

Program EVALPLOT
(Version 2018-1)

by

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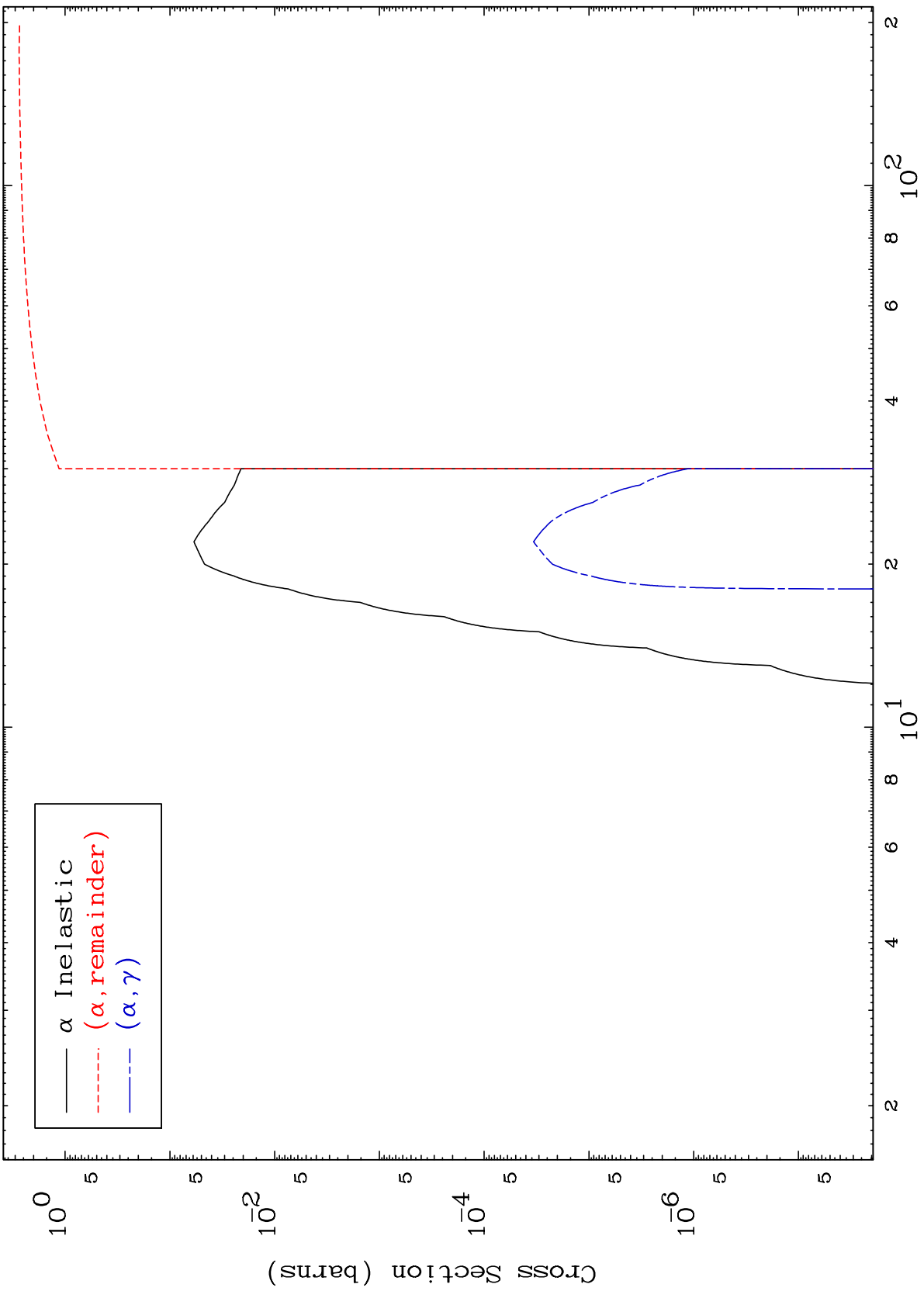
Press Mouse Button to Start

MAT 7922

α Major

79-Au-196

0 Kelvin Cross Sections

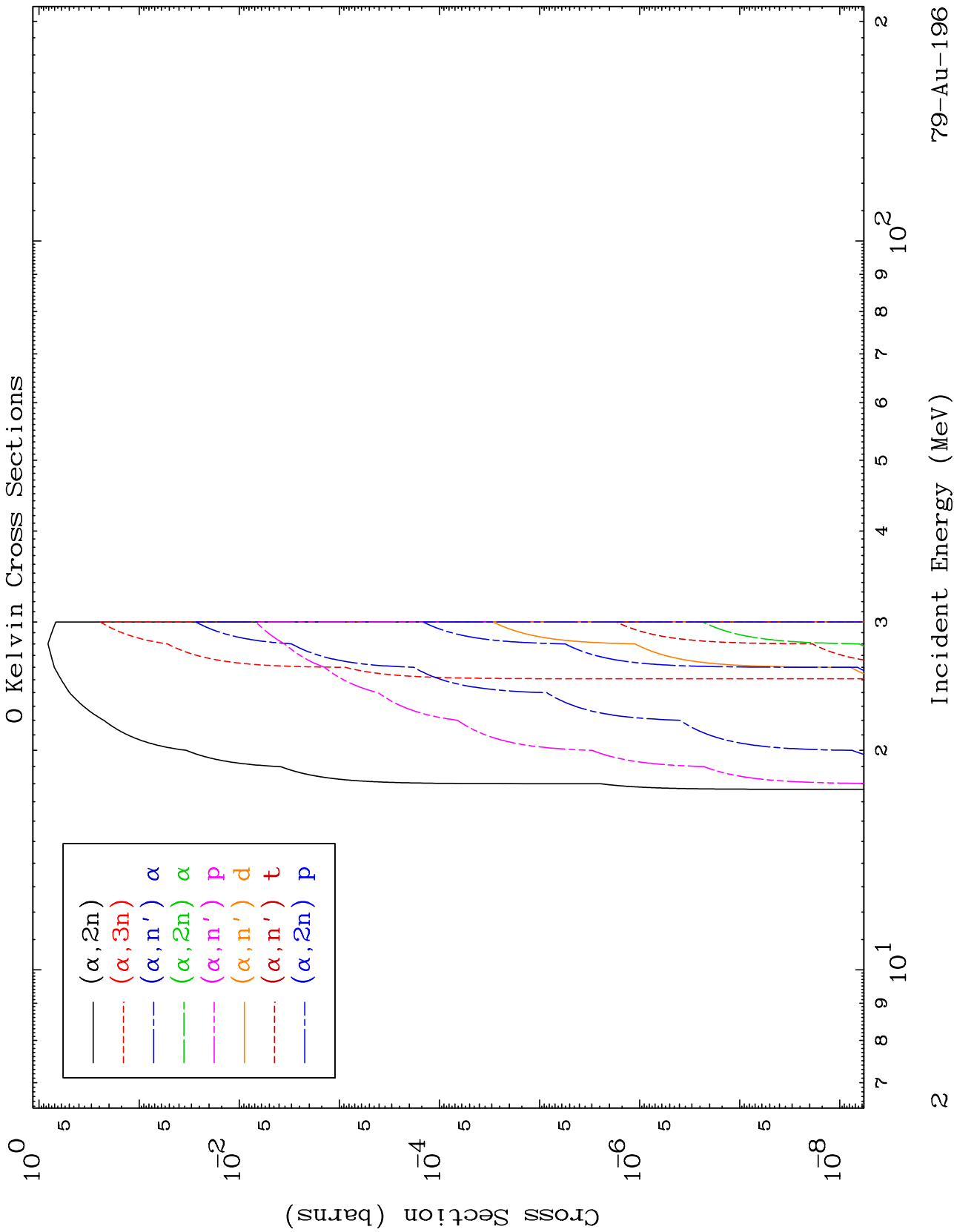


α Inelastic
 $(\alpha, \text{remainder})$
 (α, γ)

