

Program EVALPLOT  
(Version 2018-1)

by

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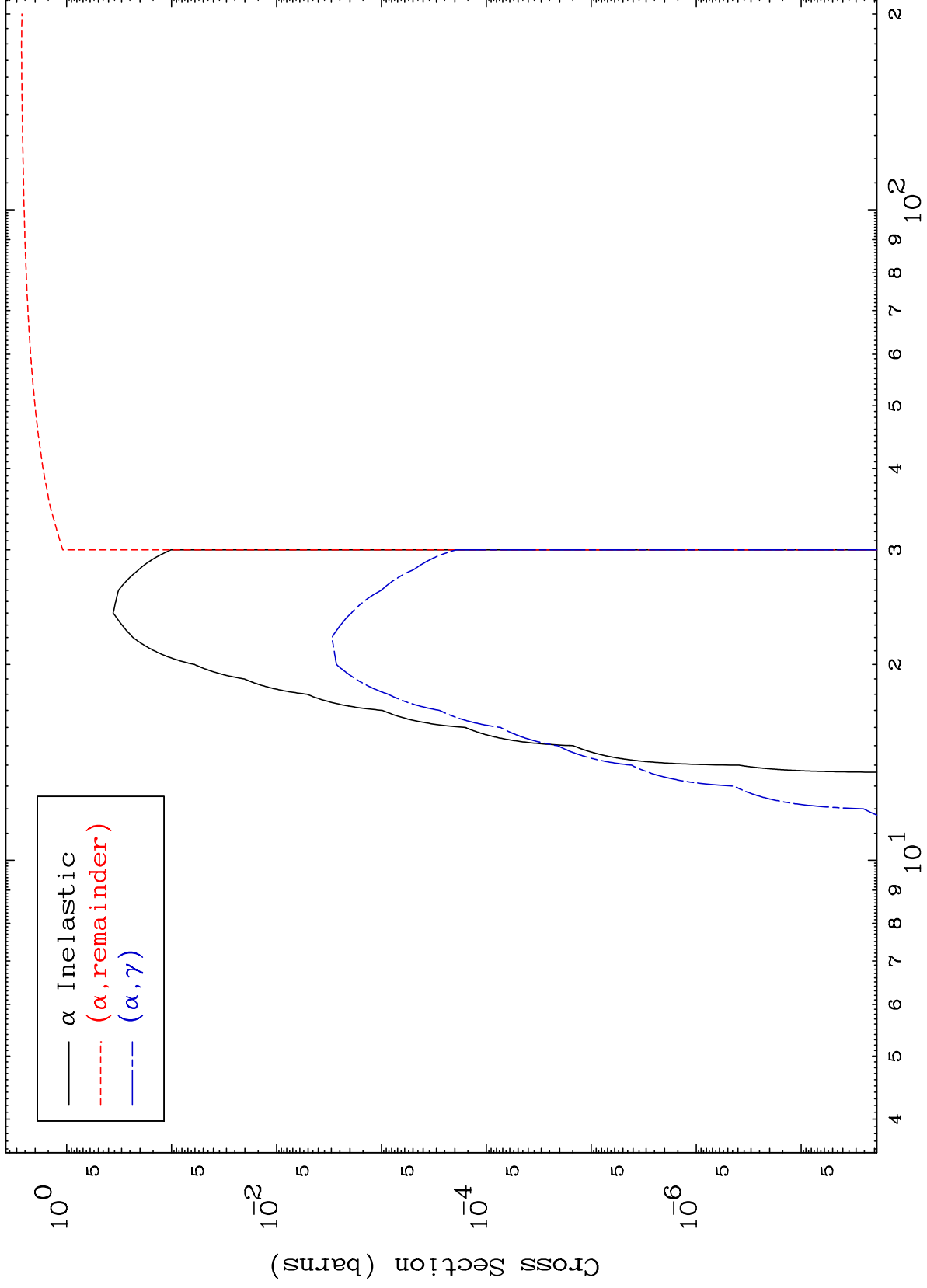
Press Mouse Button to Start

MAT 7901

$\alpha$  Major

79-Au-189

0 Kelvin Cross Sections



1

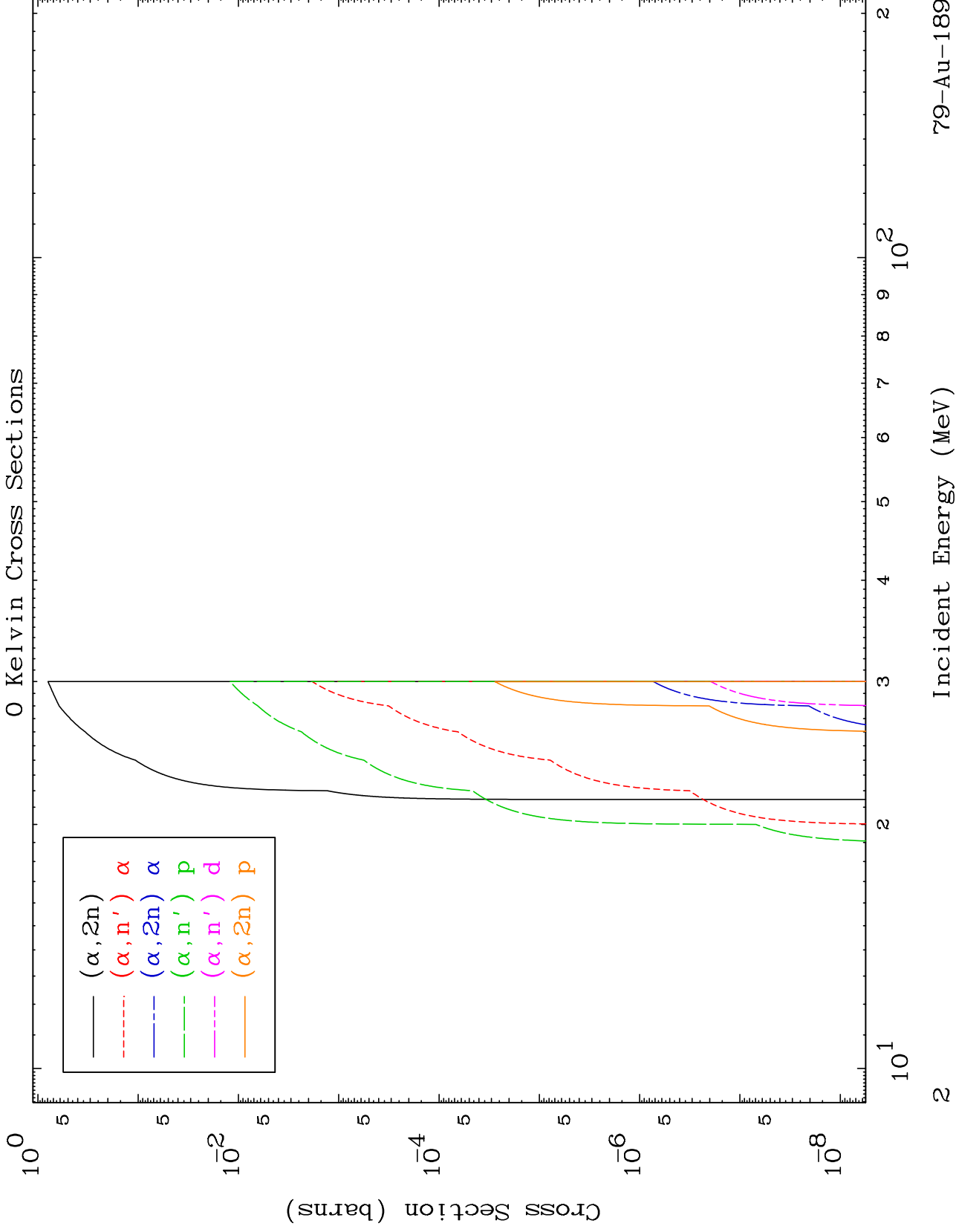
Incident Energy (MeV)

