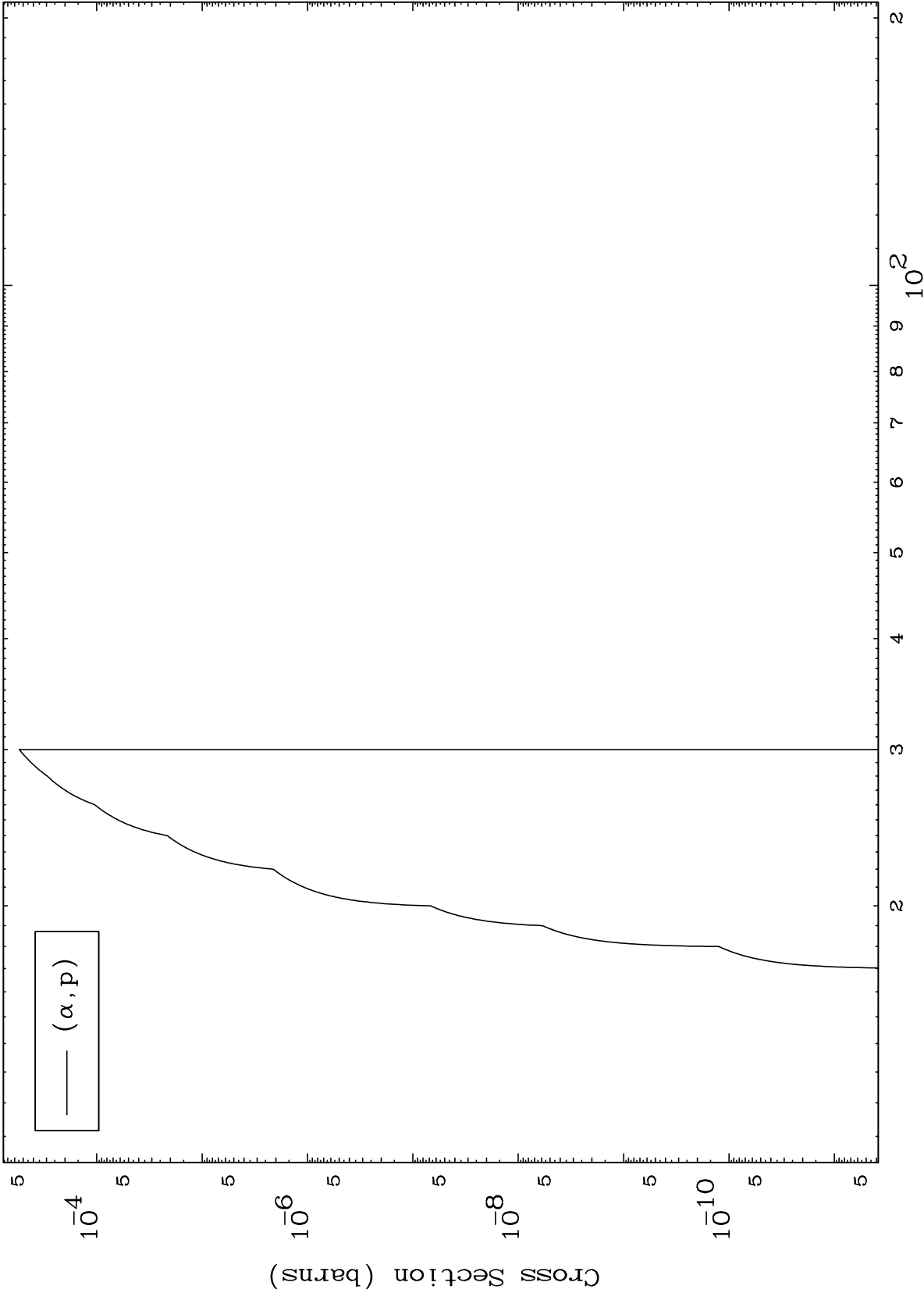
85-At-221

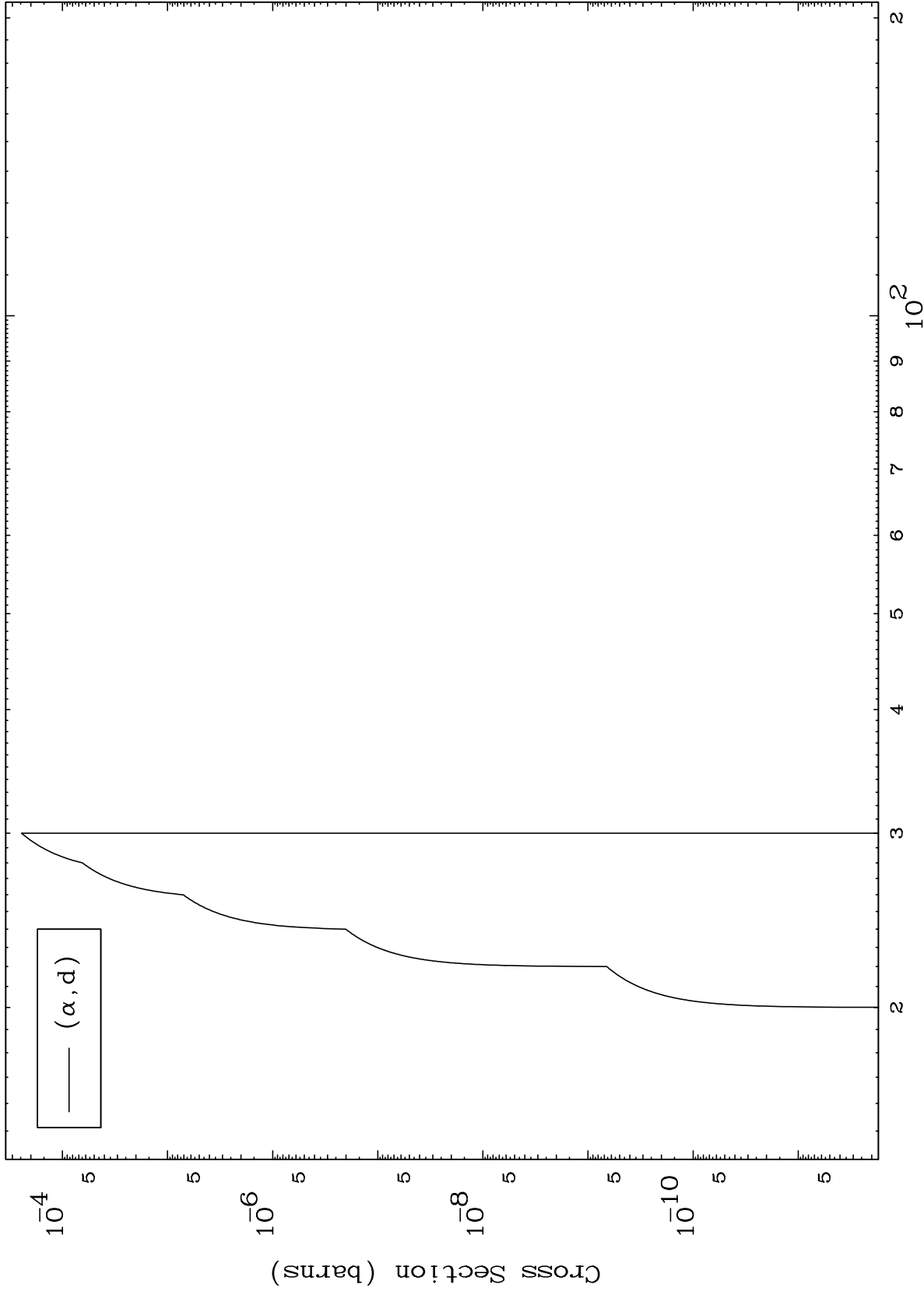




