

Program EVALPLOT
(Version 2018-1)

by

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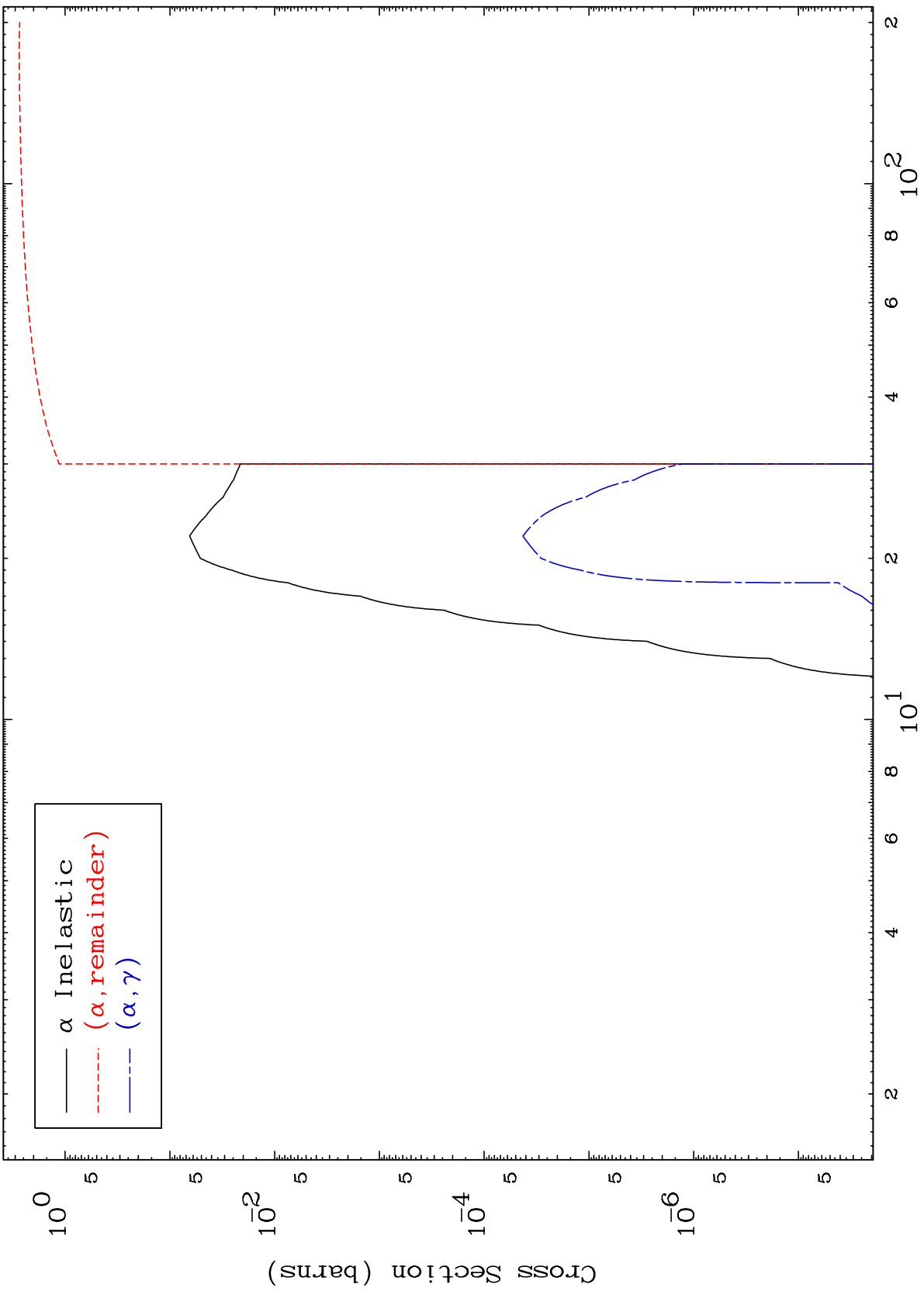
Press Mouse Button to Start

MAT 7923

α Major

79-Au-196

0 Kelvin Cross Sections

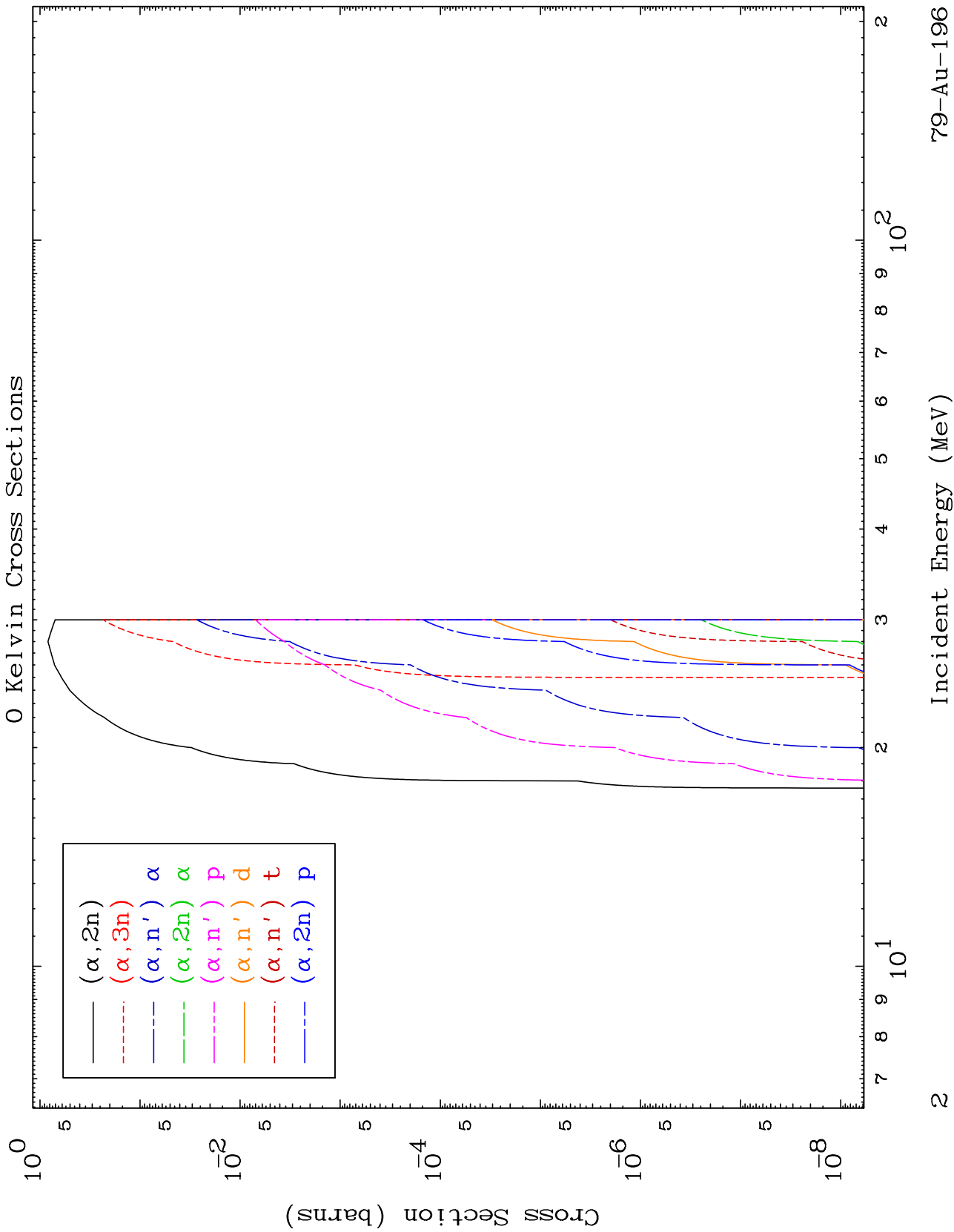


α Inelastic
 $(\alpha, \text{remainder})$
 (α, γ)