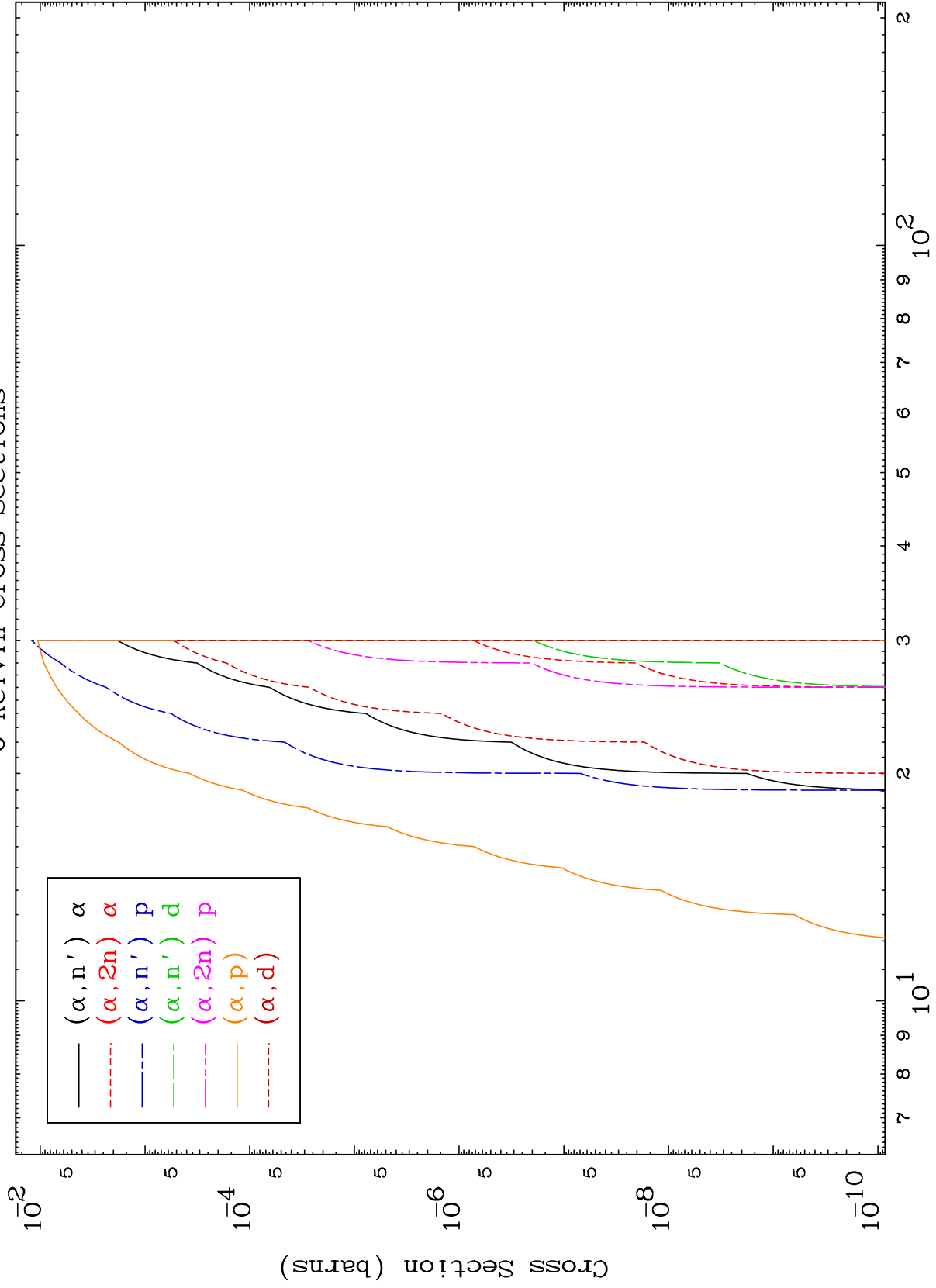
79-Au-189

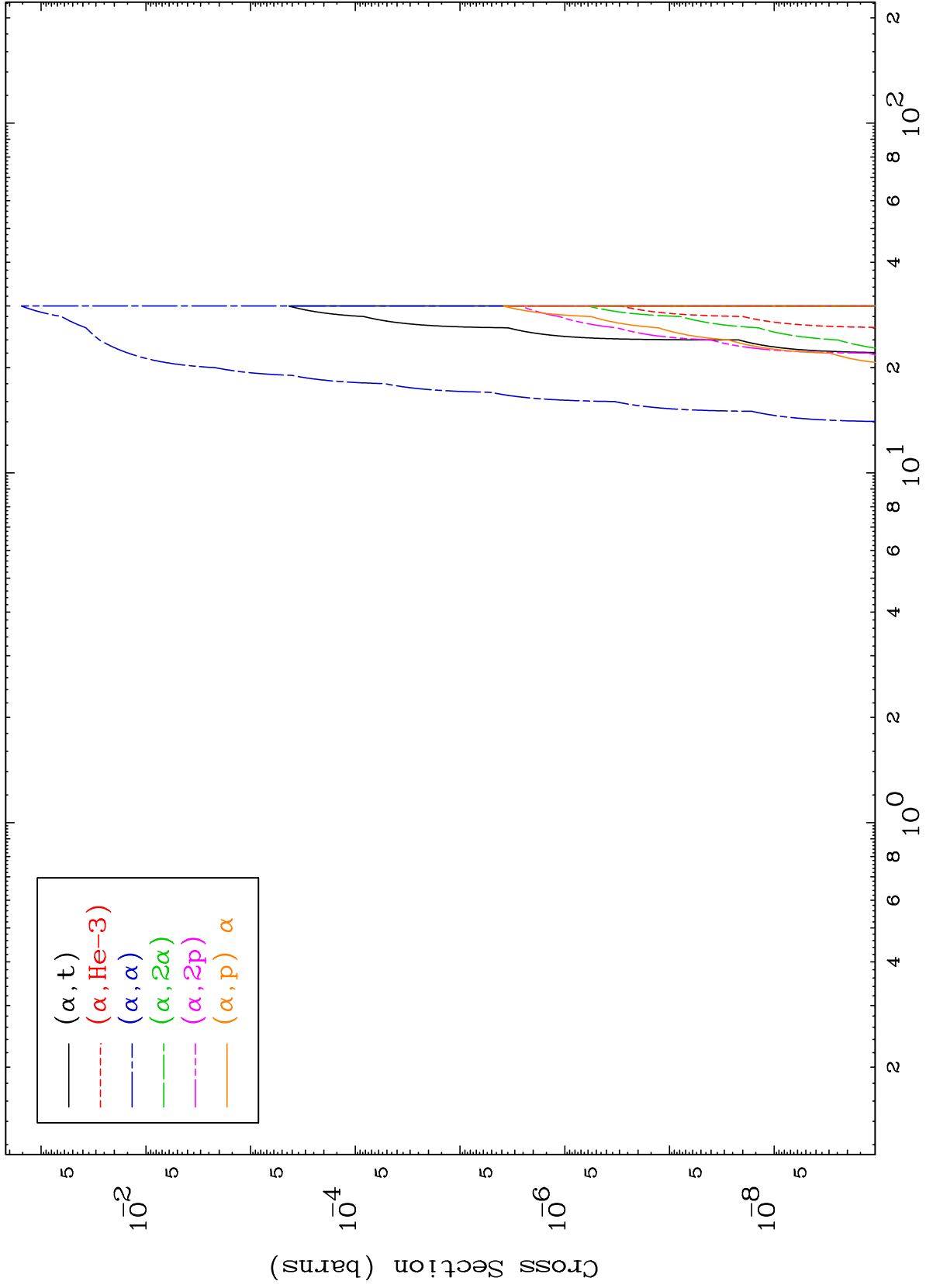
MAT 7901

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

79-Au-189



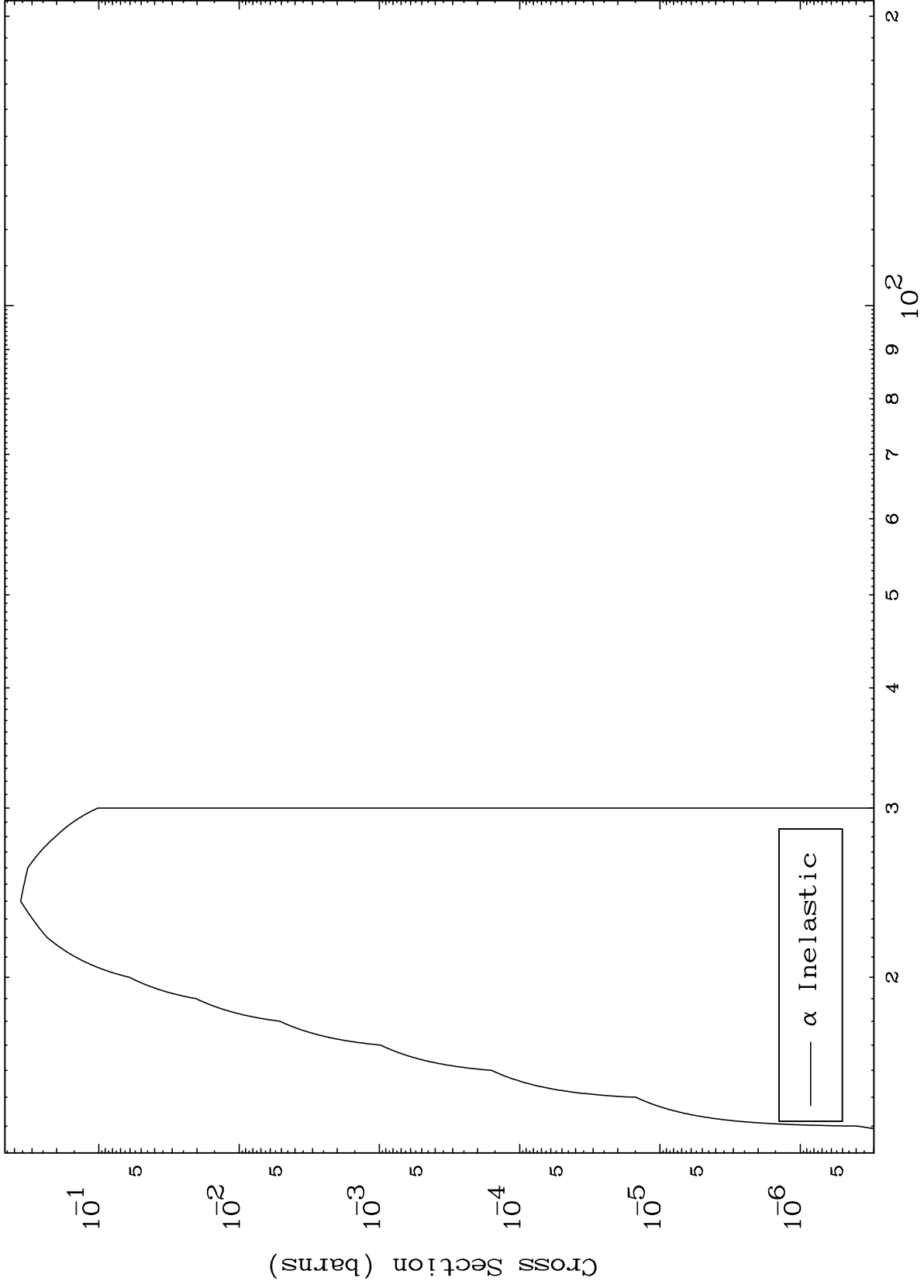




MAT 7901

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

