

Program EVALPLOT  
(Version 2018-1)

by

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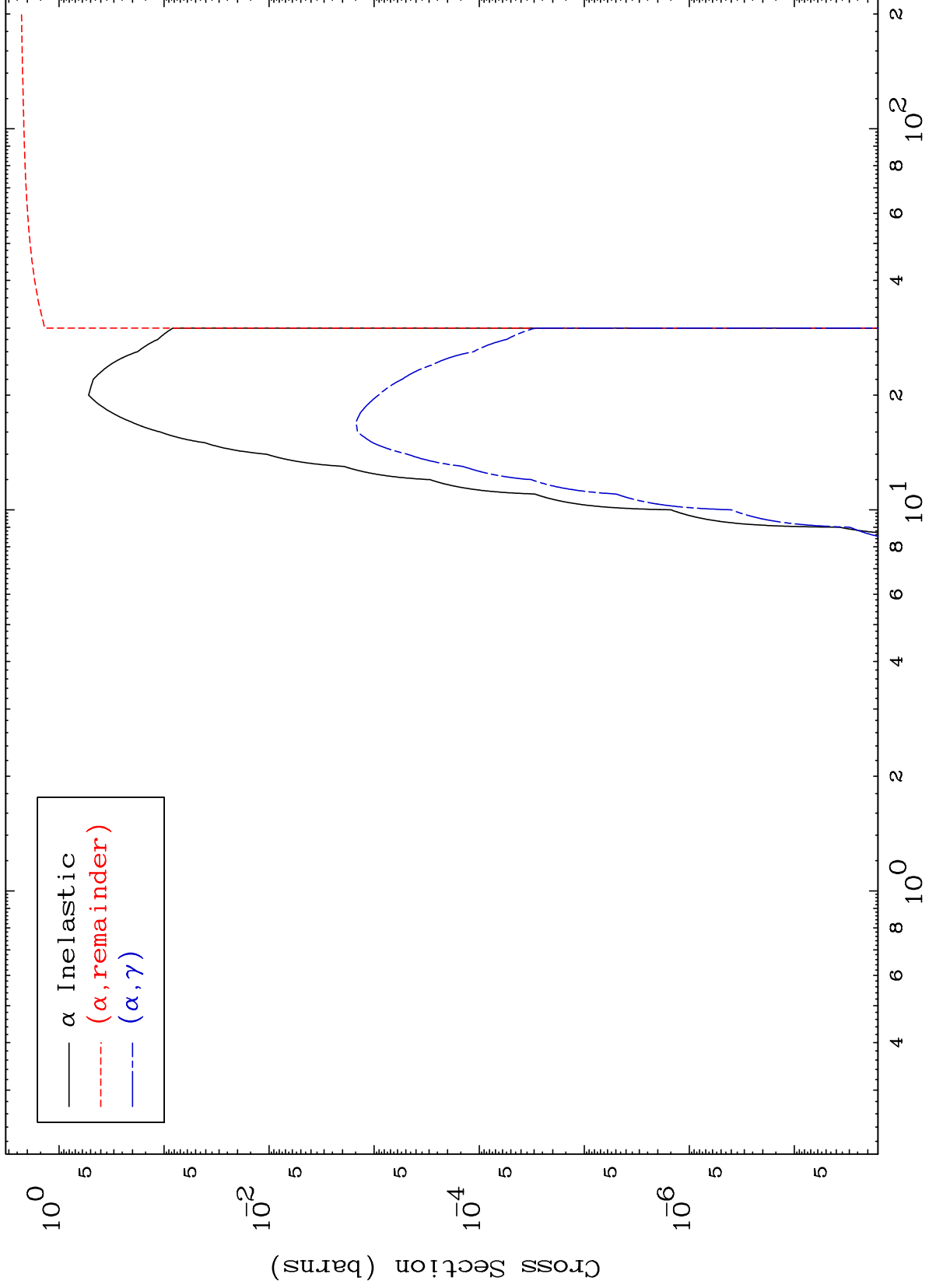
Press Mouse Button to Start