MAT 7923

α Neutron Production
0 Kelvin Cross Sections

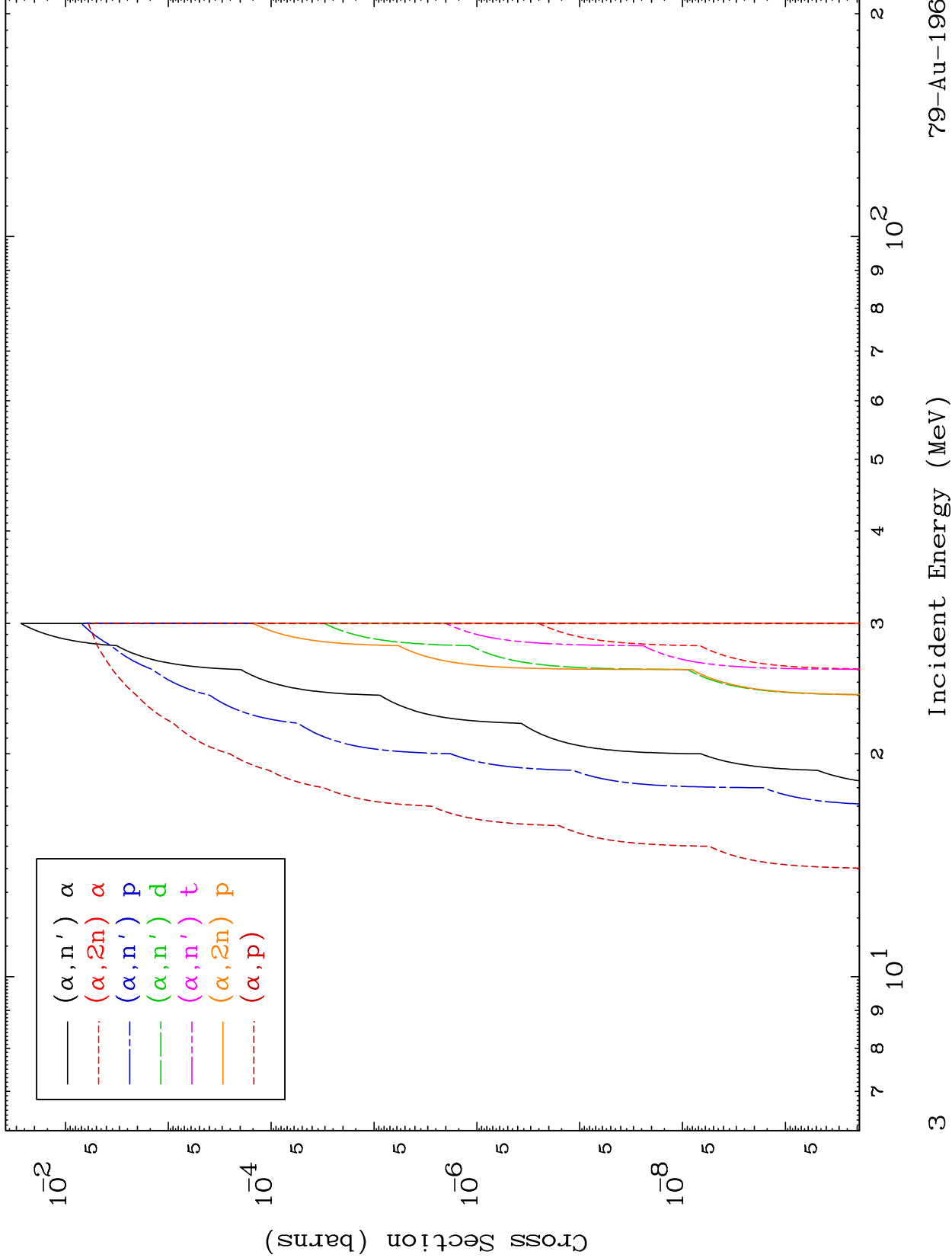
79-Au-196



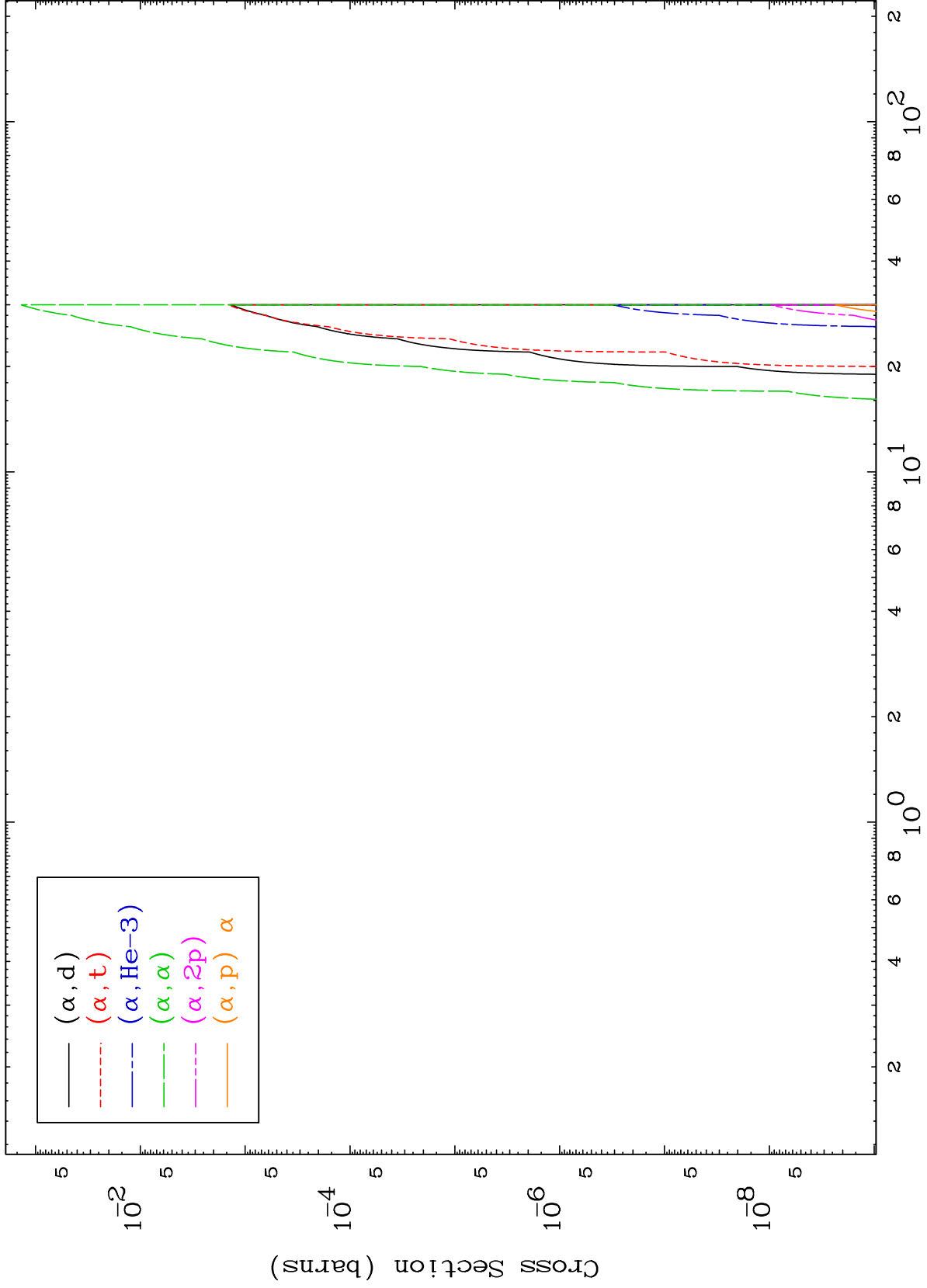
2

Incident Energy (MeV)

79-Au-196



0 Kelvin Cross Sections

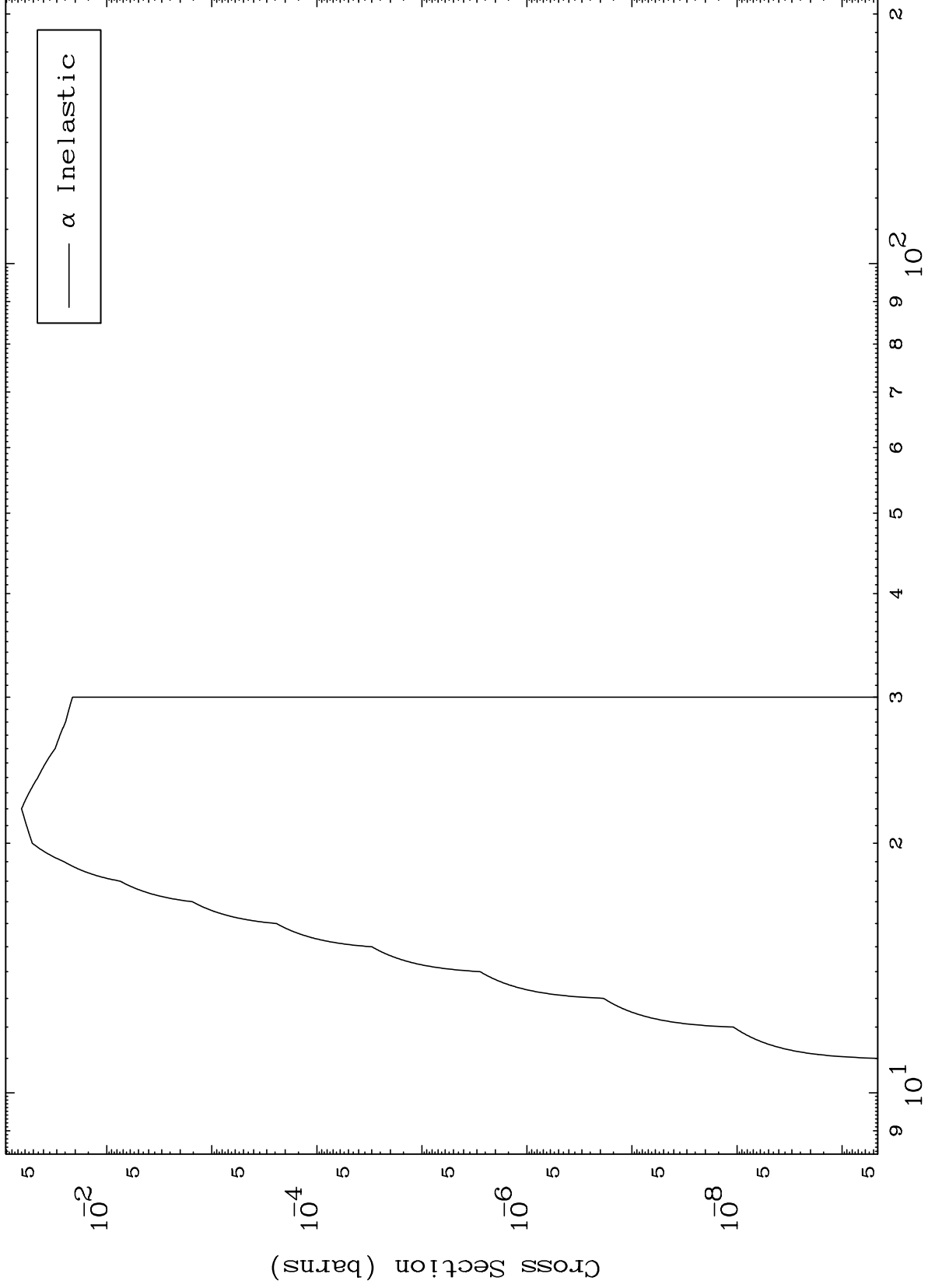


MAT 7923

(α, n') Level

79-Au-196

0 Kelvin Cross Sections



Incident Energy (MeV)