79-Au-189



5

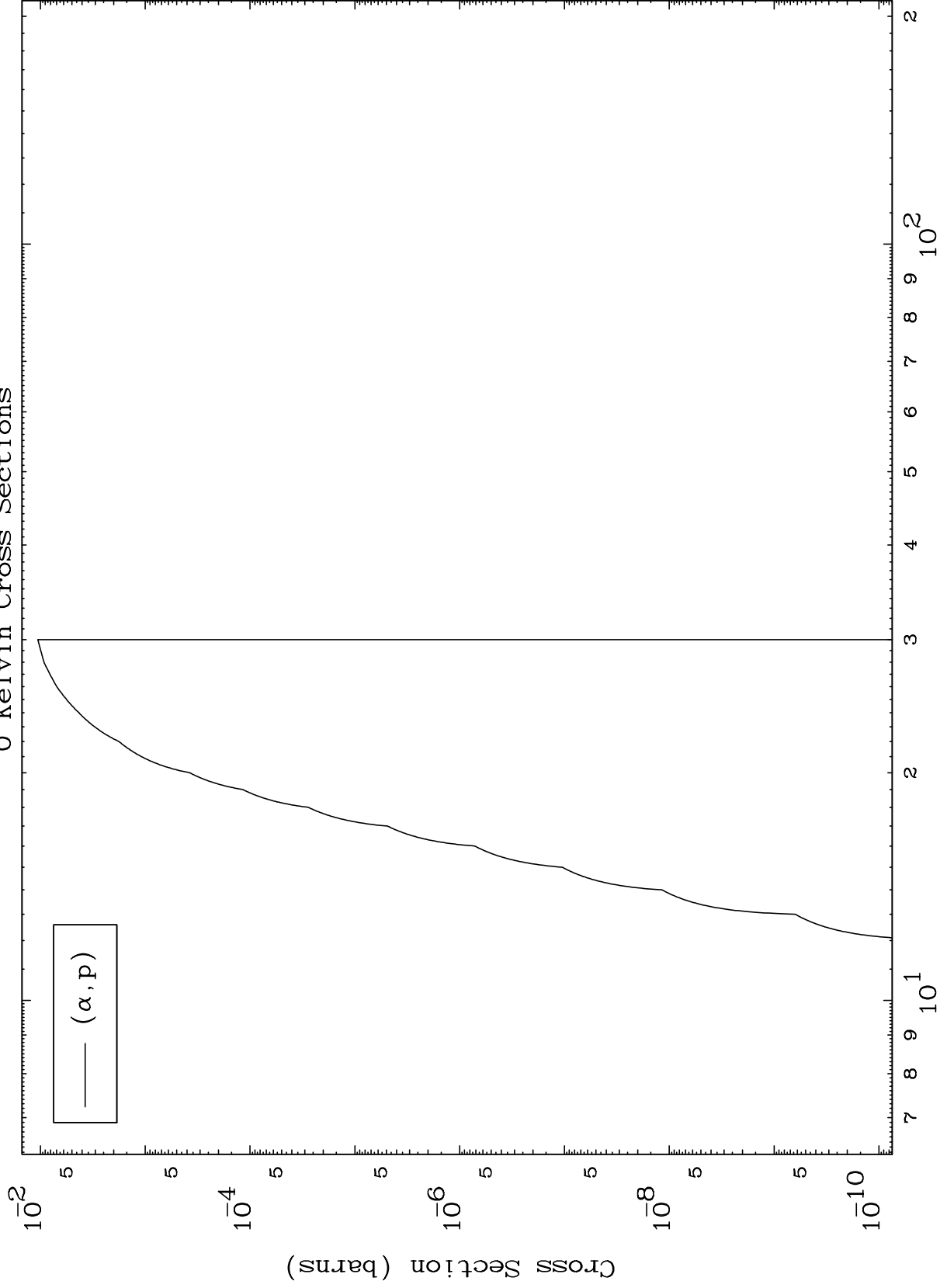
Incident Energy (MeV)

79-Au-189

MAT 7901

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

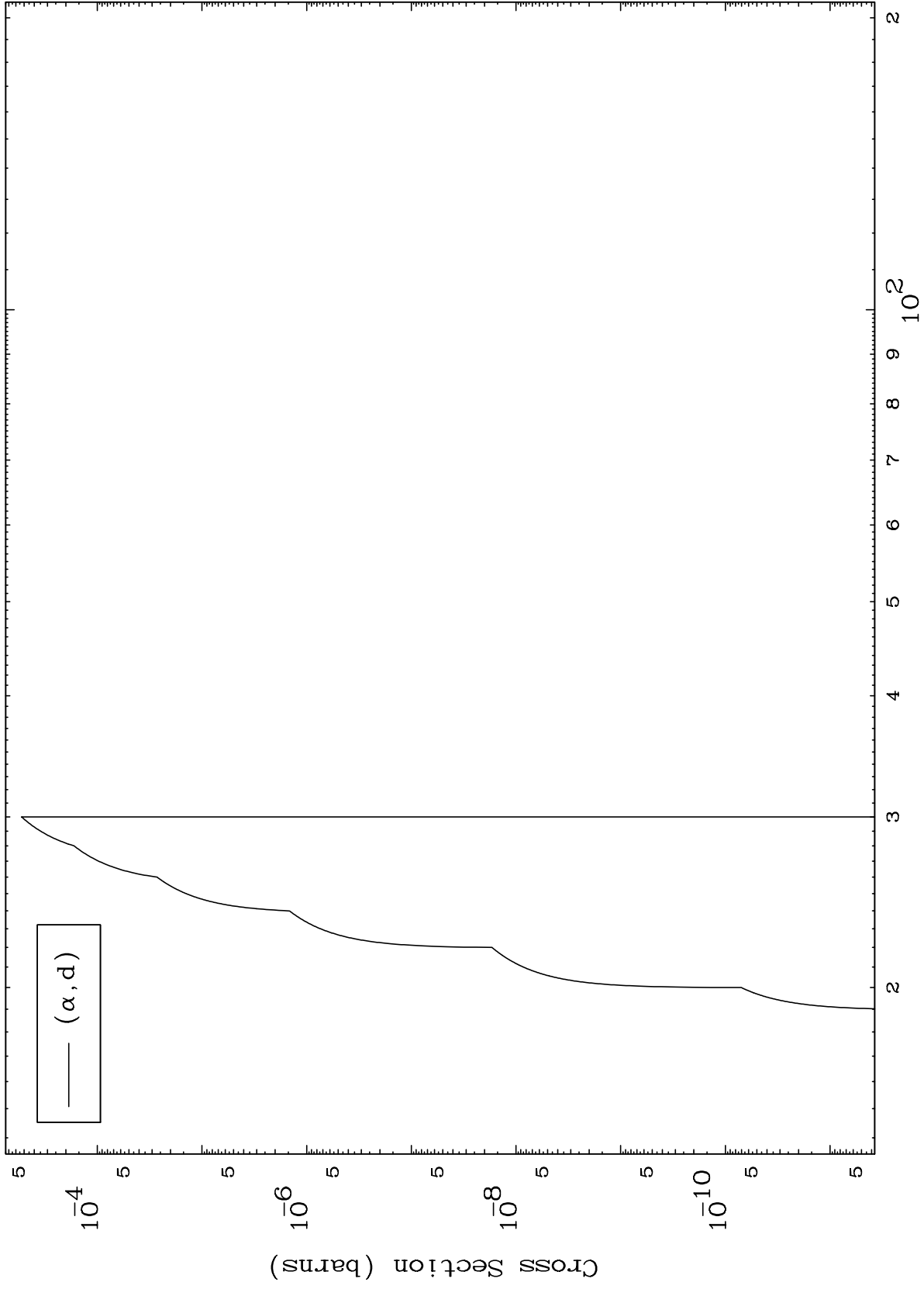
79-Au-189



6

Incident Energy (MeV)

