

Program EVALPLOT  
(Version 2018-1)

by

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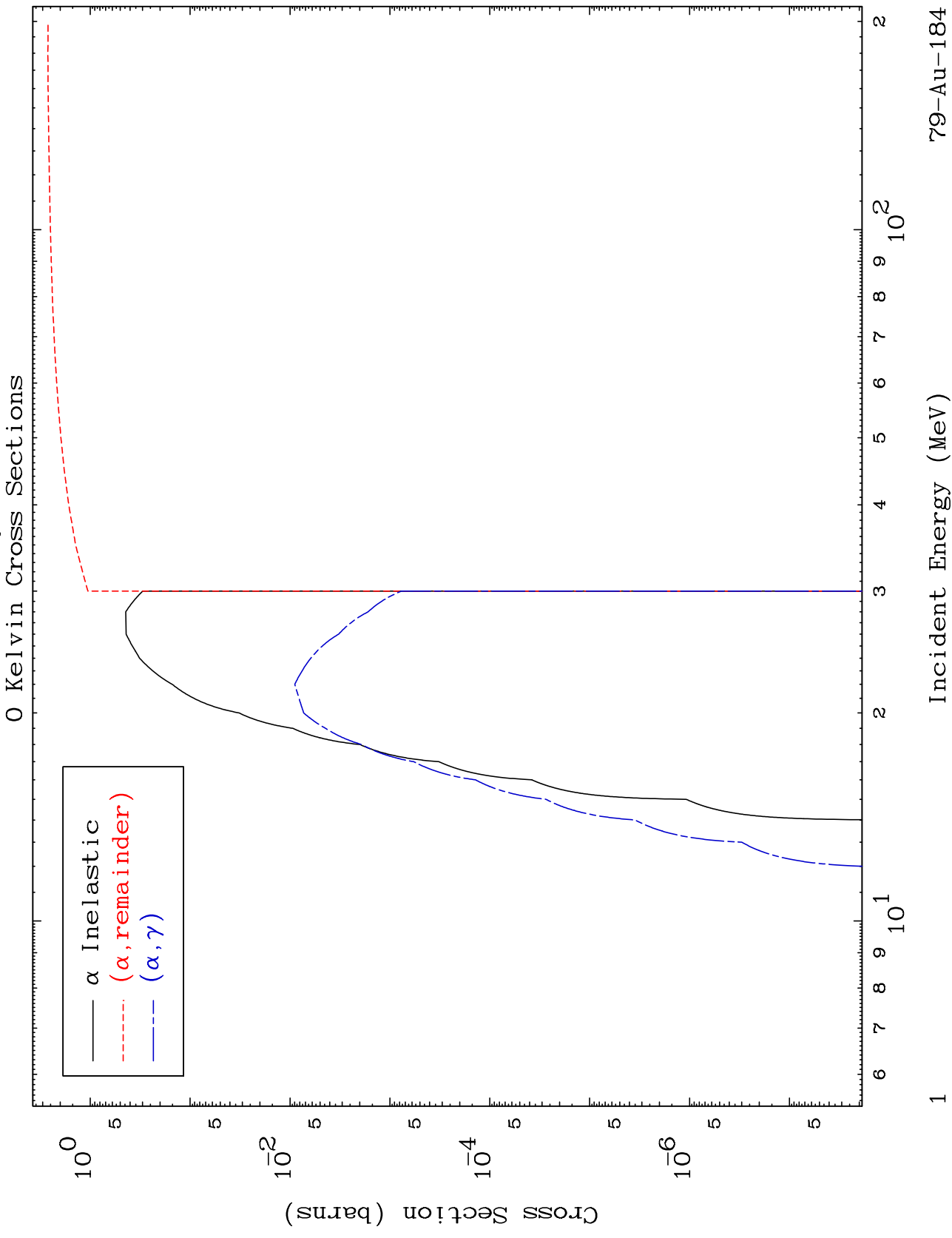
Press Mouse Button to Start