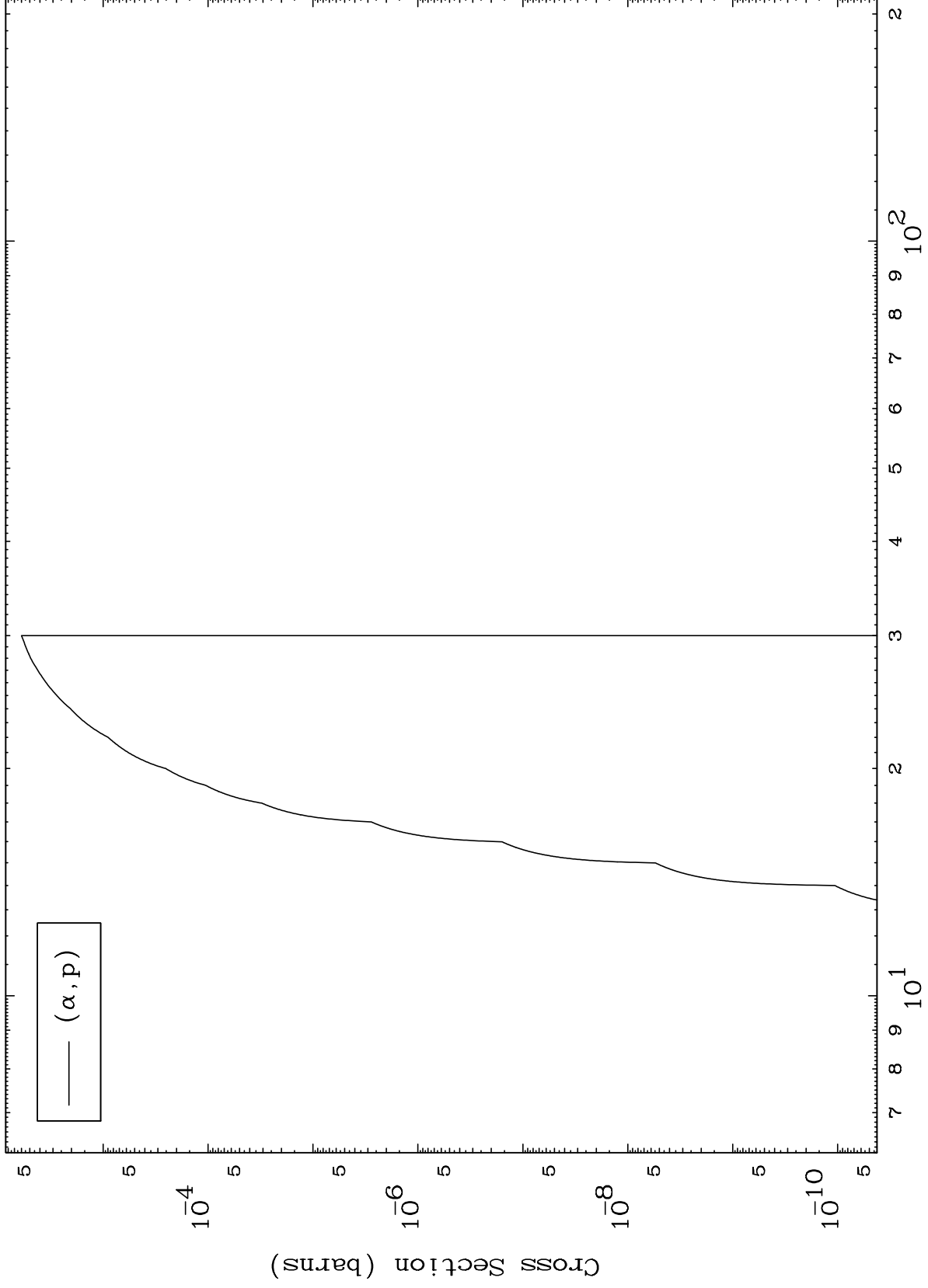
79-Au-196

5

MAT 7923

(α, p) Levels
0 Kelvin Cross Sections

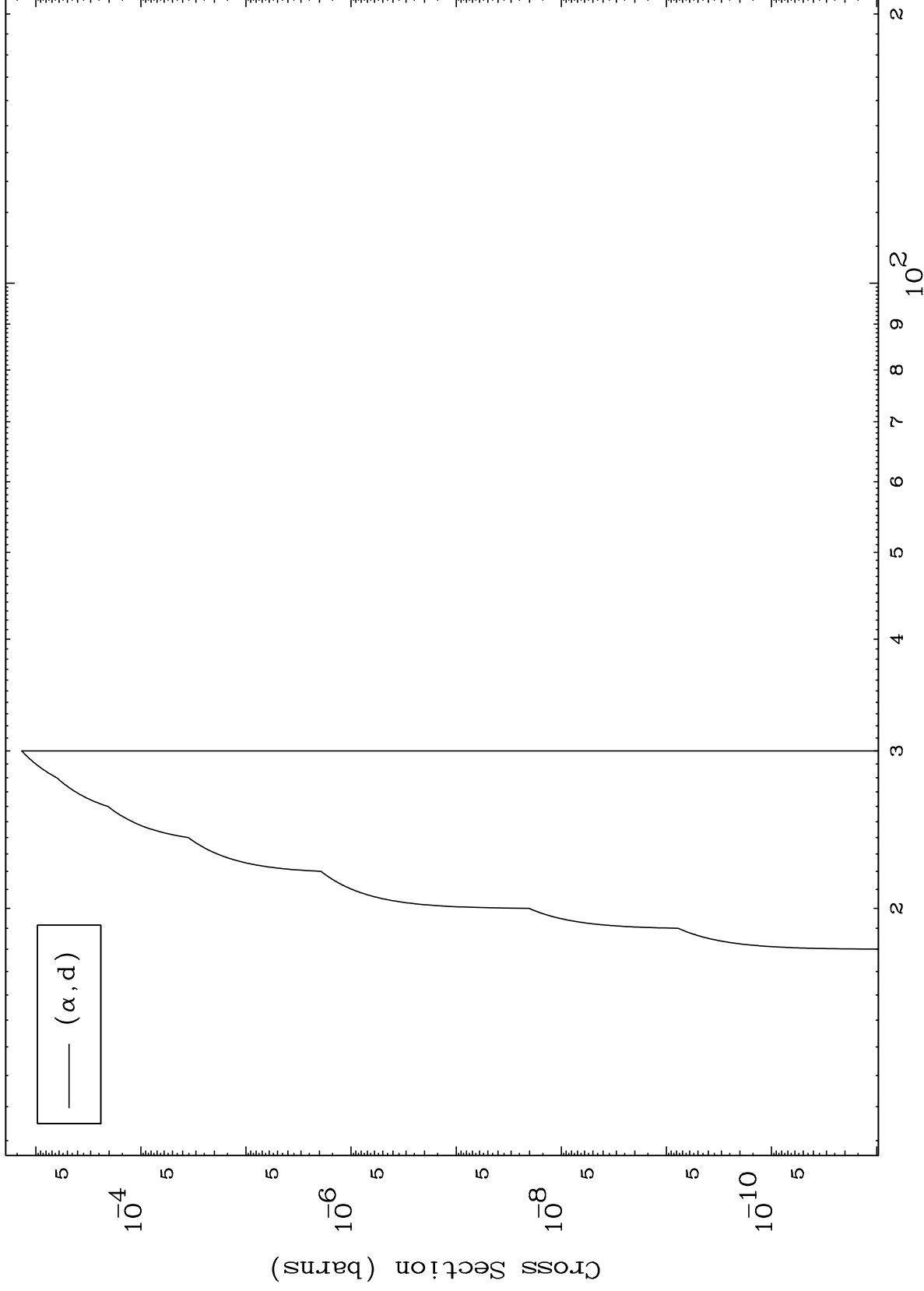
79-Au-196

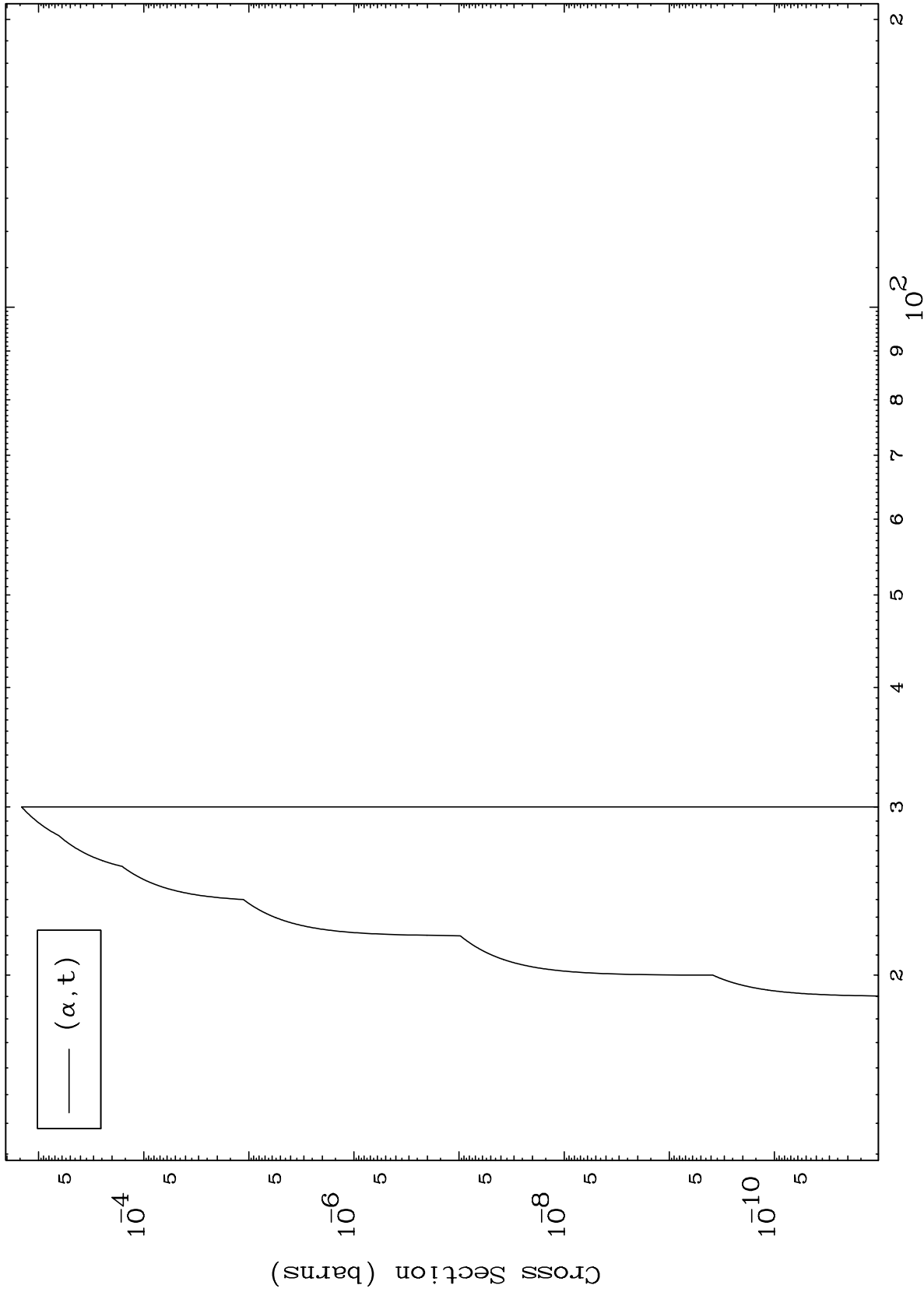


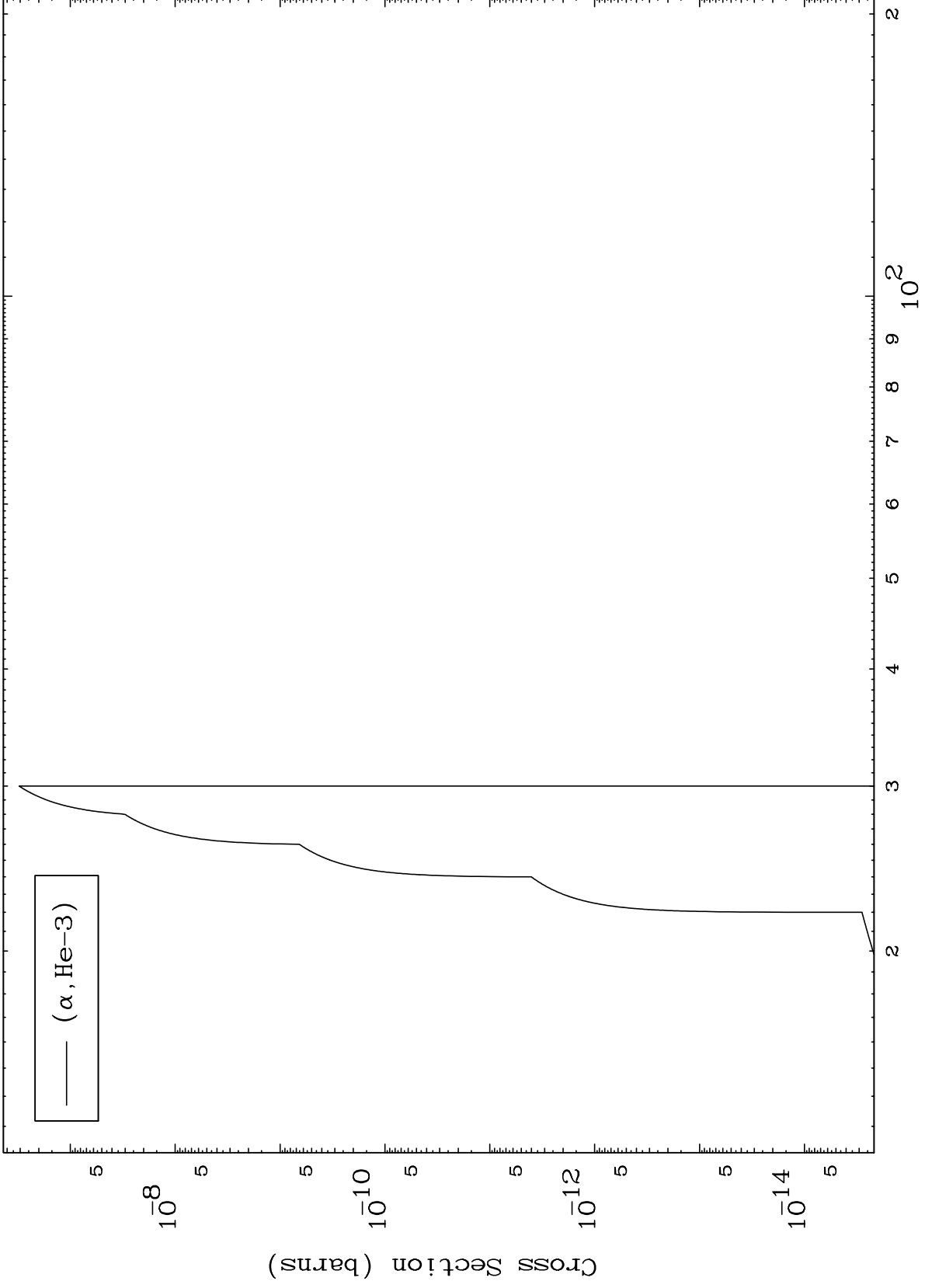
6

Incident Energy (MeV)