MAT 7922

α Neutron Production
0 Kelvin Cross Sections

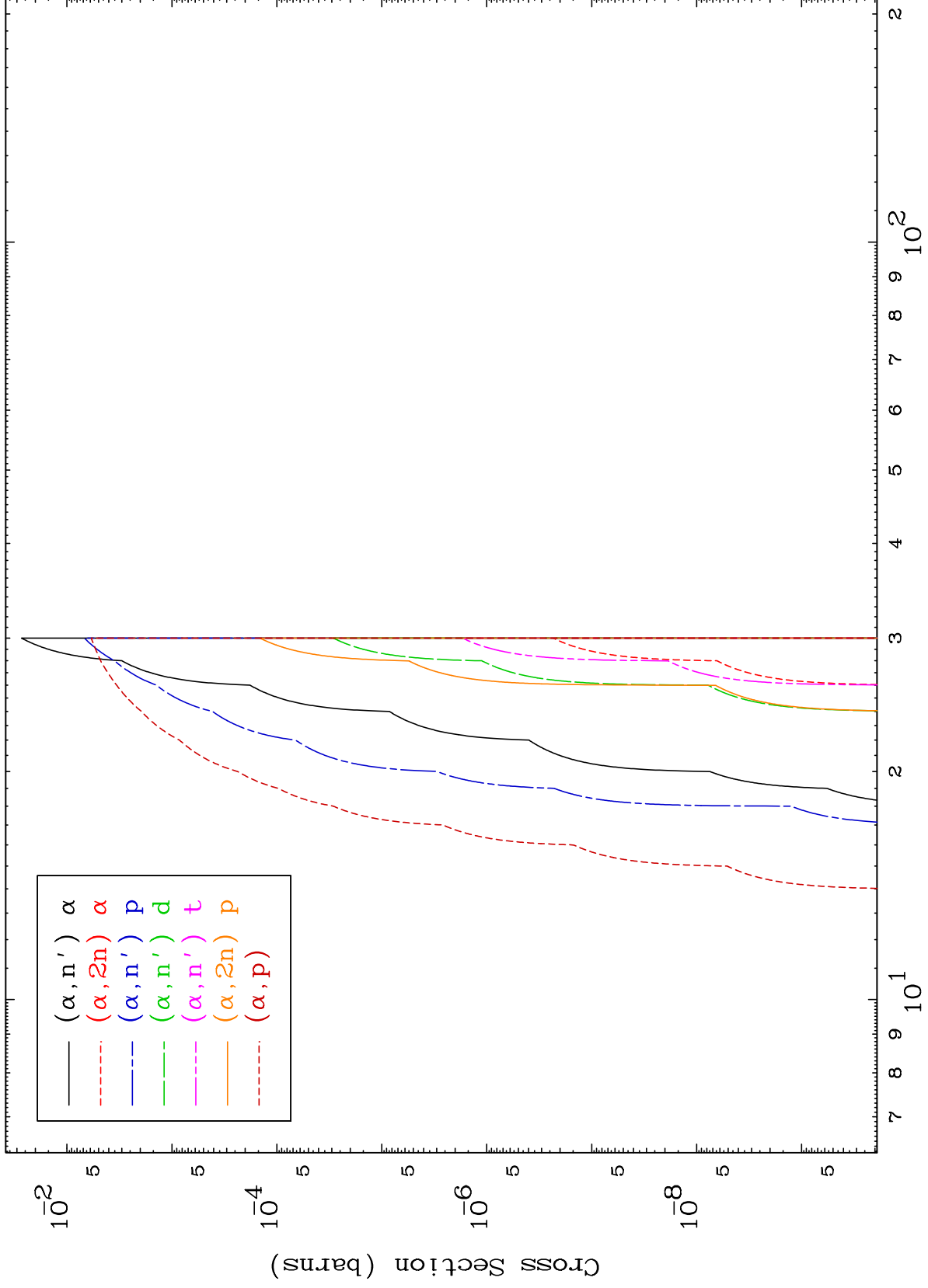
79-Au-196

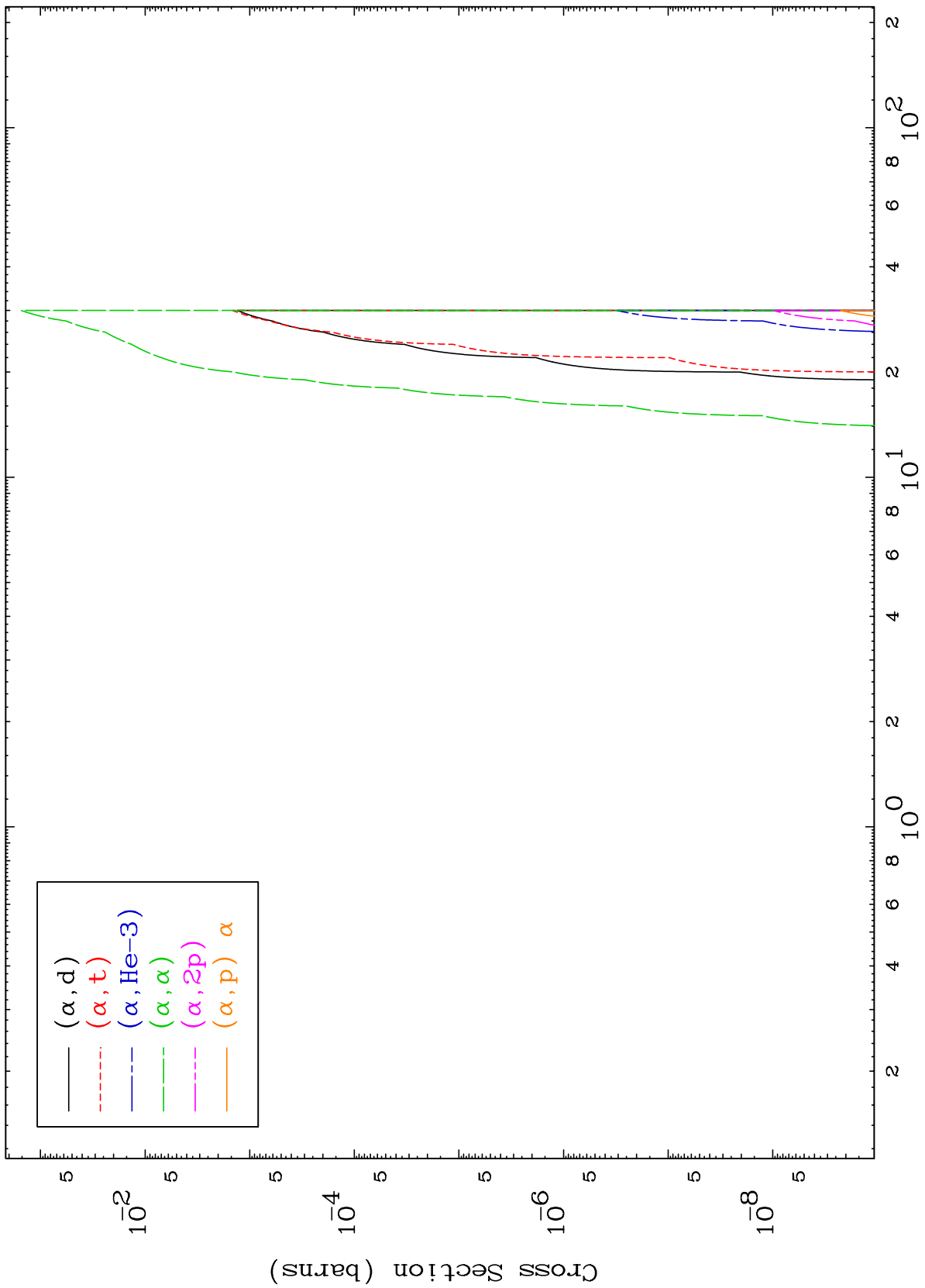


79-Au-196

Incident Energy (MeV)