MAT 7886

0 Kelvin

$\alpha$  Major

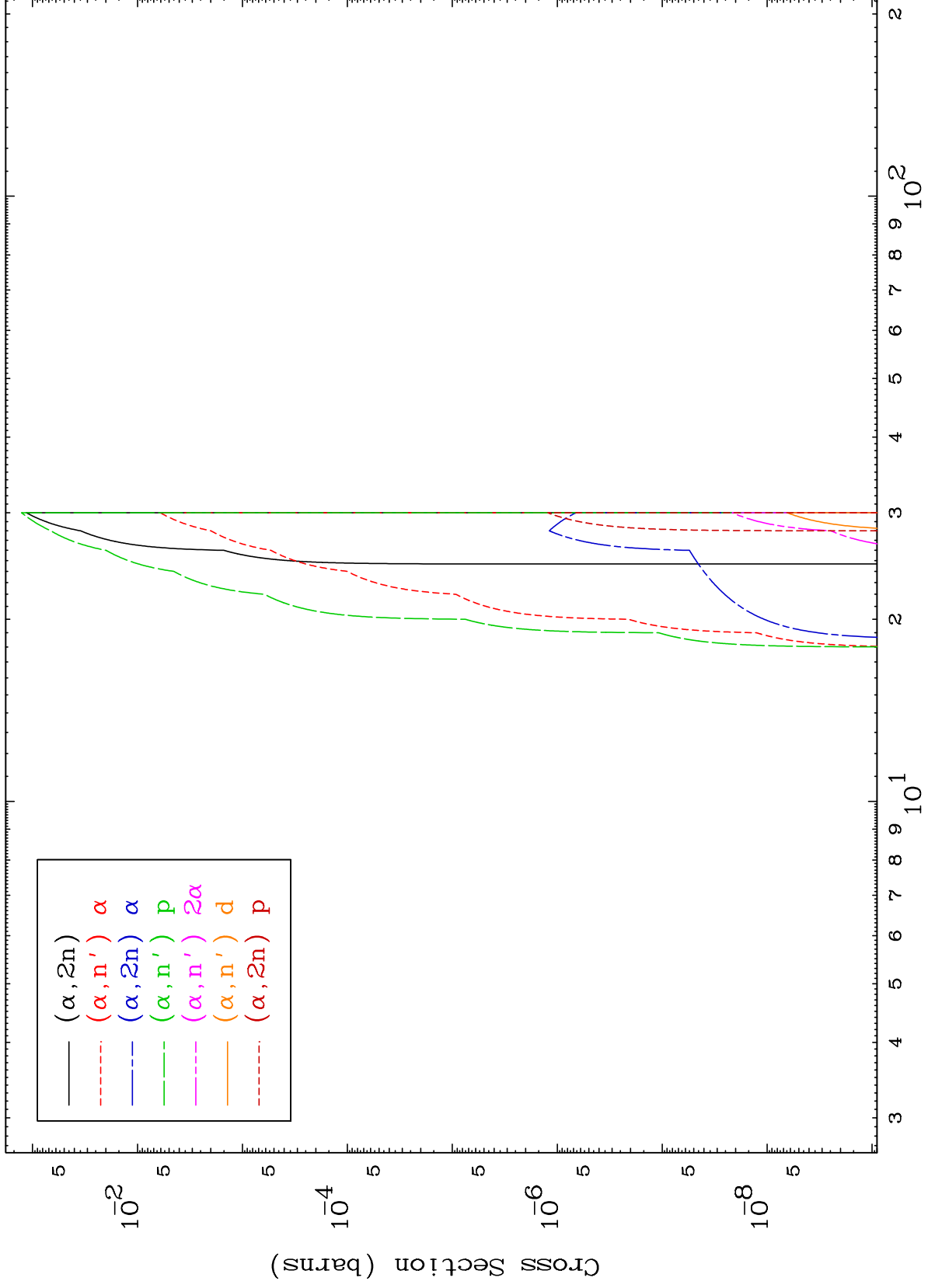
79-Au-184

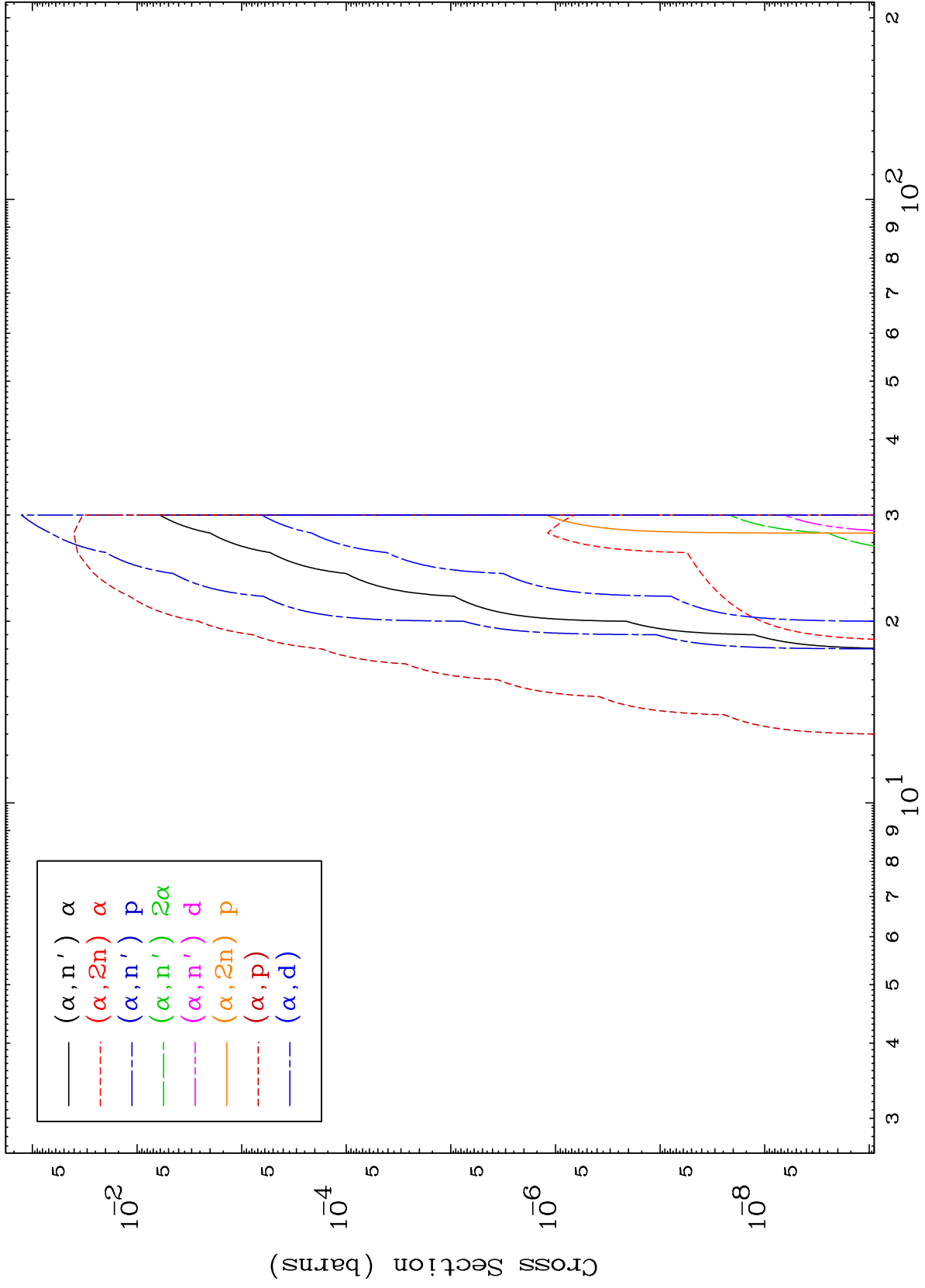


1

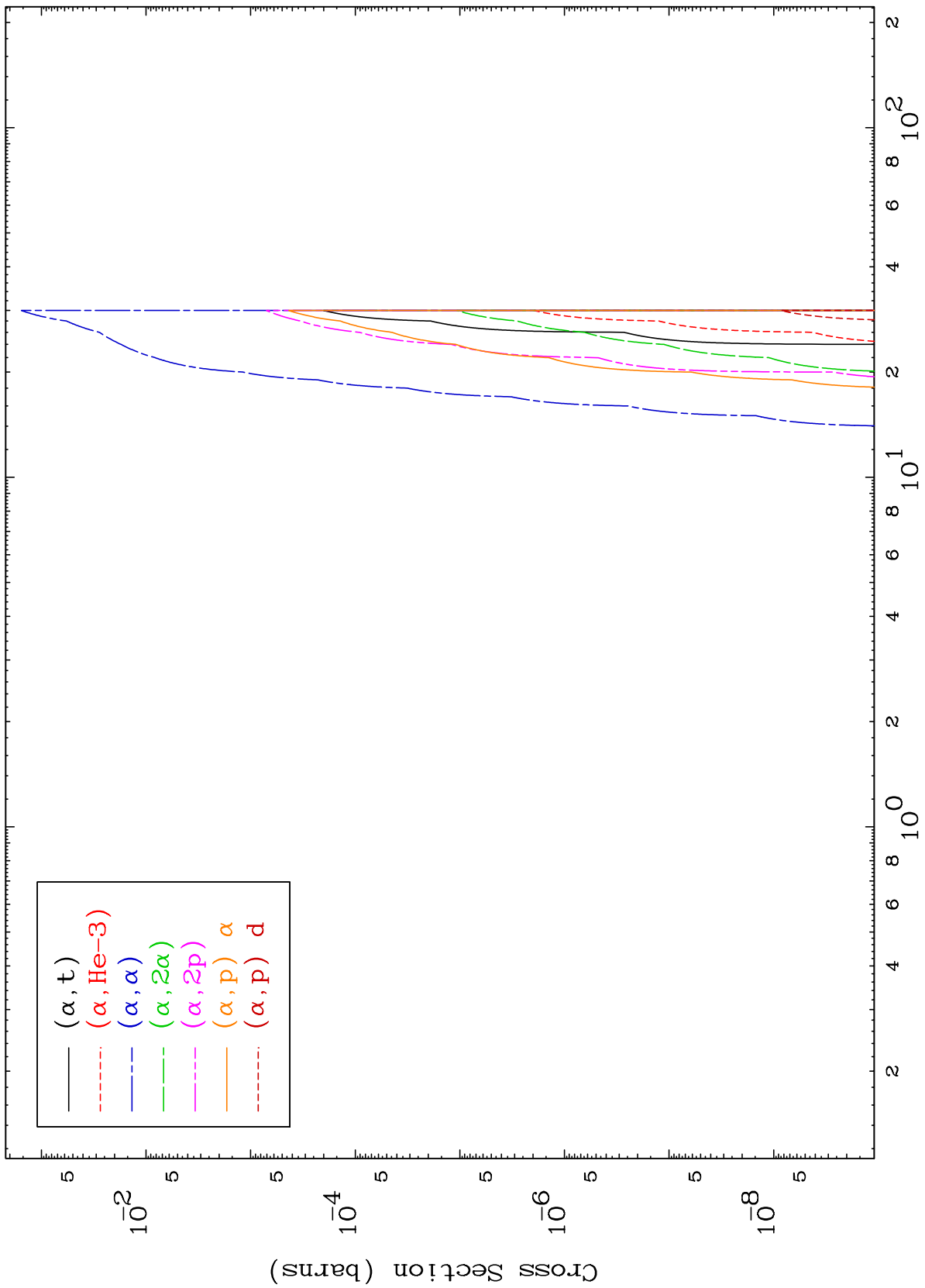
Incident Energy (MeV)

79-Au-184





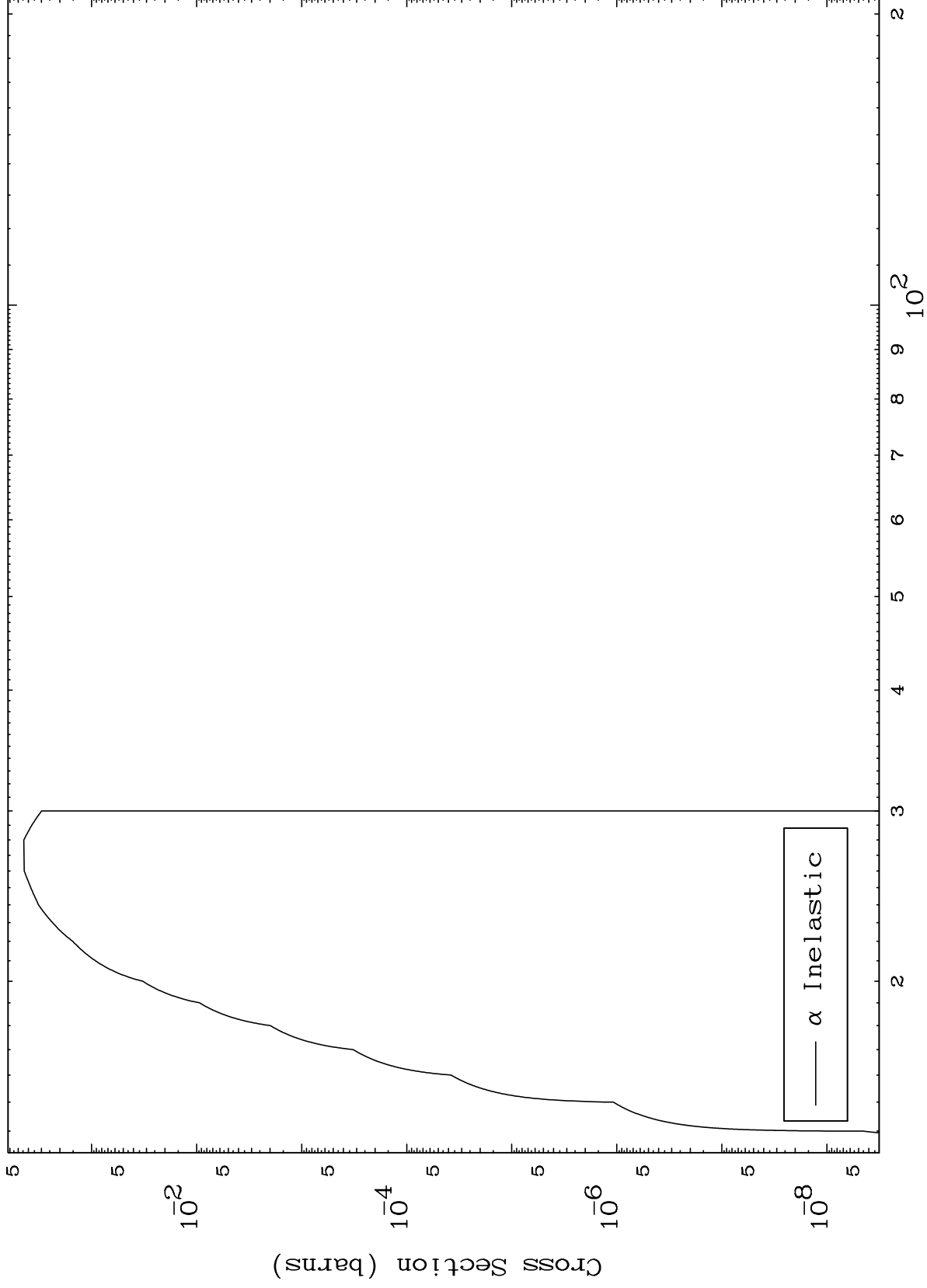
0 Kelvin Cross Sections



MAT 7886

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

79-Au-184



5

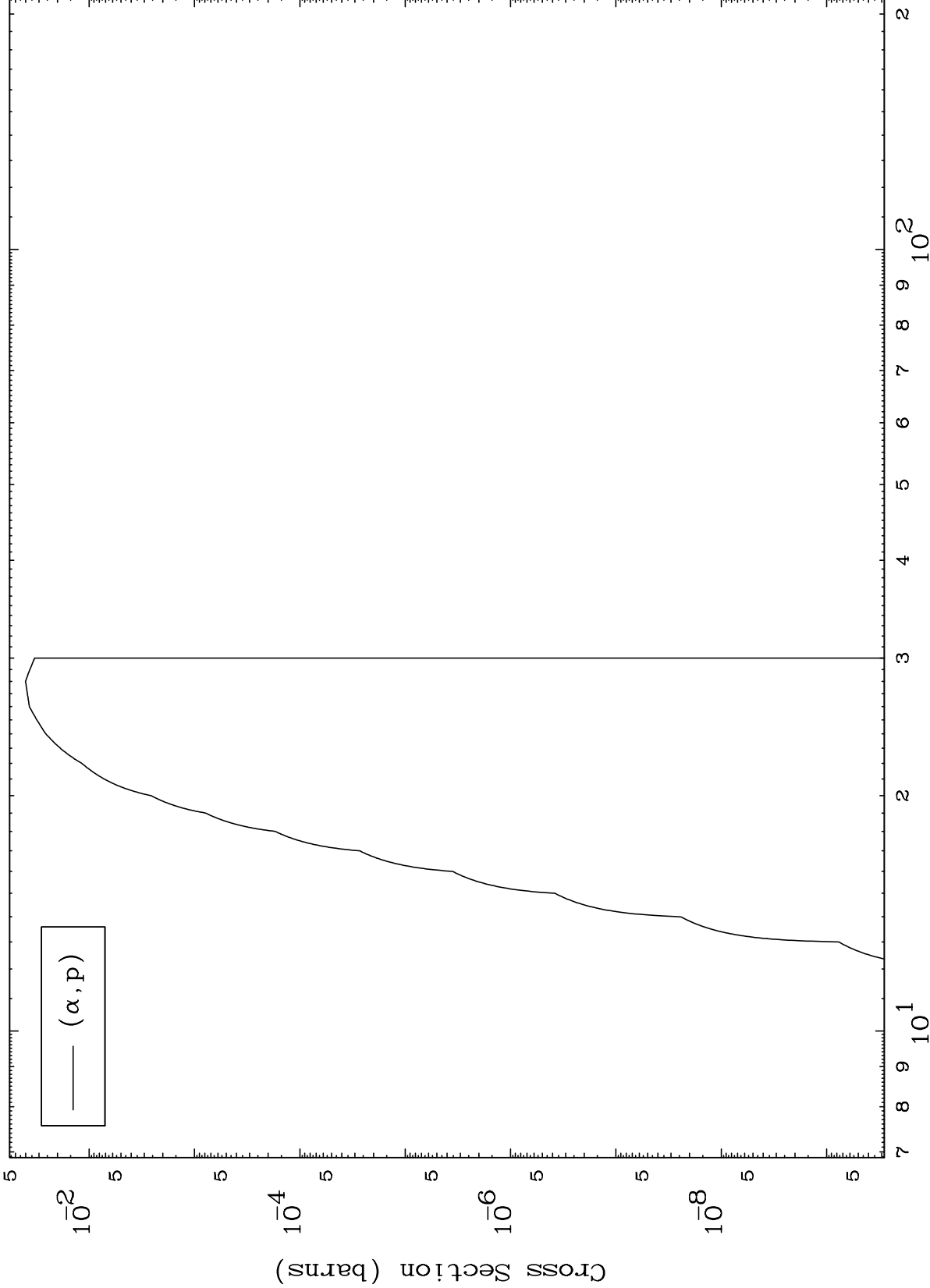
Incident Energy (MeV)