79-Au-196



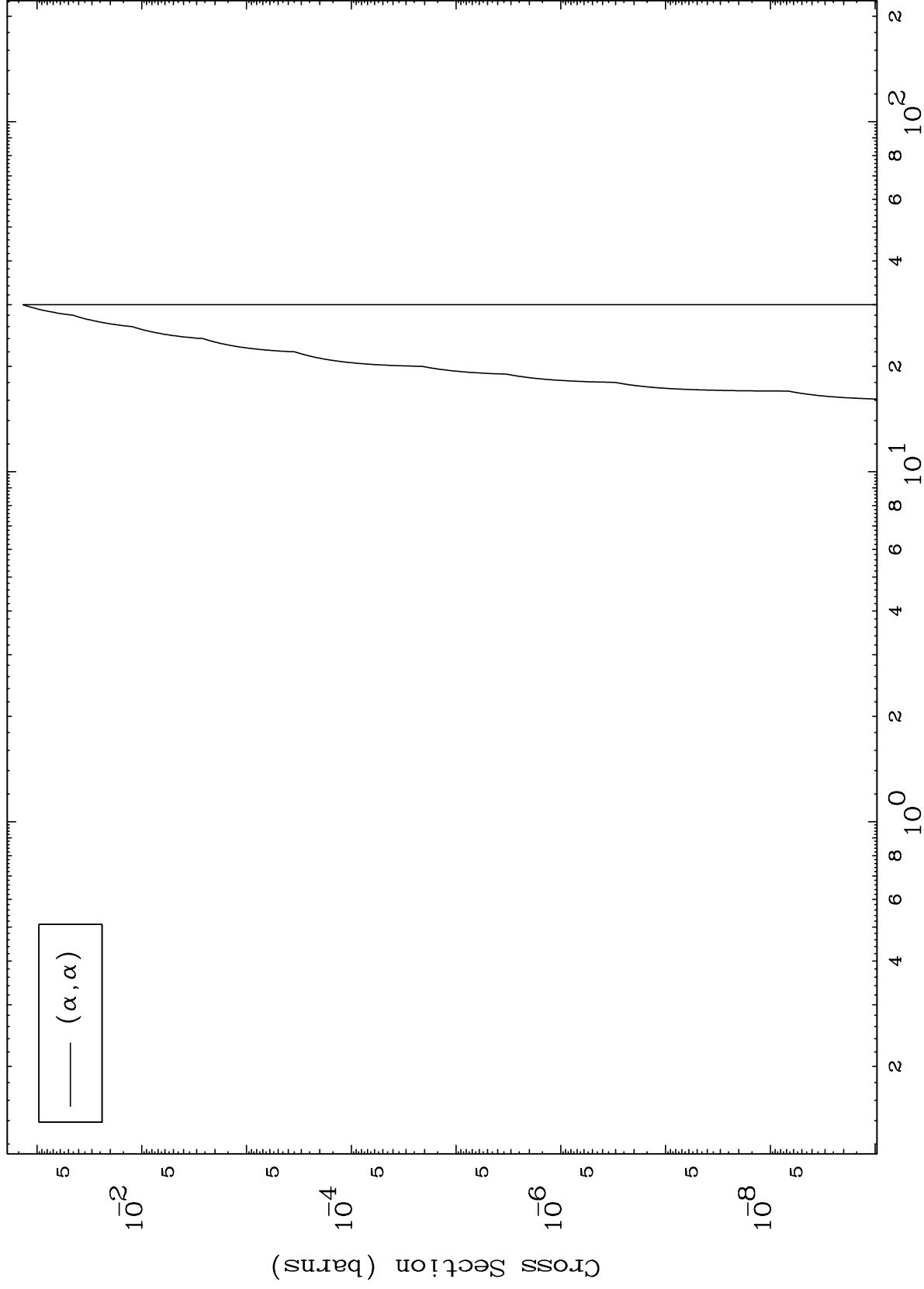




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(α, α) Levels
0 Kelvin Cross Sections

79-Au-196



10

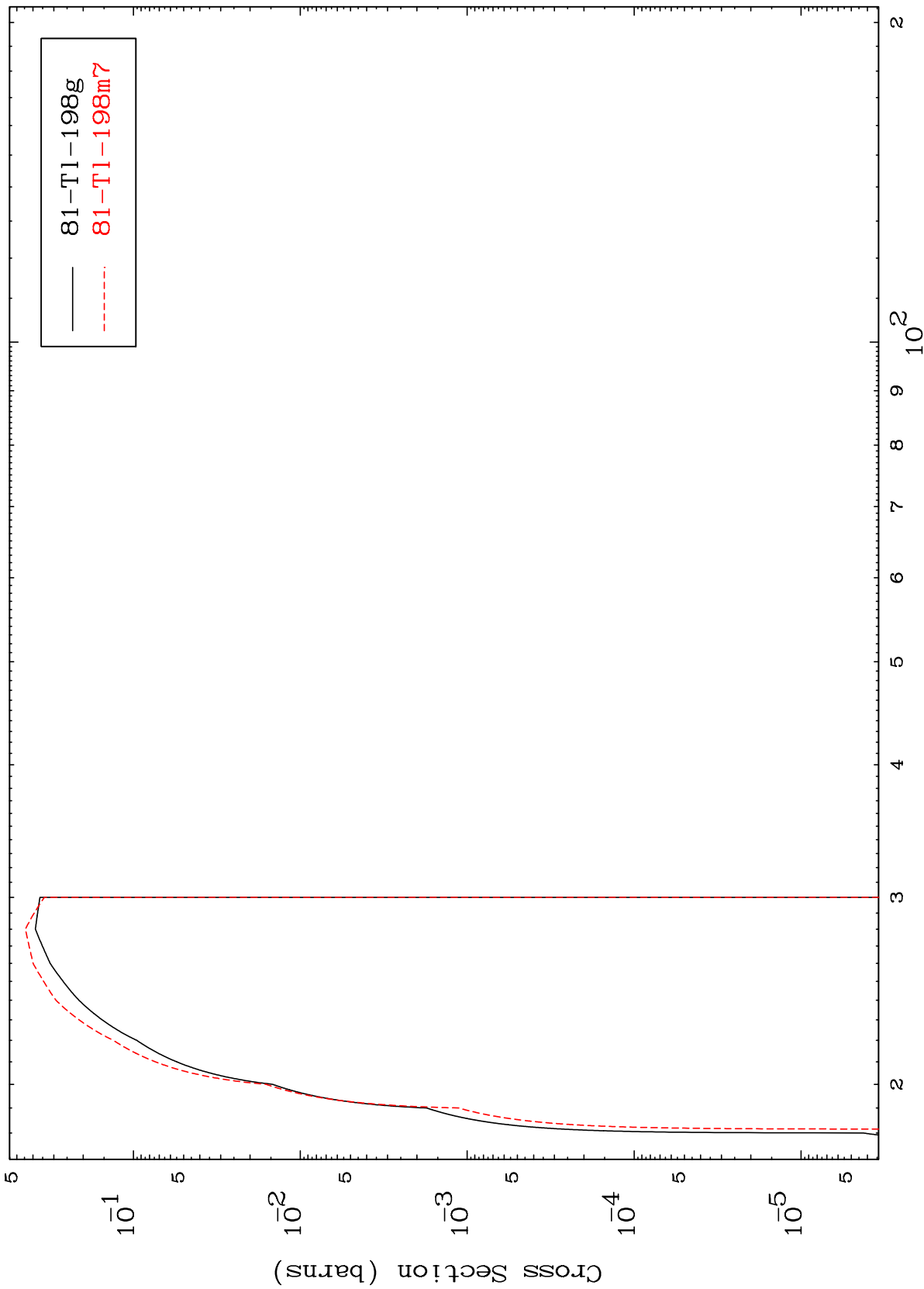
Incident Energy (MeV)

79-Au-196

MAT 7923

79-Au-196

$(\alpha, 2n)$
Radionuclide Production Cross Section



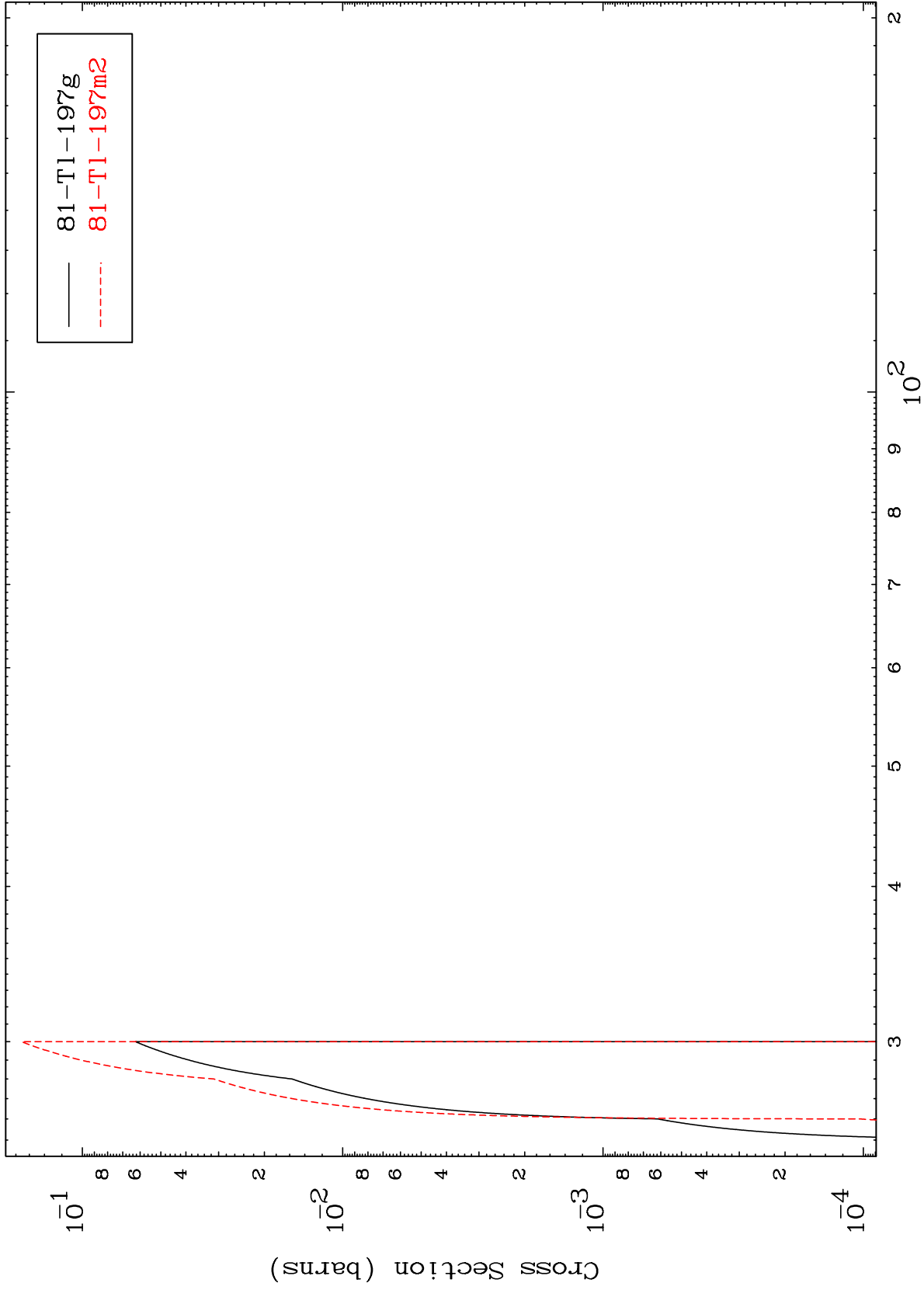
81-Tl-198g
81-Tl-198m7

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($\alpha, 3n$)