2



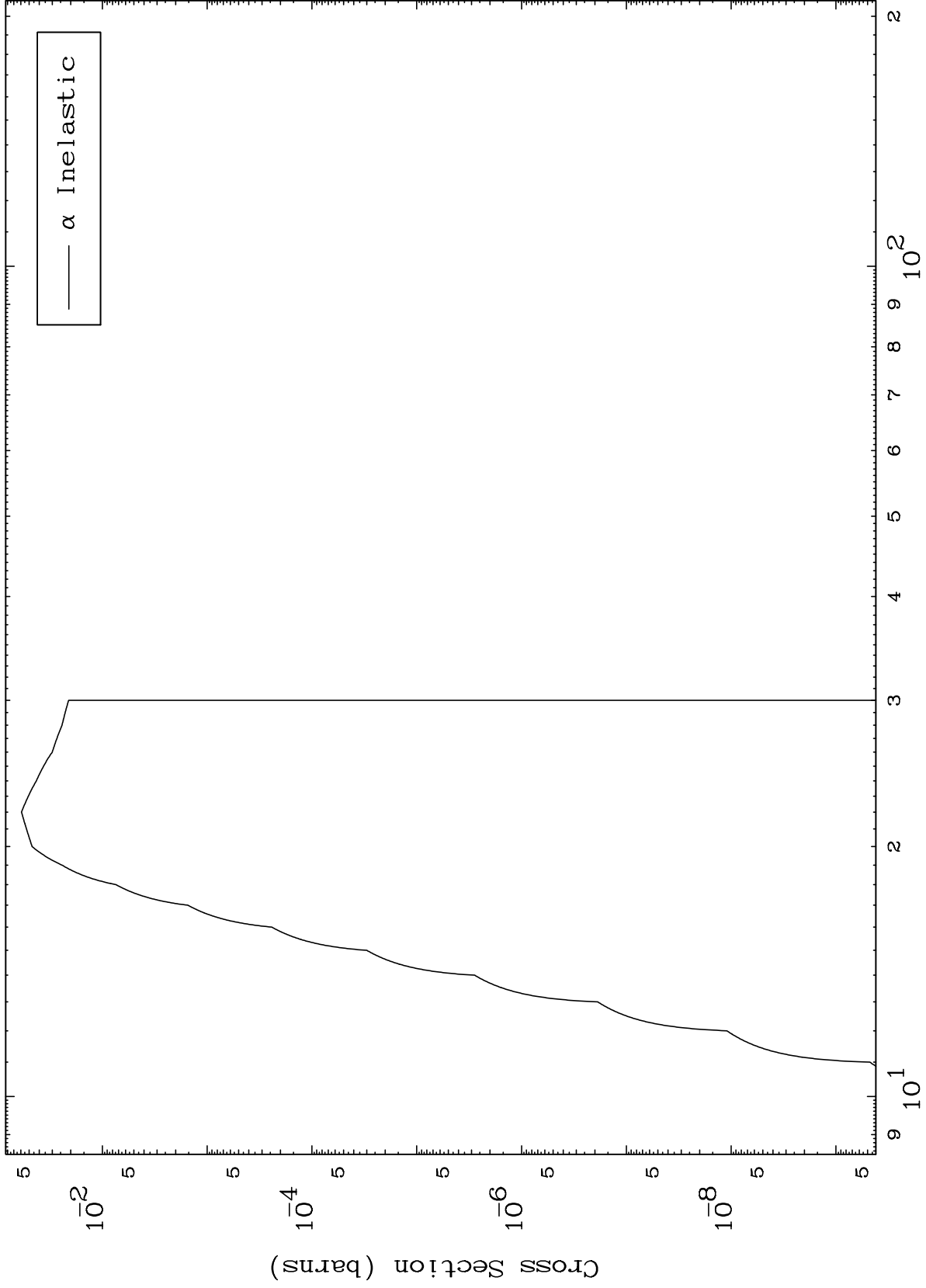


MAT 7922

(α, n') Level

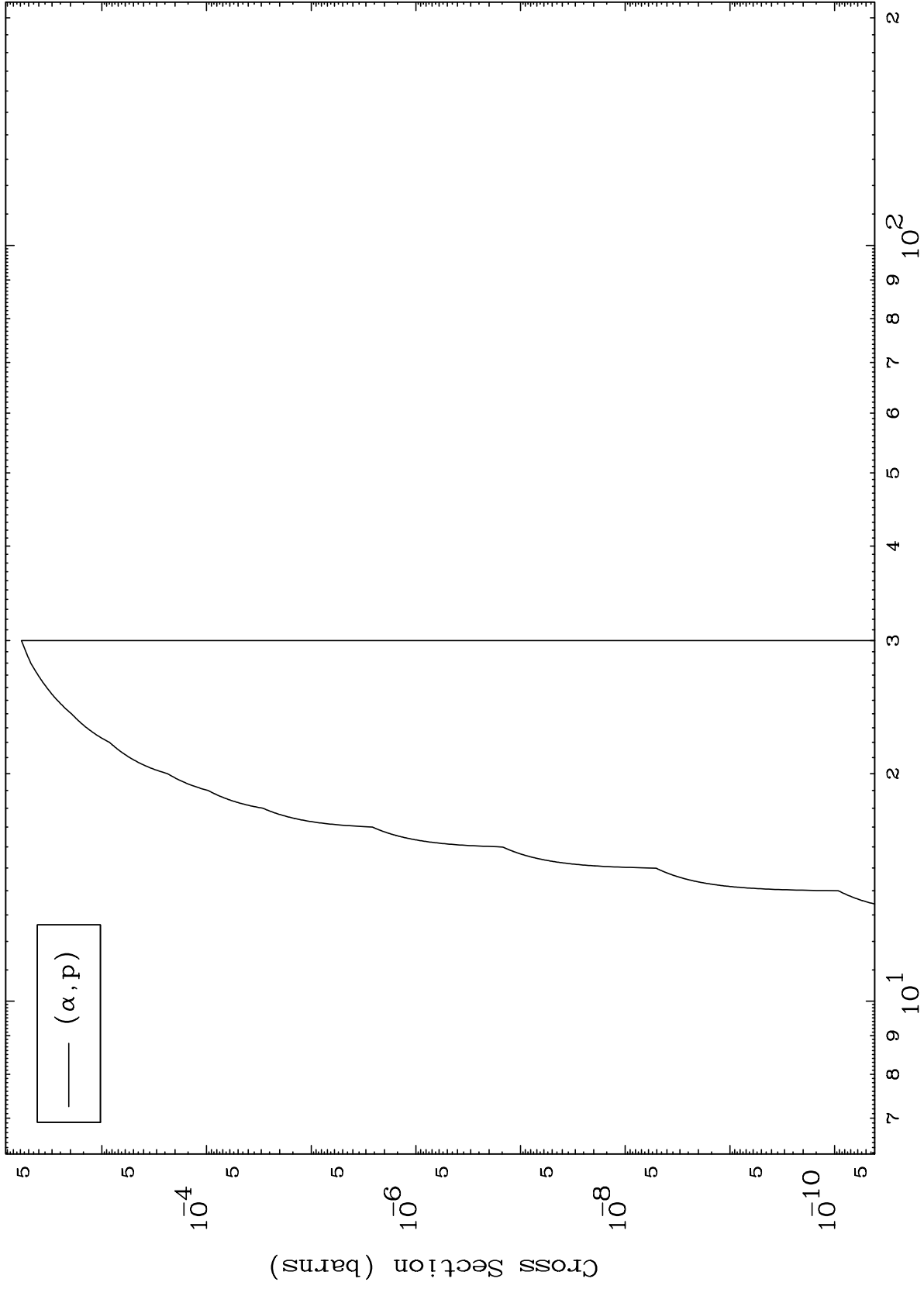
79-Au-196

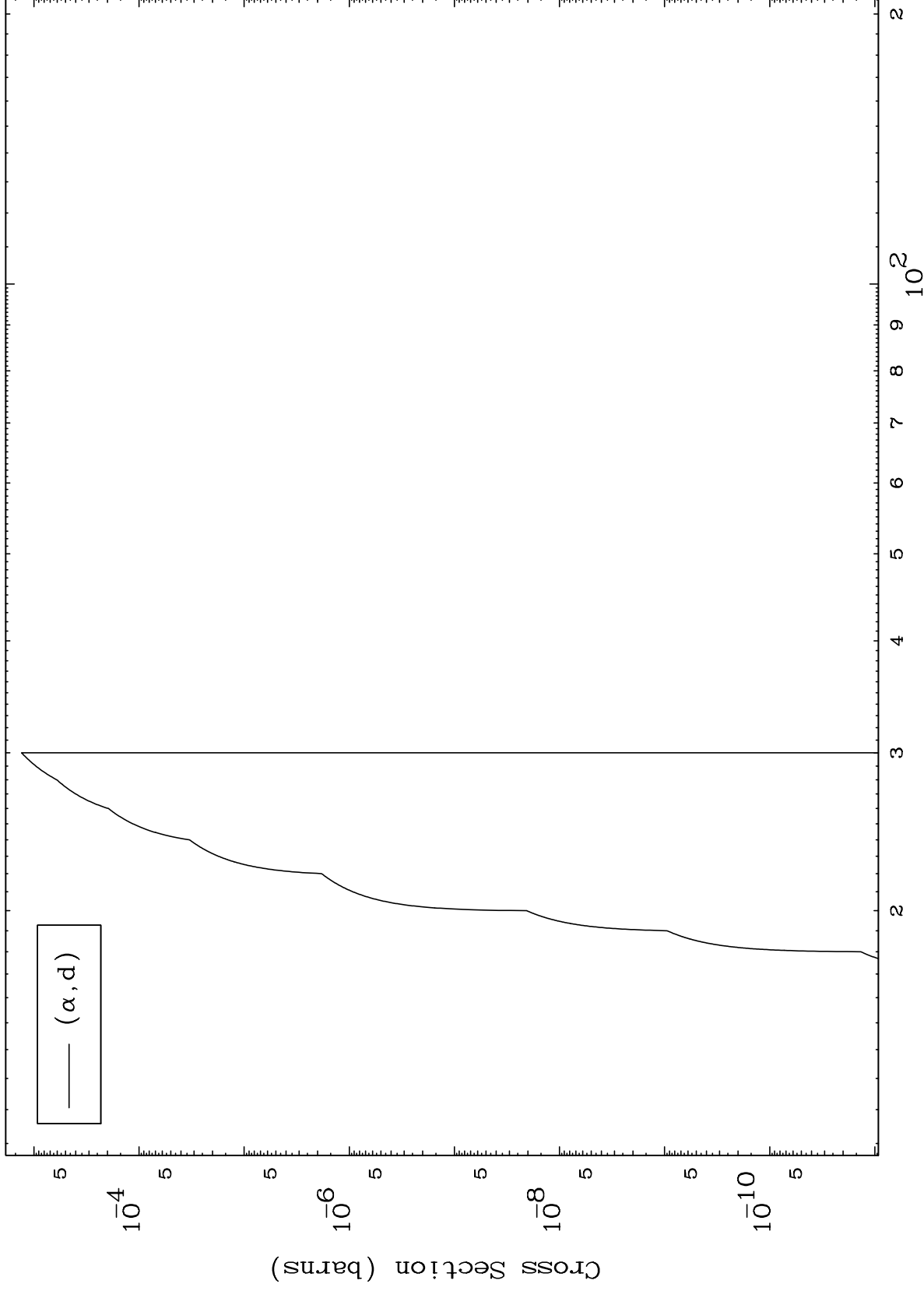
0 Kelvin Cross Sections

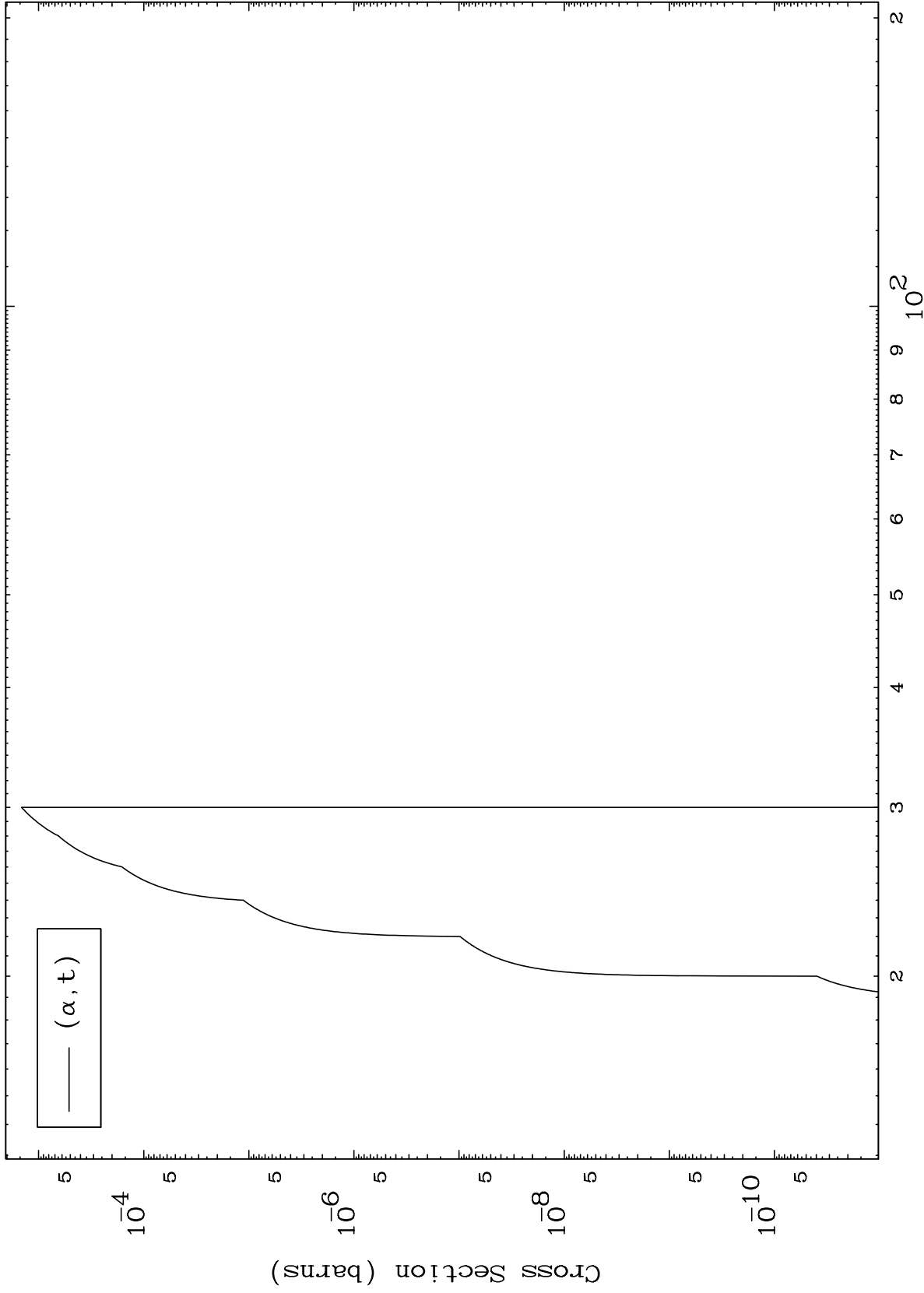


Incident Energy (MeV)

