

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

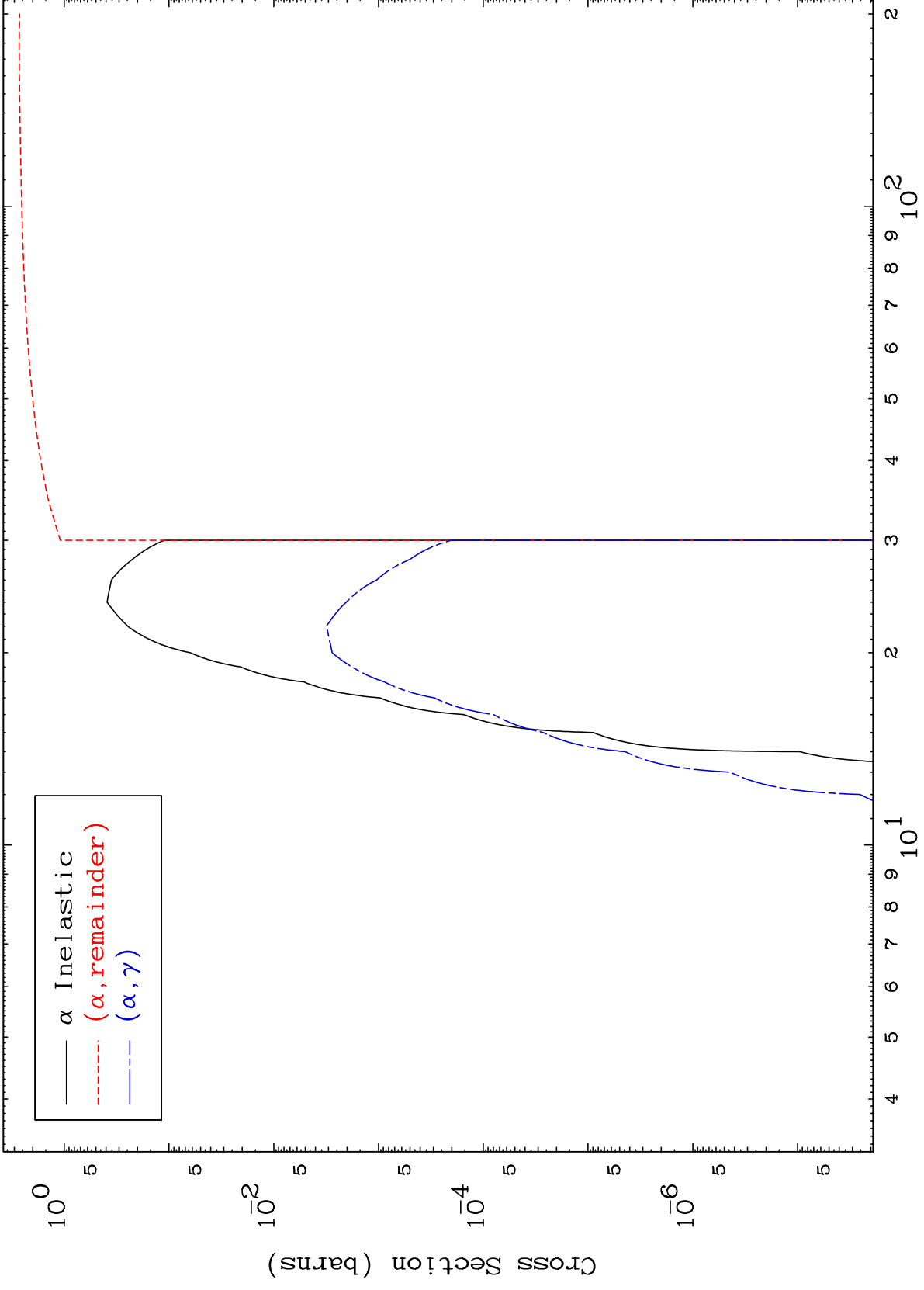
Press Mouse Button to Start

MAT 7902

α Major

79-Au-189

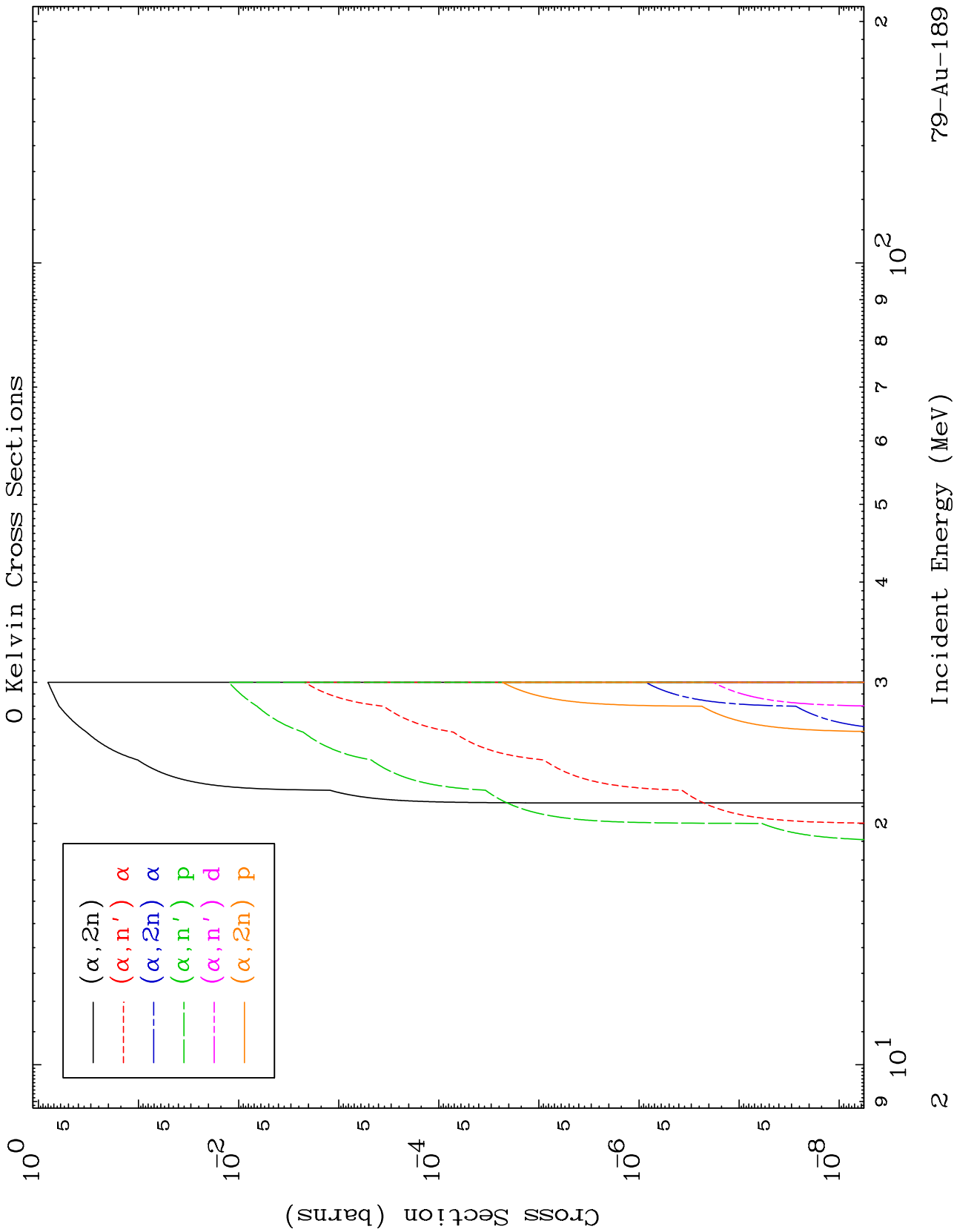