79-Au-189

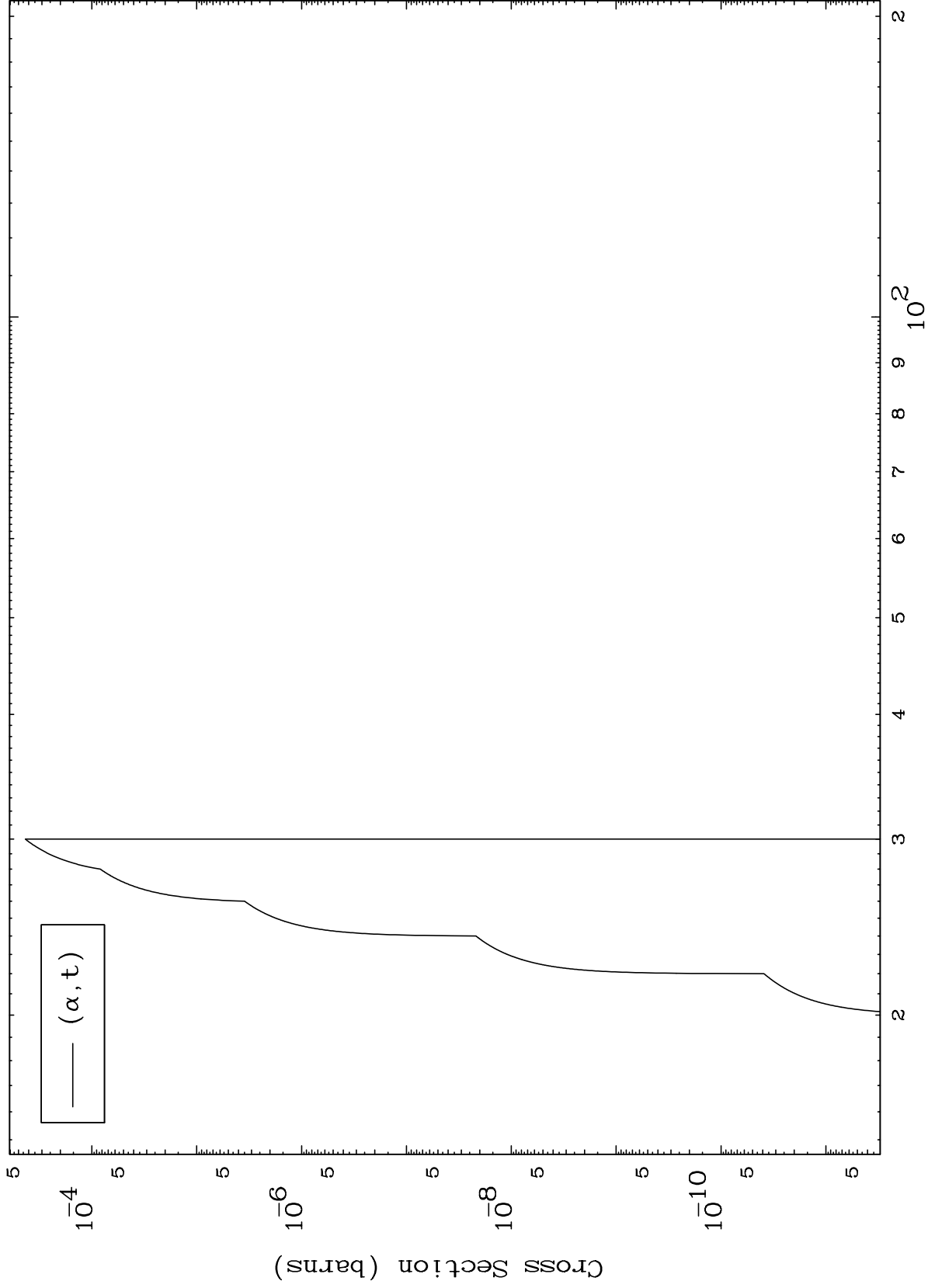




MAT 7901

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

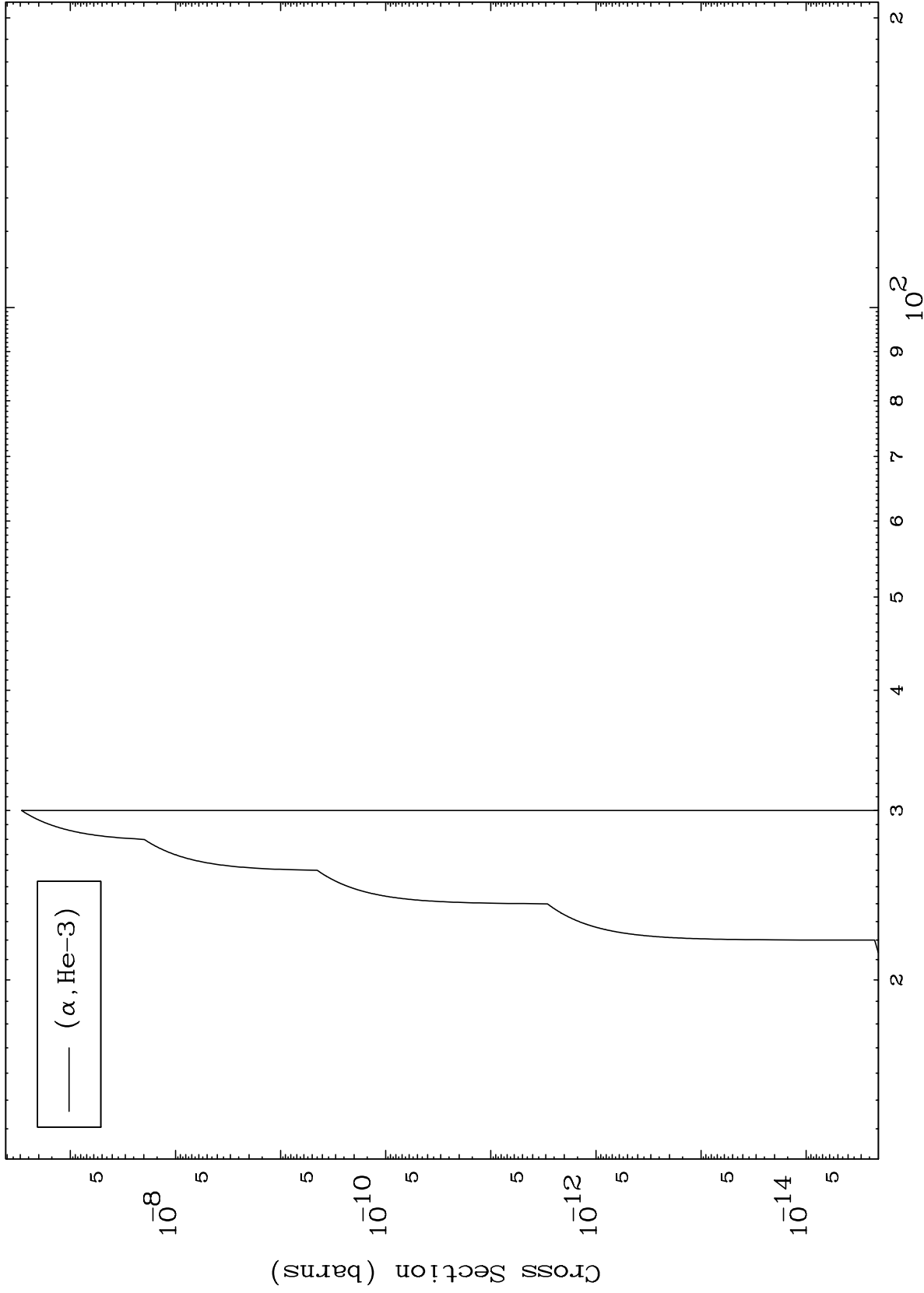
79-Au-189



8

Incident Energy (MeV)