MAT 5622

0 Kelvin

$\alpha$  Major Cross Sections

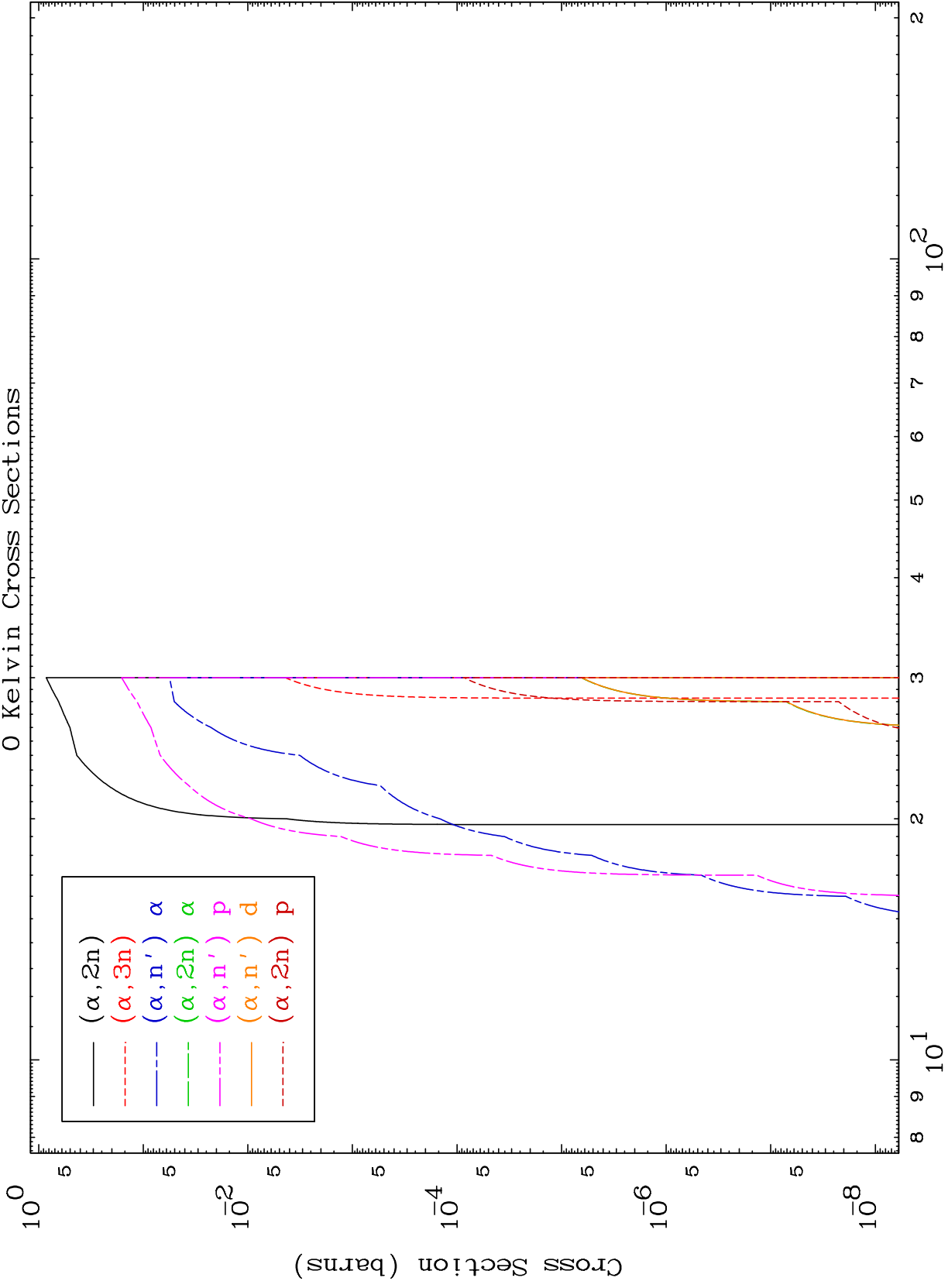