0 Kelvin Cross Sections



MAT 7902

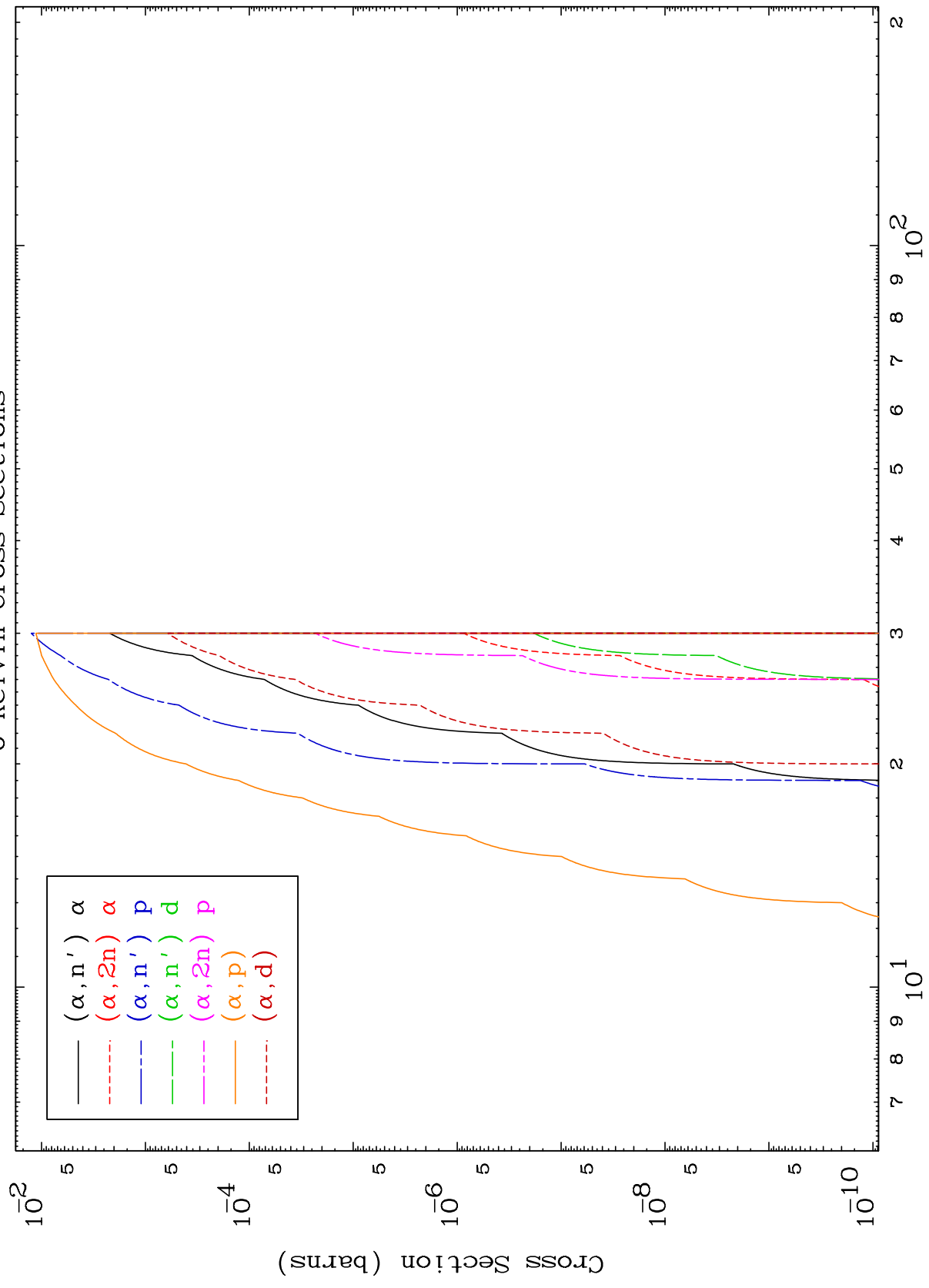
α Neutron Production
0 Kelvin Cross Sections

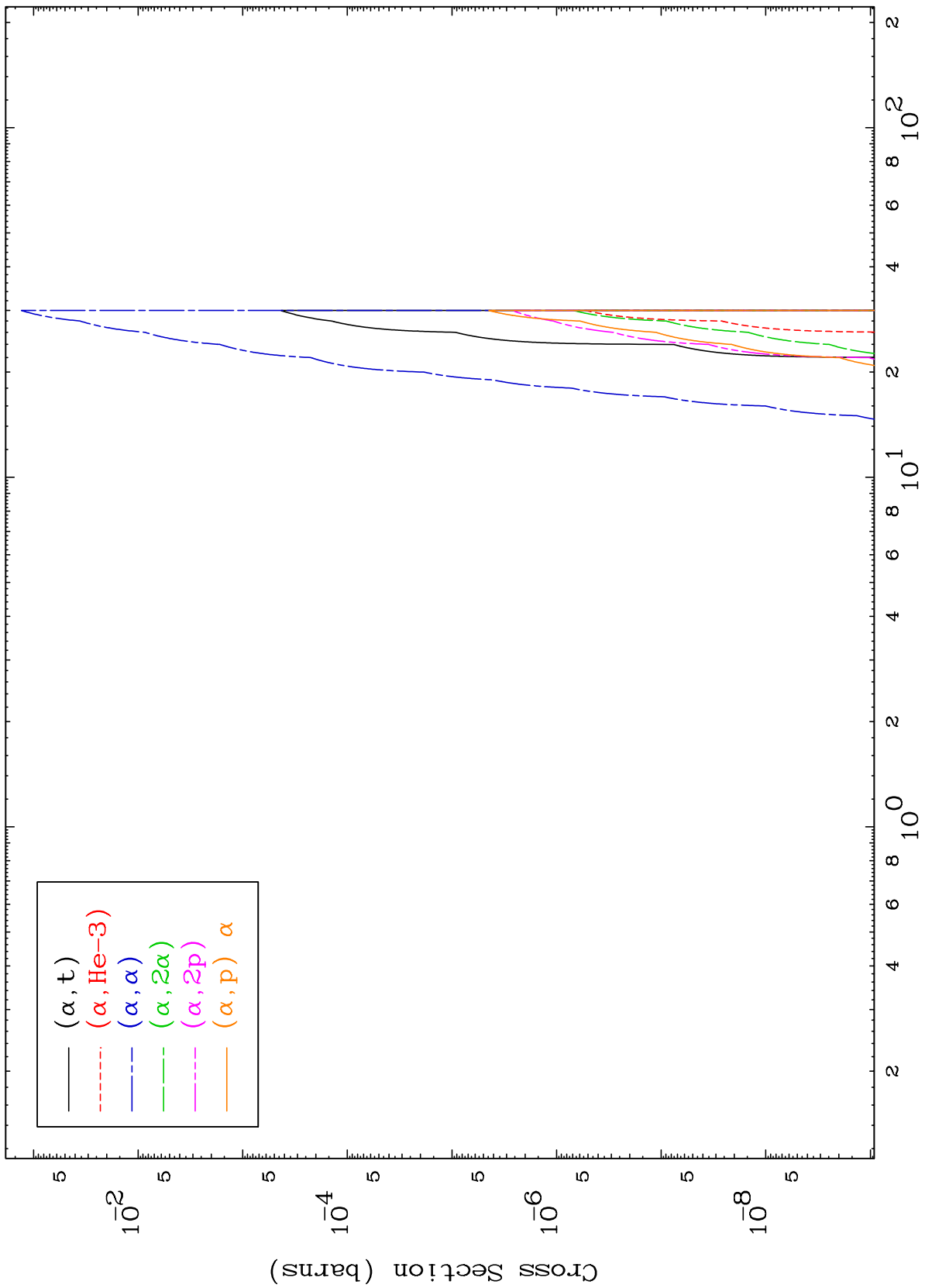
79-Au-189



79-Au-189

Incident Energy (MeV)