79-Au-184

MAT 7886

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

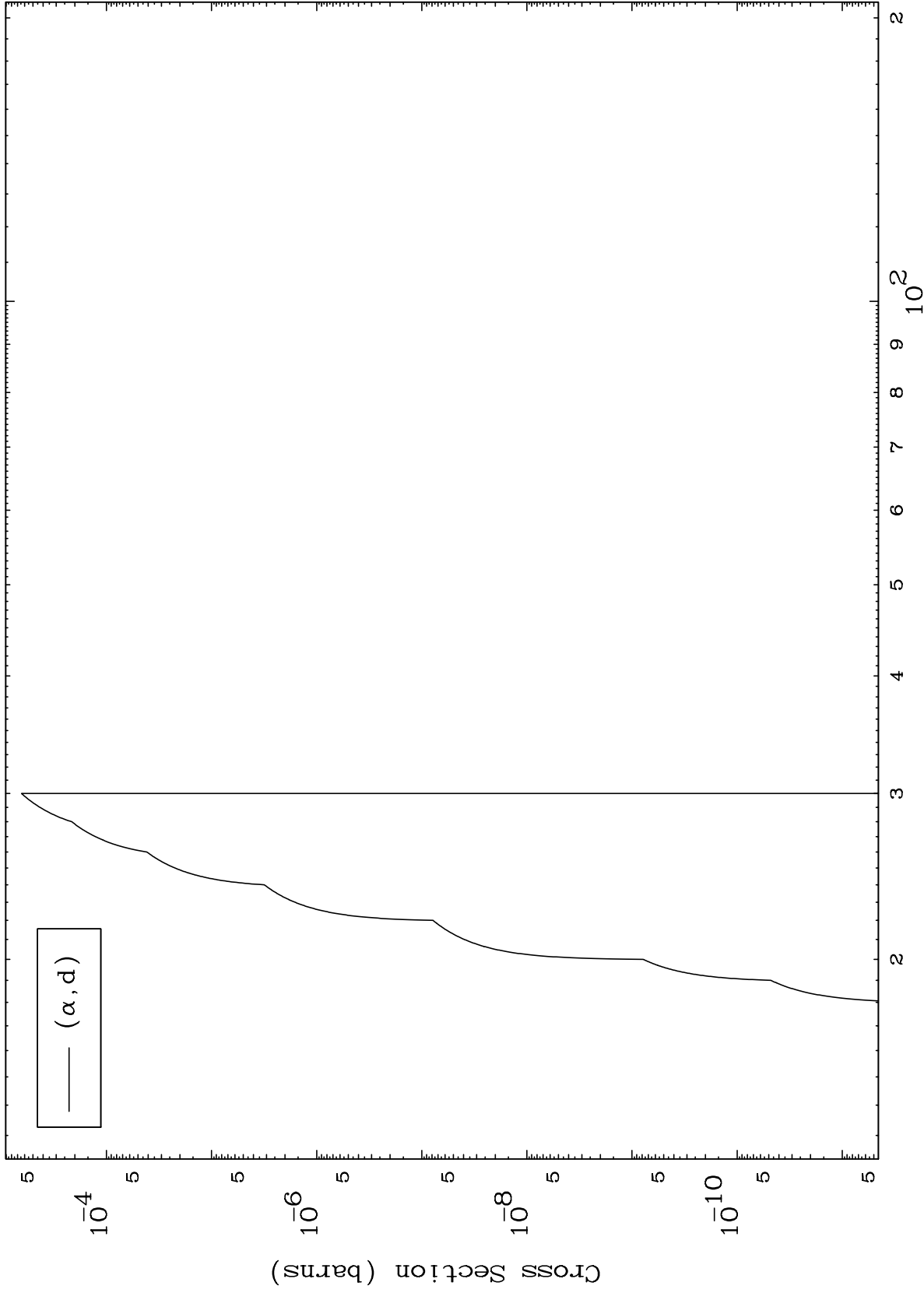
79-Au-184



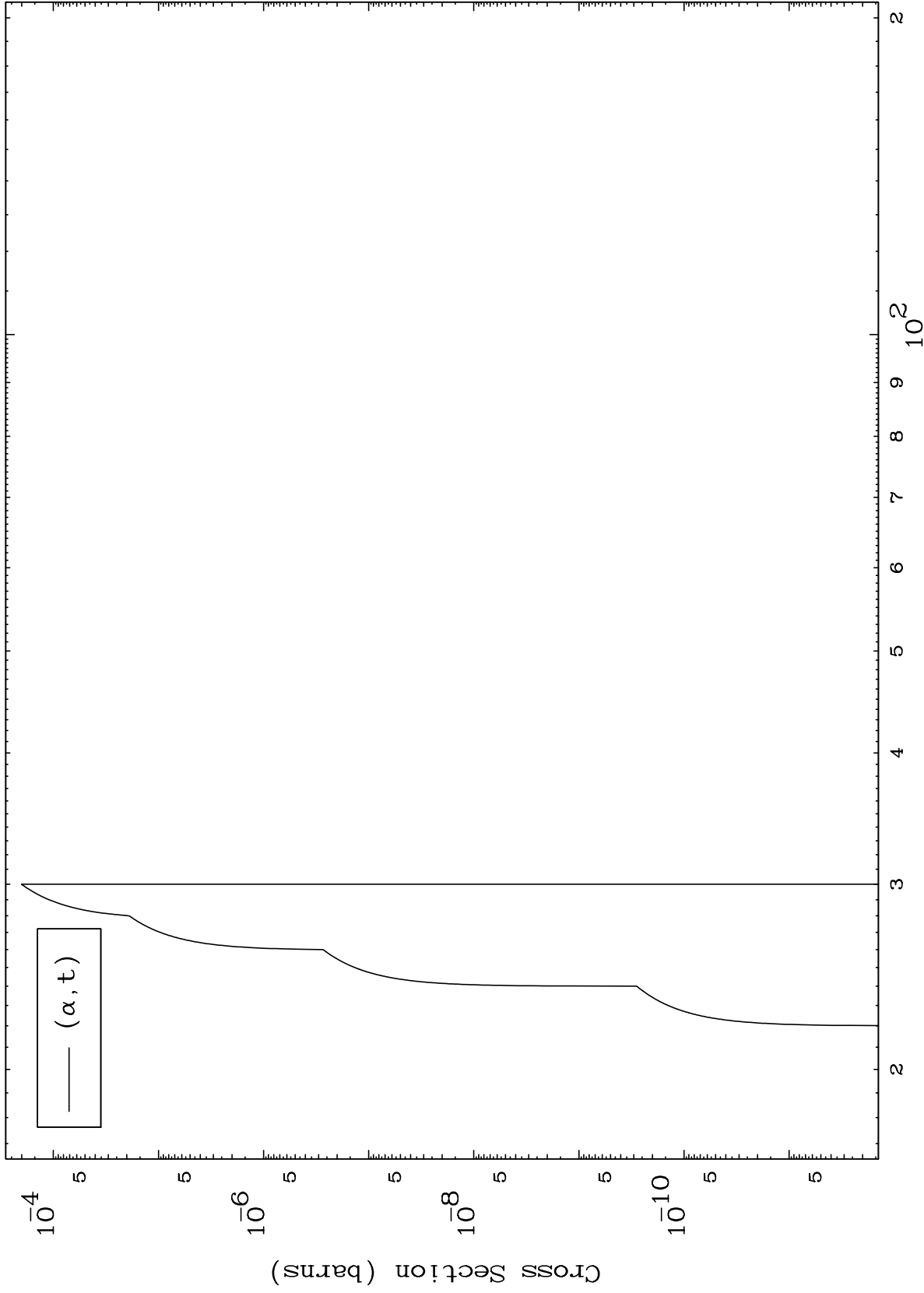
6

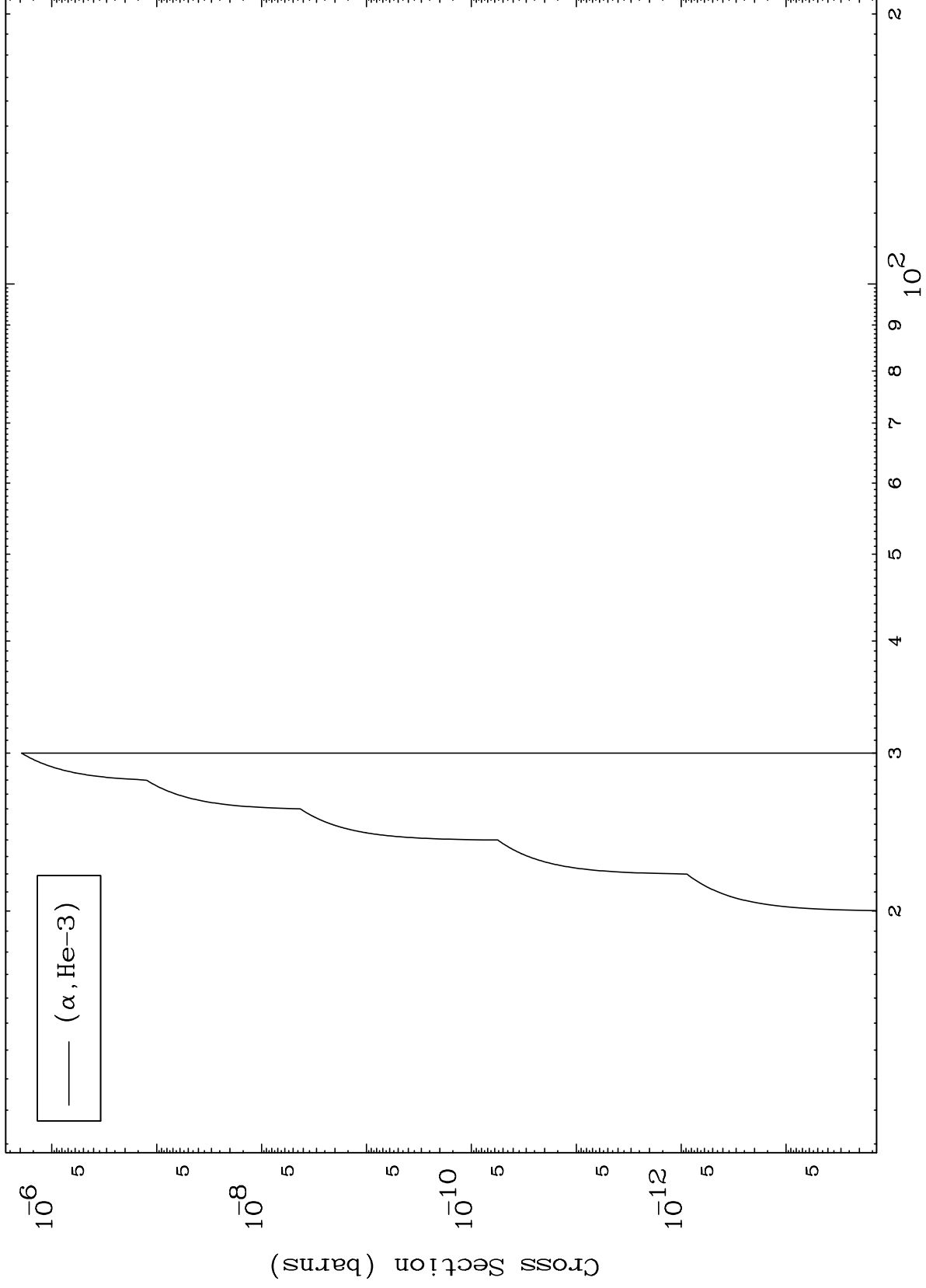
Incident Energy (MeV)