56-Ba-129



MAT 5622

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

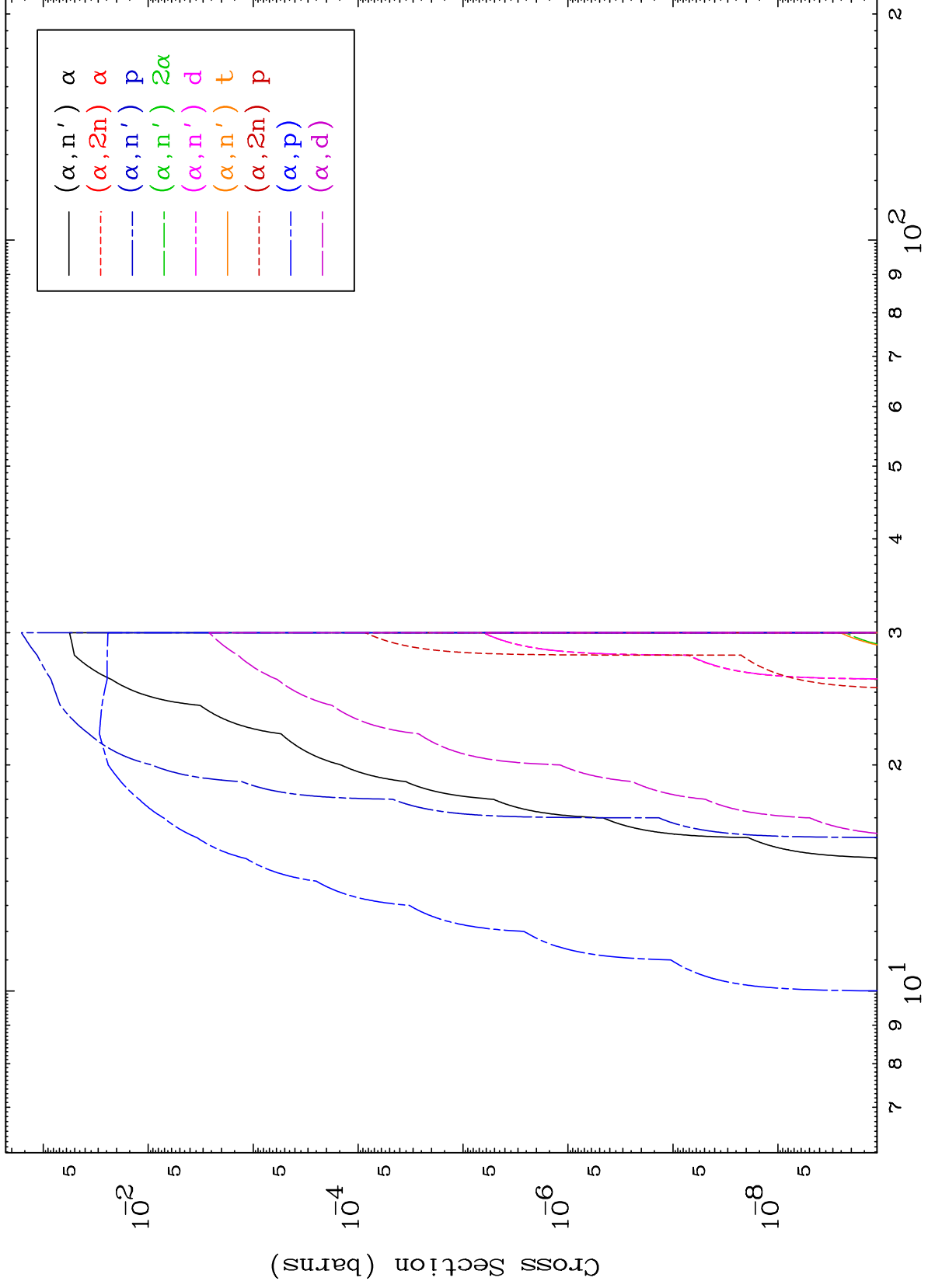