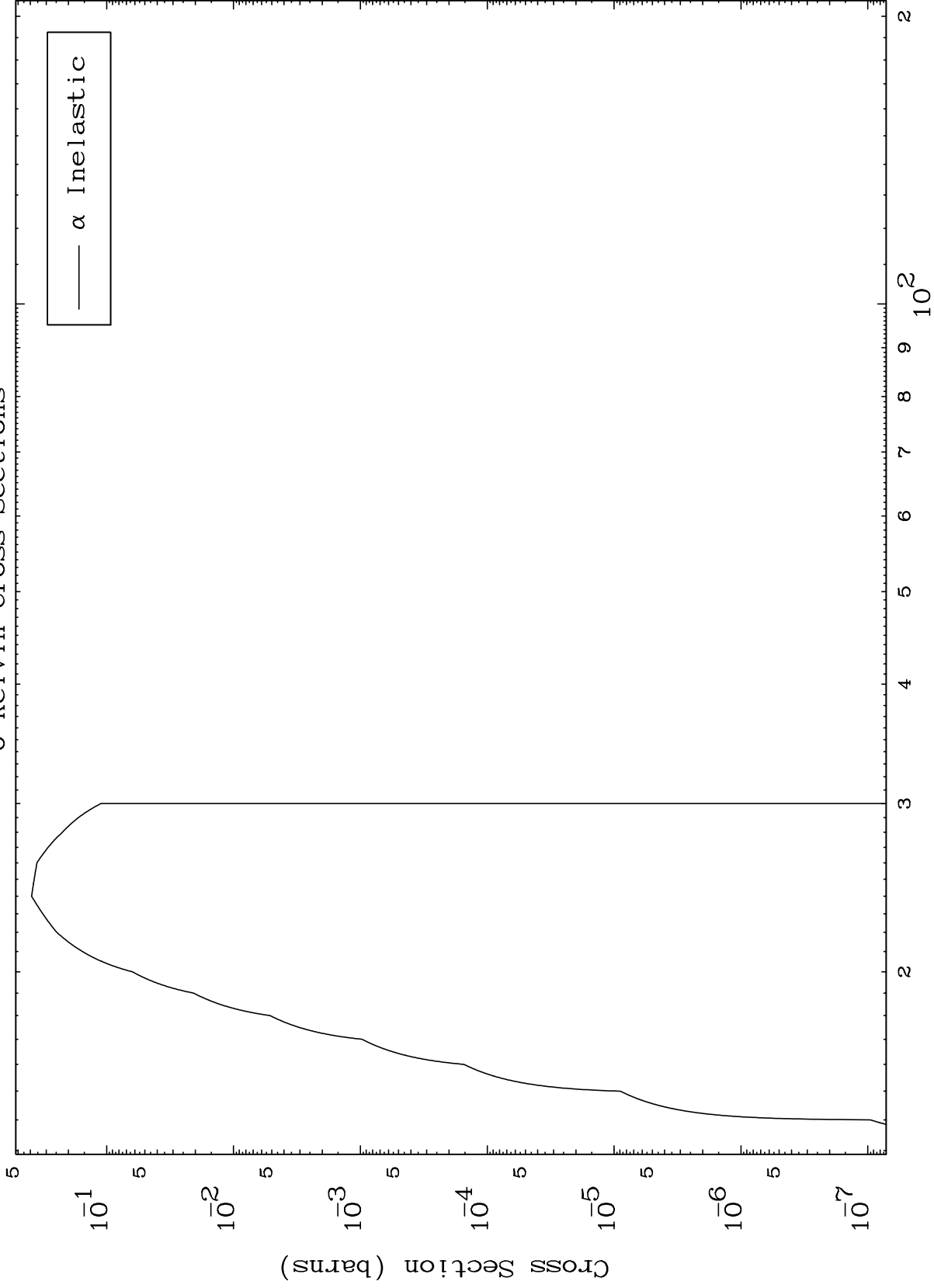


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(α, n') Level

79-Au-189

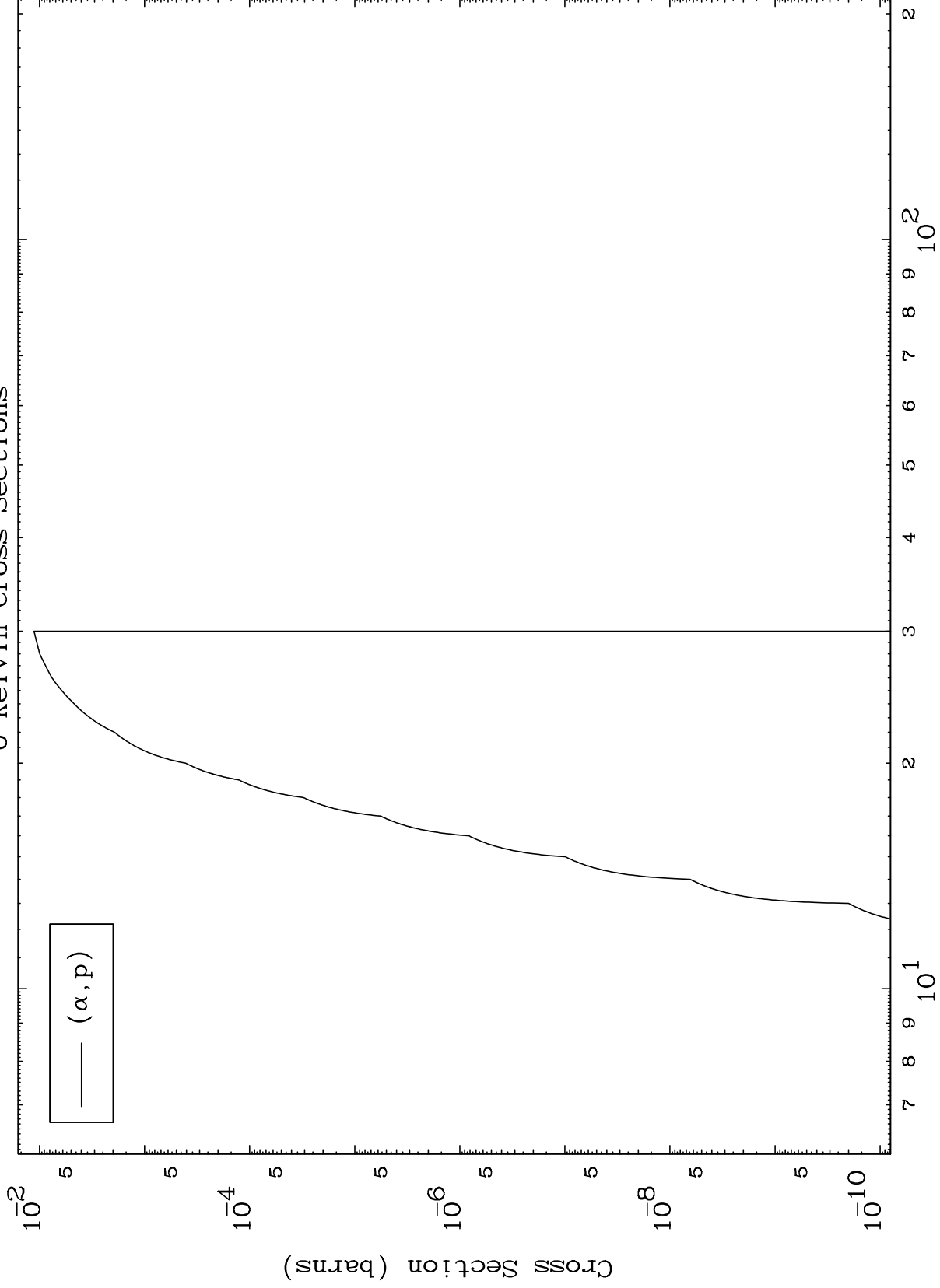
0 Kelvin Cross Sections

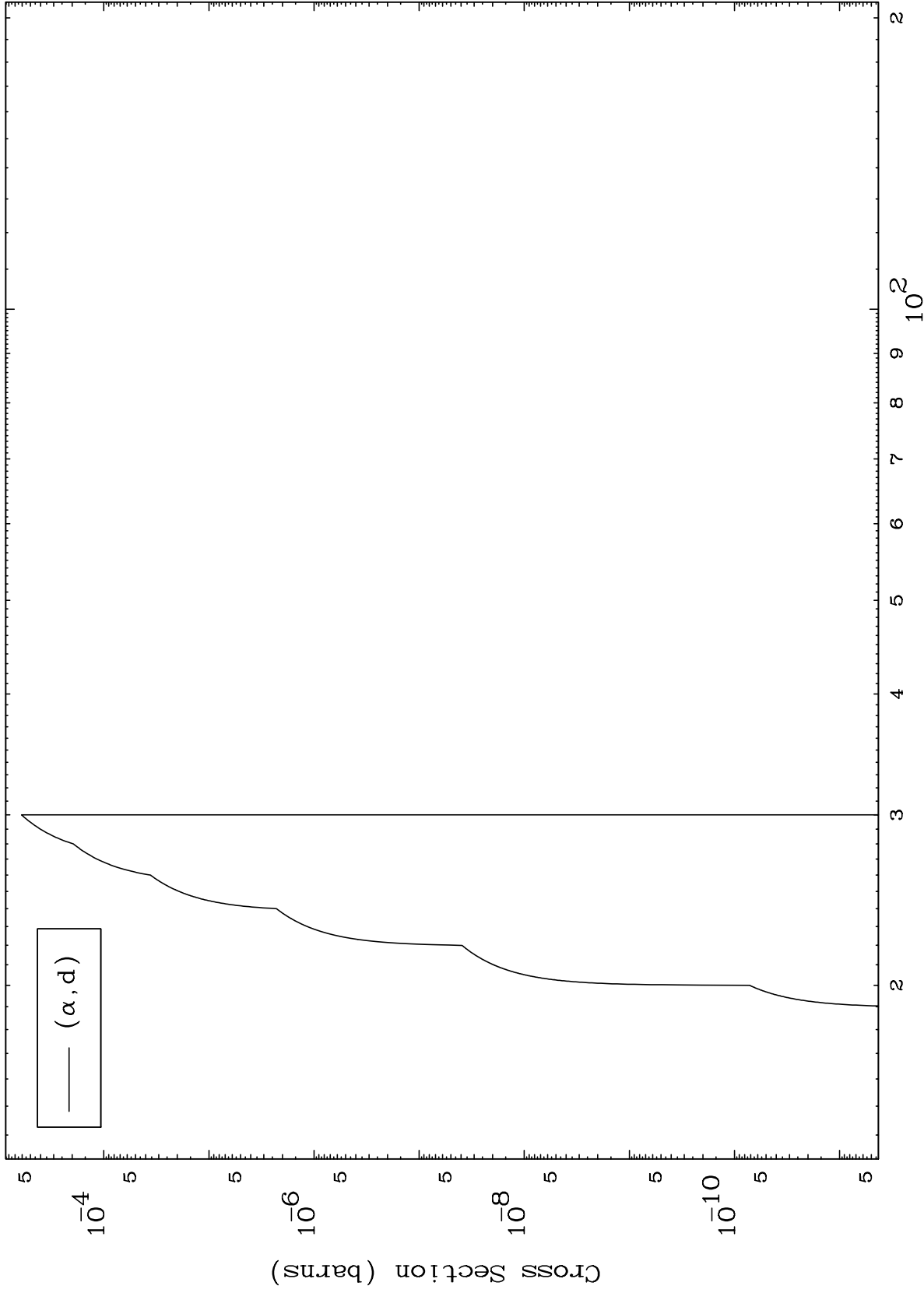


Incident Energy (MeV)