79-Au-184





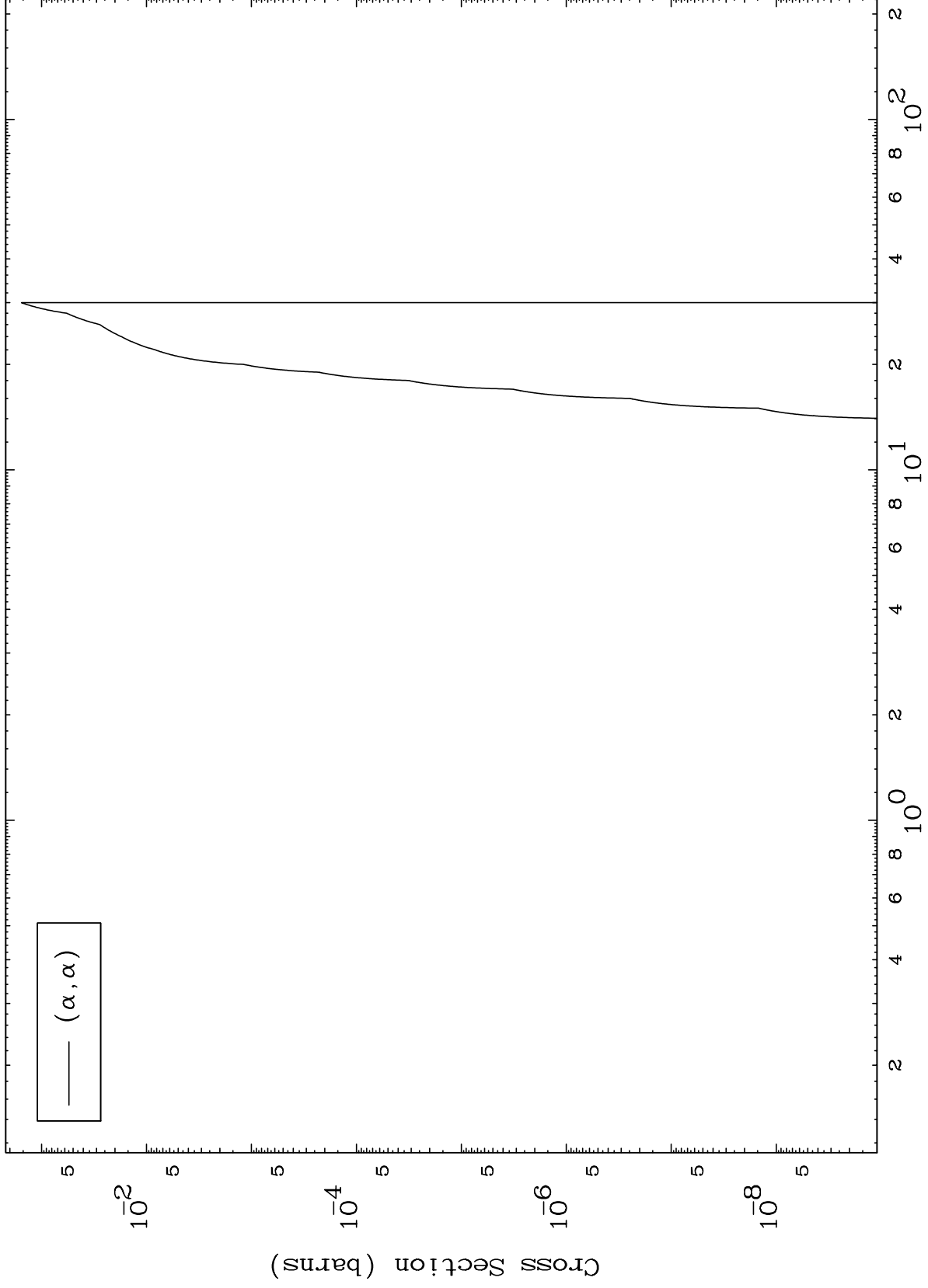




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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

79-Au-184

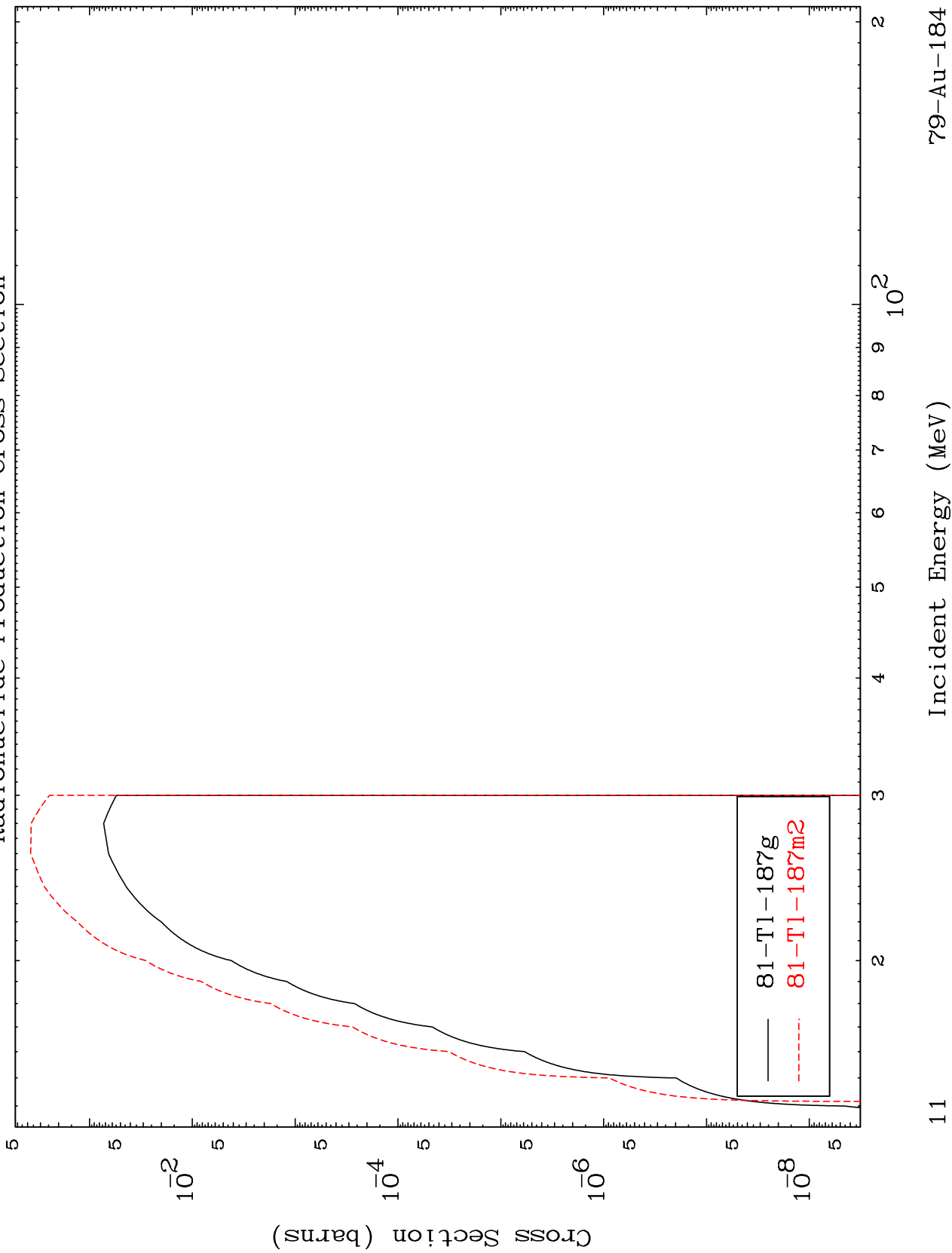


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Incident Energy (MeV)

79-Au-184

Radionuclide Production Cross Section  
 $\alpha$  Inelastic

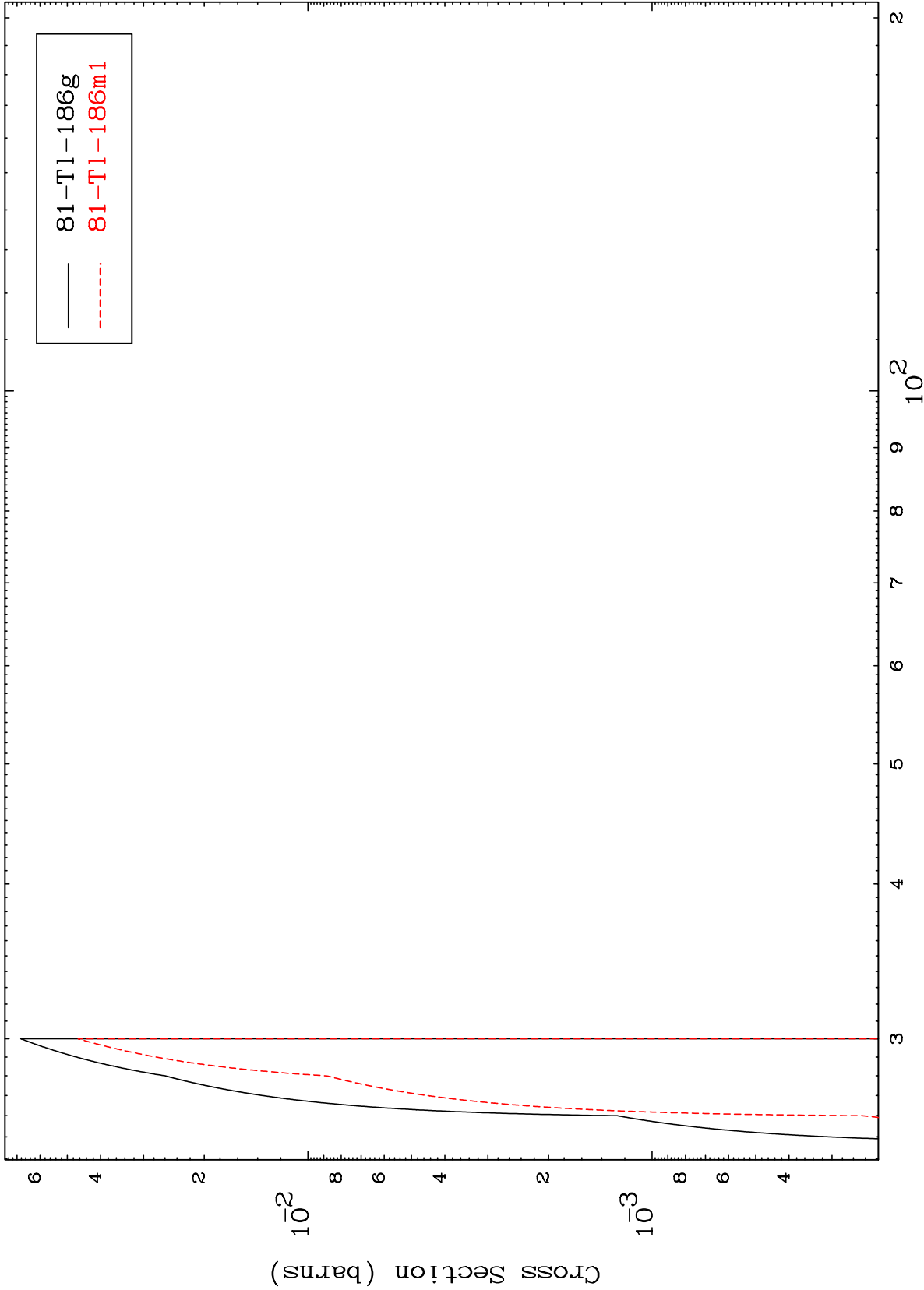


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( $\alpha, 2n$ )