79-Au-196

Radionuclide Production Cross Section



12

Incident Energy (MeV)

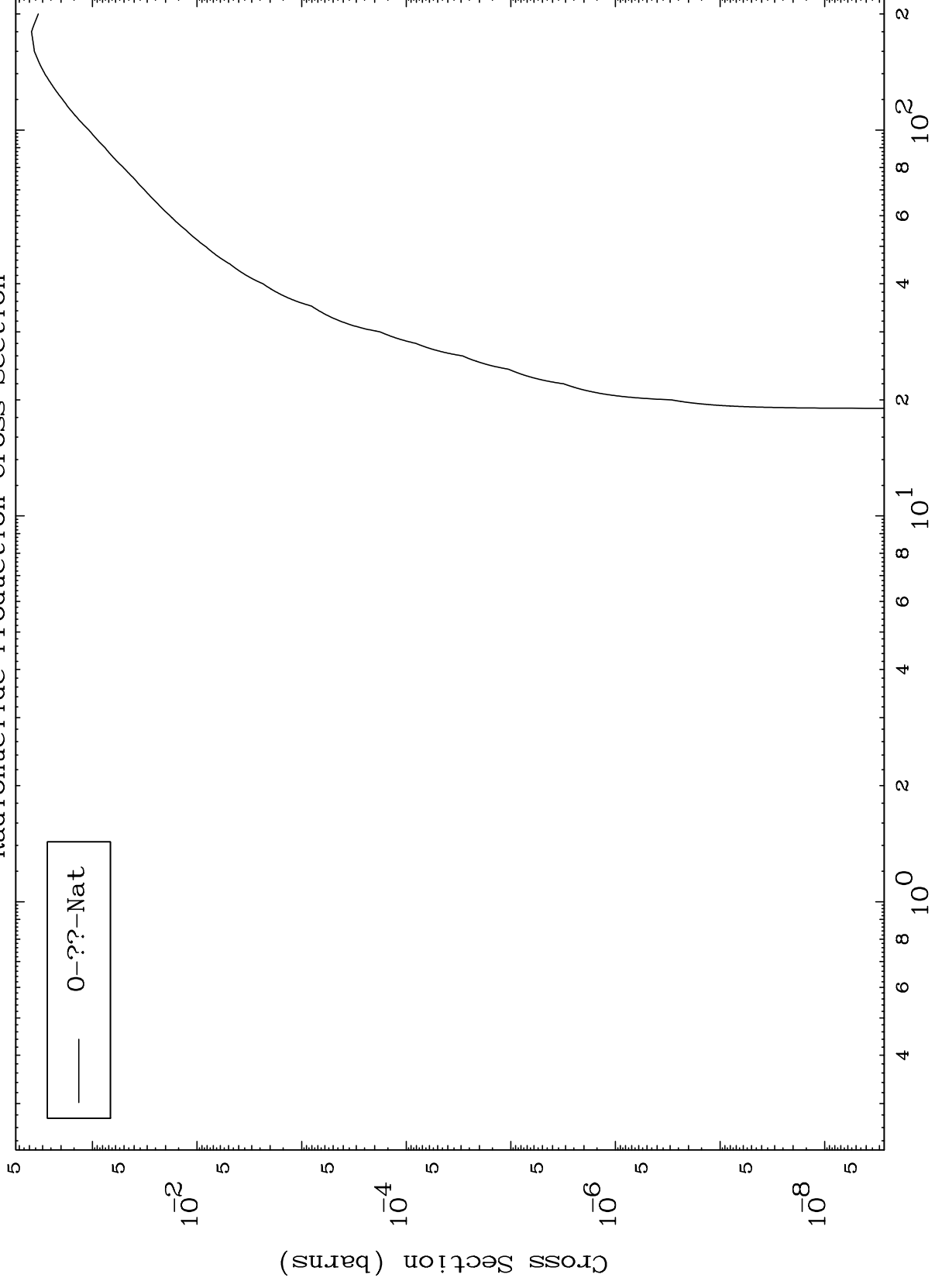
79-Au-196

MAT 7923

α Fission

79-Au-196

Radionuclide Production Cross Section

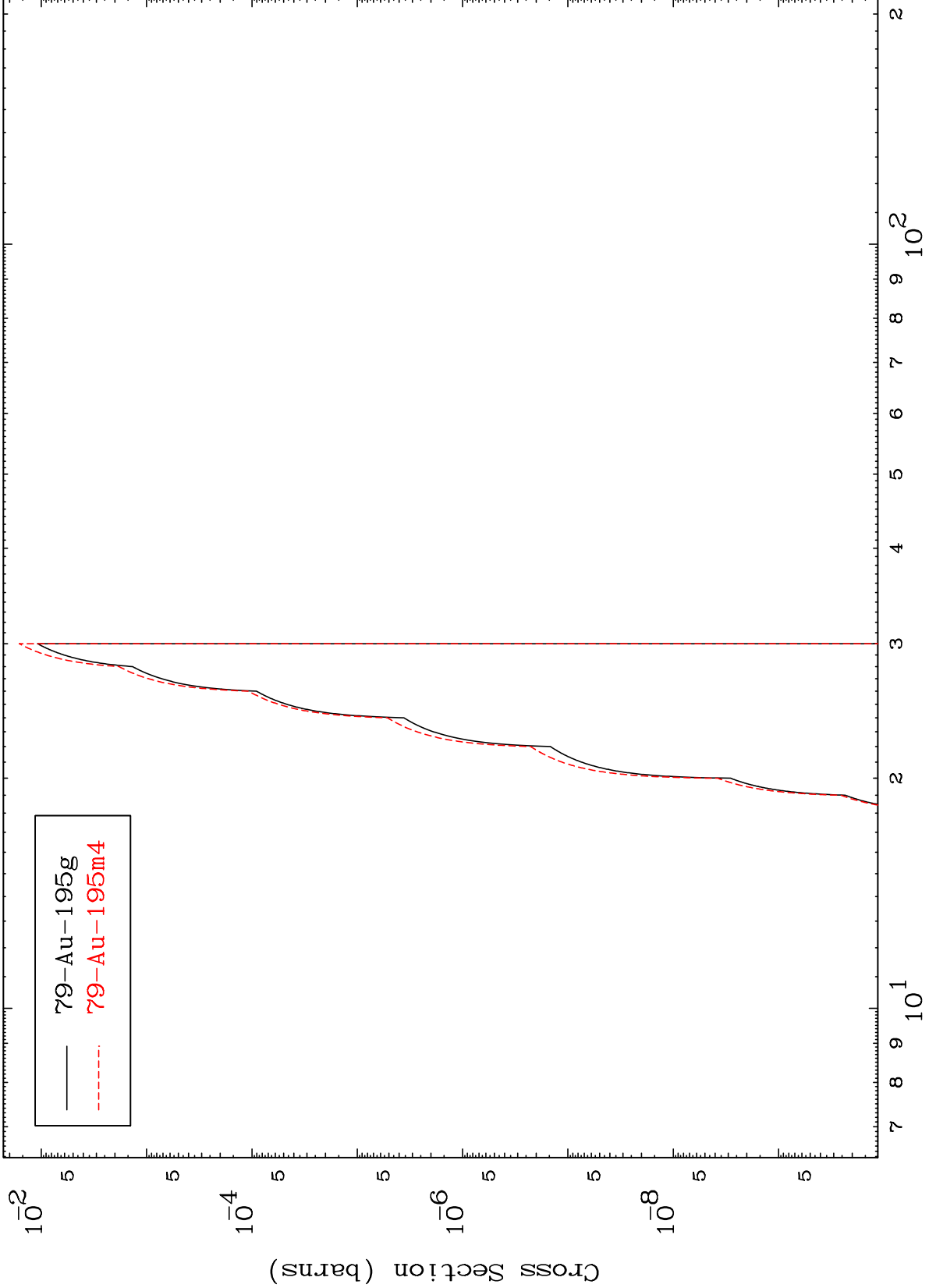


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79-Au-196

(α, n') α

Radionuclide Production Cross Section



79-Au-195g
79-Au-195m4

79-Au-196

Incident Energy (MeV)