79-Au-189

5

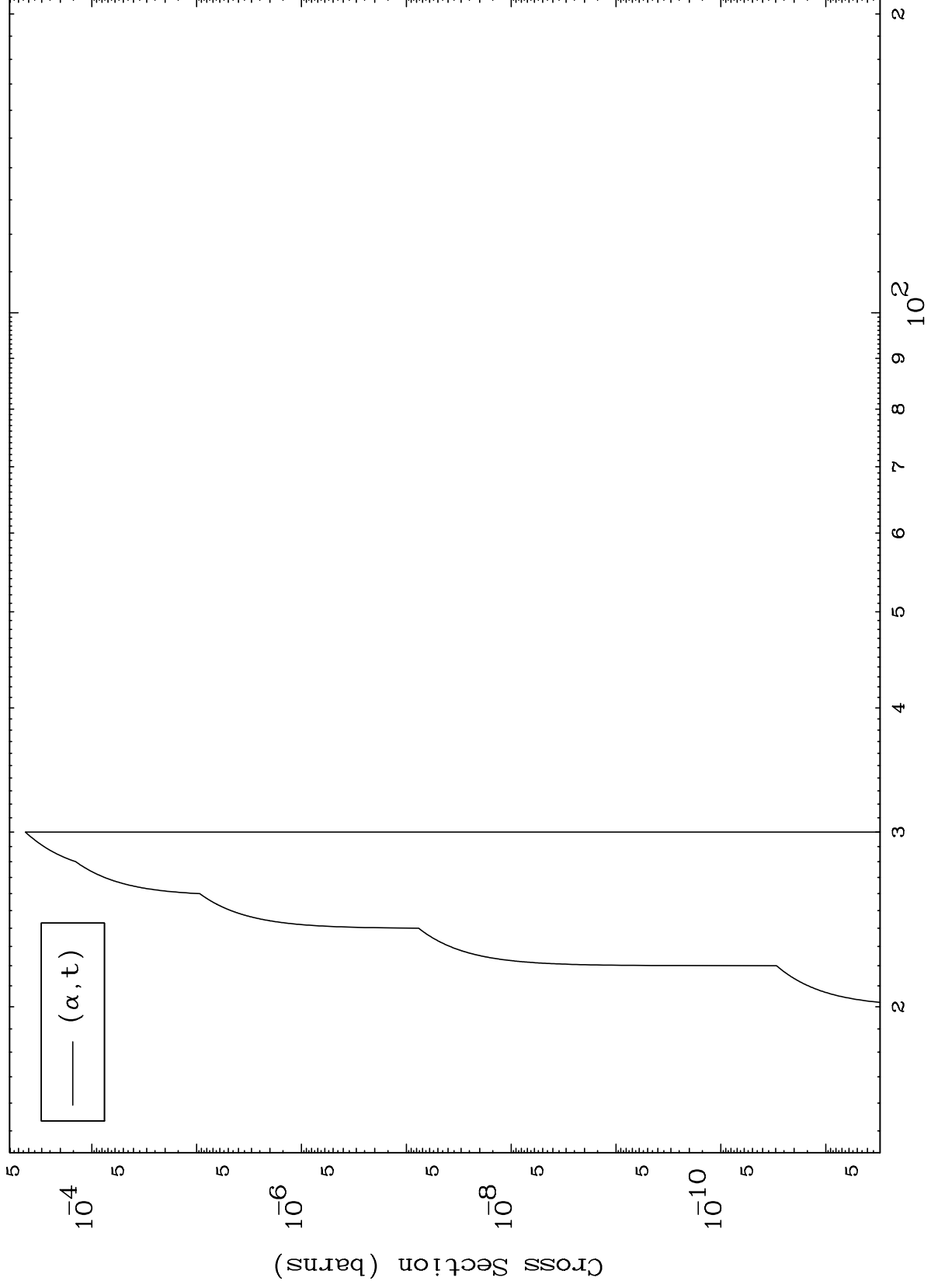


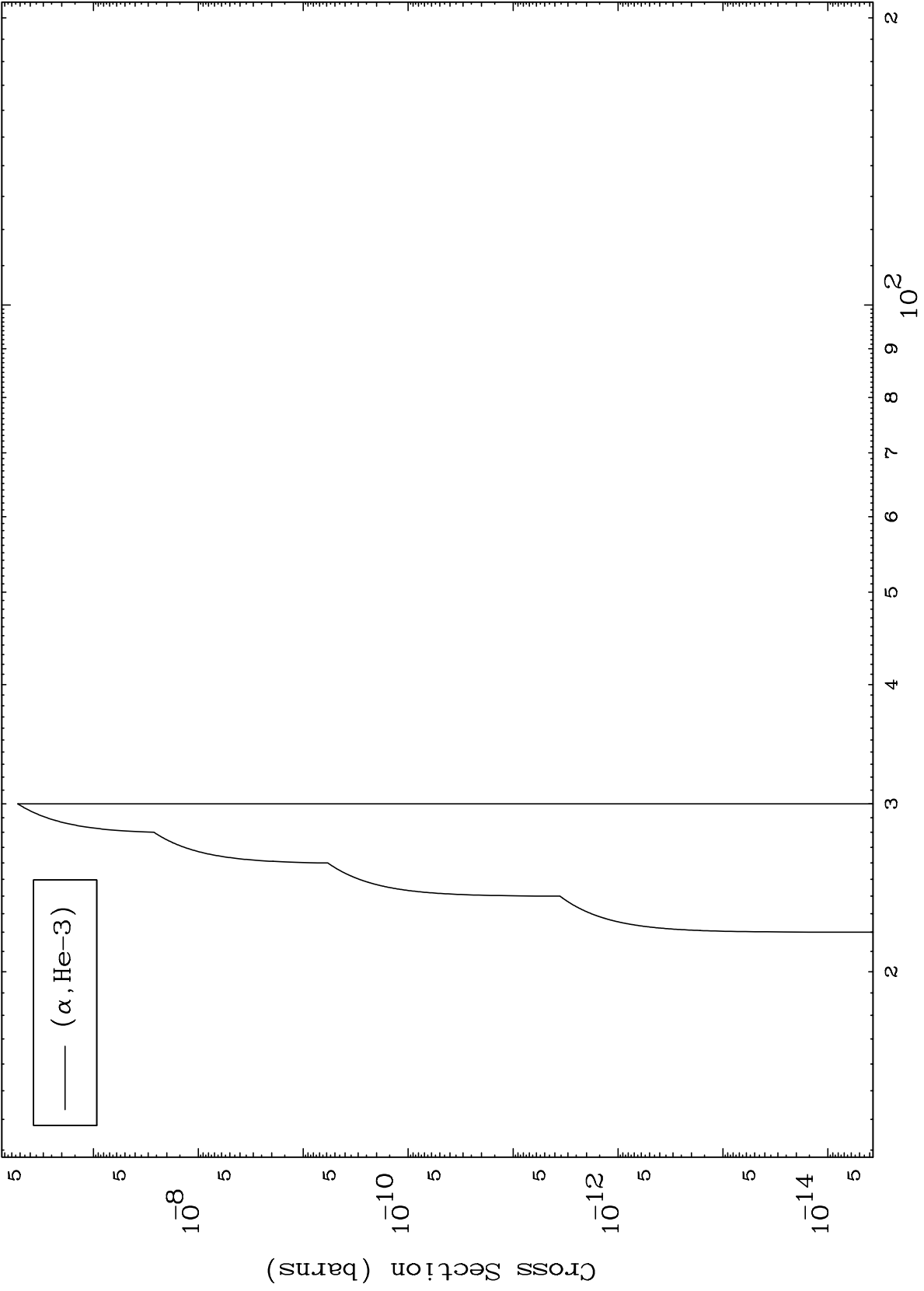


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(α, t) Levels
0 Kelvin Cross Sections