79-Au-184

Radionuclide Production Cross Section



12

Incident Energy (MeV)

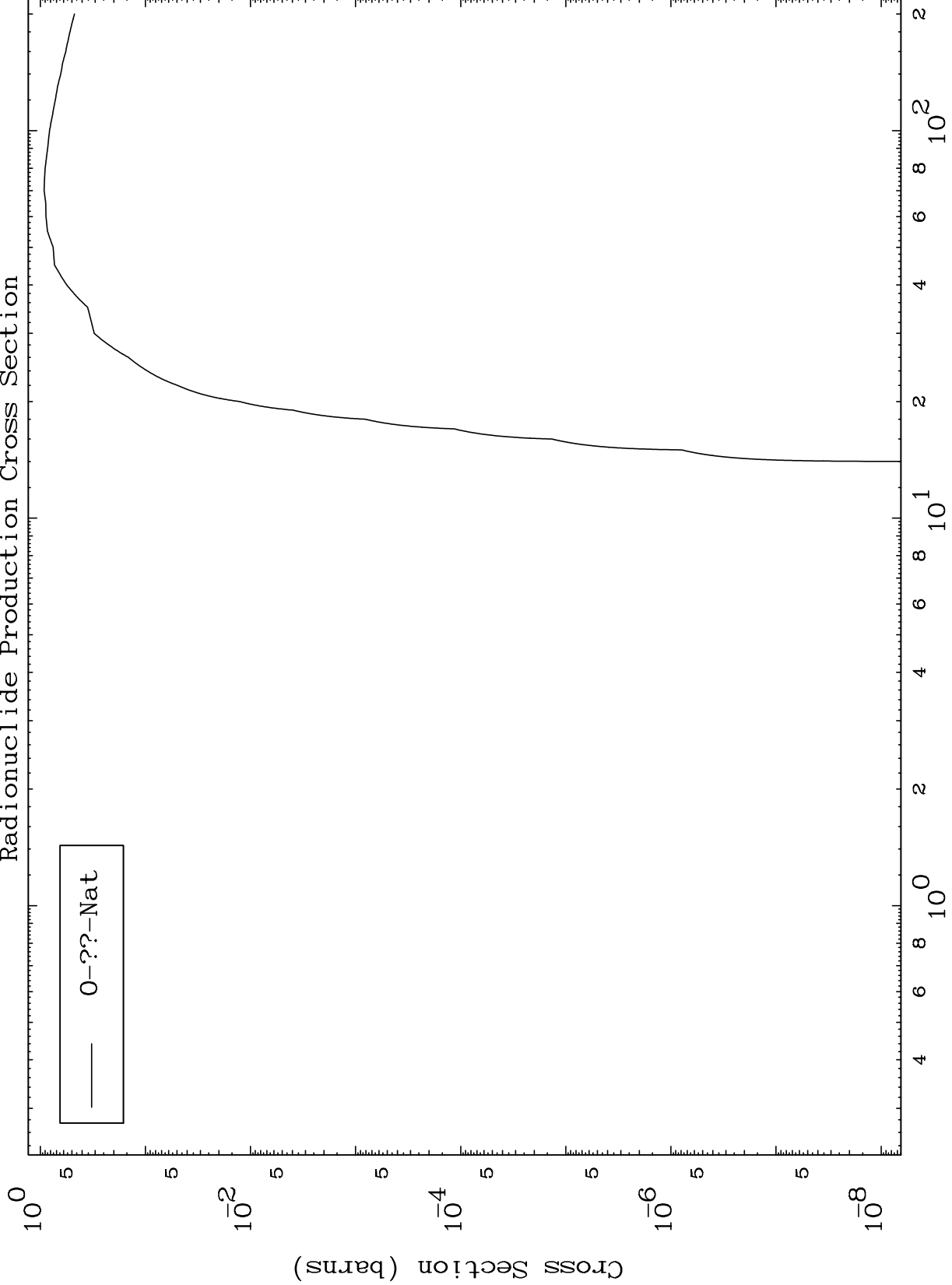
79-Au-184

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$\alpha$  Fission

79-Au-184

Radionuclide Production Cross Section

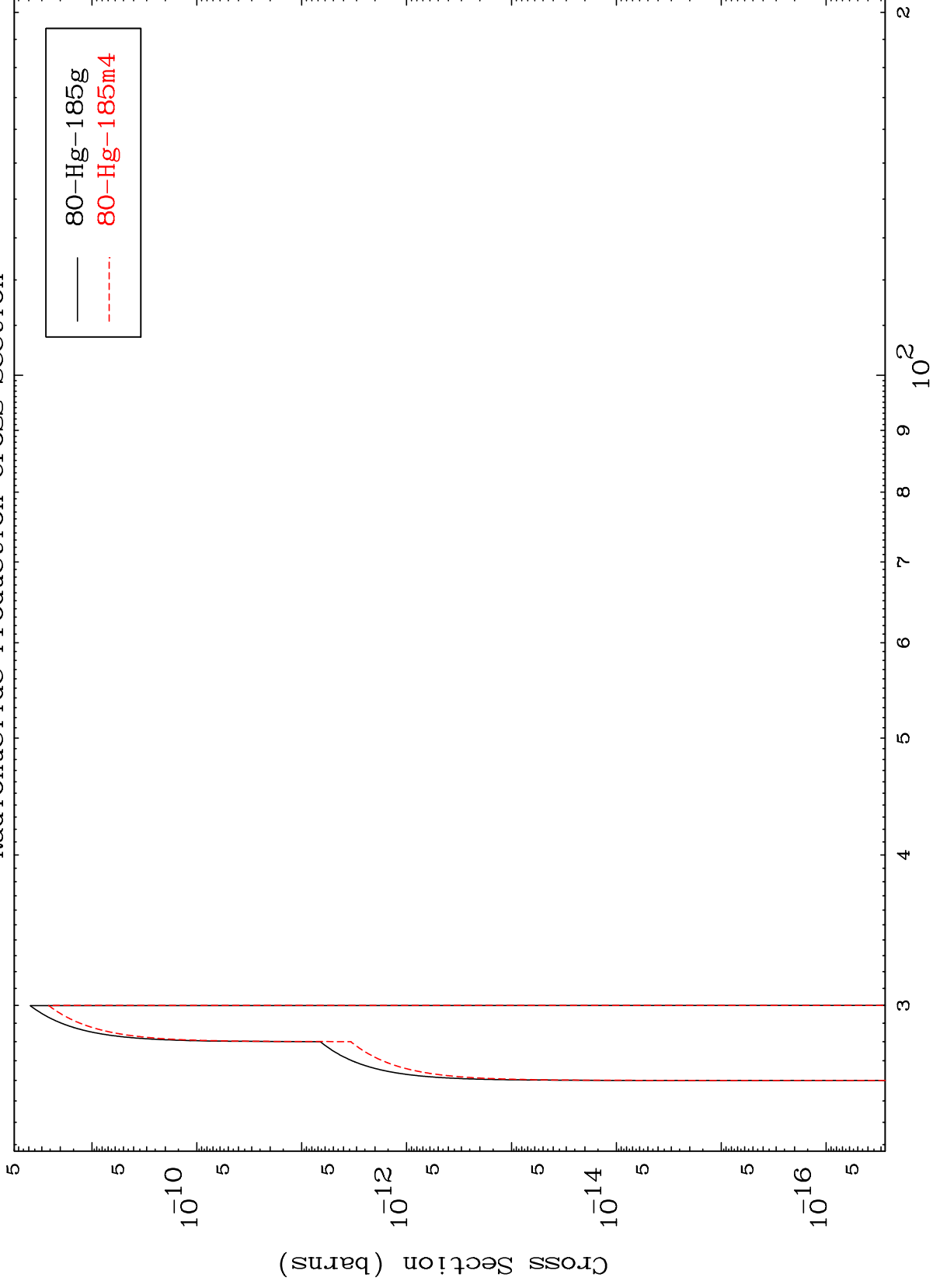


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$(\alpha, n')$  d

<sup>79</sup>Au-184

Radionuclide Production Cross Section



14

Incident Energy (MeV)