56-Ba-129



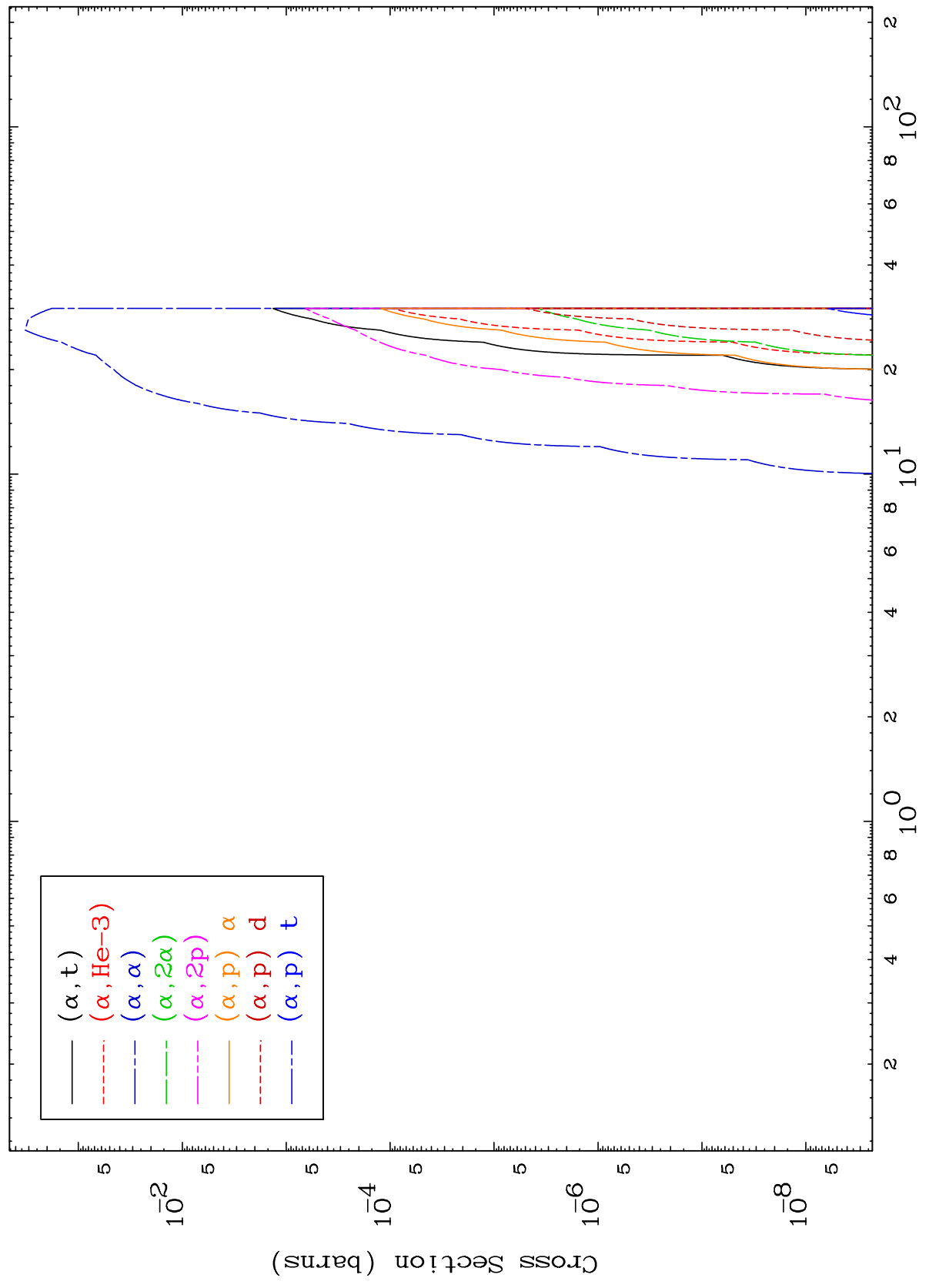
Incident Energy (MeV)