79-Au-189



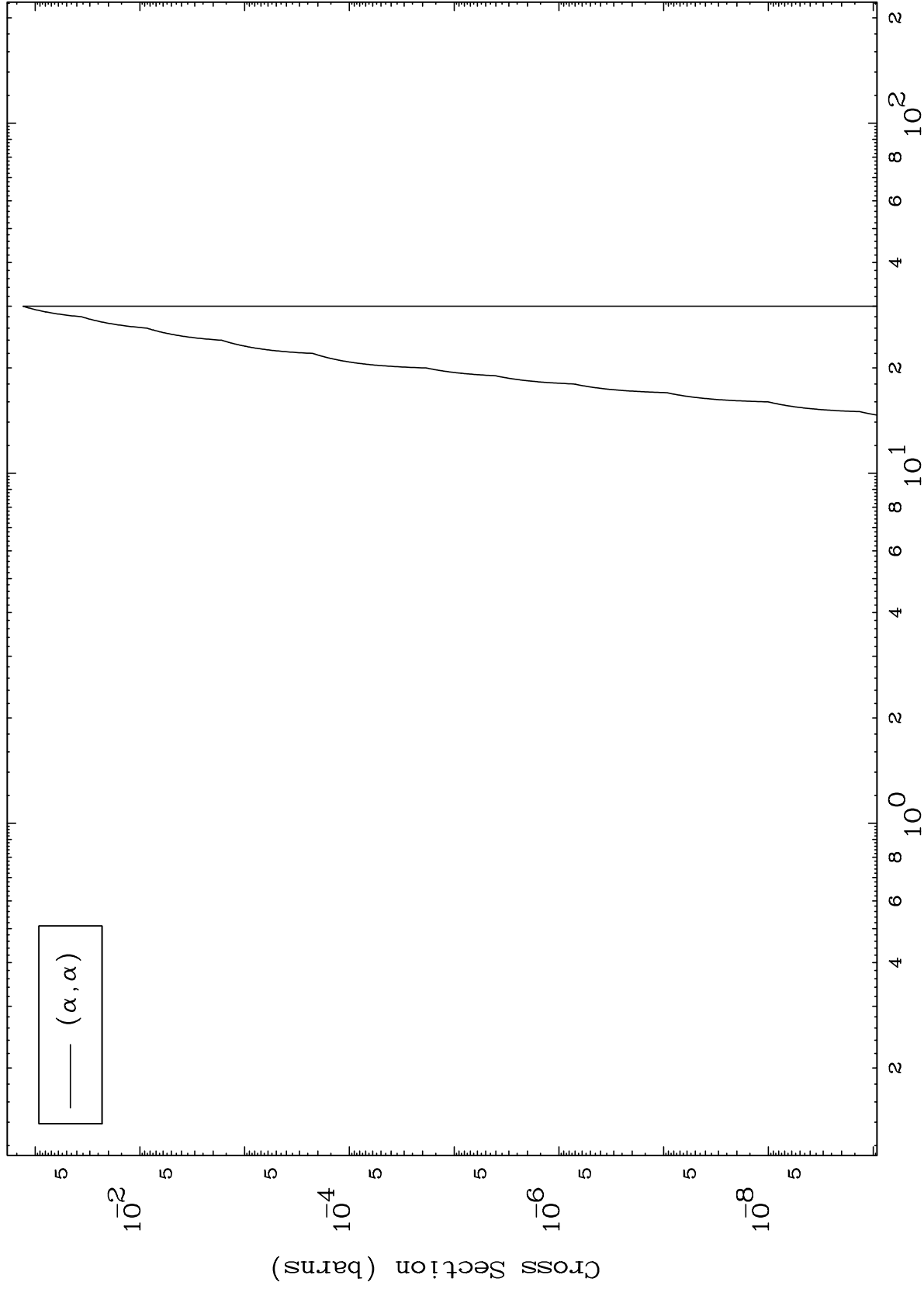


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(α, α) Levels

79-Au-189

0 Kelvin Cross Sections

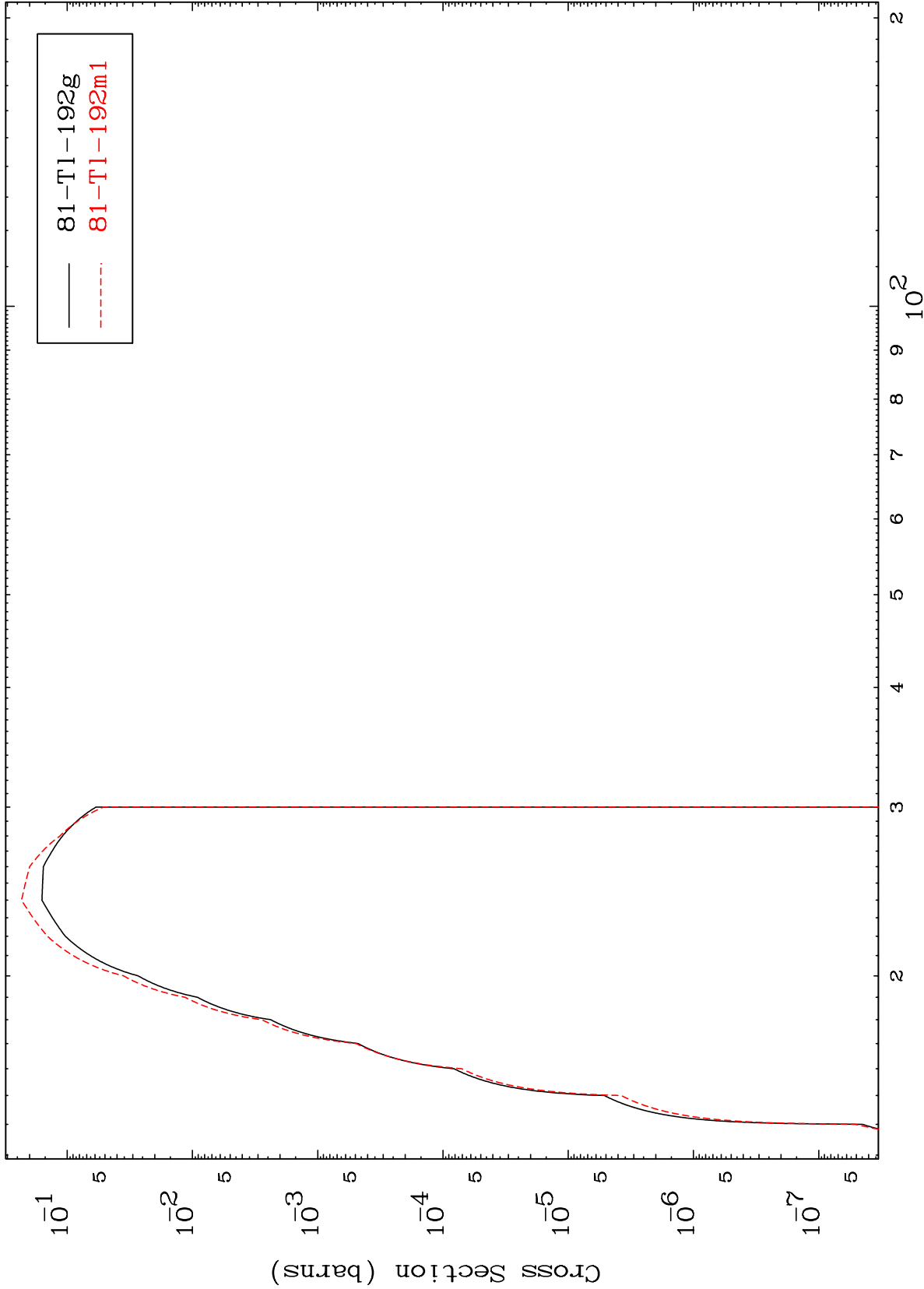


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Incident Energy (MeV)