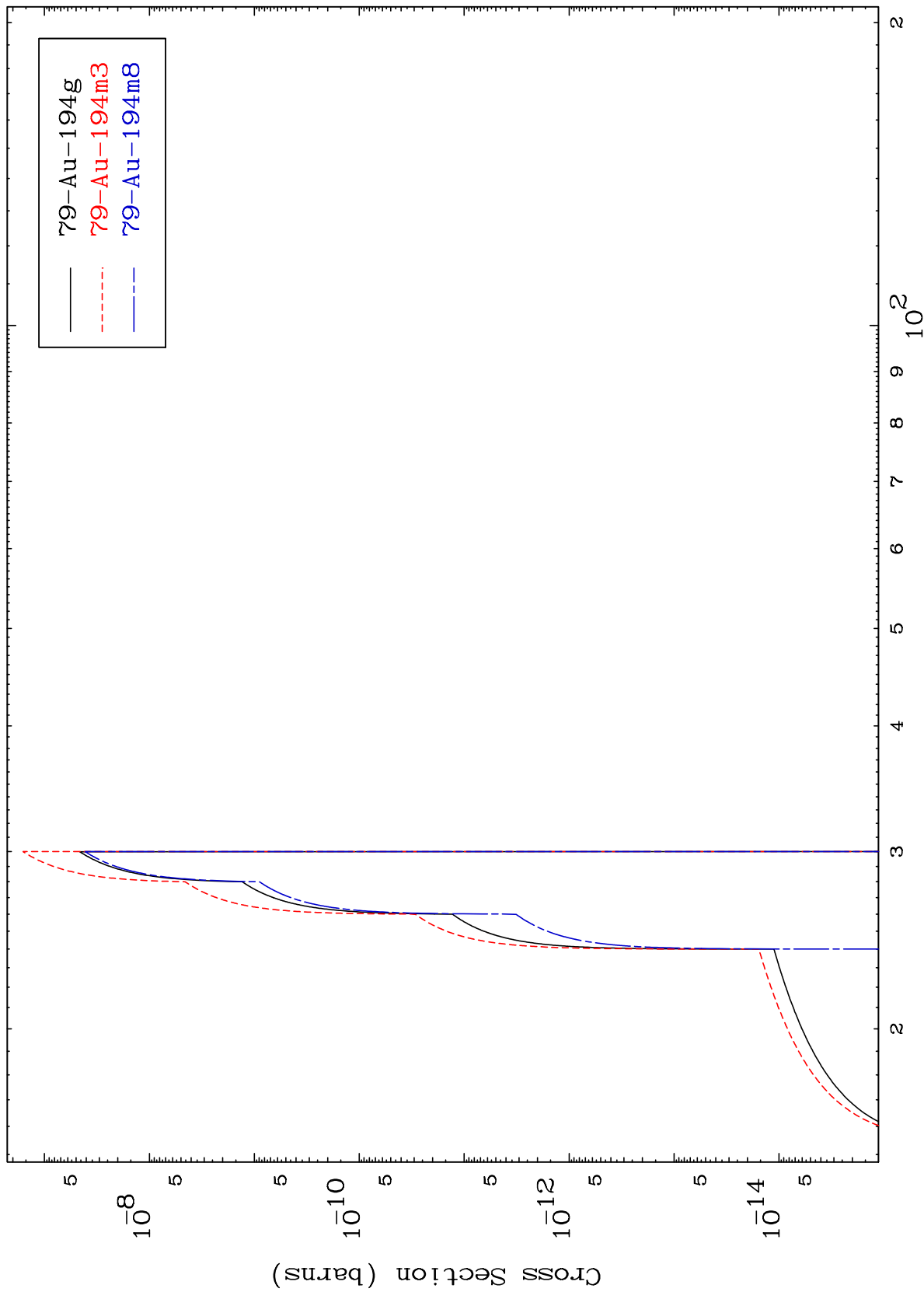
14

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$(\alpha, 2n) \alpha$

$^{79}\text{Au-196}$

Radionuclide Production Cross Section

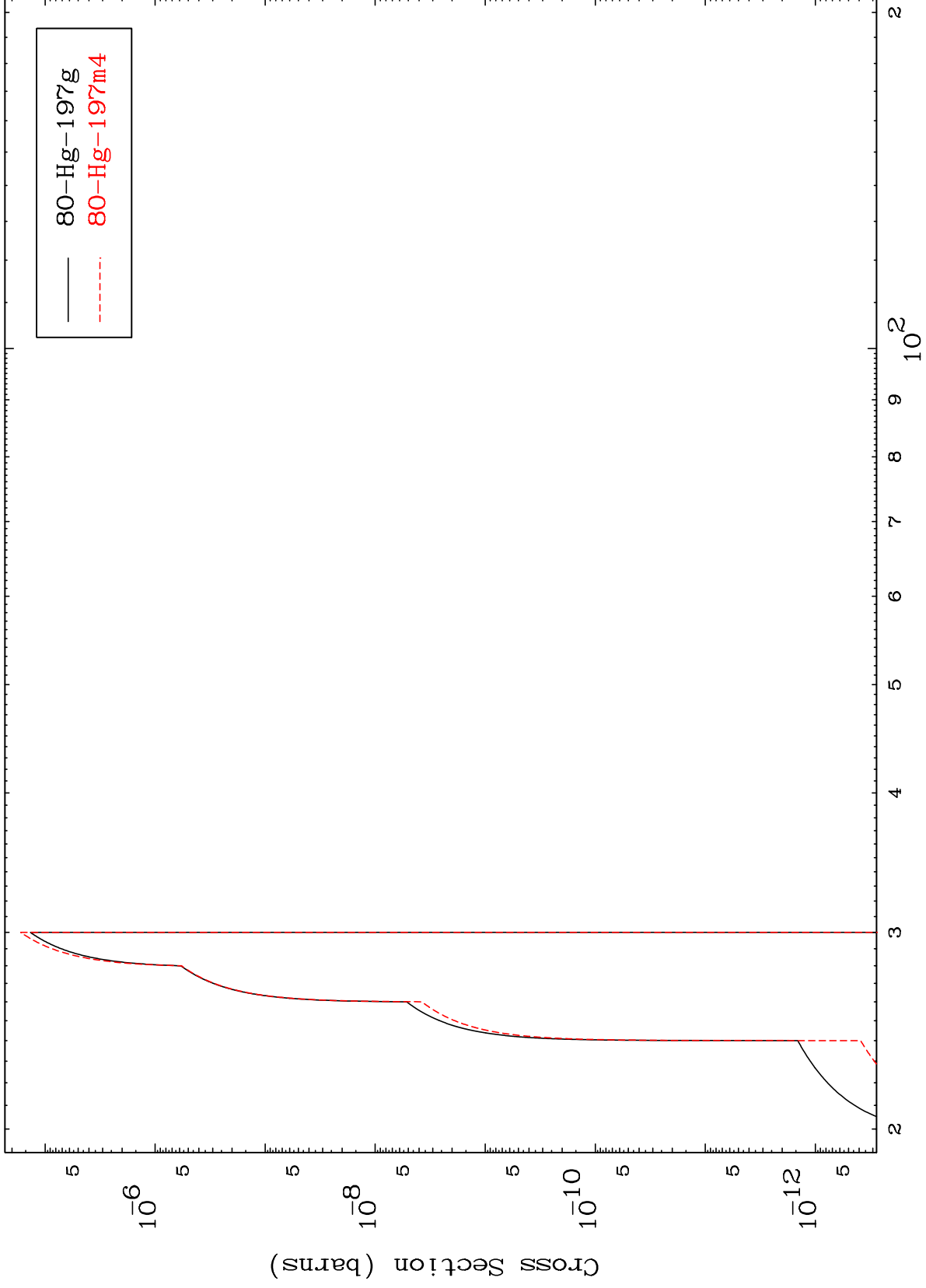


15

Incident Energy (MeV)