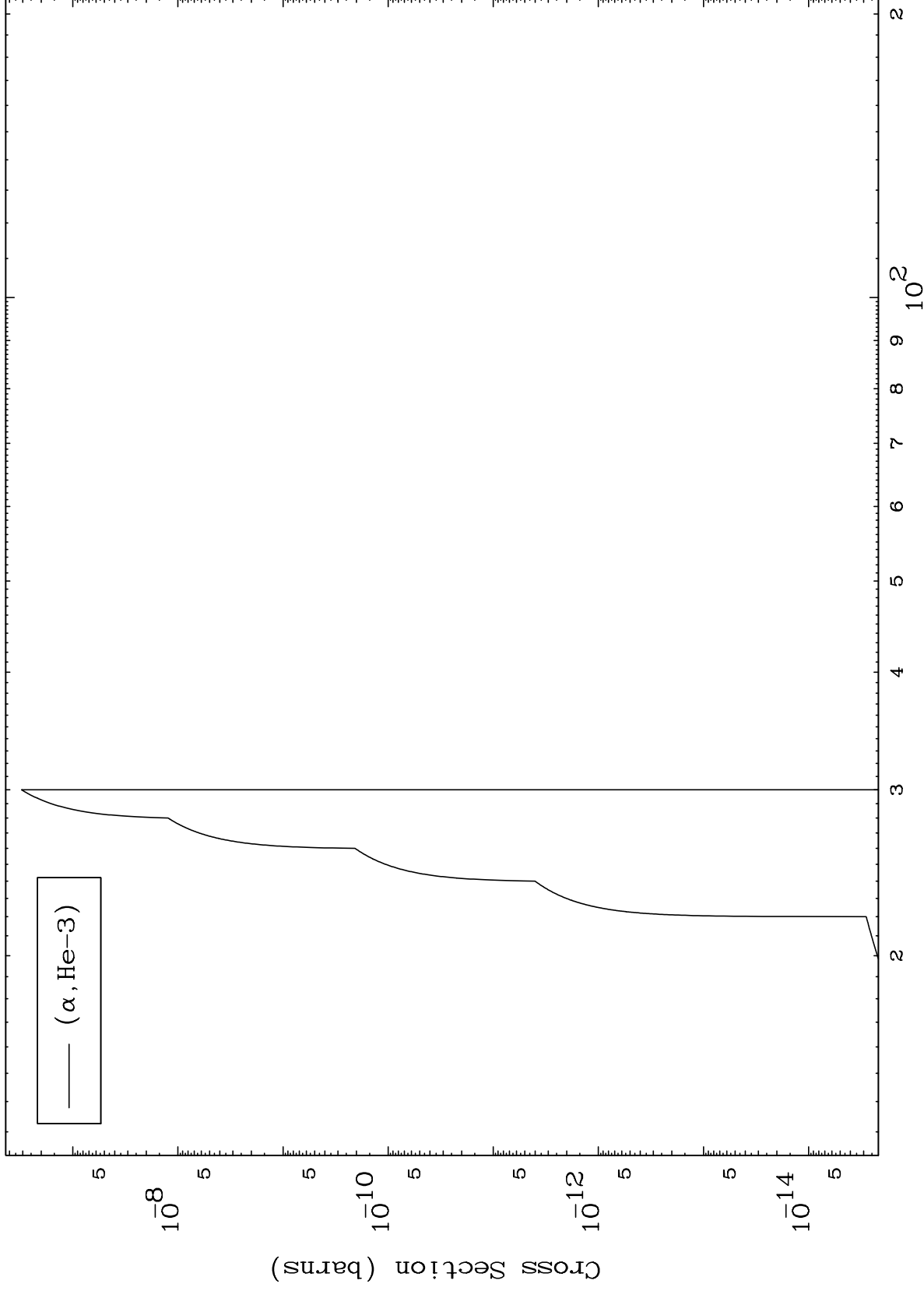
79-Au-196







0 Kelvin Cross Sections

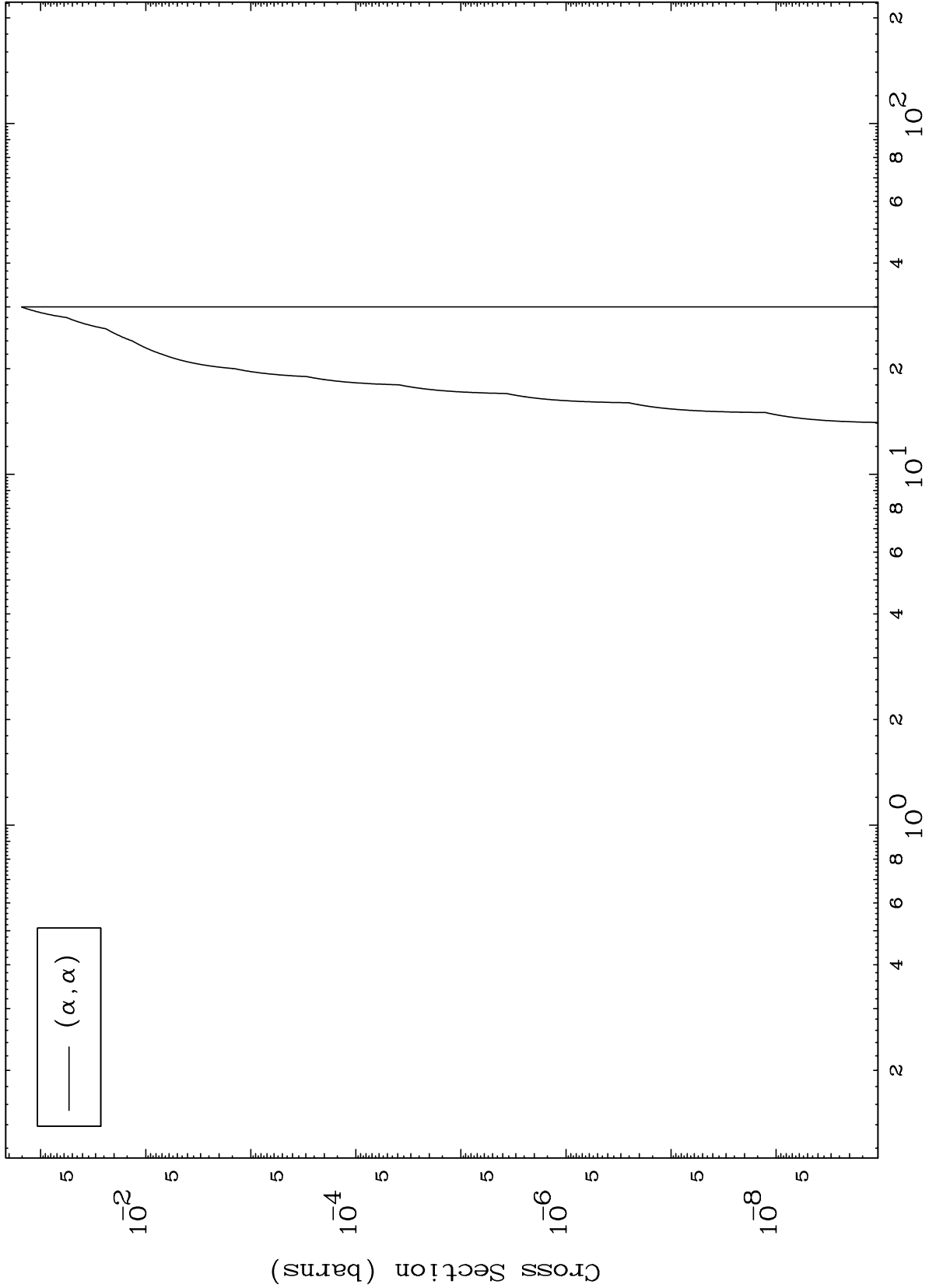


MAT 7922

(α, α) Levels

79-Au-196

0 Kelvin Cross Sections

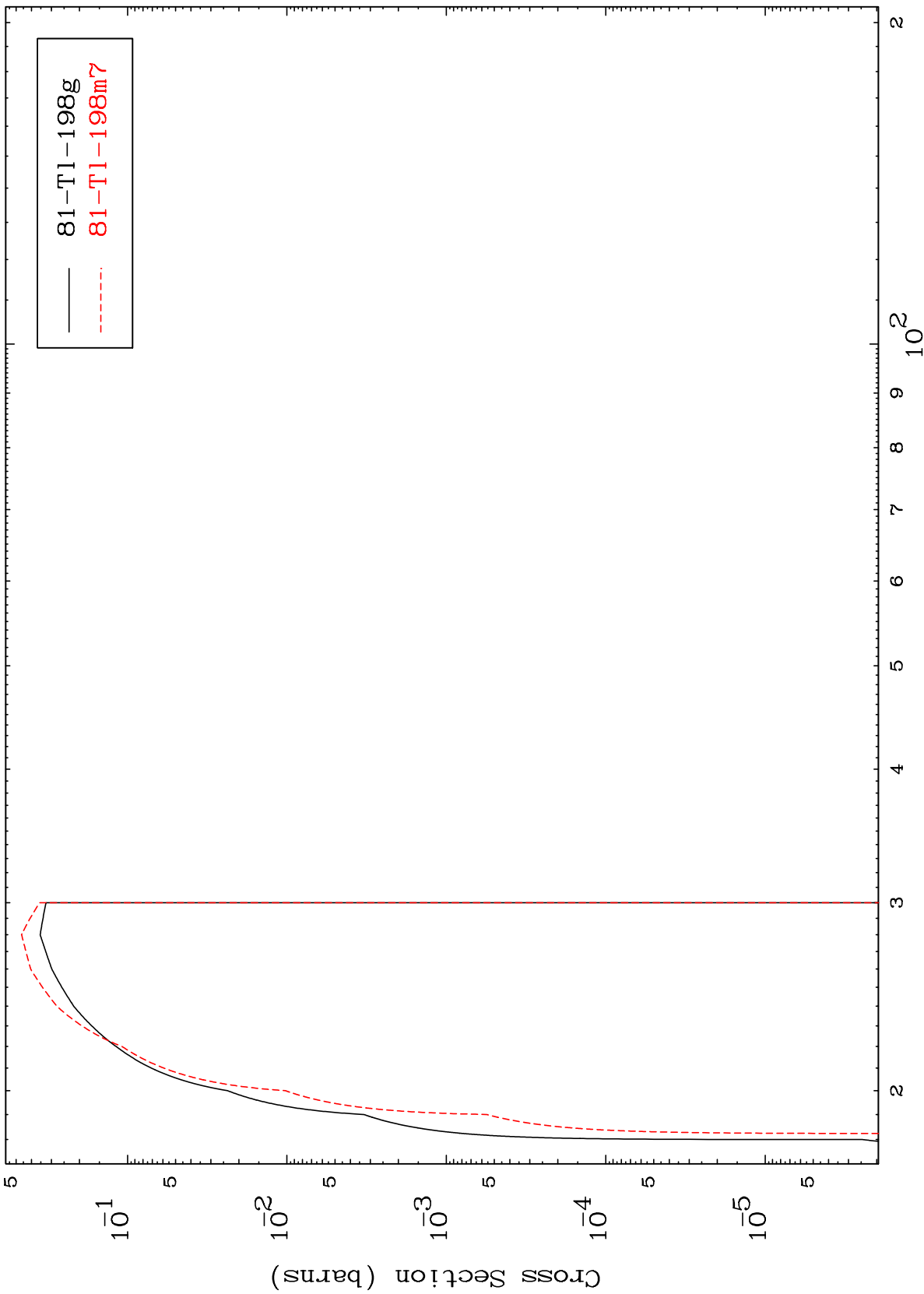


10

Incident Energy (MeV)

79-Au-196

($\alpha, 2n$)
Radionuclide Production Cross Section



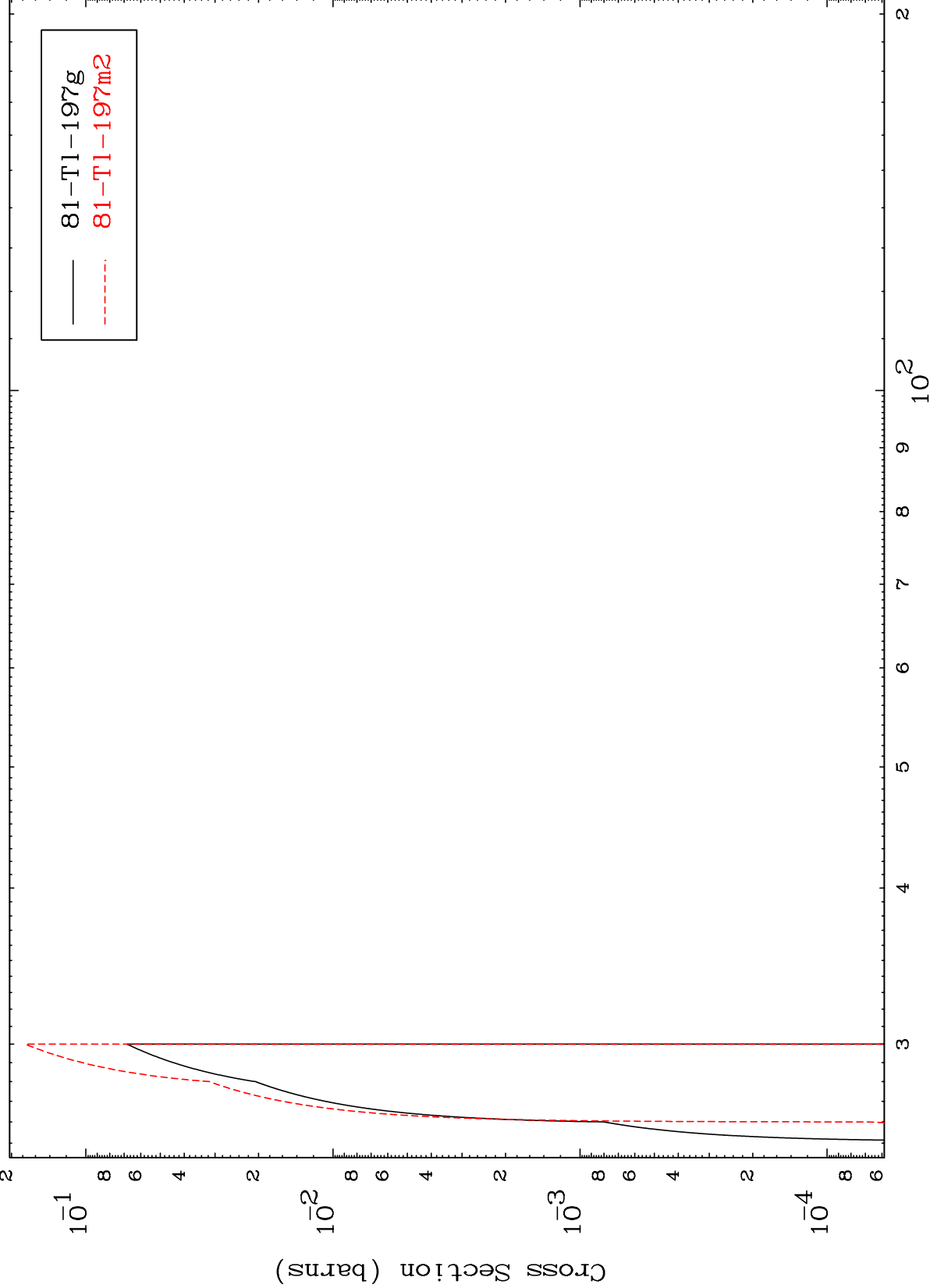
81-Tl-198g
81-Tl-198m7

MAT 7922

($\alpha, 3n$)