79-Au-189

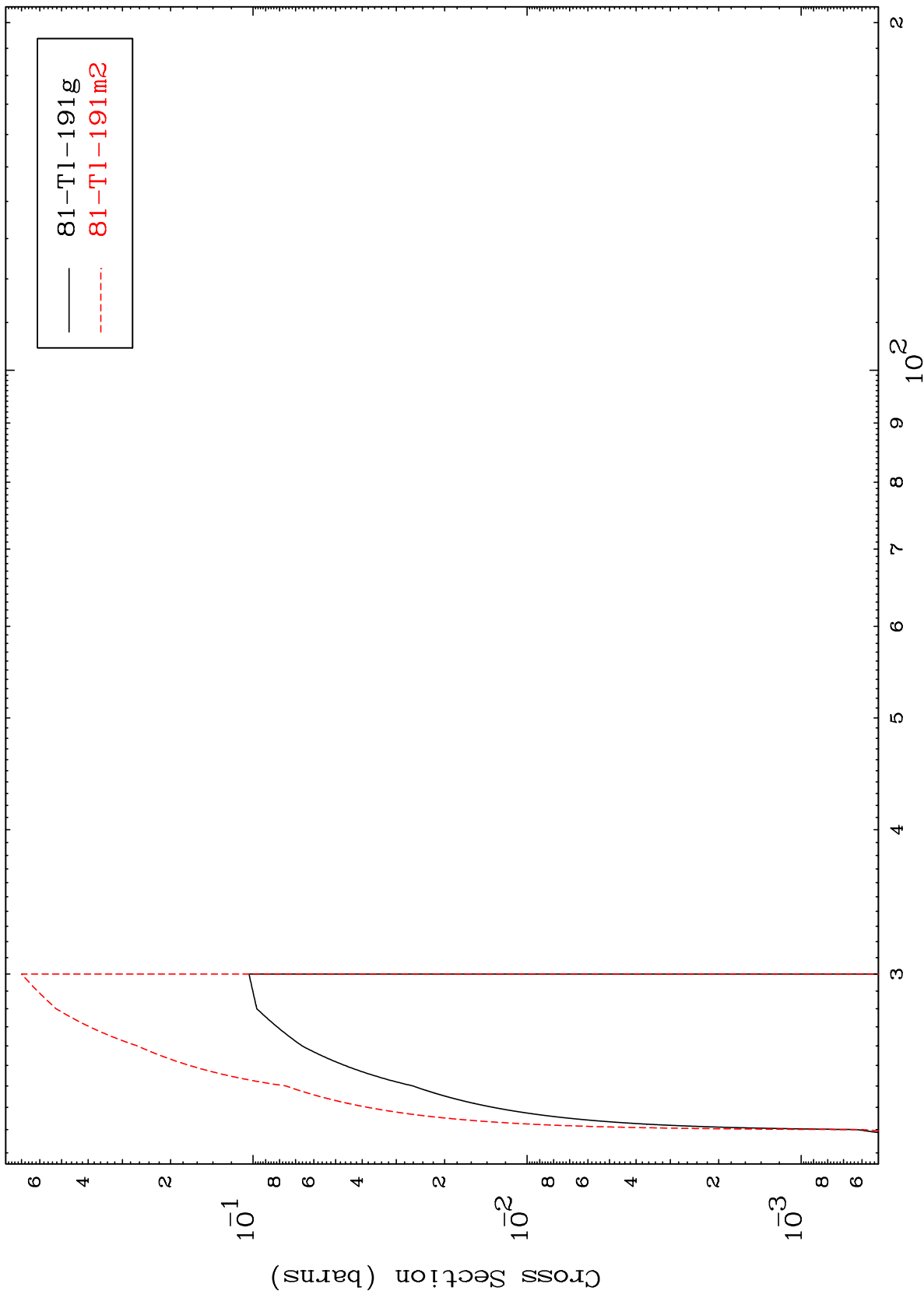
Radionuclide Production Cross Section
 α Inelastic



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79-Au-189

$(\alpha, 2n)$
Radionuclide Production Cross Section



79-Au-189

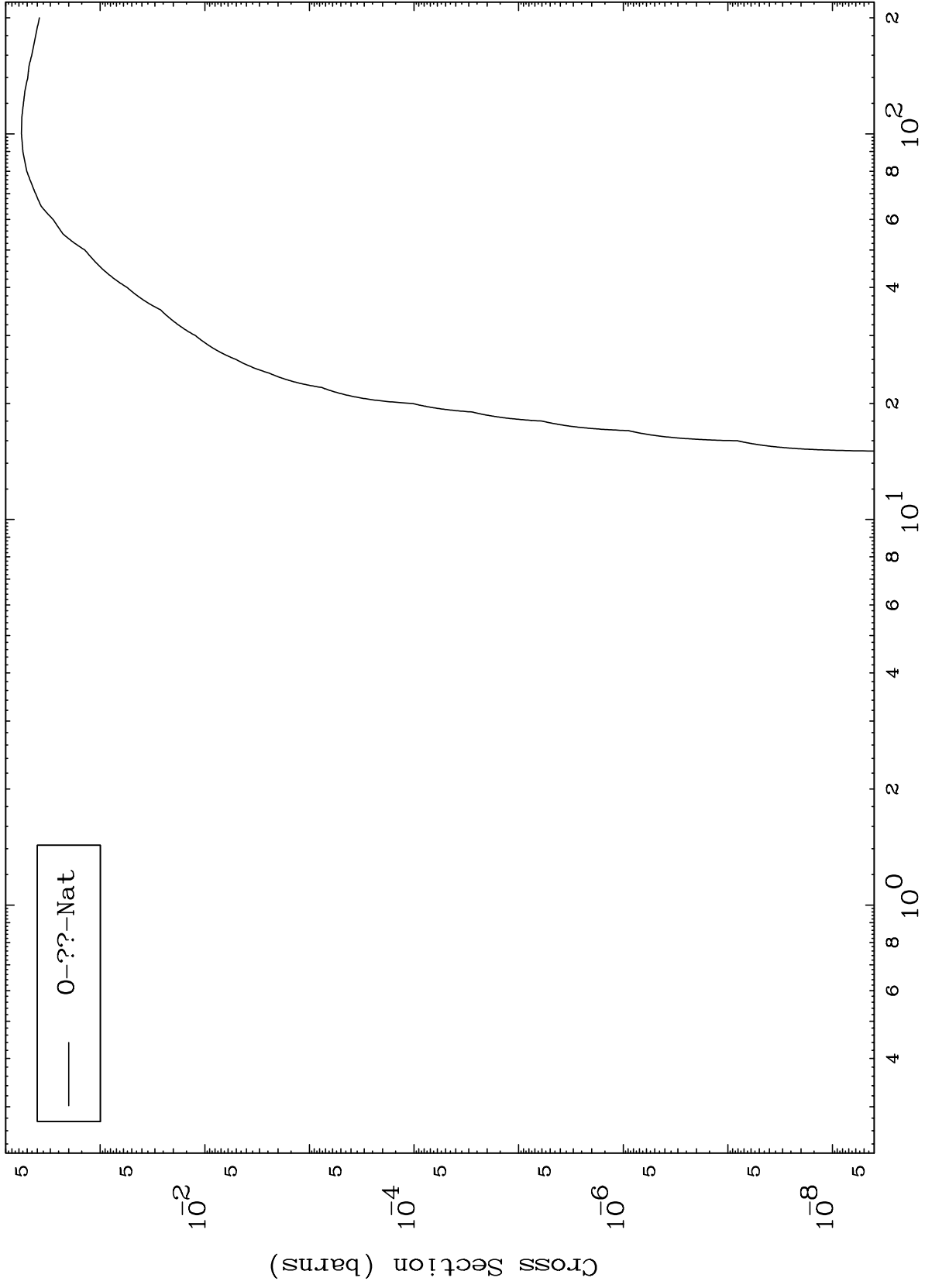
Incident Energy (MeV)

12

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79-Au-189

α Fission
Radionuclide Production Cross Section



13

79-Au-189

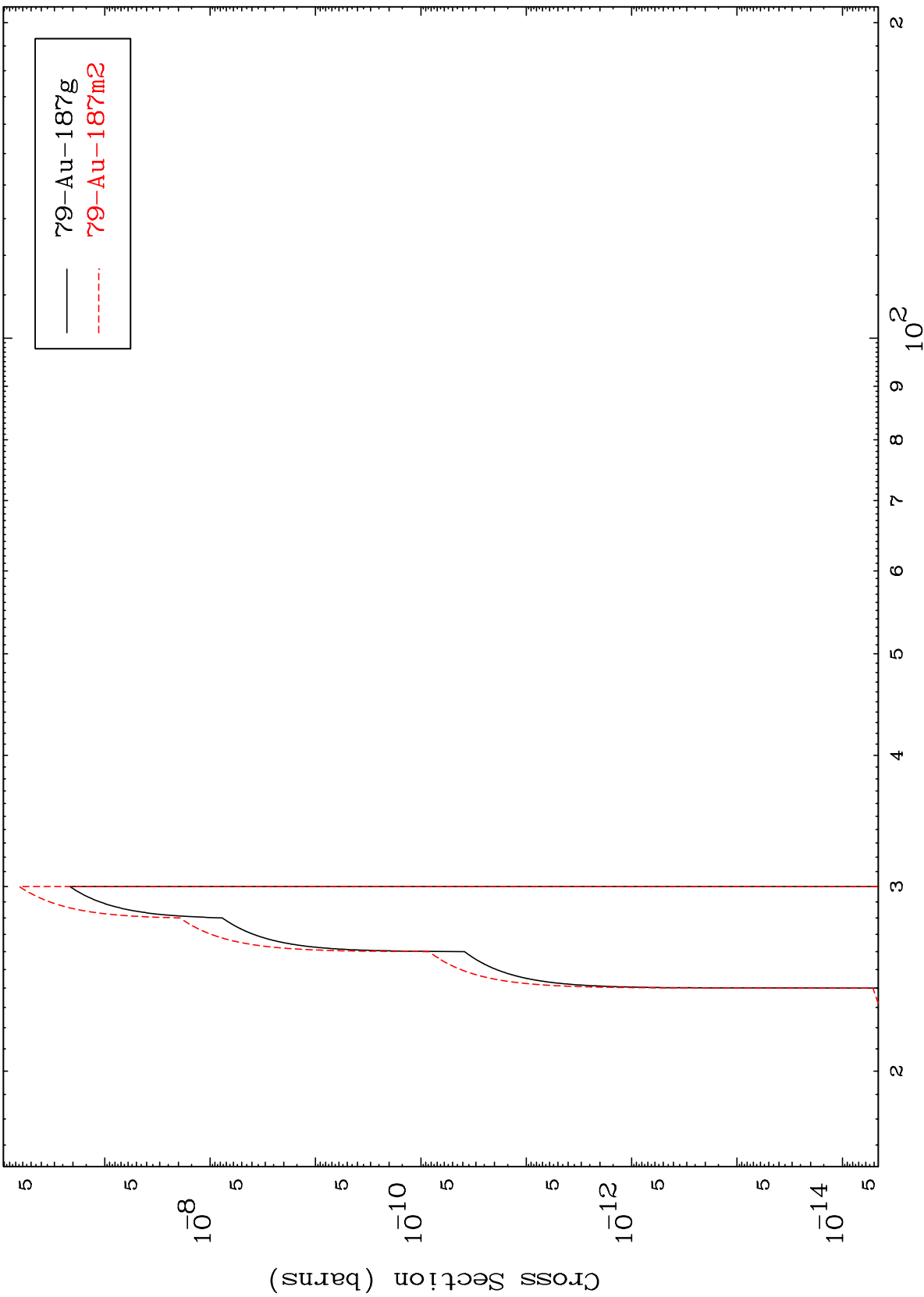
Incident Energy (MeV)