$^{79}\text{Au-196}$

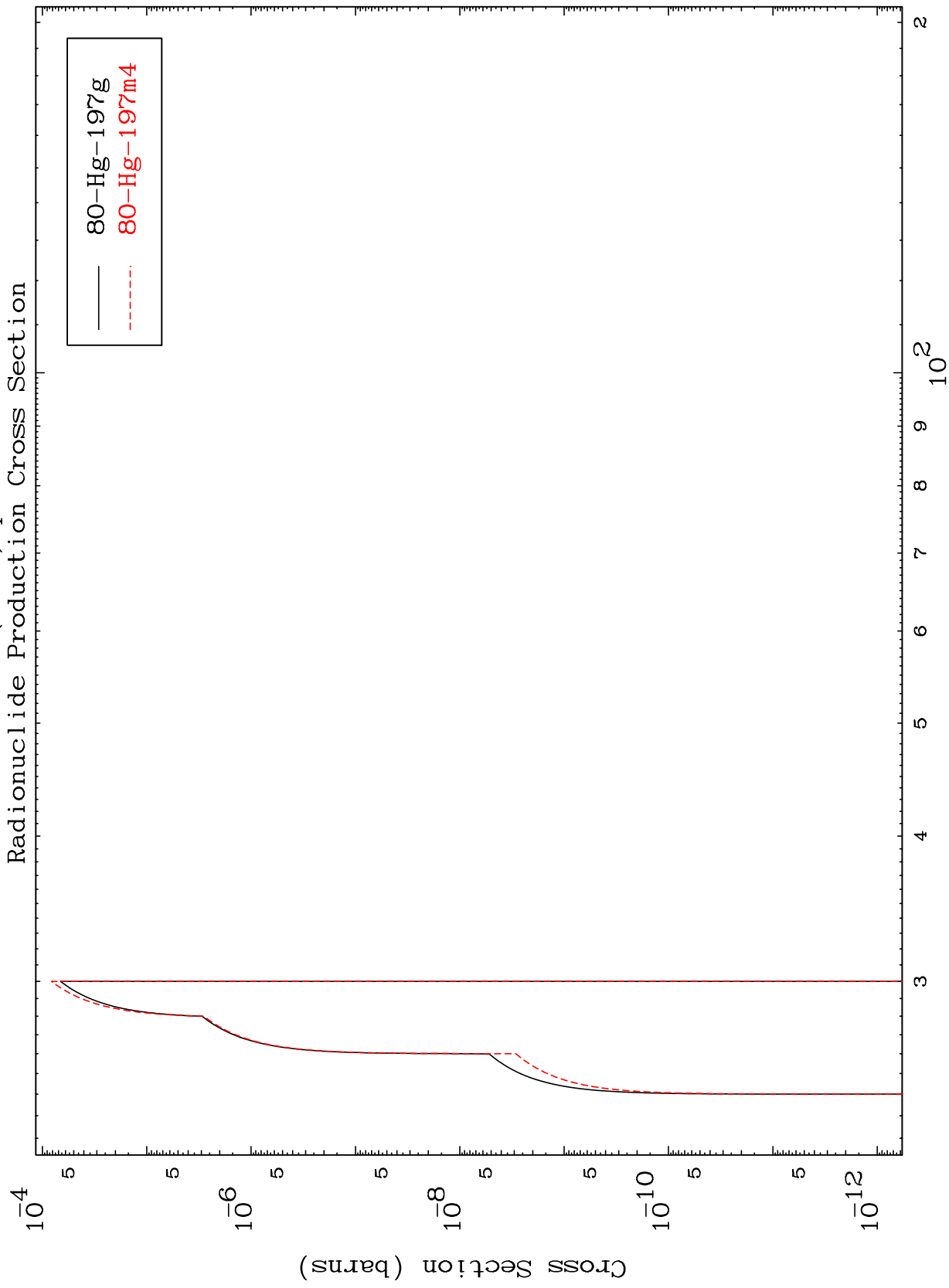
Radionuclide Production Cross Section



MAT 7923

79-Au-196

$(\alpha, 2n)$ p
Radionuclide Production Cross Section



79-Au-196

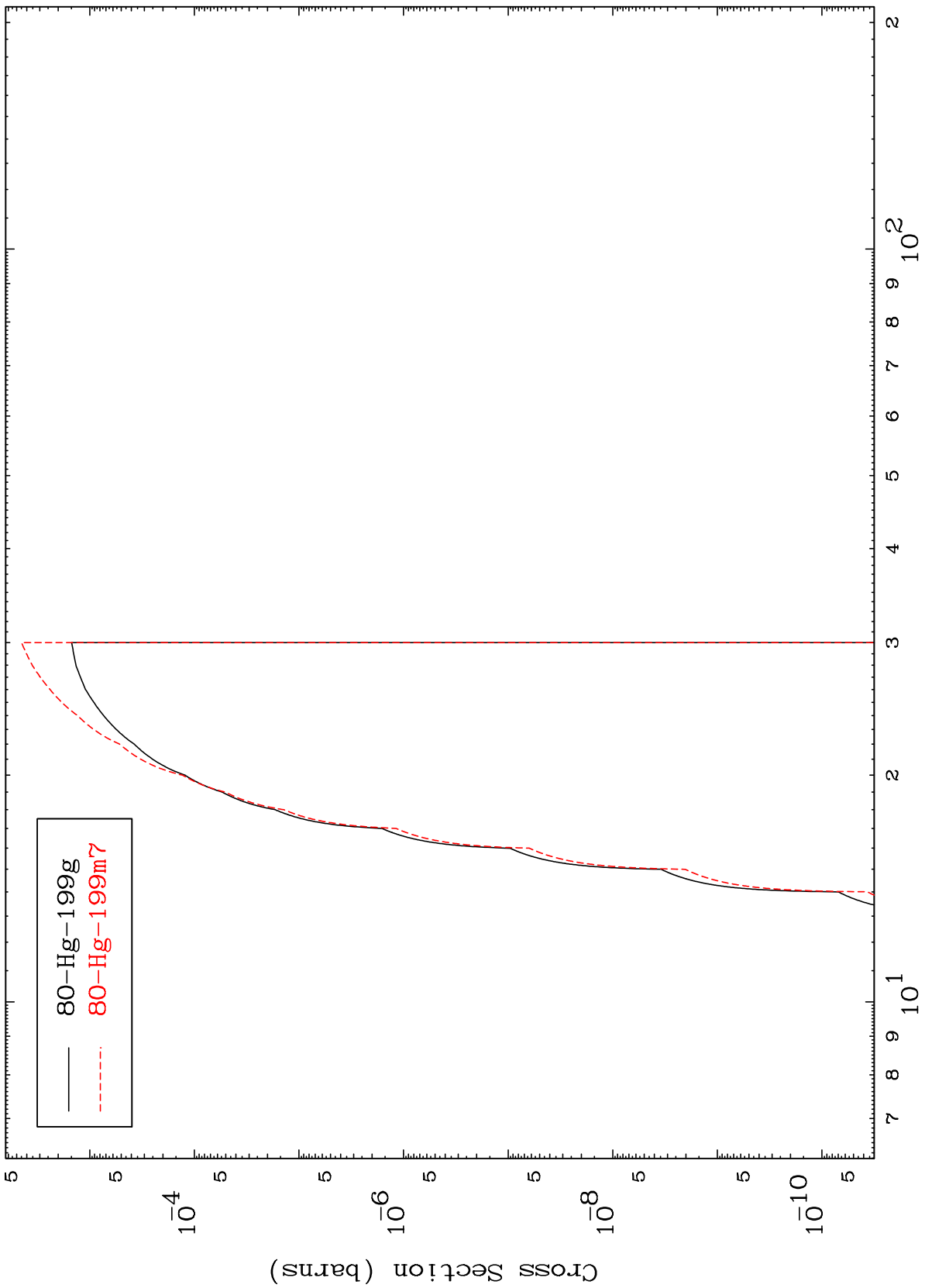
Incident Energy (MeV)

17

MAT 7923

79-Au-196

(α, p)
Radionuclide Production Cross Section



18

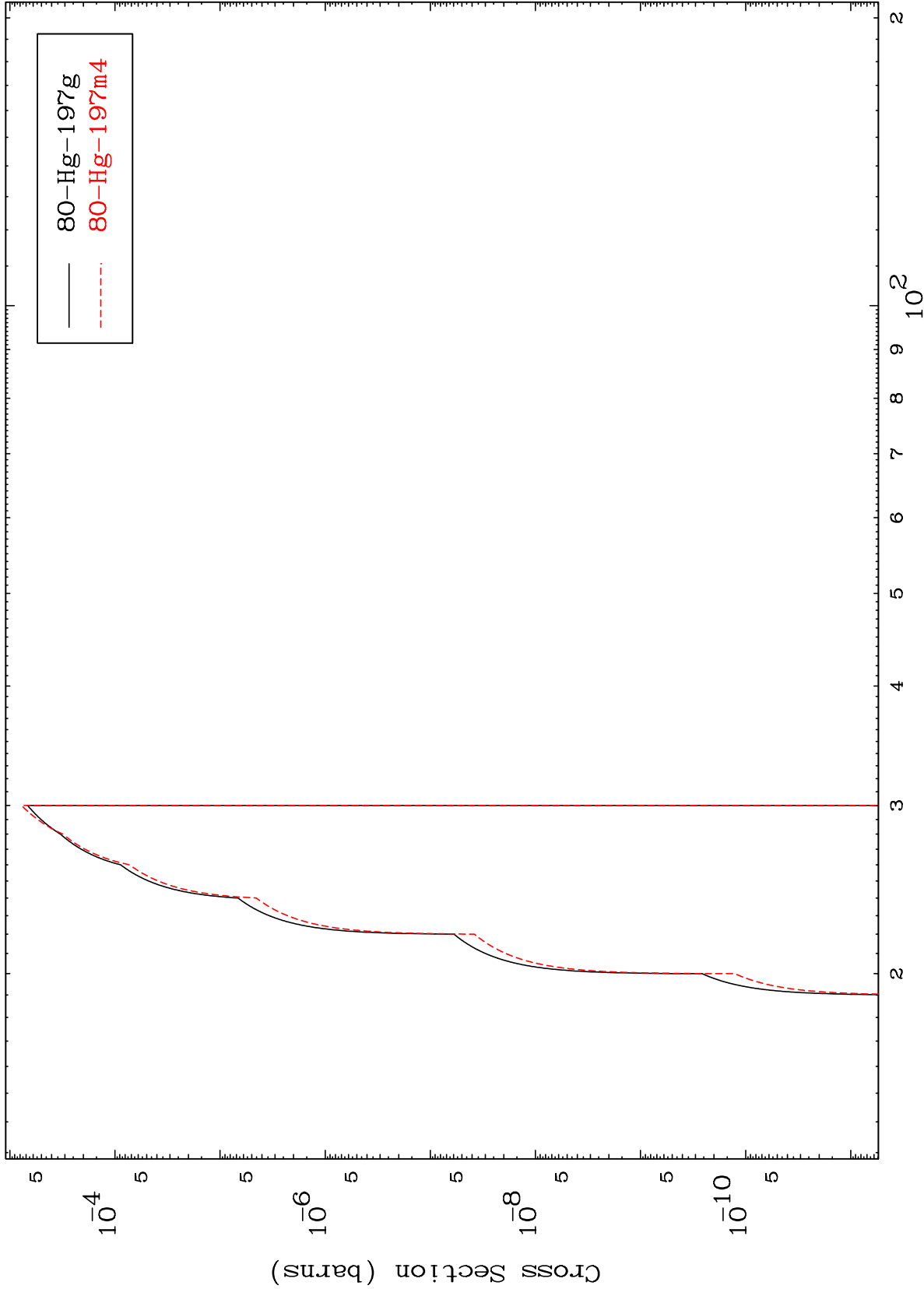
Incident Energy (MeV)

79-Au-196

MAT 7923

79-Au-196

(α, t)
Radionuclide Production Cross Section

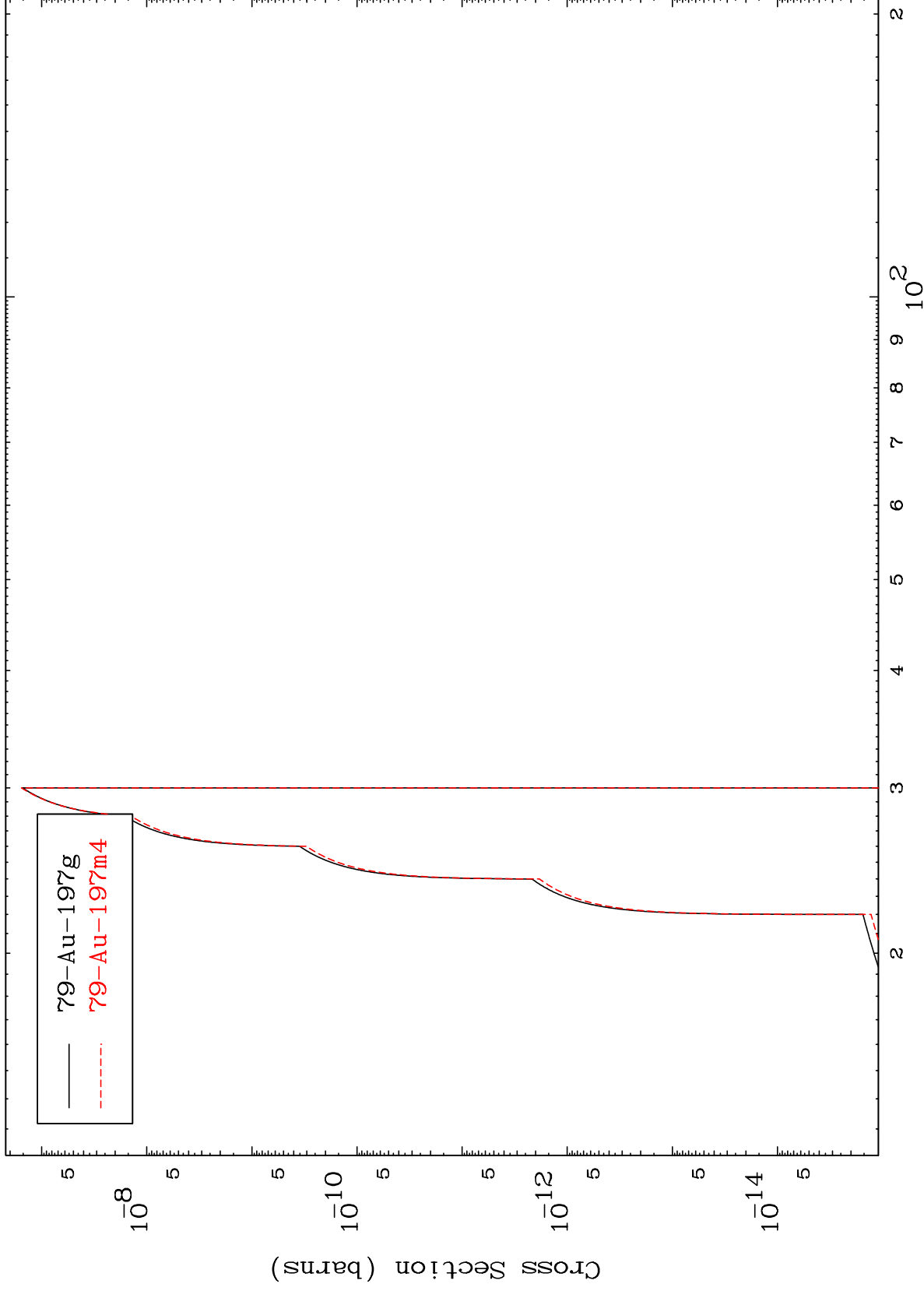


19

79-Au-196

Incident Energy (MeV)