79-Au-196

Radionuclide Production Cross Section



12

Incident Energy (MeV)

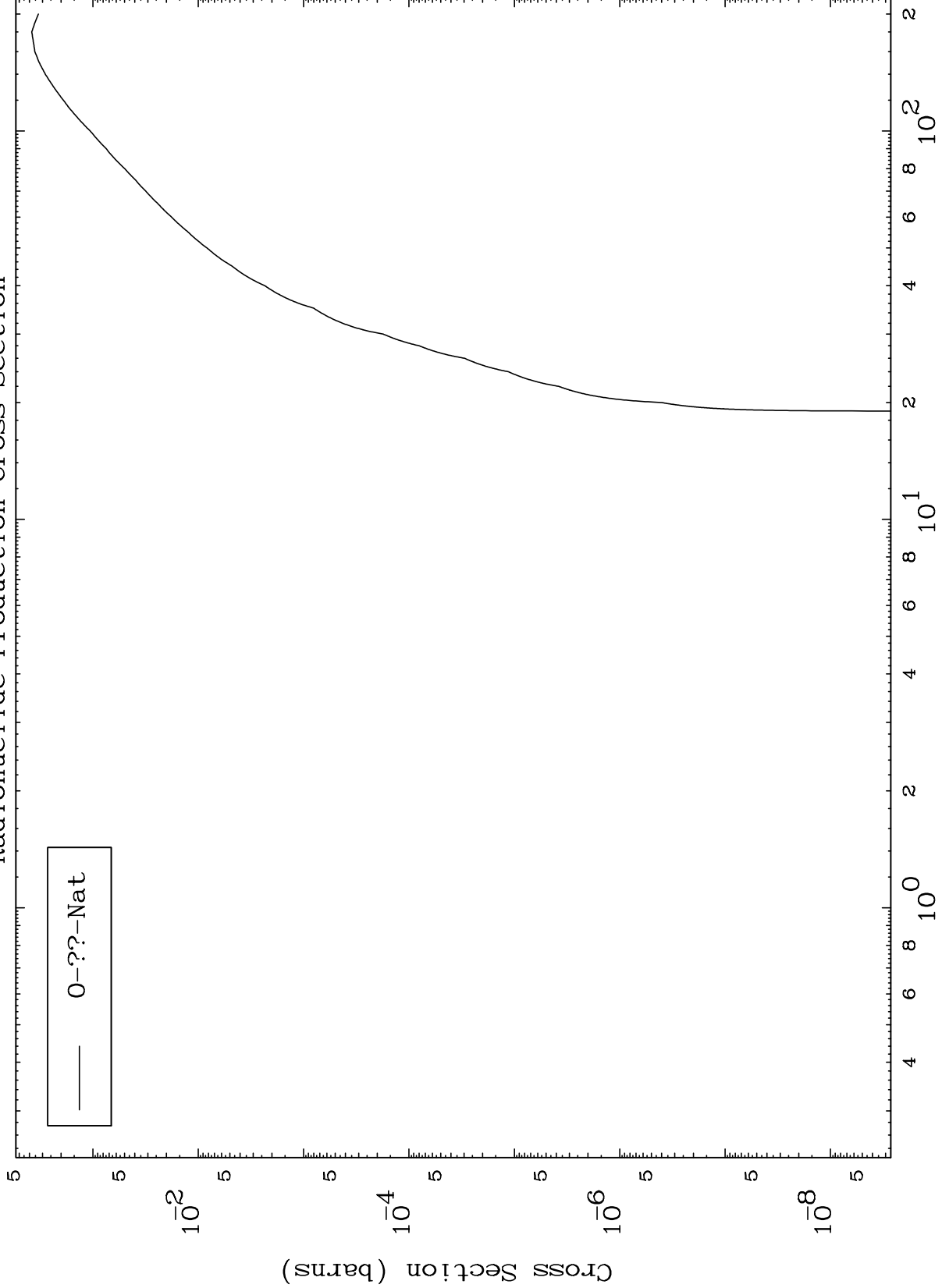
79-Au-196

MAT 7922

α Fission

79-Au-196

Radionuclide Production Cross Section

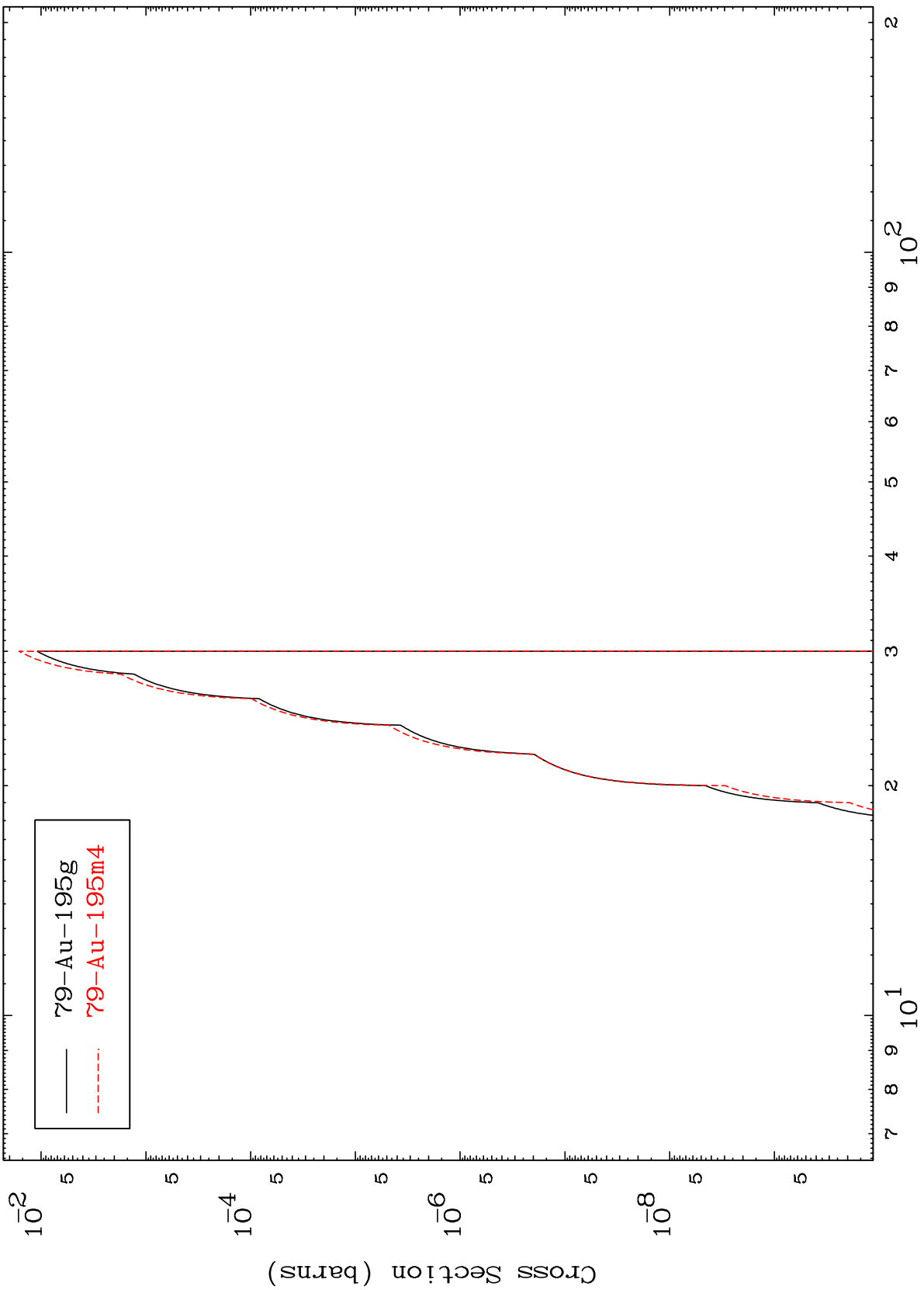


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79-Au-196

(α, n') α

Radionuclide Production Cross Section



79-Au-195g
79-Au-195m4

79-Au-196

Incident Energy (MeV)