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$(\alpha, 2n) \alpha$

$^{79}\text{Au-189}$

Radionuclide Production Cross Section



14

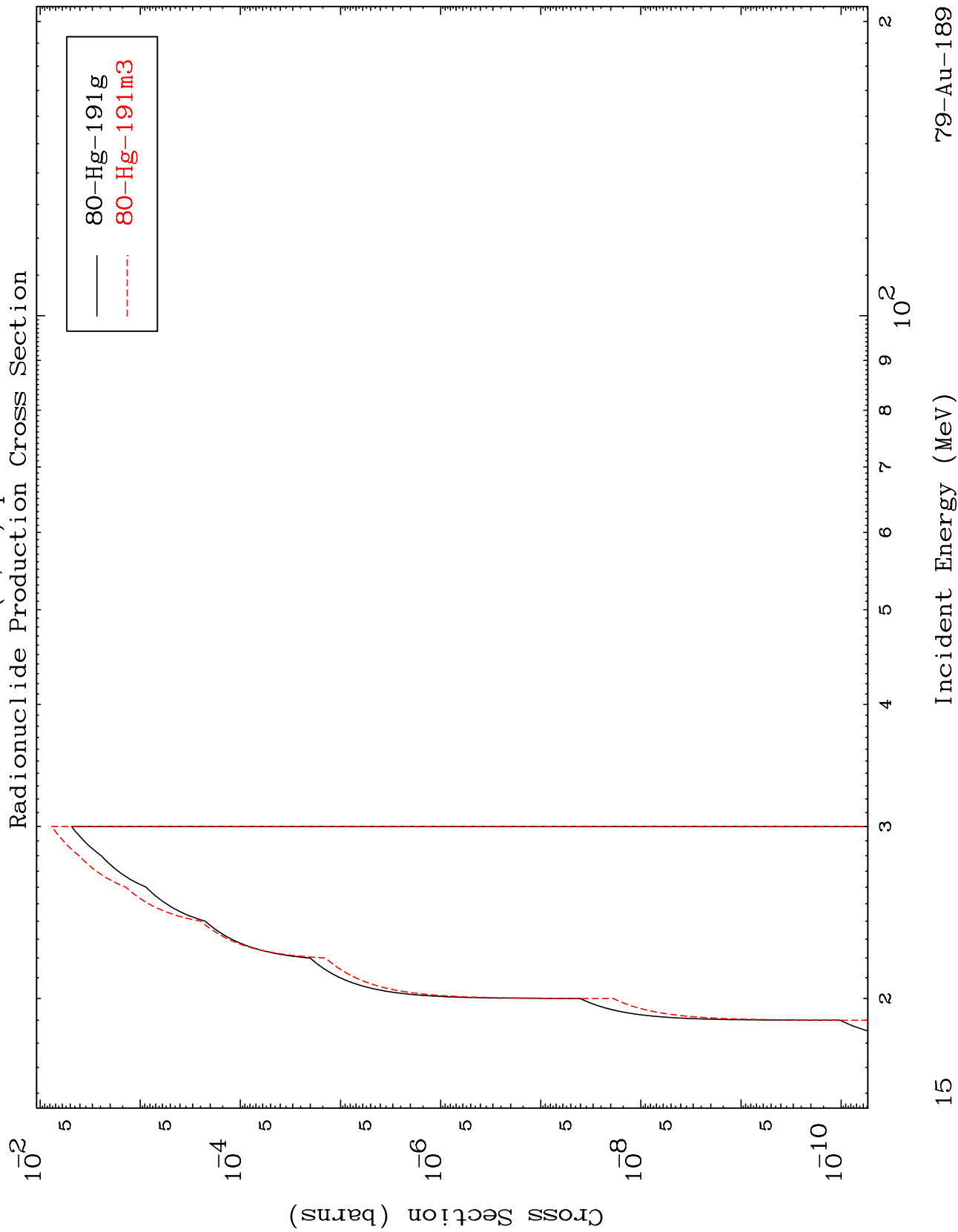
Incident Energy (MeV)