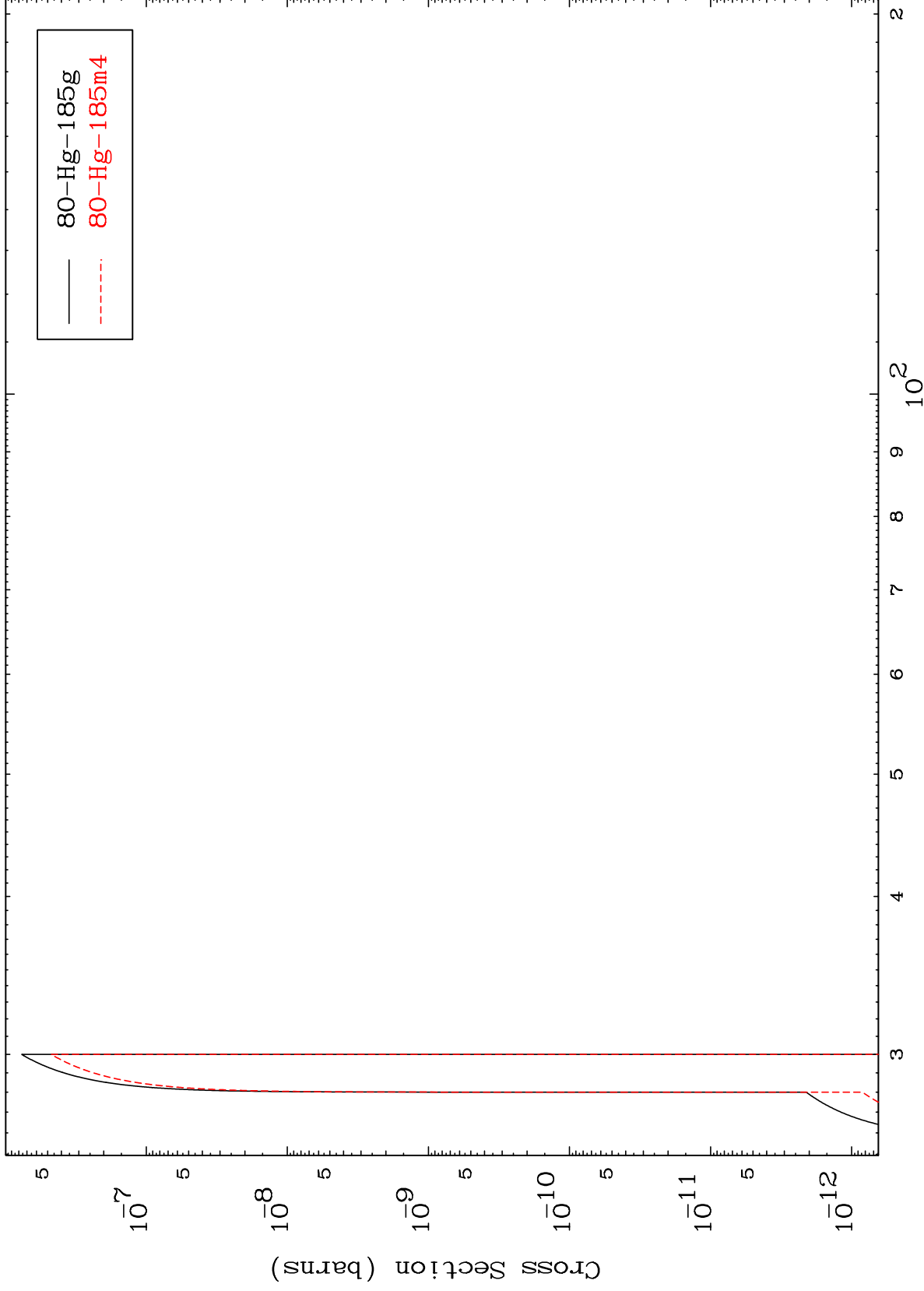
<sup>79</sup>Au-184

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$(\alpha, 2n)$  p

79-Au-184

Radionuclide Production Cross Section



15

Incident Energy (MeV)

79-Au-184

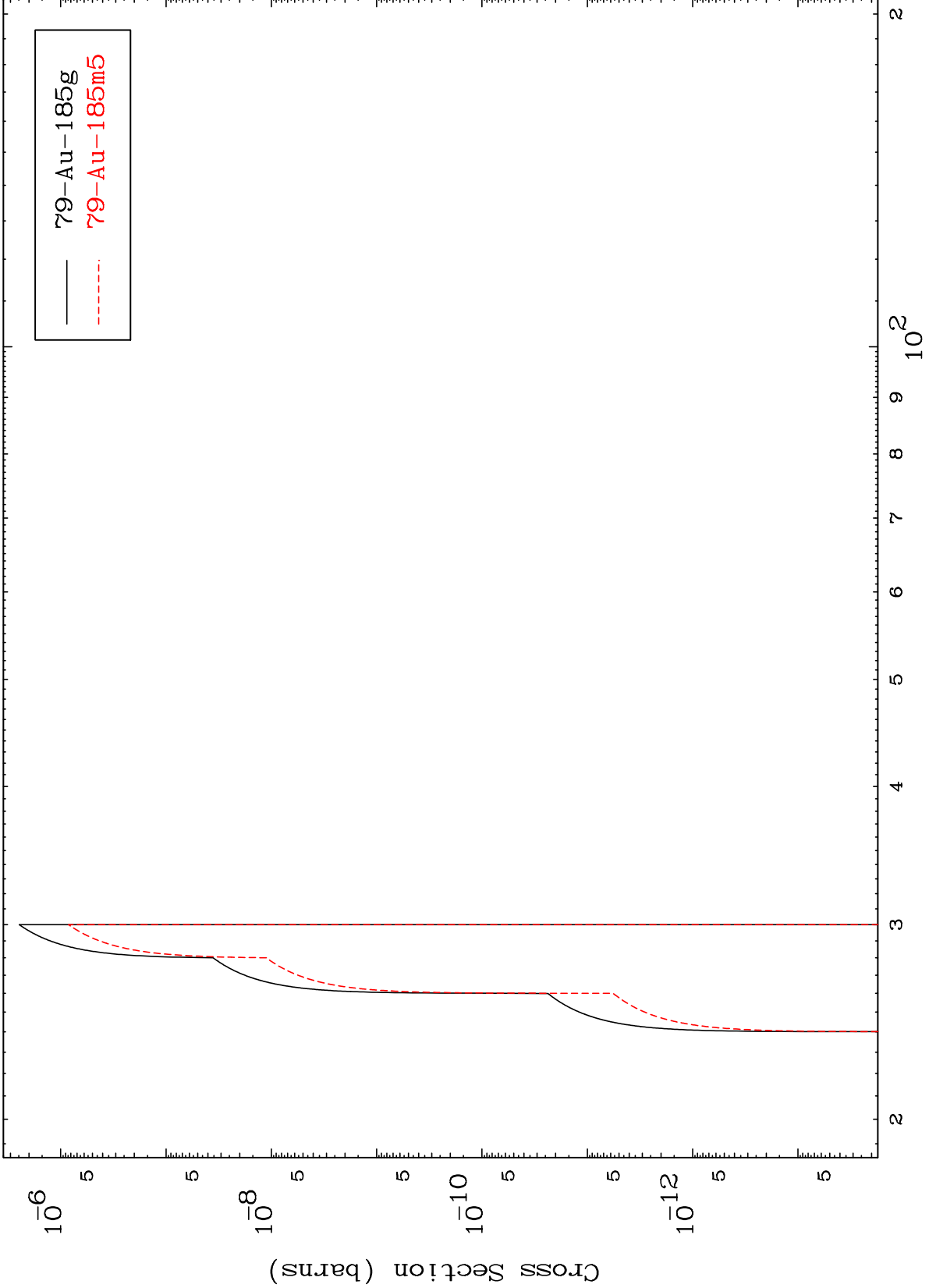


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$(\alpha, 2n)$  p

<sup>79</sup>Au-184

Radionuclide Production Cross Section



16

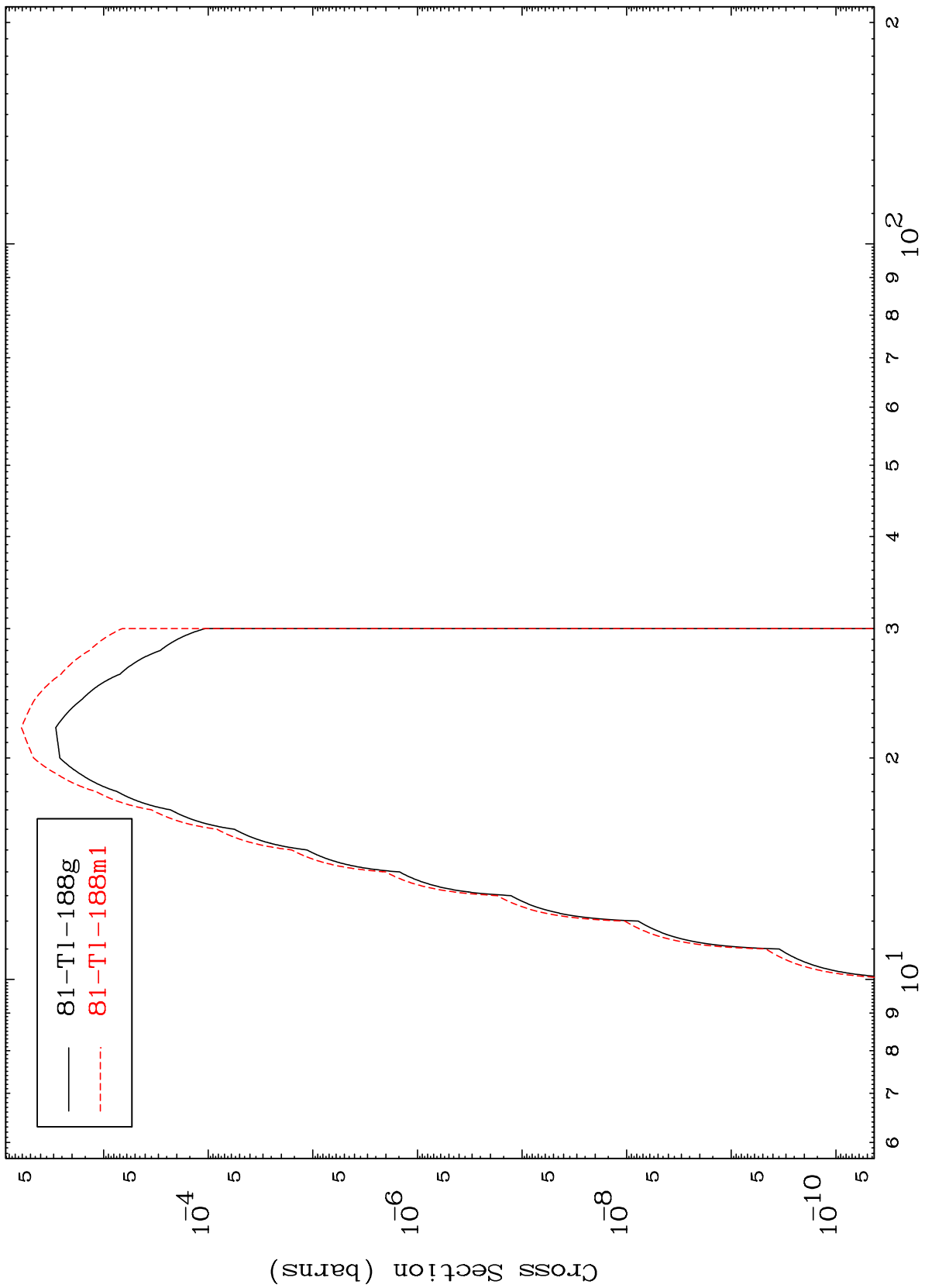
Incident Energy (MeV)