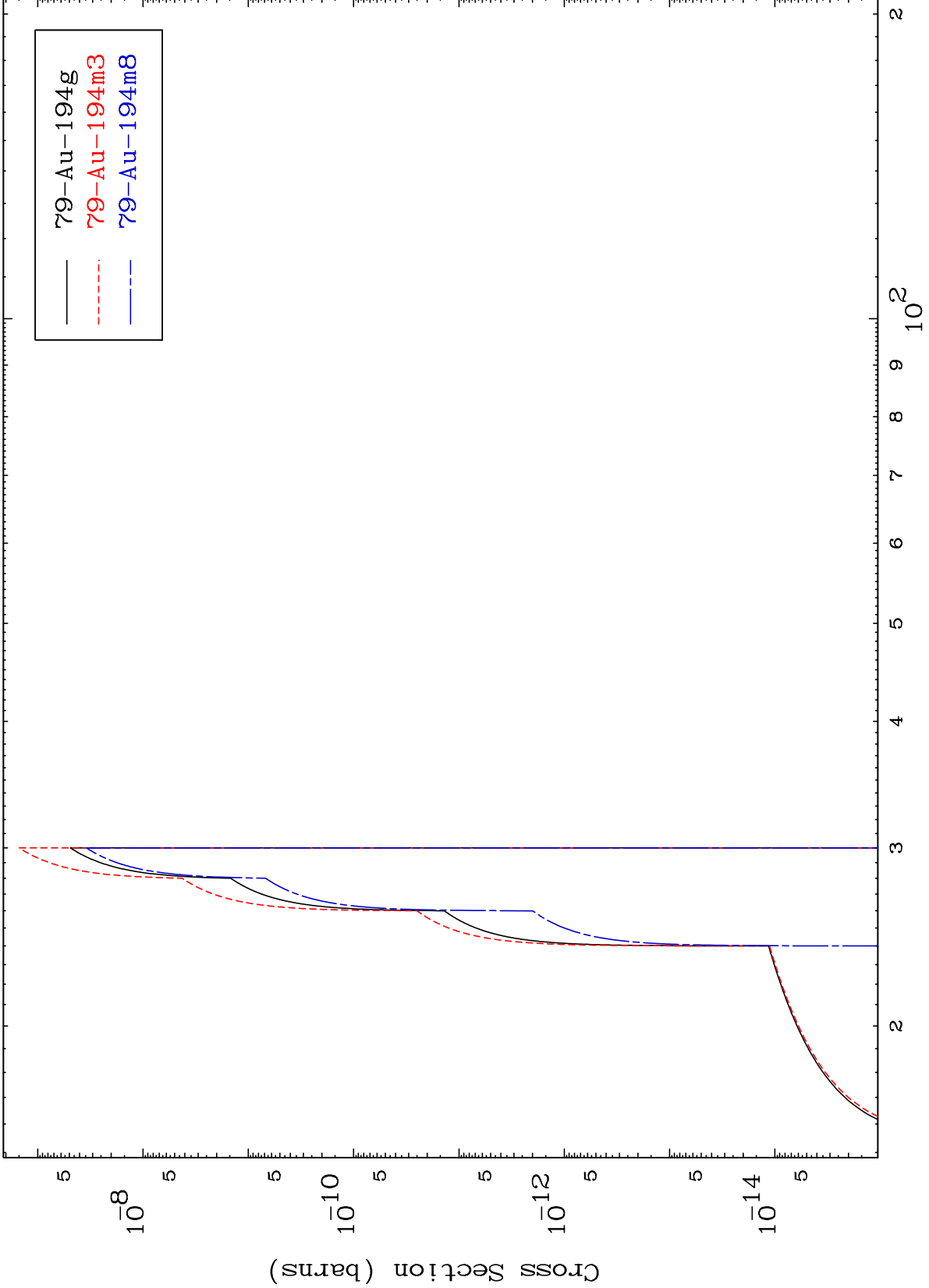
14

MAT 7922

$(\alpha, 2n) \alpha$

$^{79}\text{Au-196}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

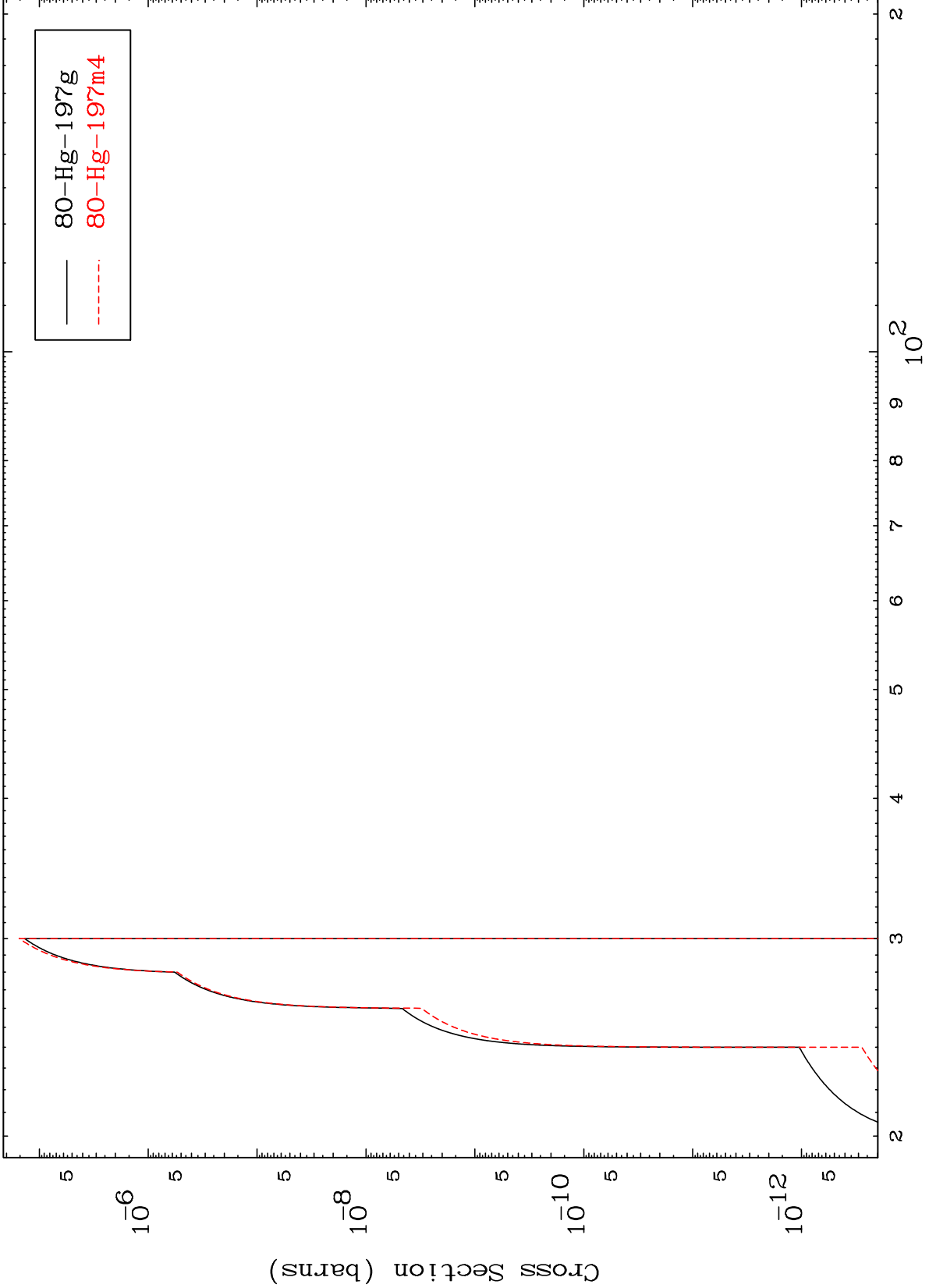
$^{79}\text{Au-196}$

MAT 7922

(α, n') d

79-Au-196

Radionuclide Production Cross Section



16

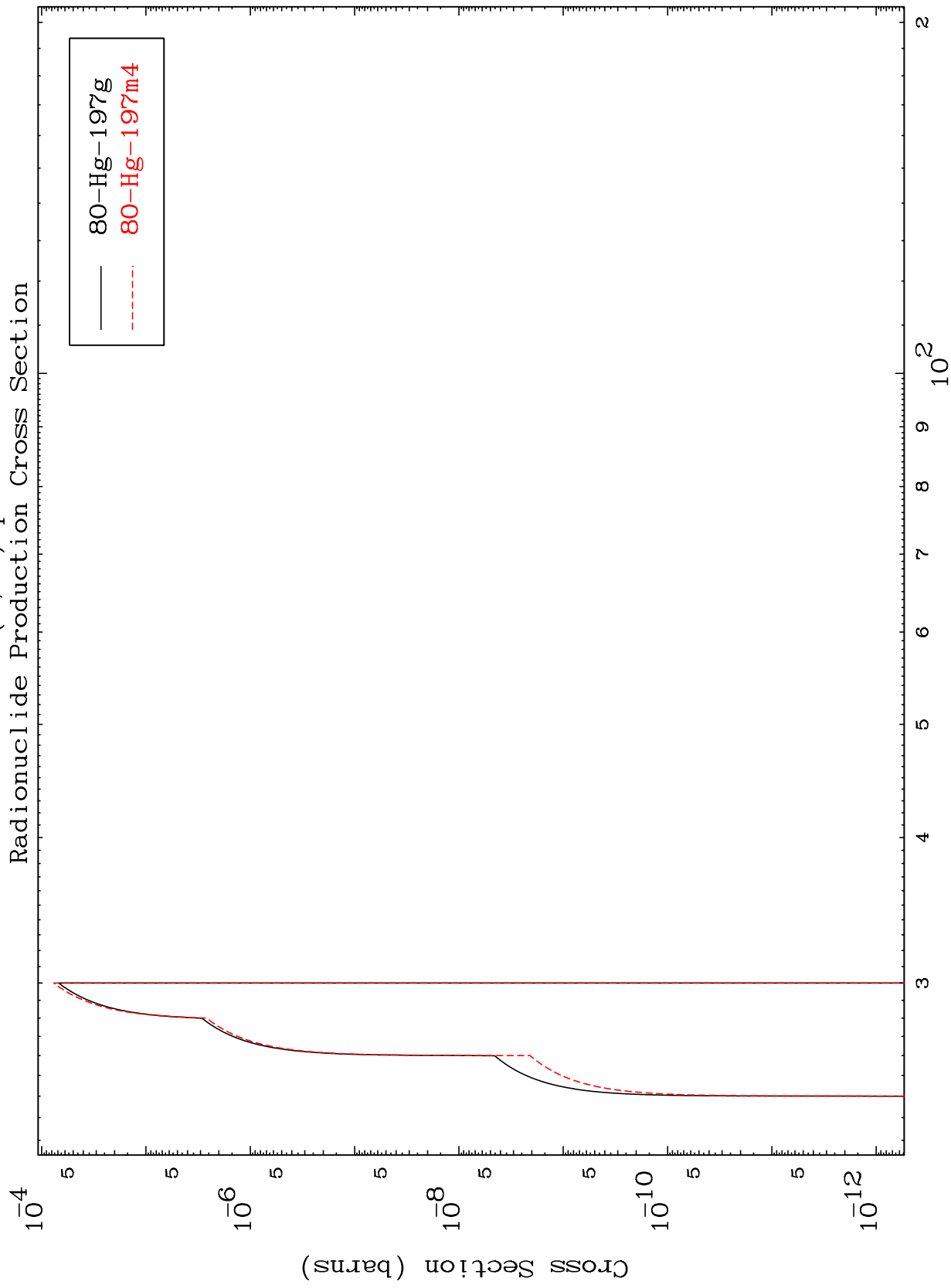
Incident Energy (MeV)