$^{79}\text{Au-189}$

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(α, n') p

79-Au-189

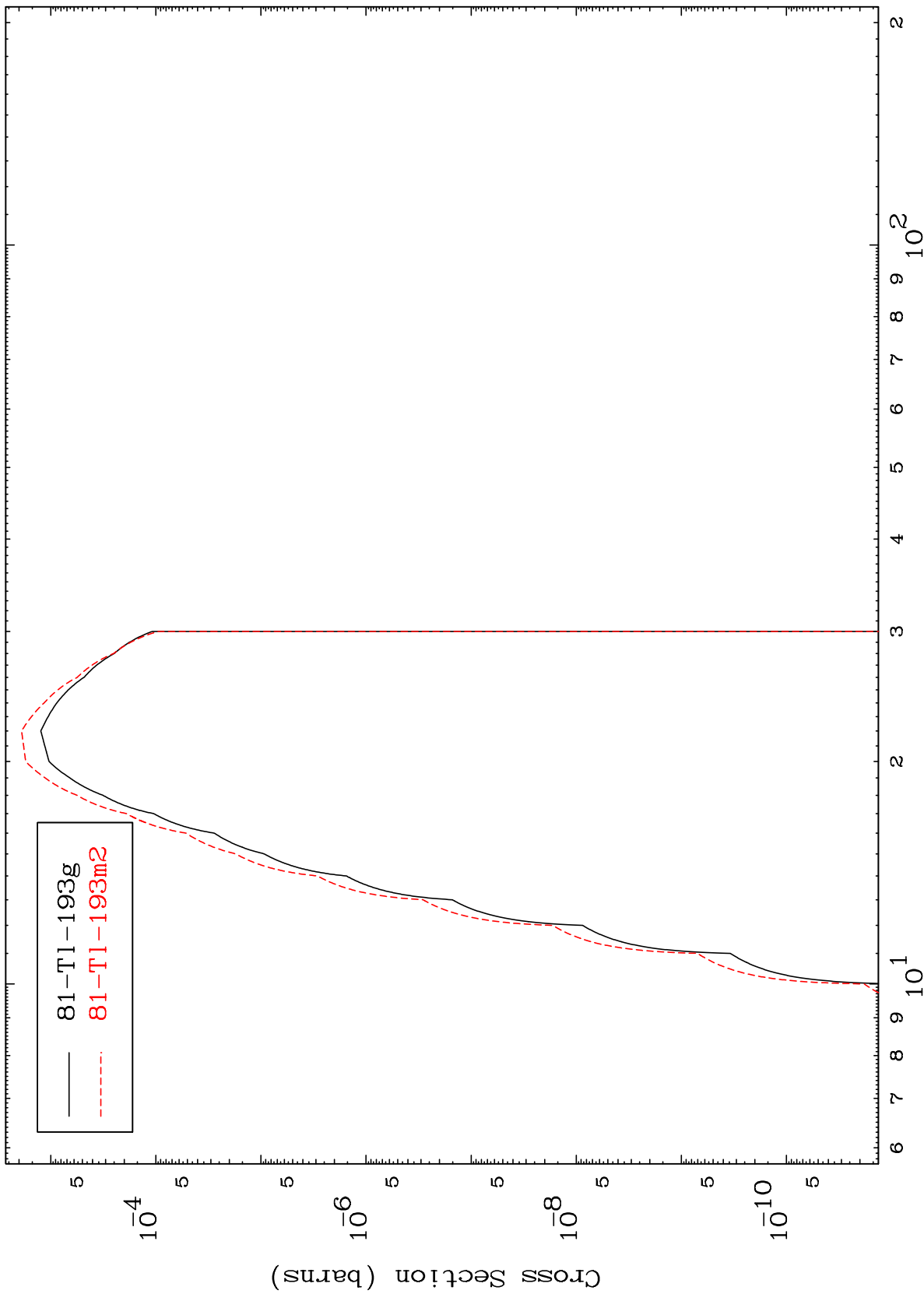


15

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79-Au-189

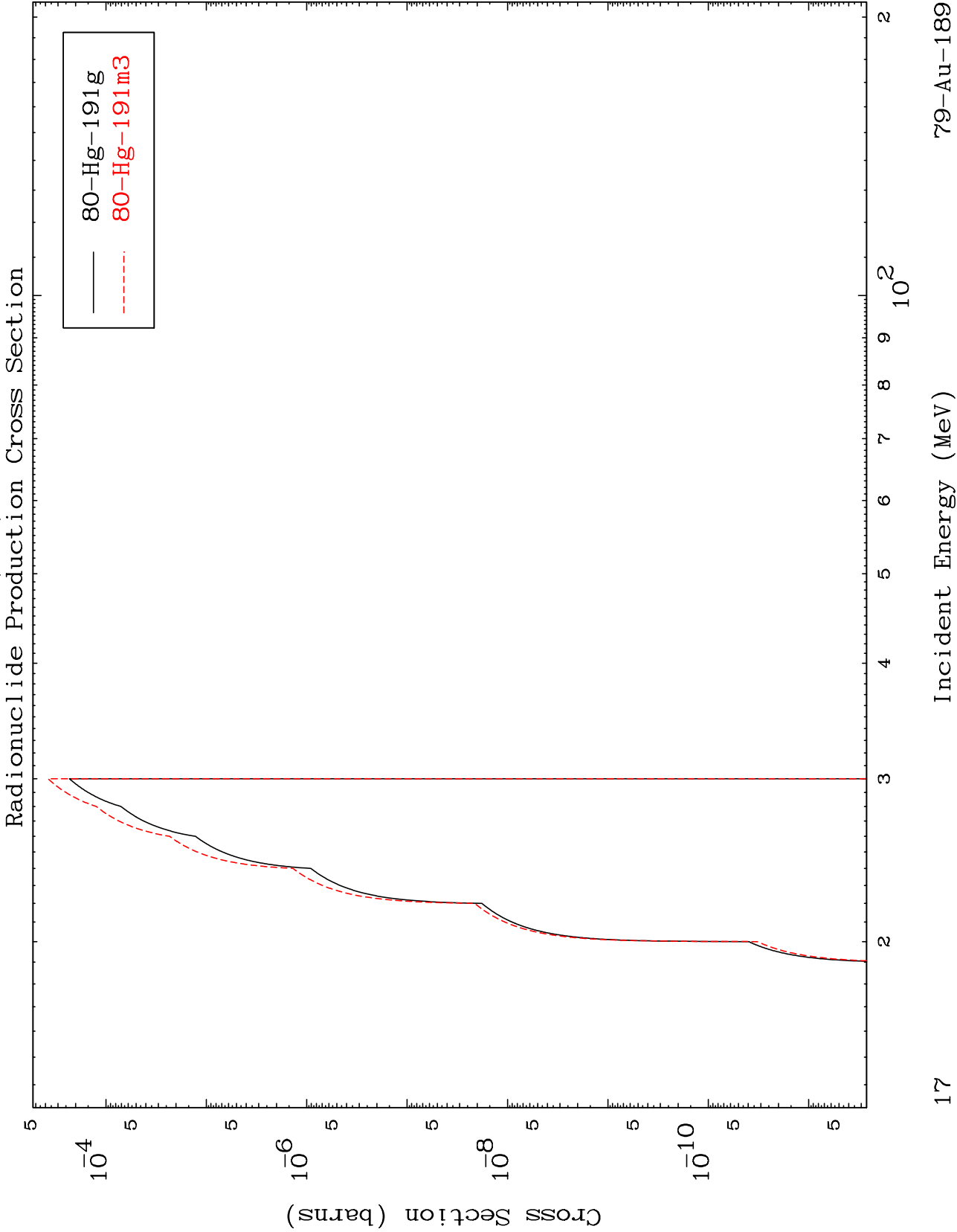
(α, γ)
Radionuclide Production Cross Section



16

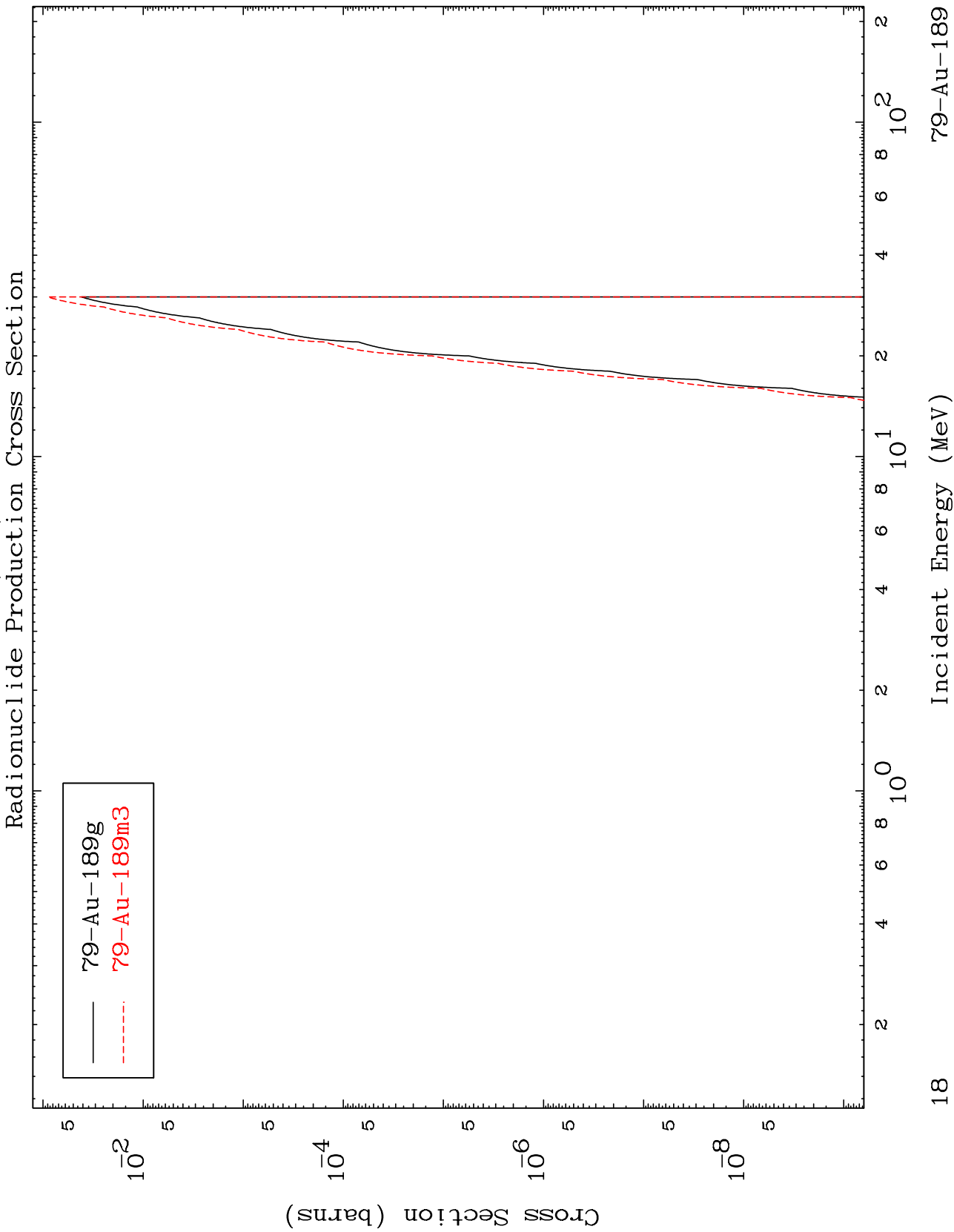
Incident Energy (MeV)

79-Au-189



MAT 7902

$^{79}\text{Au-189}$



$^{79}\text{Au-189}$

Incident Energy (MeV)

18

($\alpha, 2p$)
Radionuclide Production Cross Section