56-Ba-129

2



0 Kelvin Cross Sections

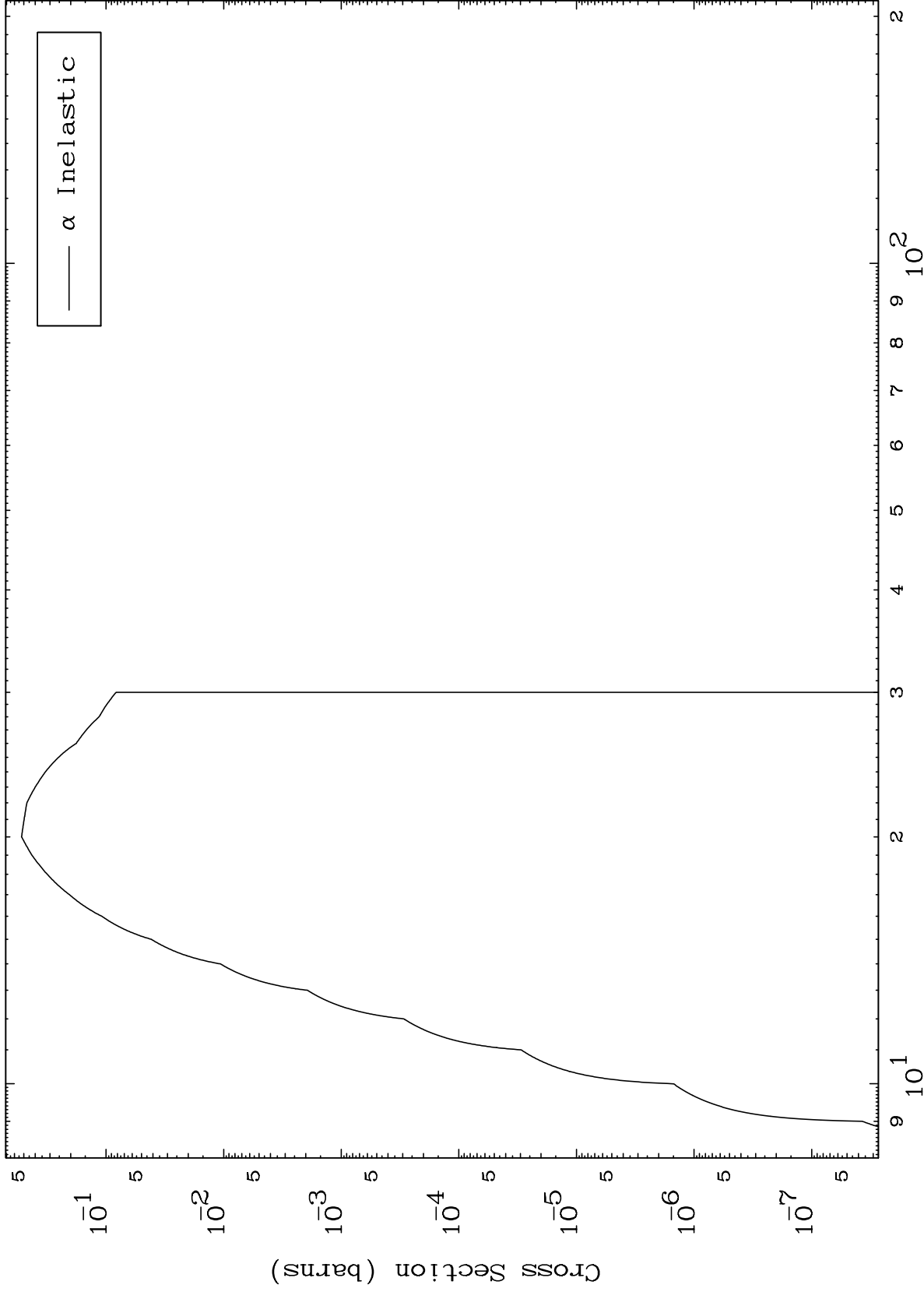


MAT 5622

( $\alpha, n'$ ) Level

56-Ba-129

0 Kelvin Cross Sections