Radionuclide Production Cross Section

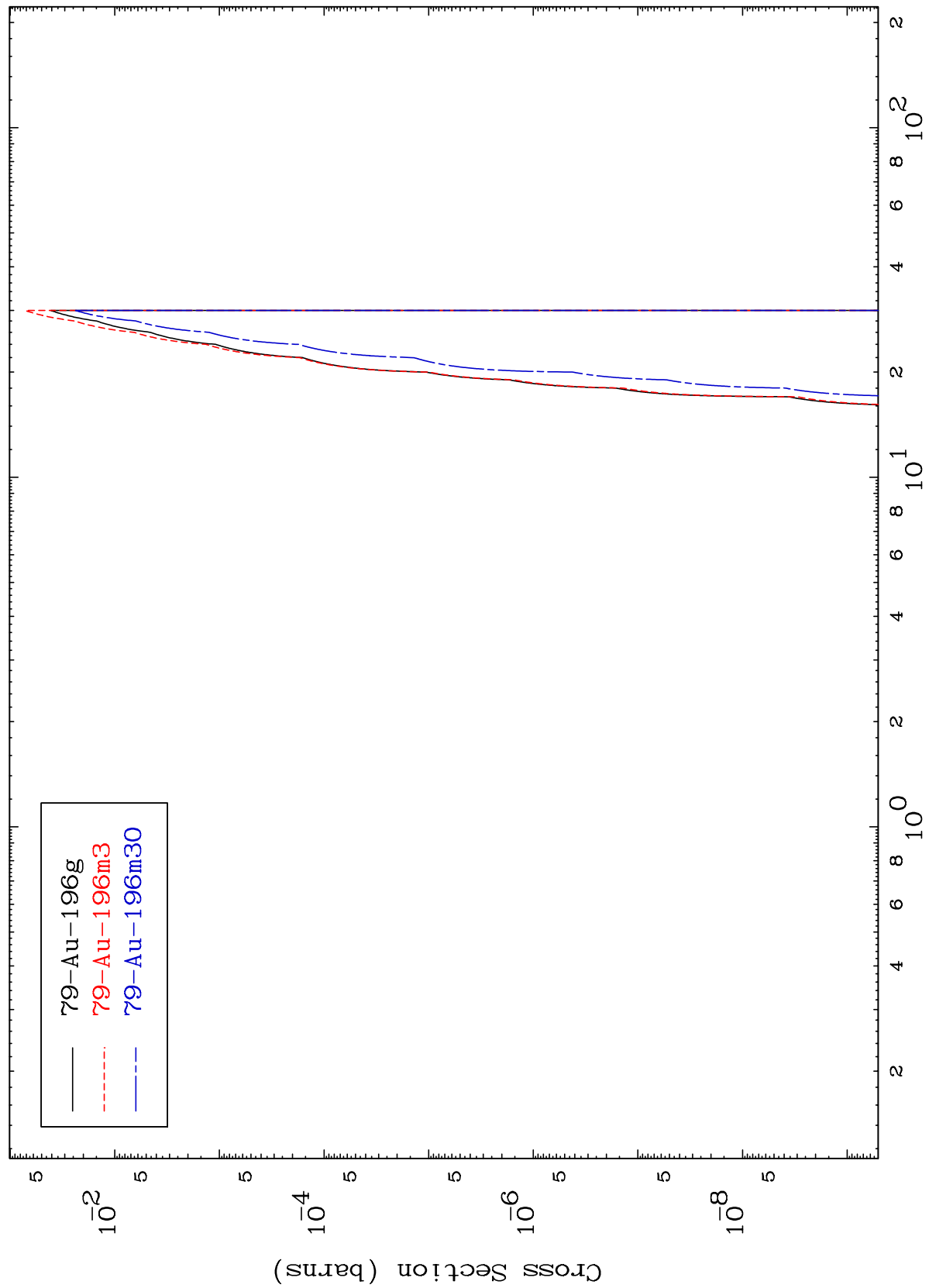


MAT 7923

79-Au-196

(α, α)

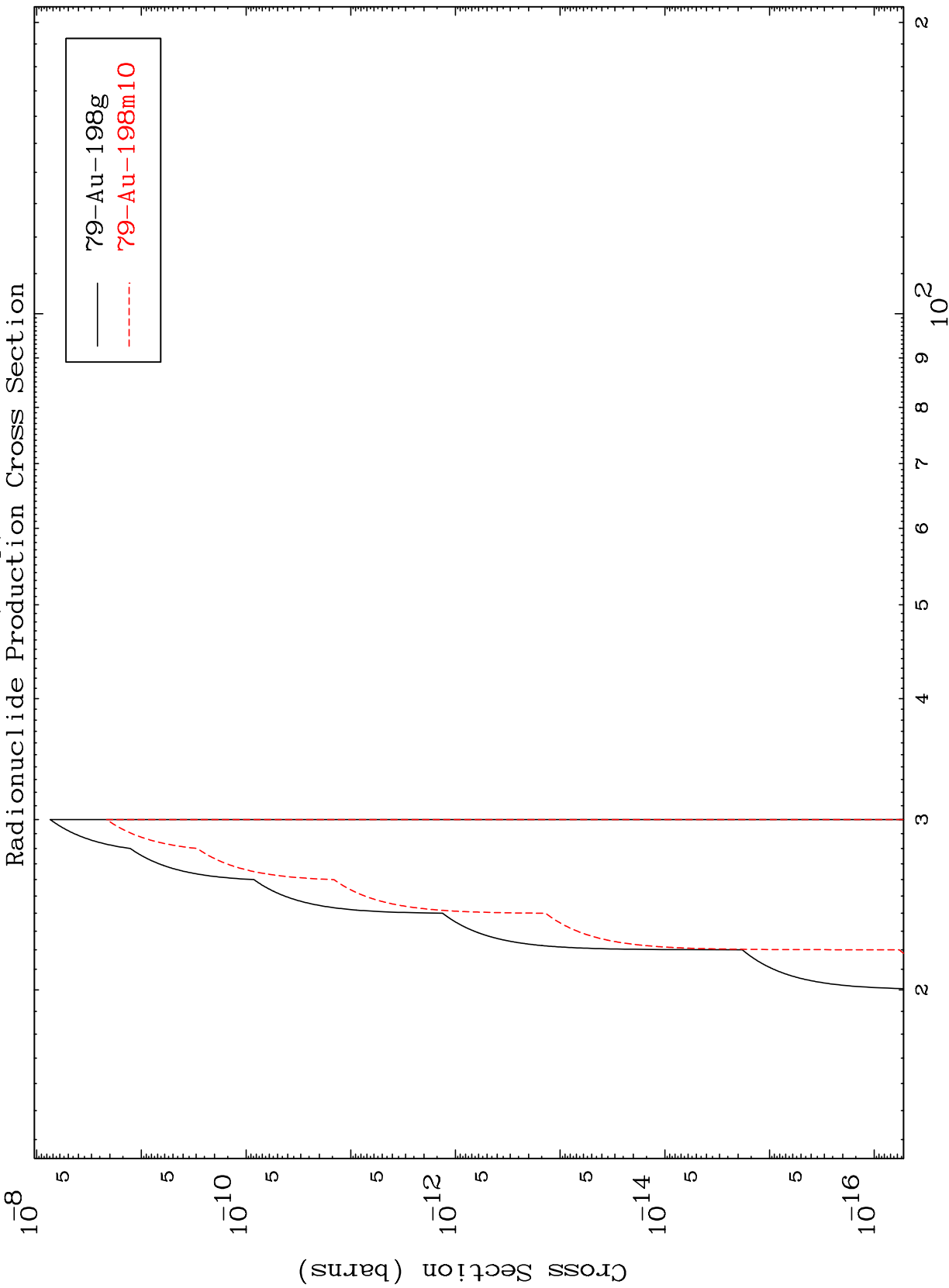
Radionuclide Production Cross Section



MAT 7923

79-Au-196

($\alpha, 2p$)
Radionuclide Production Cross Section



22

79-Au-196

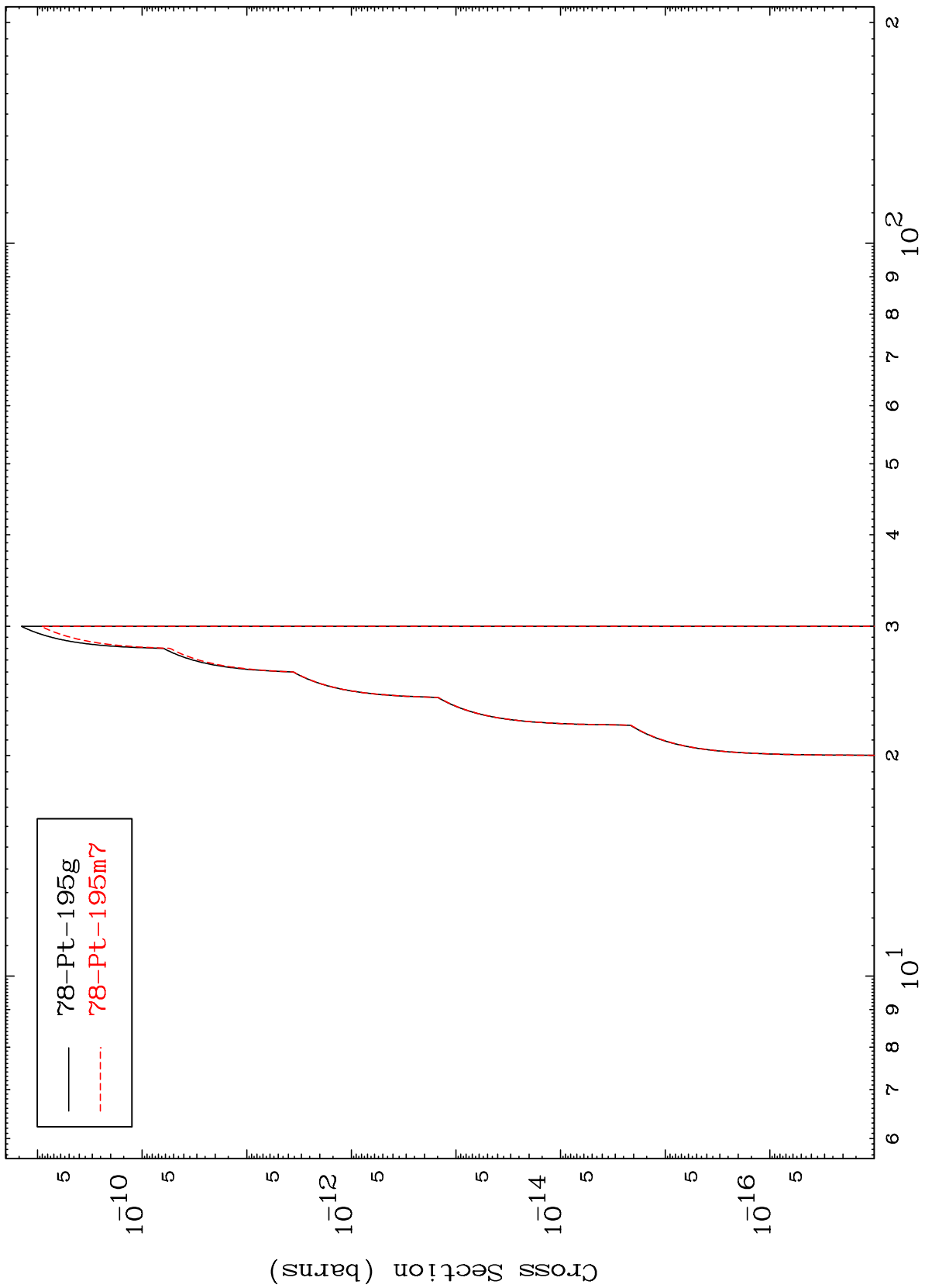
Incident Energy (MeV)

MAT 7923

$(\alpha, p) \alpha$

$^{79}\text{Au-196}$

Radionuclide Production Cross Section



— 78-Pt-195g
- - - 78-Pt-195m7