79-Au-189

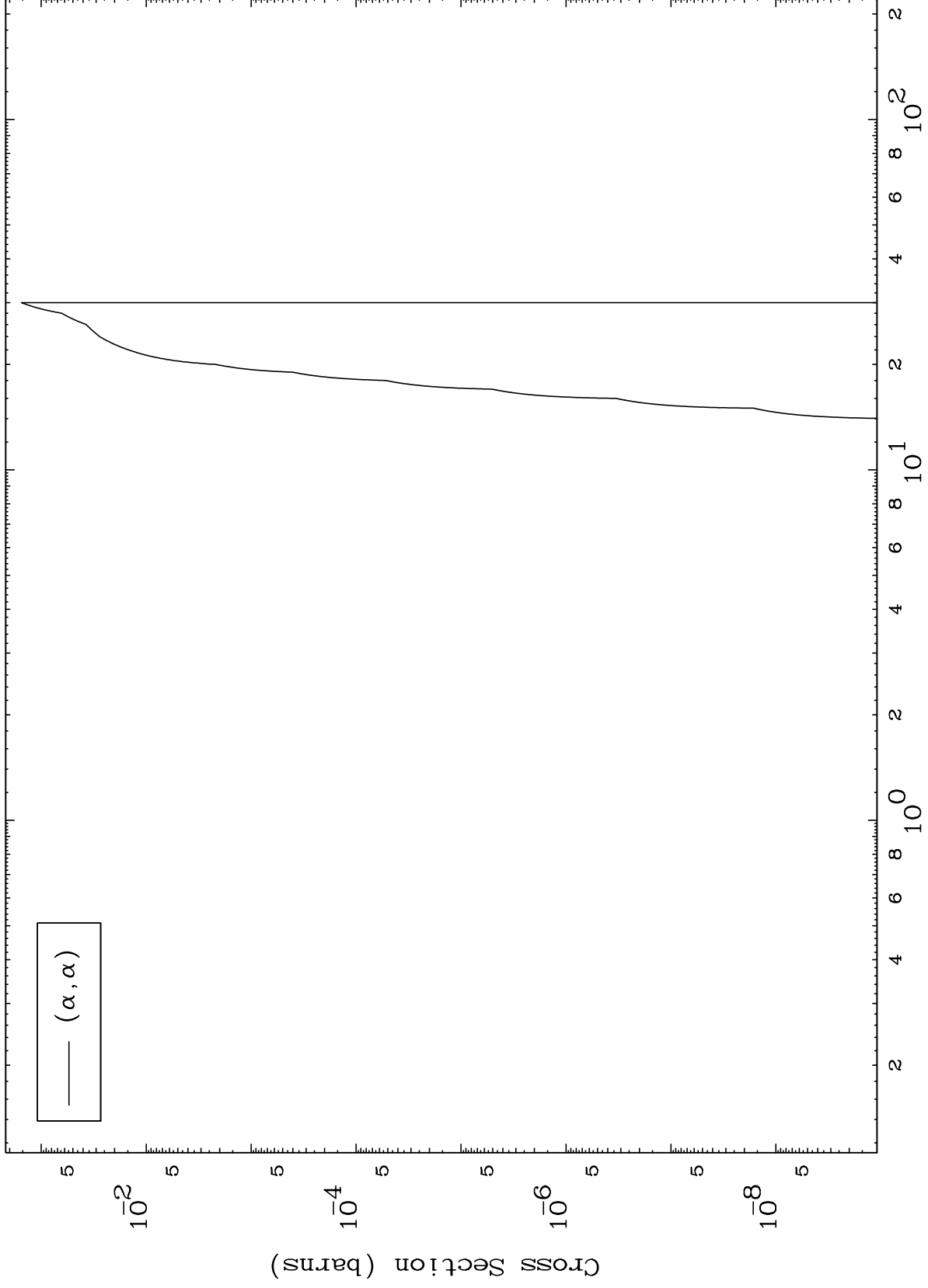


MAT 7901

( $\alpha, \alpha$ ) Levels

79-Au-189

0 Kelvin Cross Sections



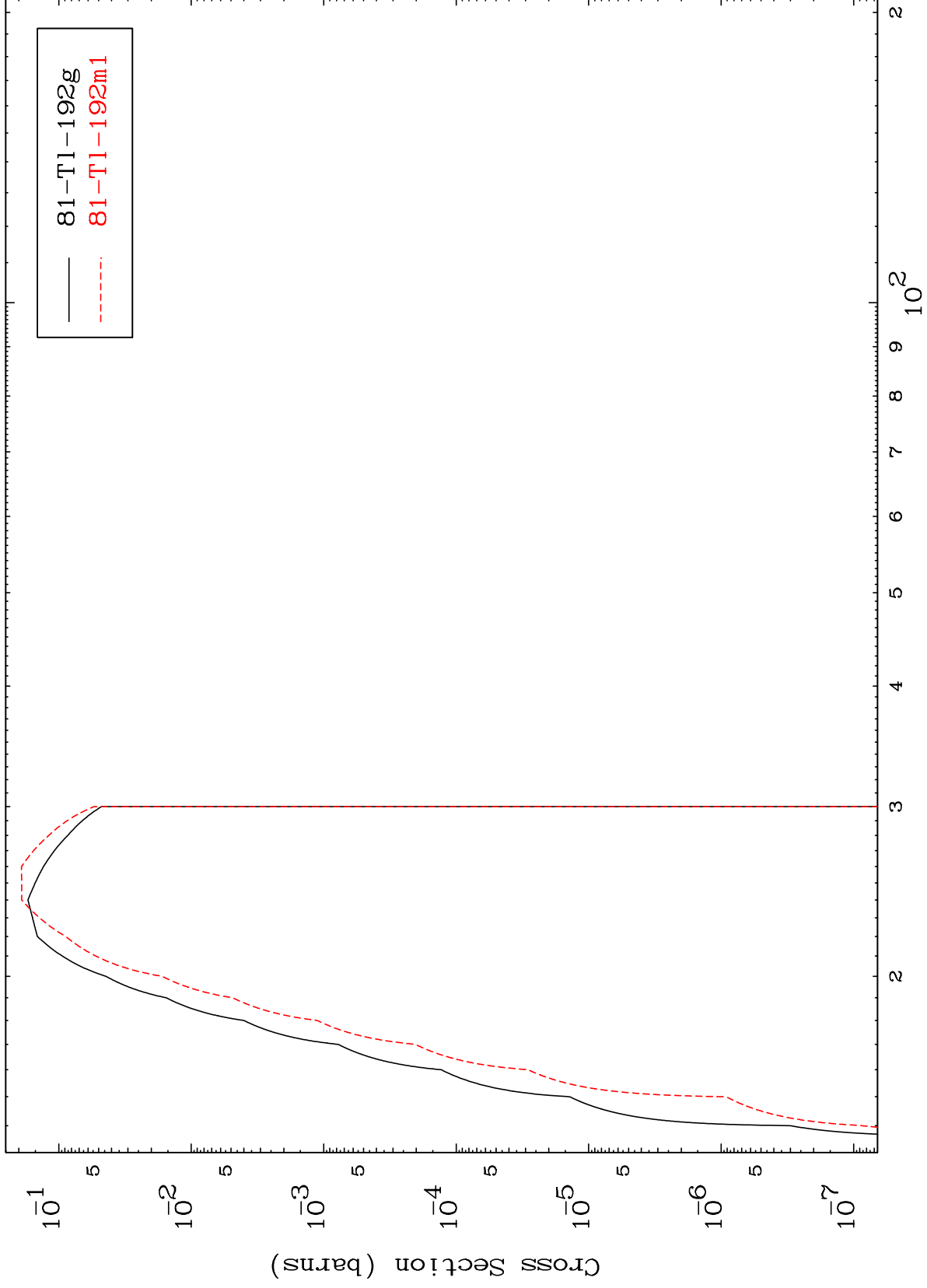
10

Incident Energy (MeV)