0 Kelvin Cross Sections



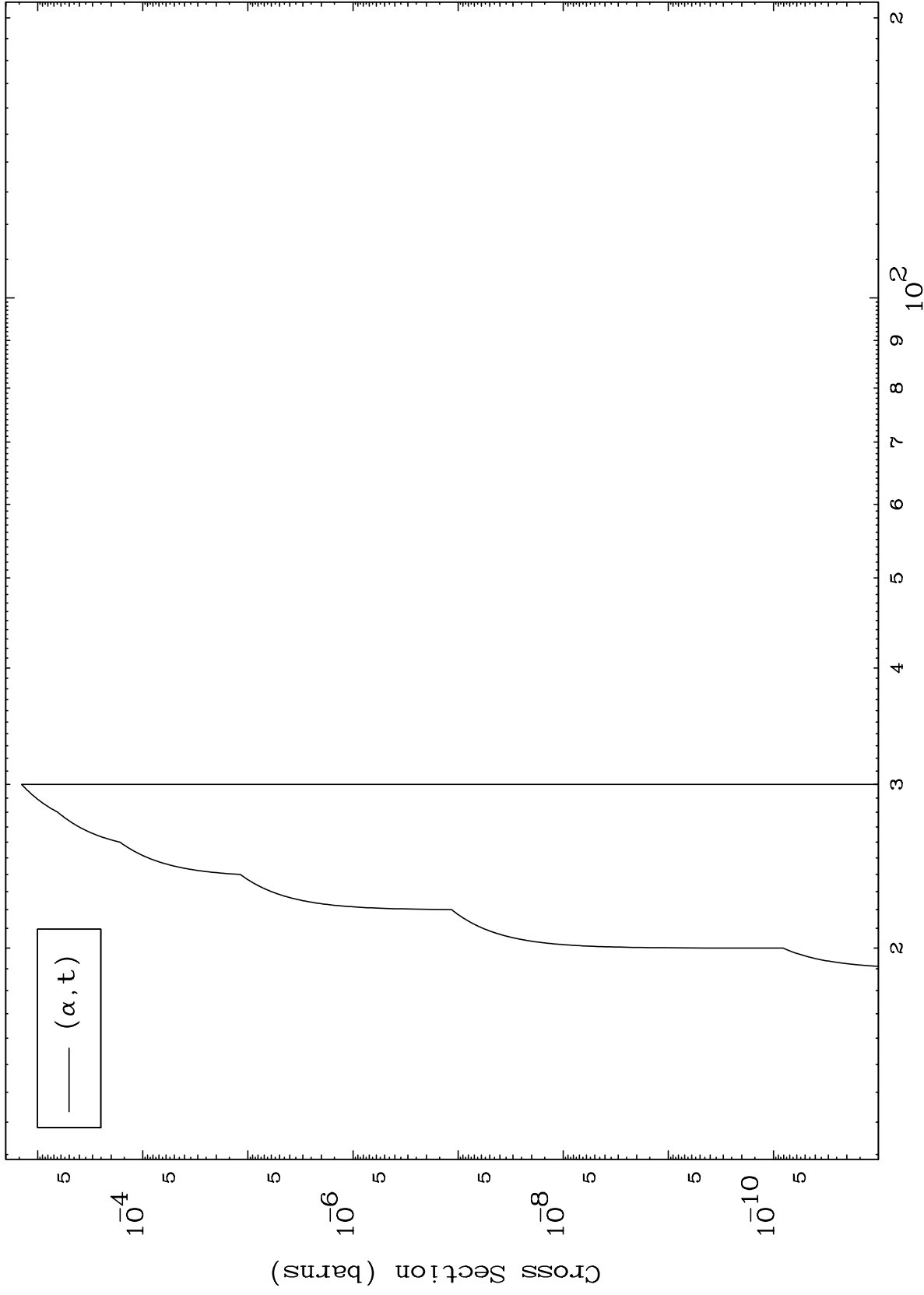




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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

85-At-221



7

Incident Energy (MeV)