79-Au-196

MAT 7922

79-Au-196

$(\alpha, 2n)$ p
Radionuclide Production Cross Section



17

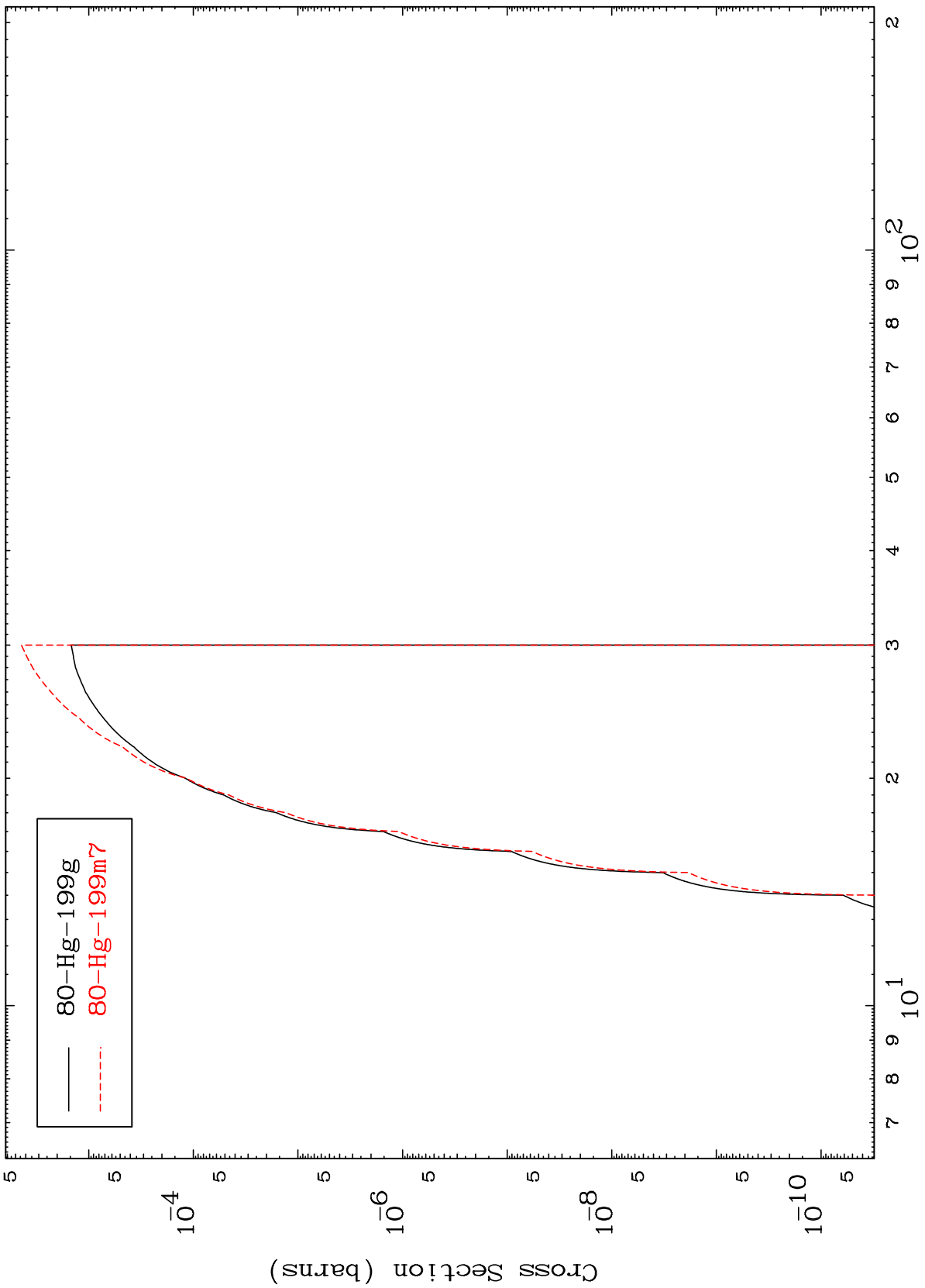
79-Au-196

Incident Energy (MeV)

MAT 7922

79-Au-196

(α, p)
Radionuclide Production Cross Section

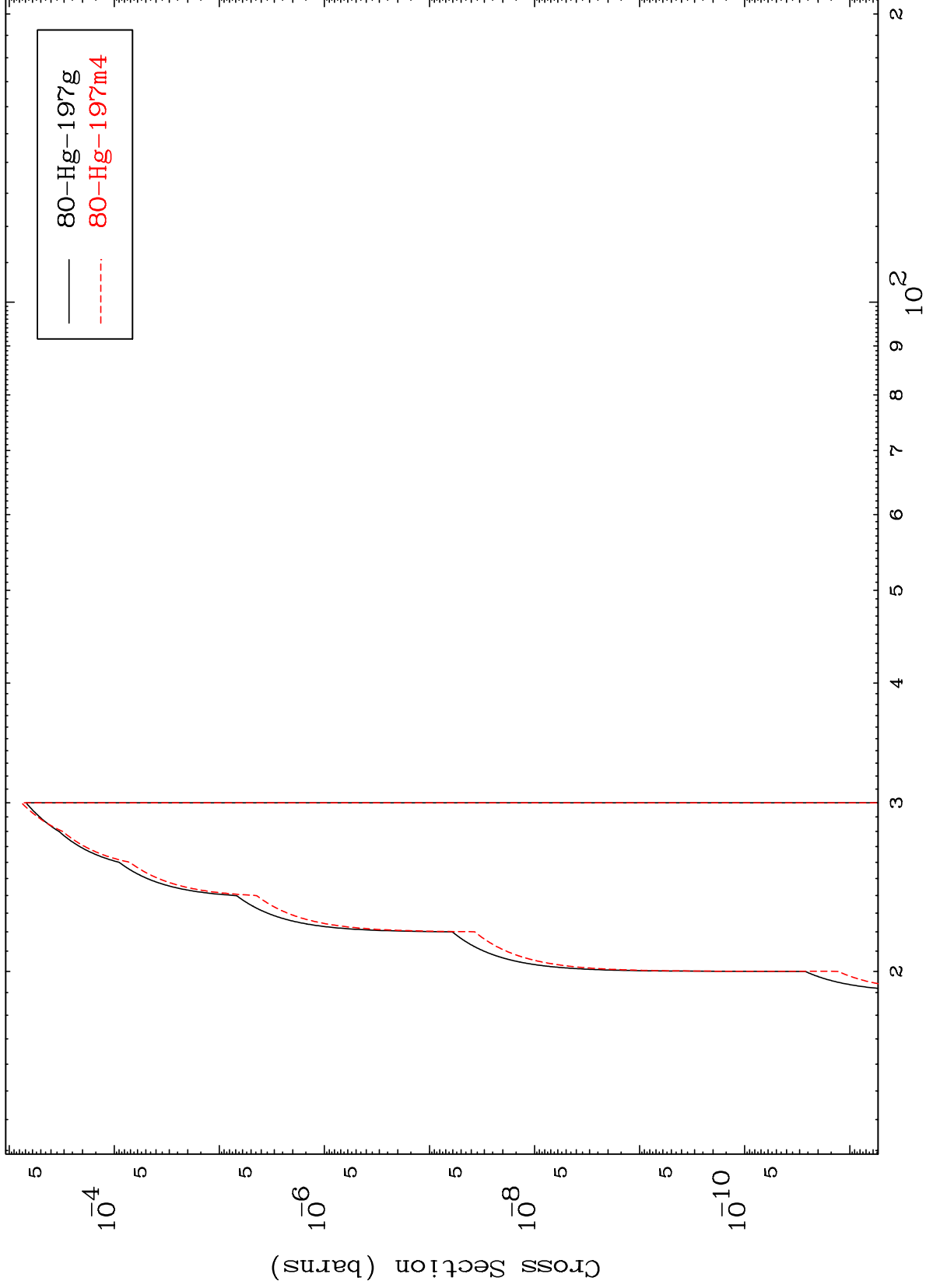


18

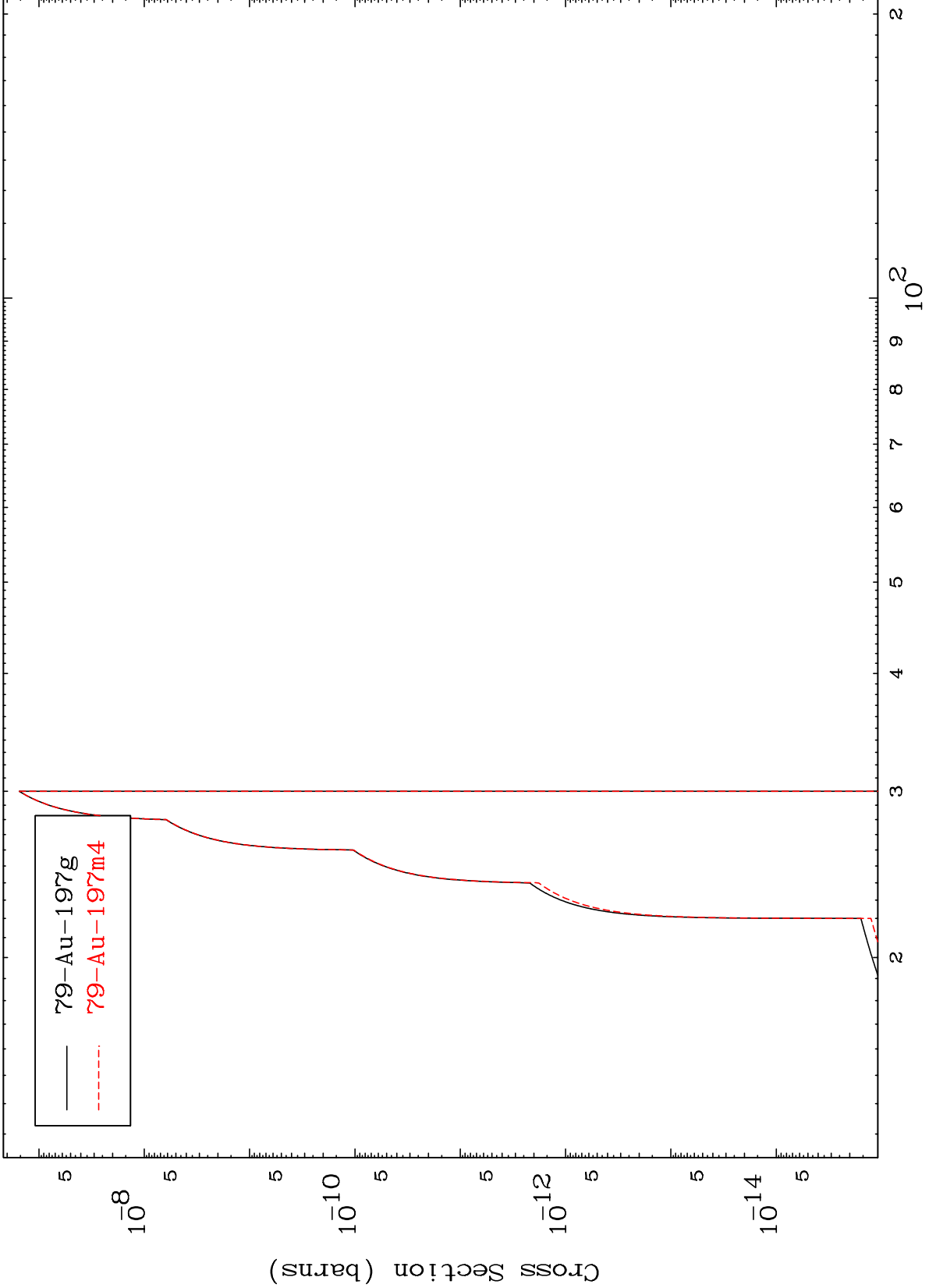
Incident Energy (MeV)