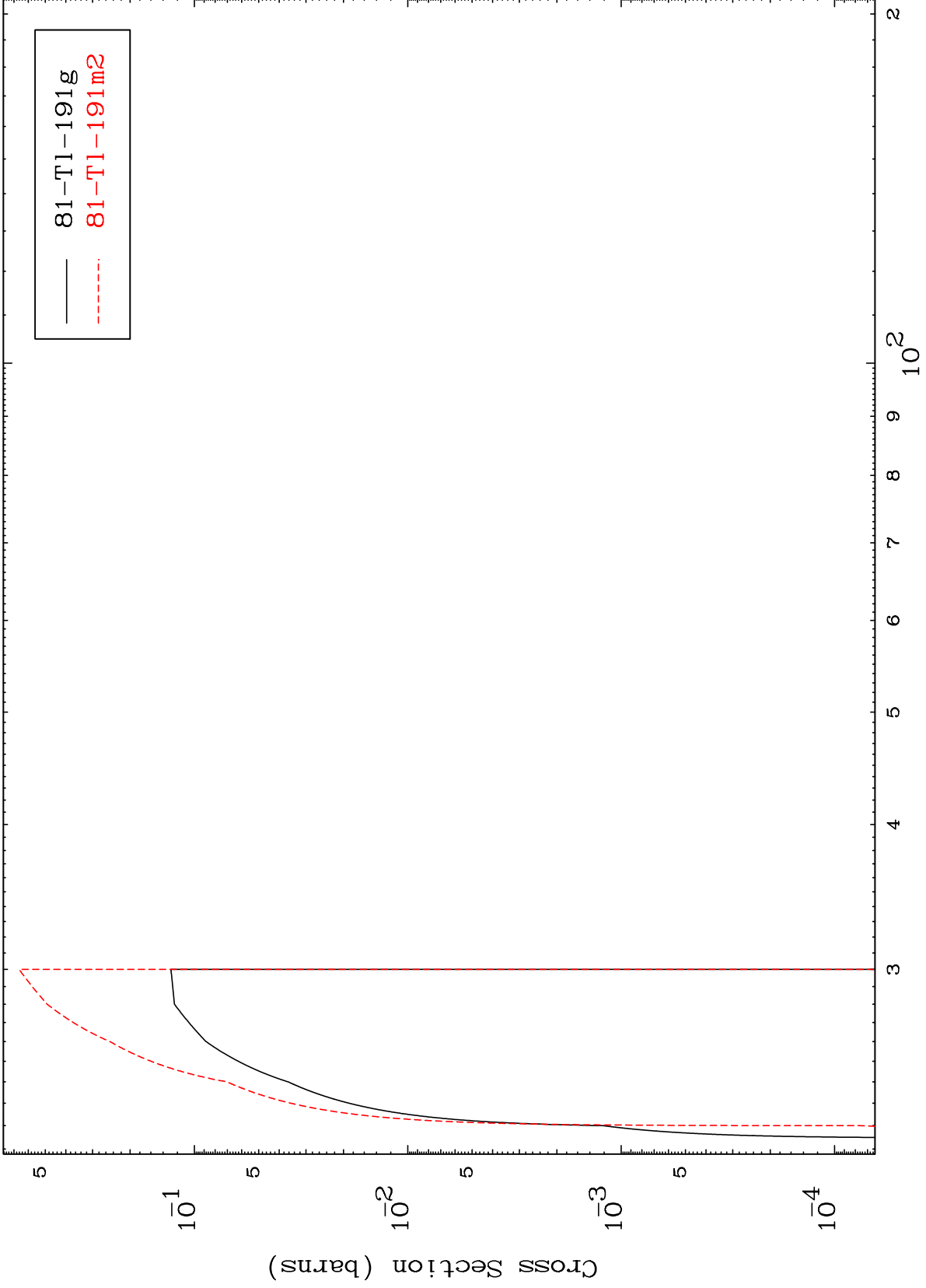
79-Au-189

Radionuclide Production Cross Section

$\alpha$  Inelastic



Radionuclide Production Cross Section

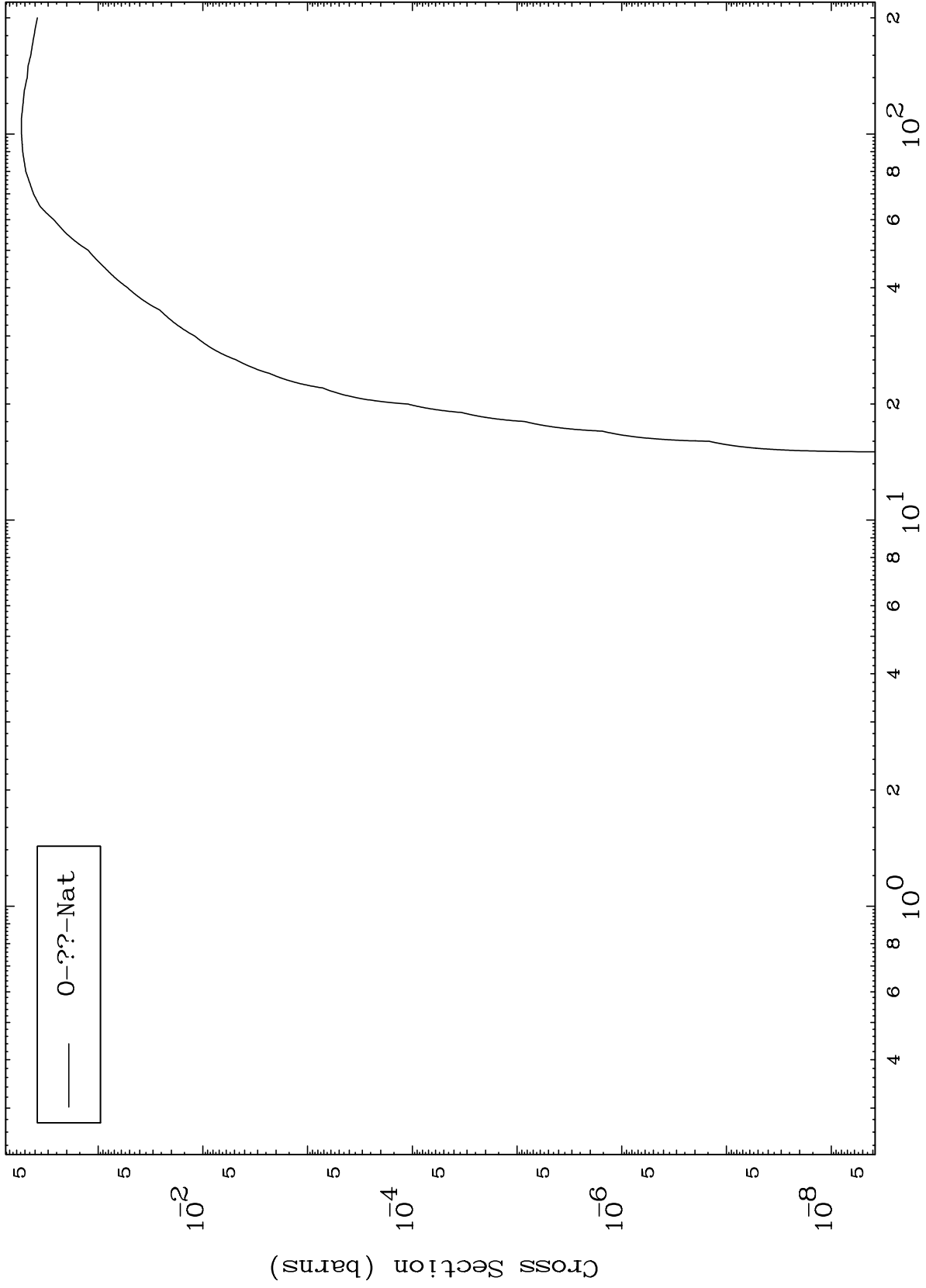


81-Tl-191g  
81-Tl-191m2

MAT 7901

79-Au-189

$\alpha$  Fission  
Radionuclide Production Cross Section



13

79-Au-189

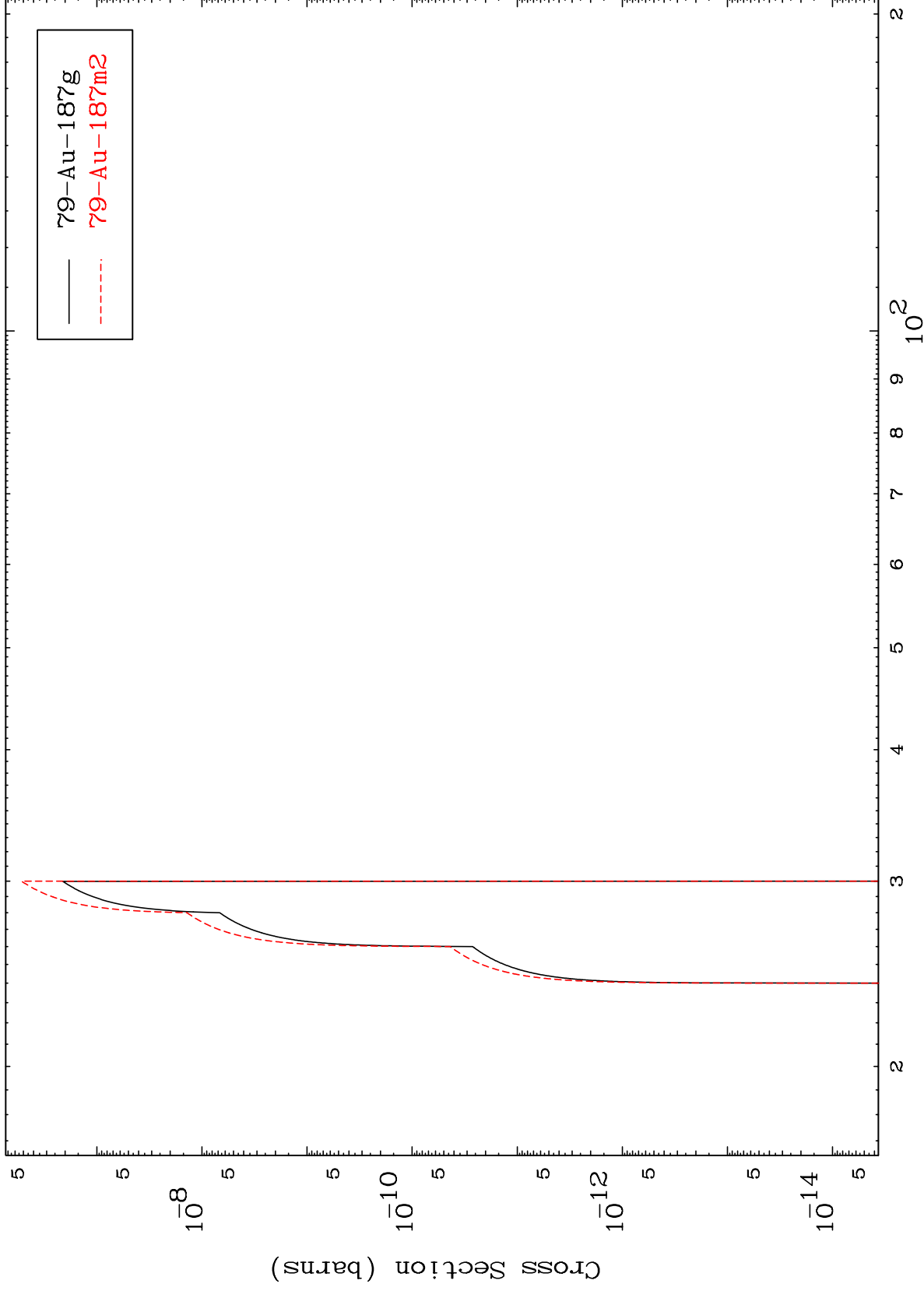
Incident Energy (MeV)