<sup>79</sup>Au-184

MAT 7886

79-Au-184

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



17

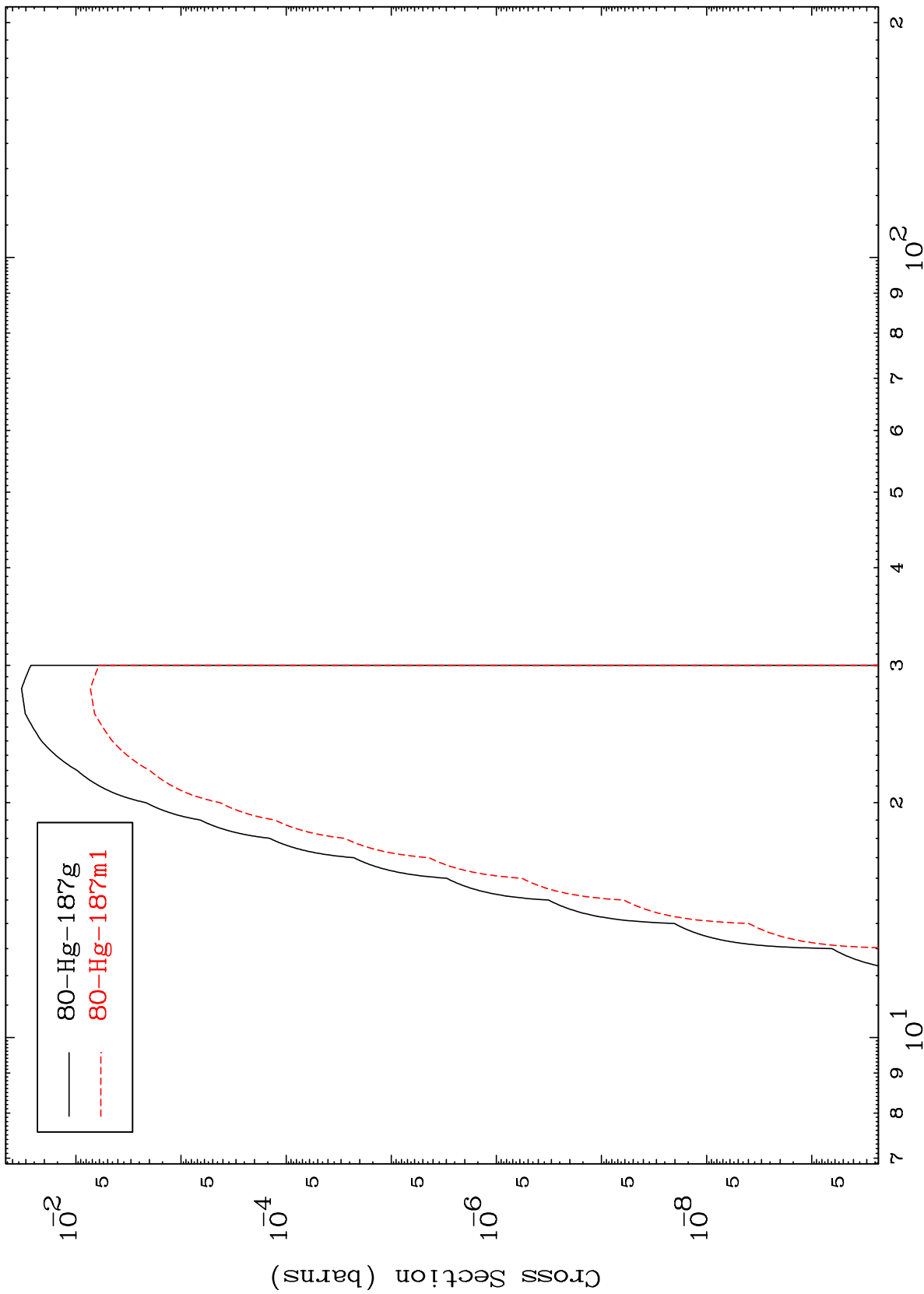
Incident Energy (MeV)

79-Au-184

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79-Au-184

( $\alpha, p$ )  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

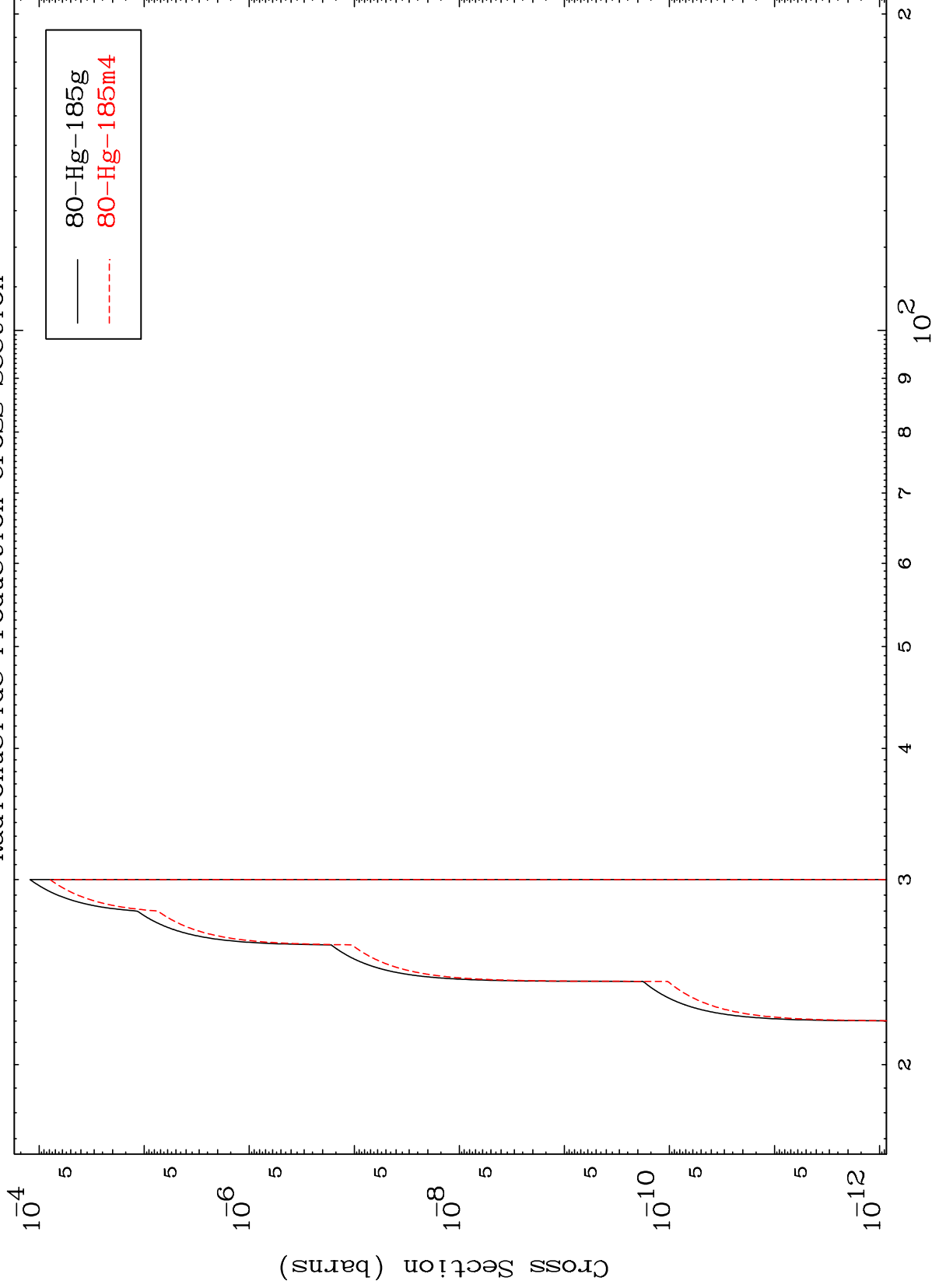
79-Au-184

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( $\alpha, t$ )

79-Au-184

Radionuclide Production Cross Section

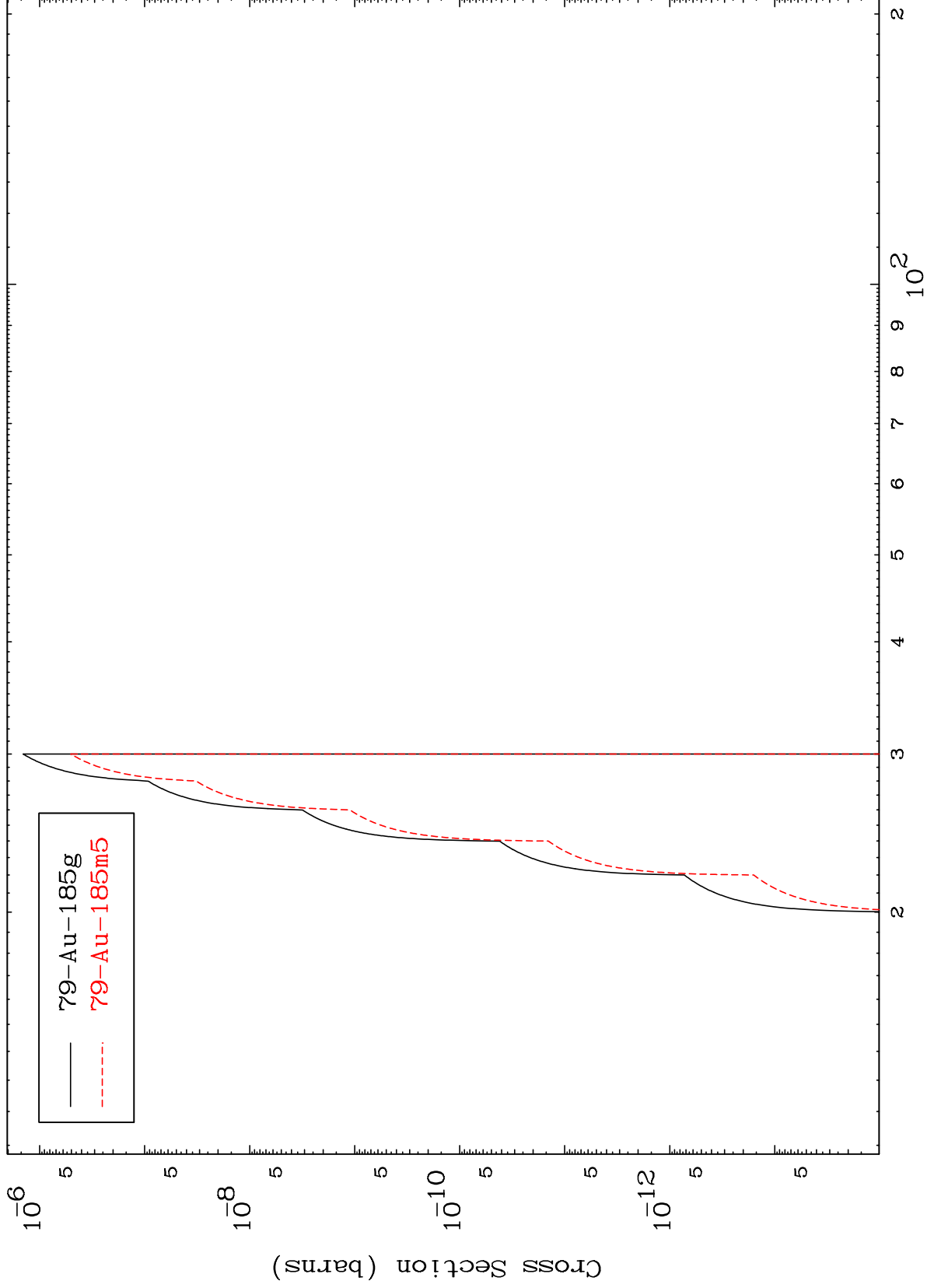


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Incident Energy (MeV)