85-At-221

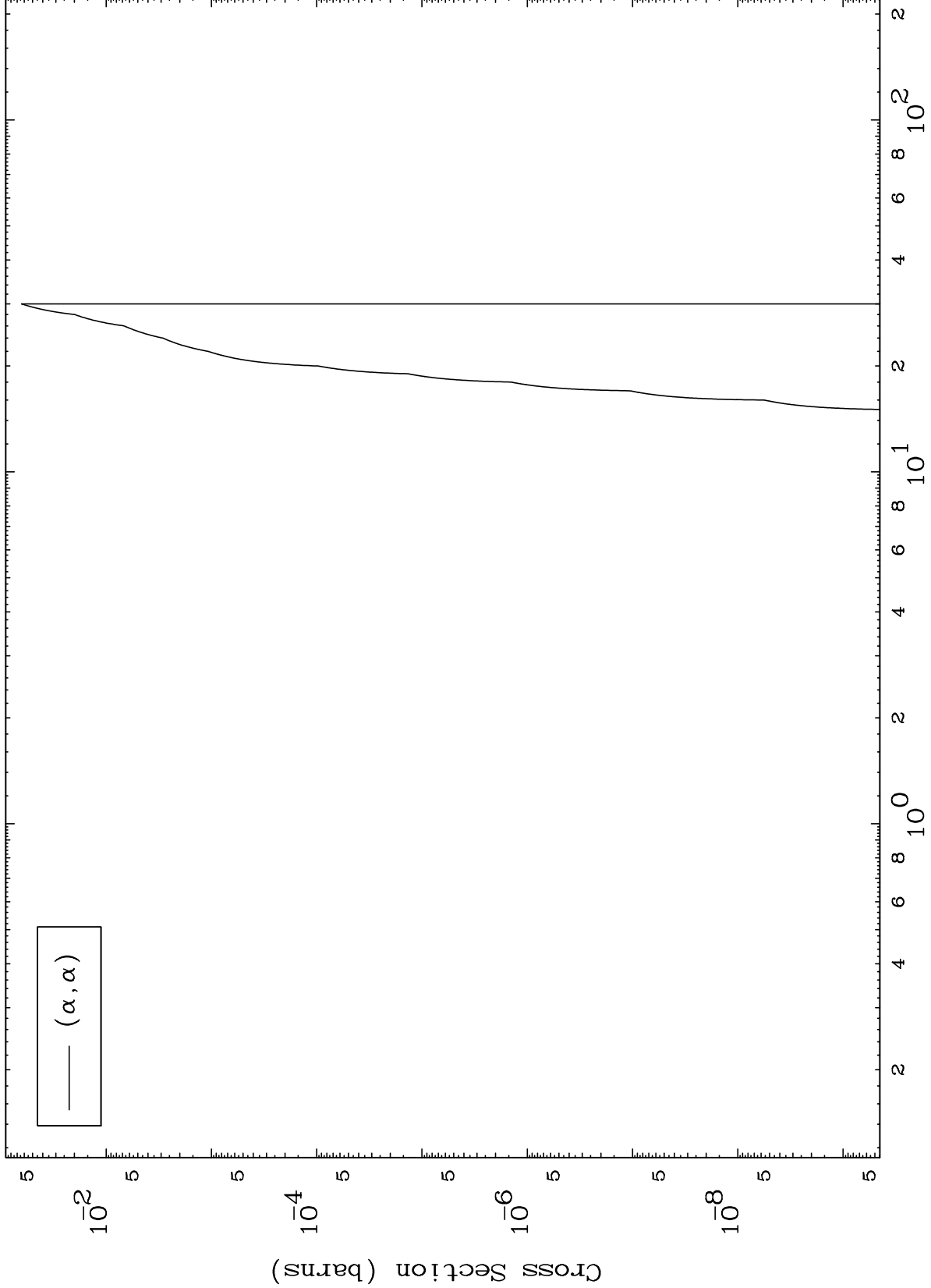


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( $\alpha, \alpha$ ) Levels

85-At-221

0 Kelvin Cross Sections



MAT 8579

85-At-221

$\alpha$  Fission  
Radionuclide Production Cross Section