MAT 7901

$(\alpha, 2n) \alpha$

$^{79}\text{Au-189}$

Radionuclide Production Cross Section



14

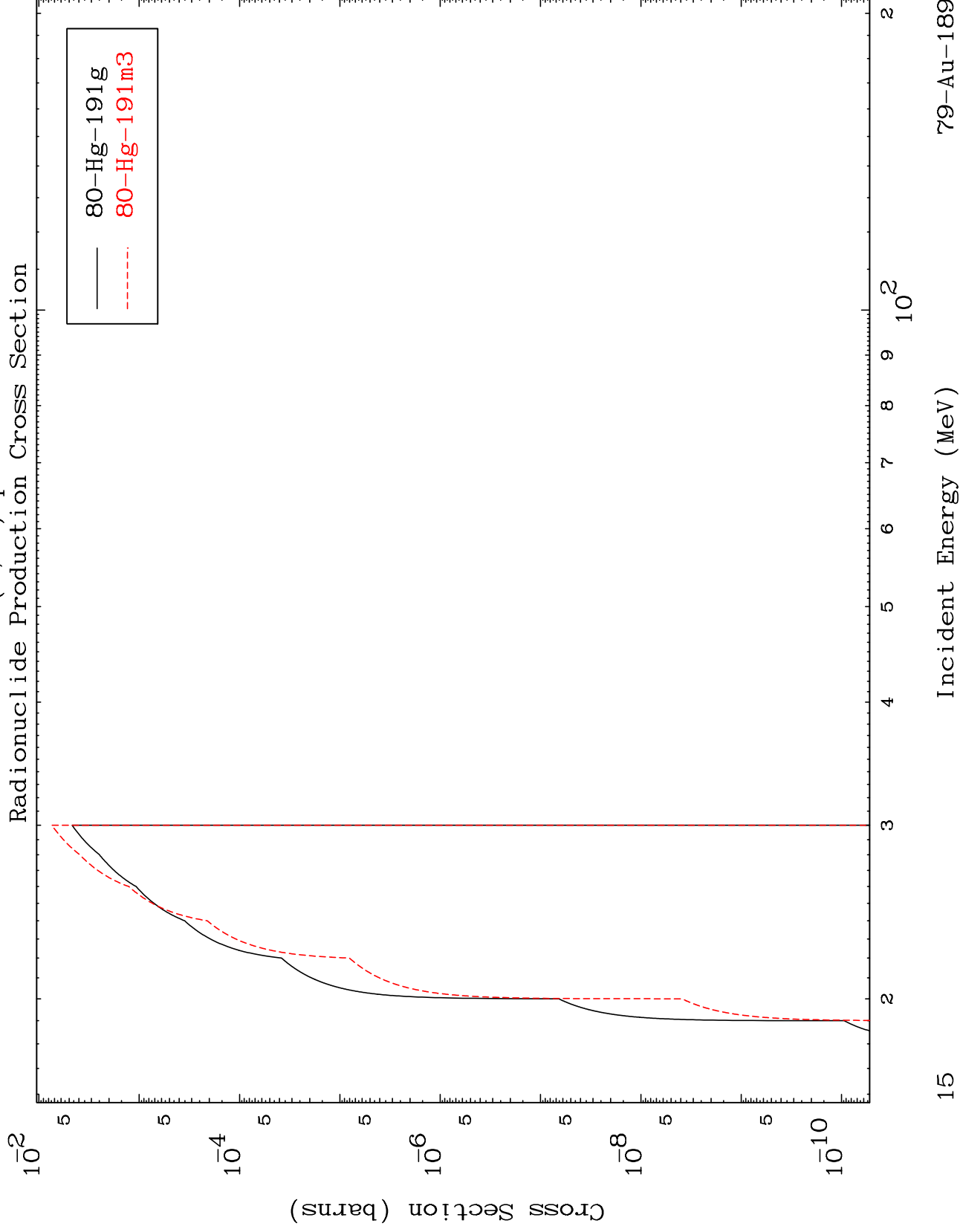
Incident Energy (MeV)