79-Au-184

Radionuclide Production Cross Section

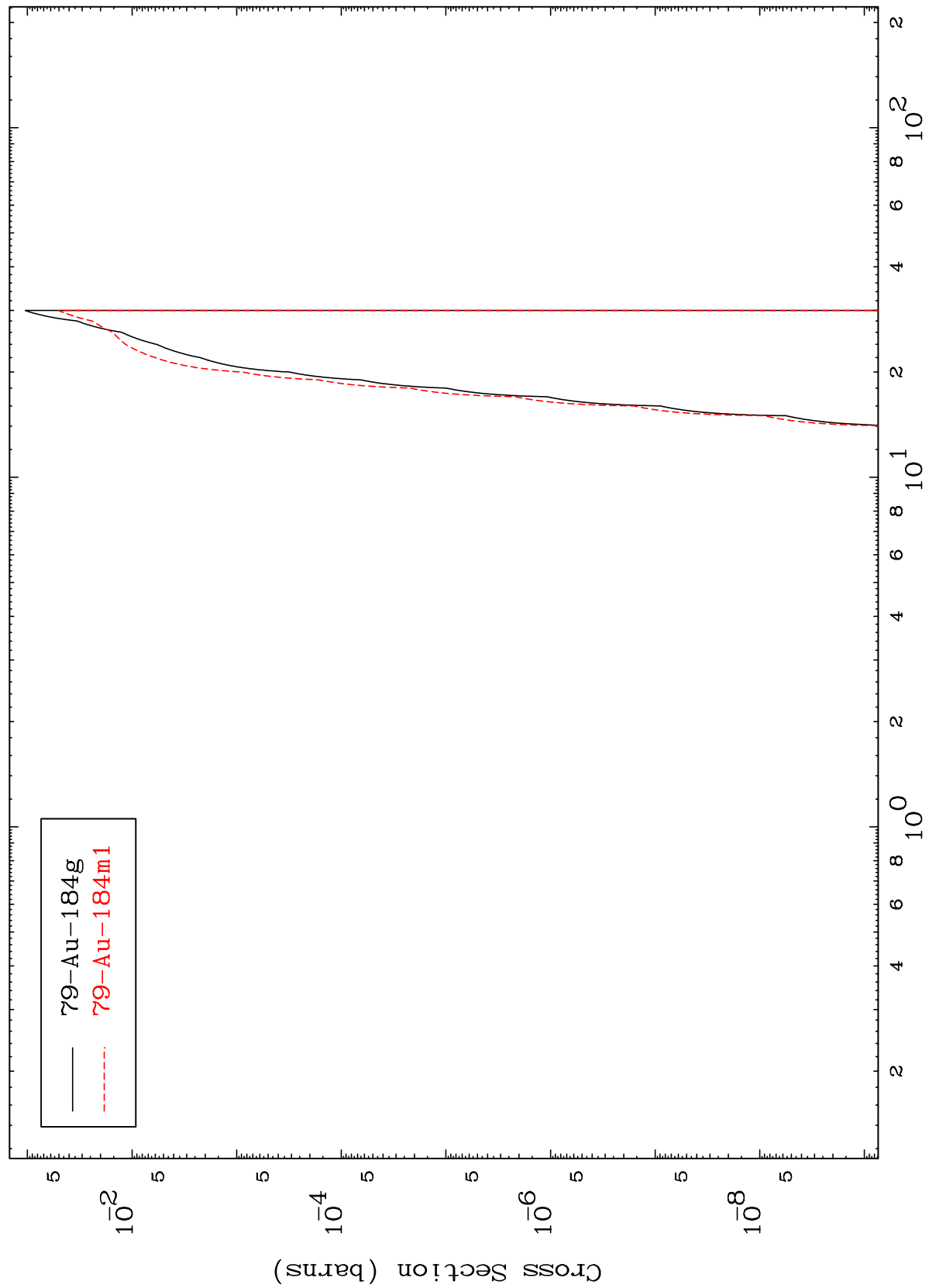


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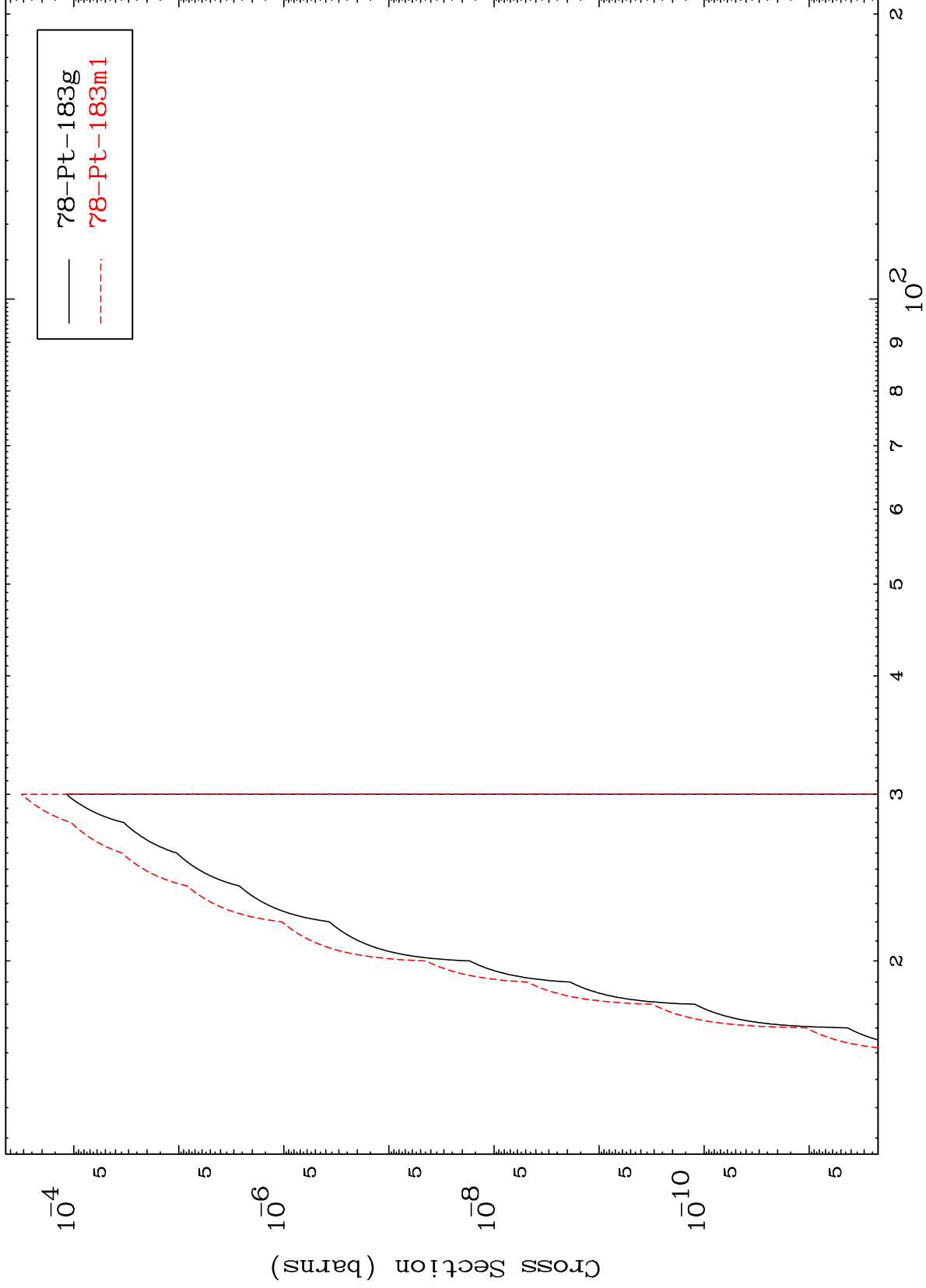
( $\alpha, \alpha$ )

<sup>79</sup>Au-184

Radionuclide Production Cross Section



Radionuclide Production Cross Section

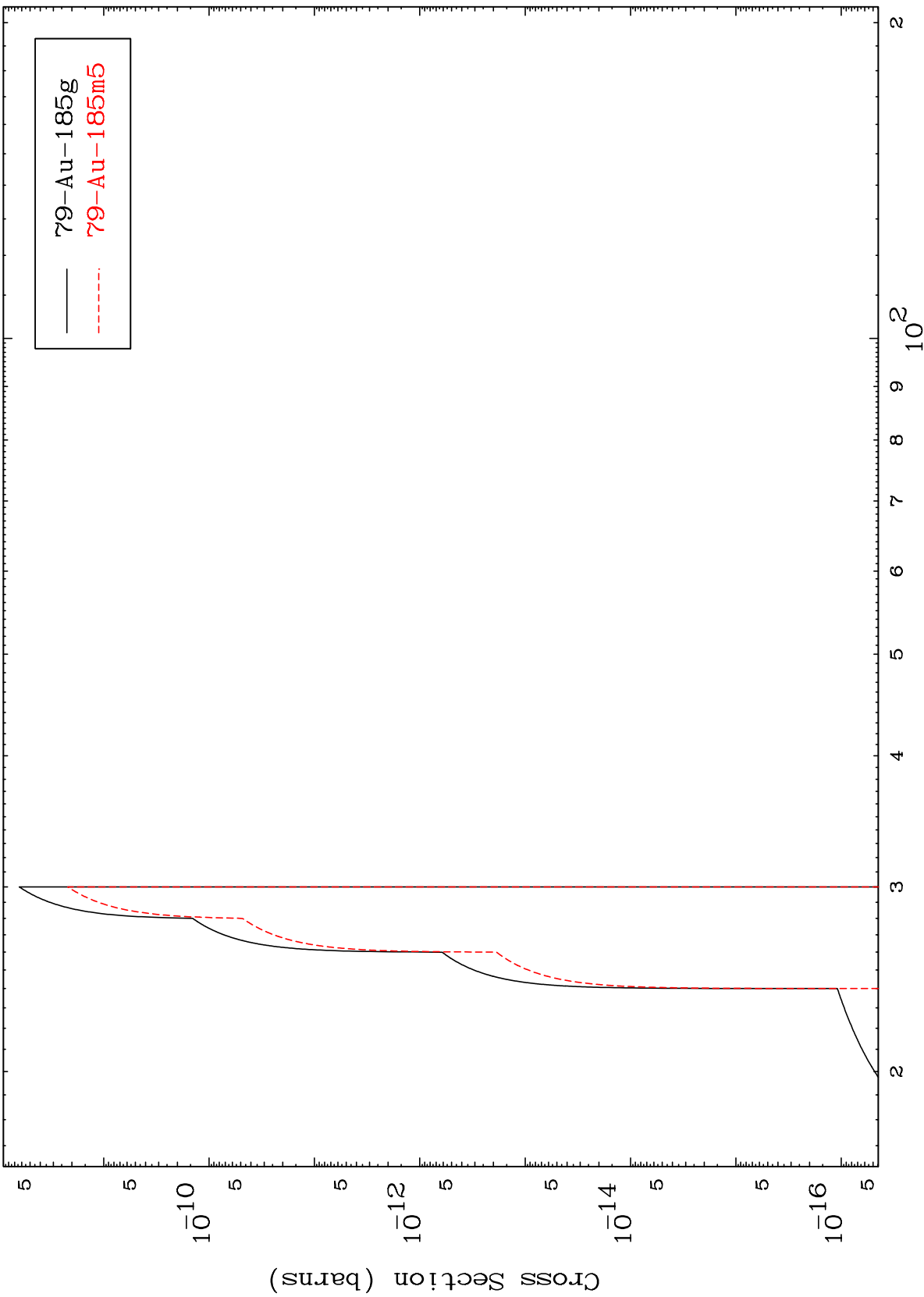


MAT 7886

( $\alpha, p$ ) d

<sup>79</sup>Au-184

Radionuclide Production Cross Section



23

Incident Energy (MeV)

<sup>79</sup>Au-184