79-Au-196

Radionuclide Production Cross Section



Radionuclide Production Cross Section

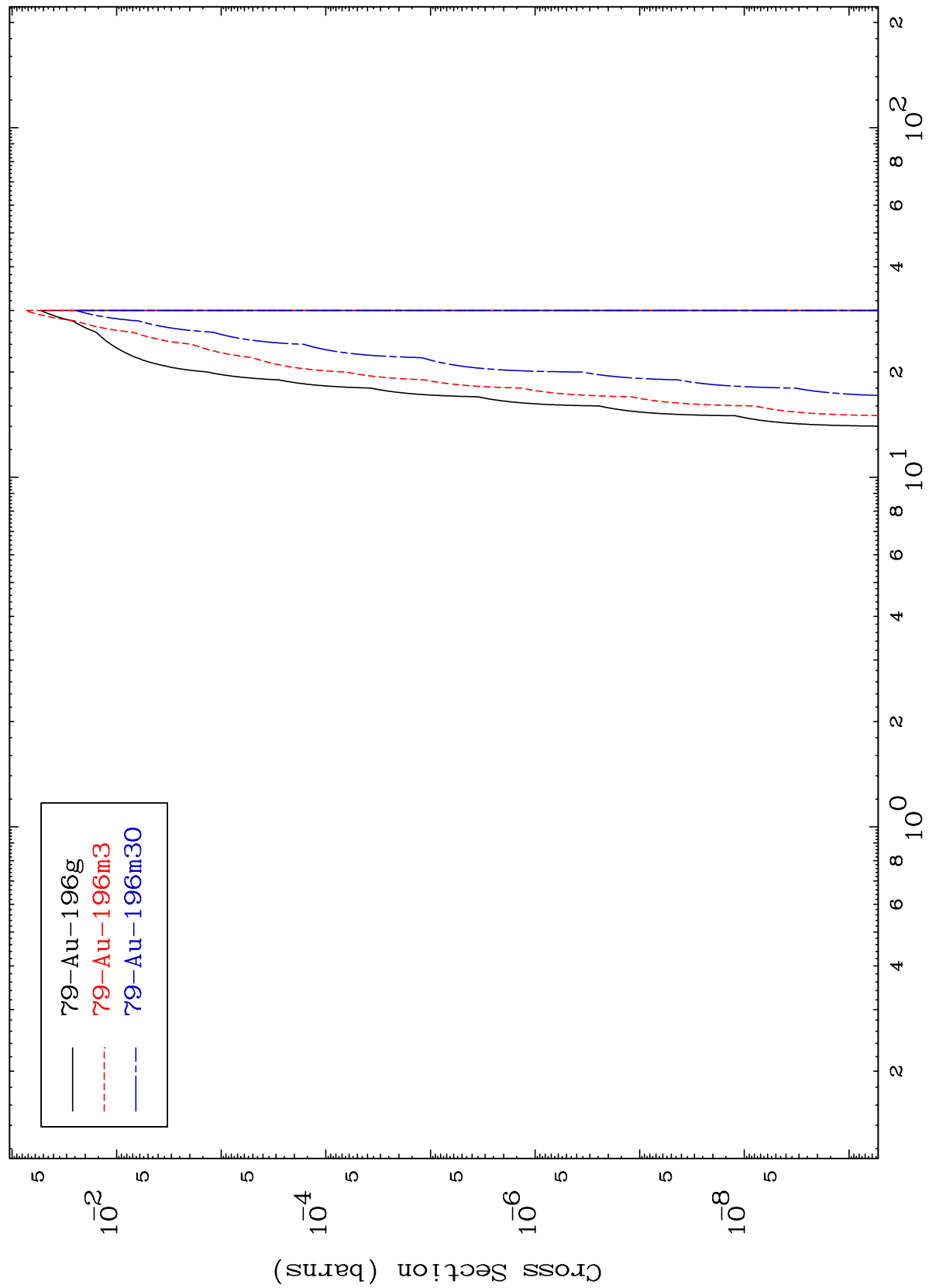


MAT 7922

(α, α)

⁷⁹Au-196

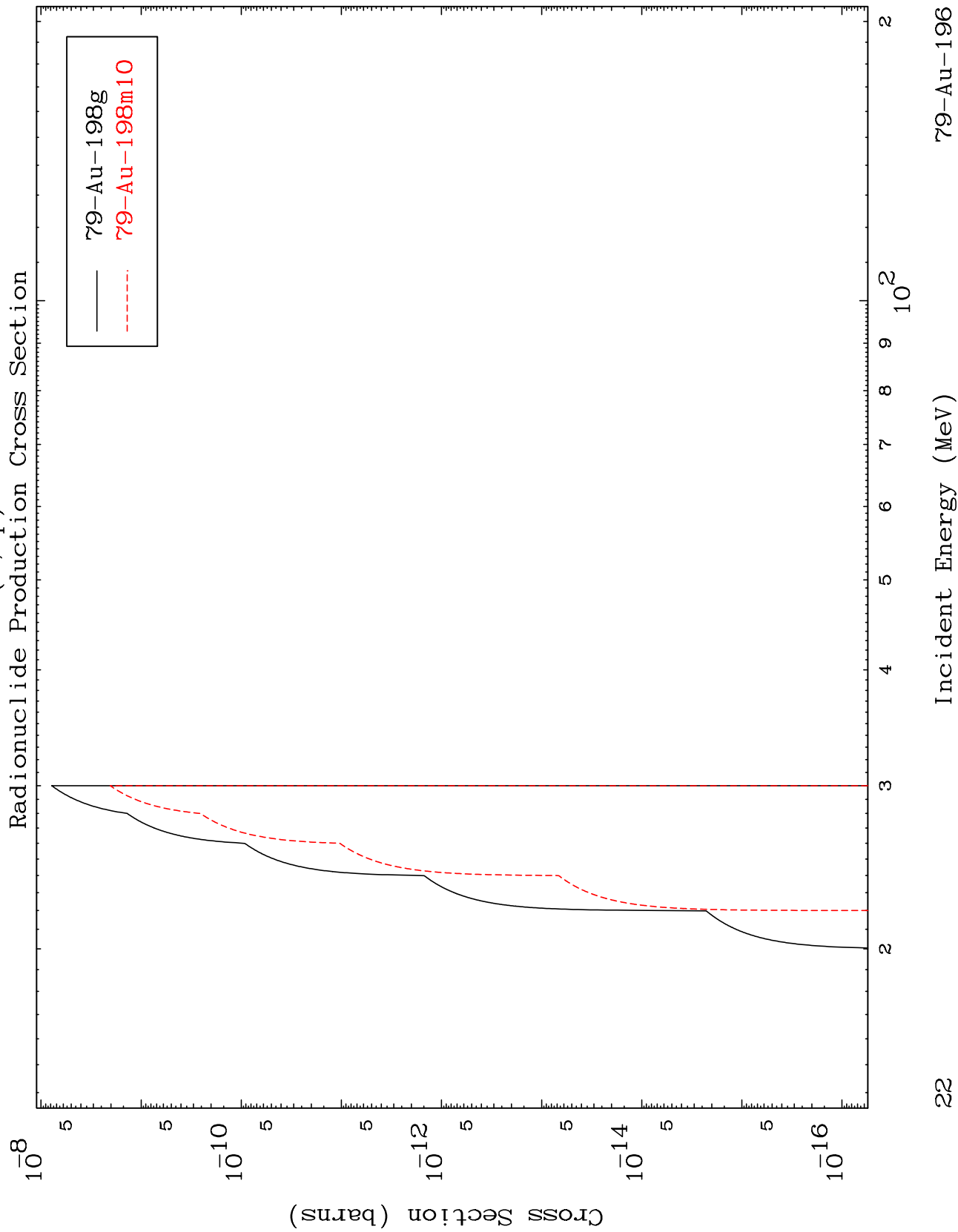
Radionuclide Production Cross Section



MAT 7922

79-Au-196

($\alpha, 2p$)



22

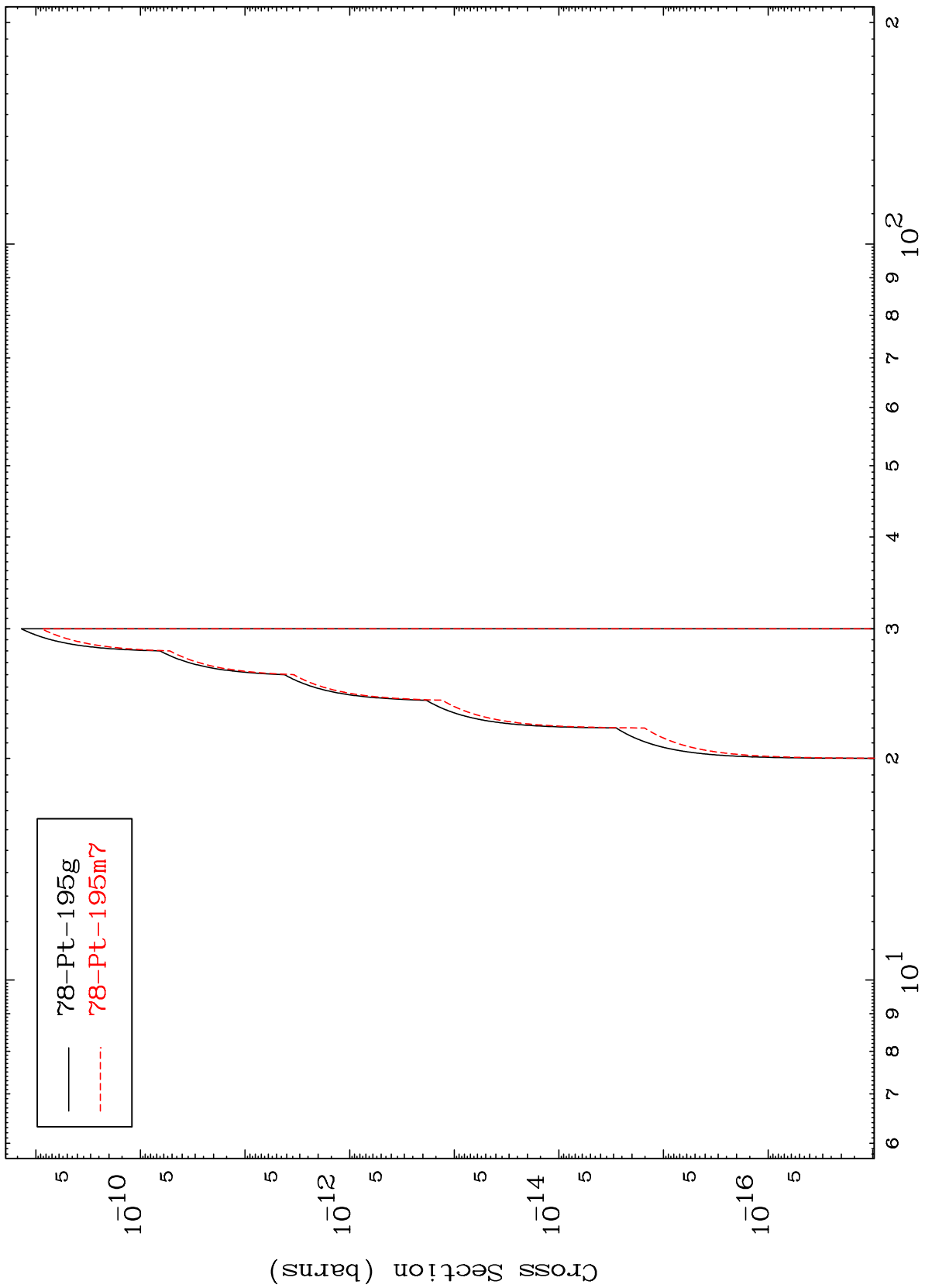
79-Au-196

MAT 7922

$(\alpha, p) \alpha$

$^{79}\text{Au-196}$

Radionuclide Production Cross Section



23

Incident Energy (MeV)

$^{79}\text{Au-196}$