$^{79}\text{Au-189}$

MAT 7901

$(\alpha, n')$  p

79-Au-189



15

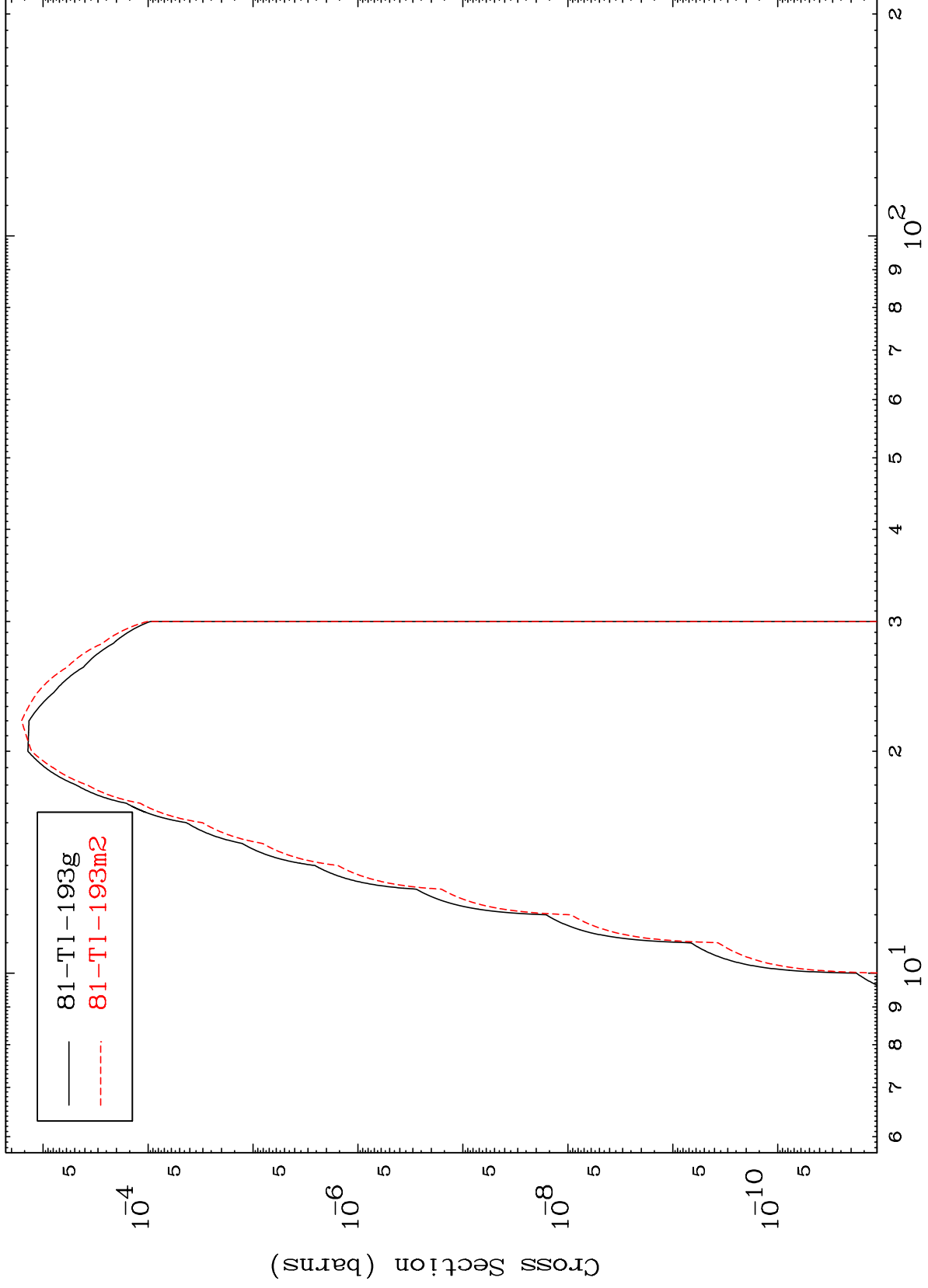
79-Au-189



MAT 7901

79-Au-189

$(\alpha, \gamma)$   
Radionuclide Production Cross Section

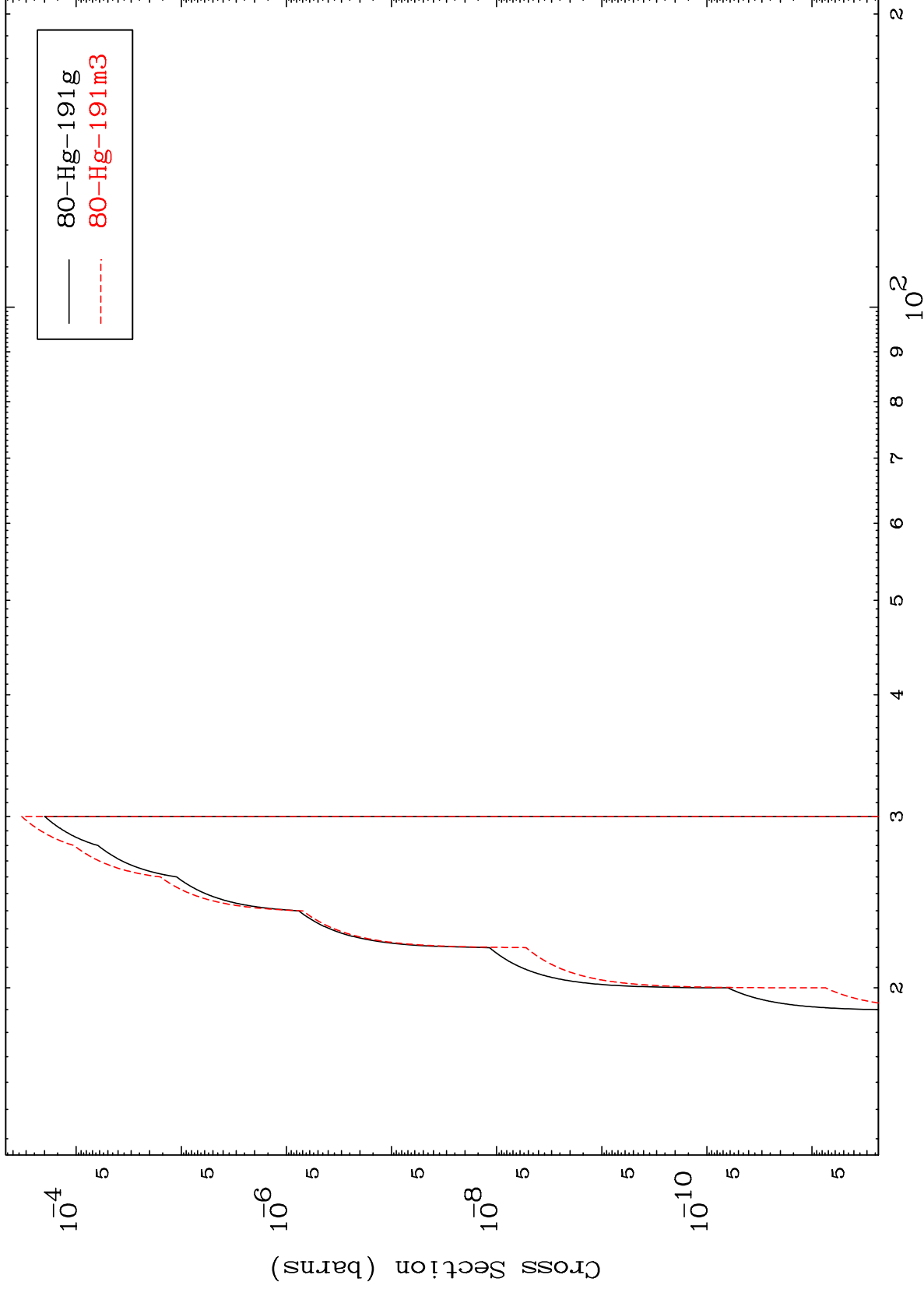


16

Incident Energy (MeV)

79-Au-189

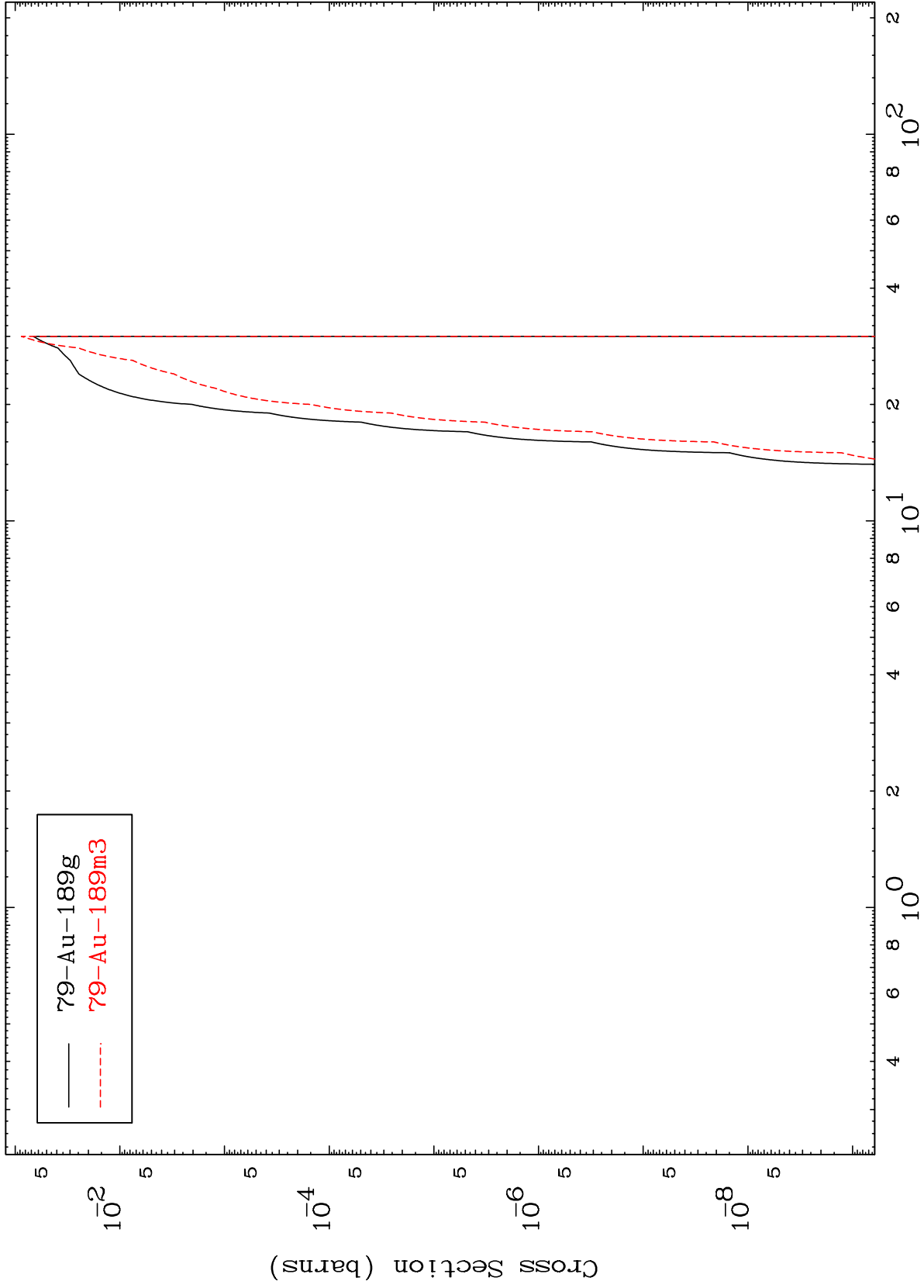
Radionuclide Production Cross Section



MAT 7901

$^{79}\text{Au-189}$

$(\alpha, \alpha)$   
Radionuclide Production Cross Section



18

$^{79}\text{Au-189}$

Incident Energy (MeV)

( $\alpha, 2p$ )  
Radionuclide Production Cross Section