5

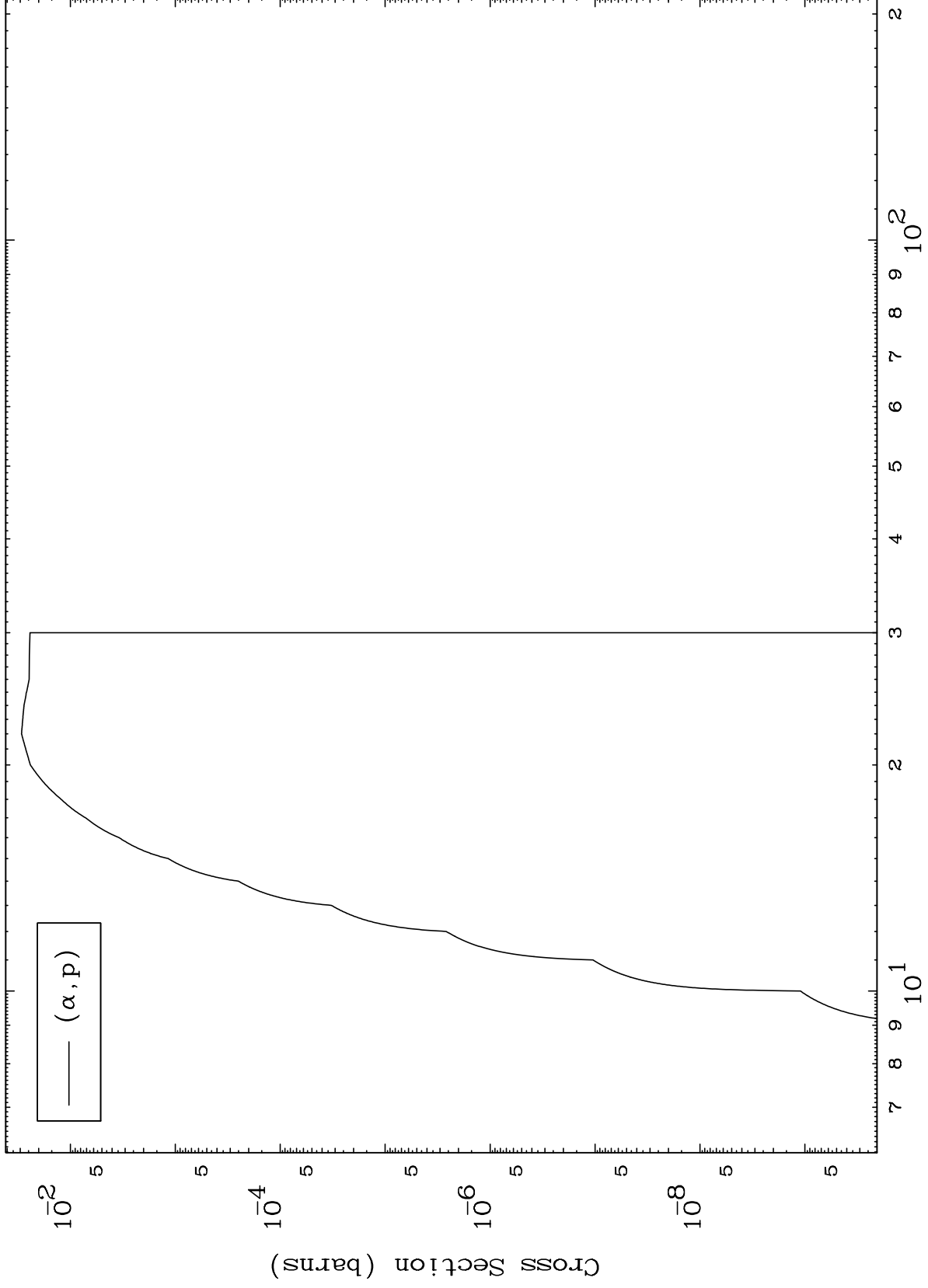
Incident Energy (MeV)

56-Ba-129

MAT 5622

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

56-Ba-129

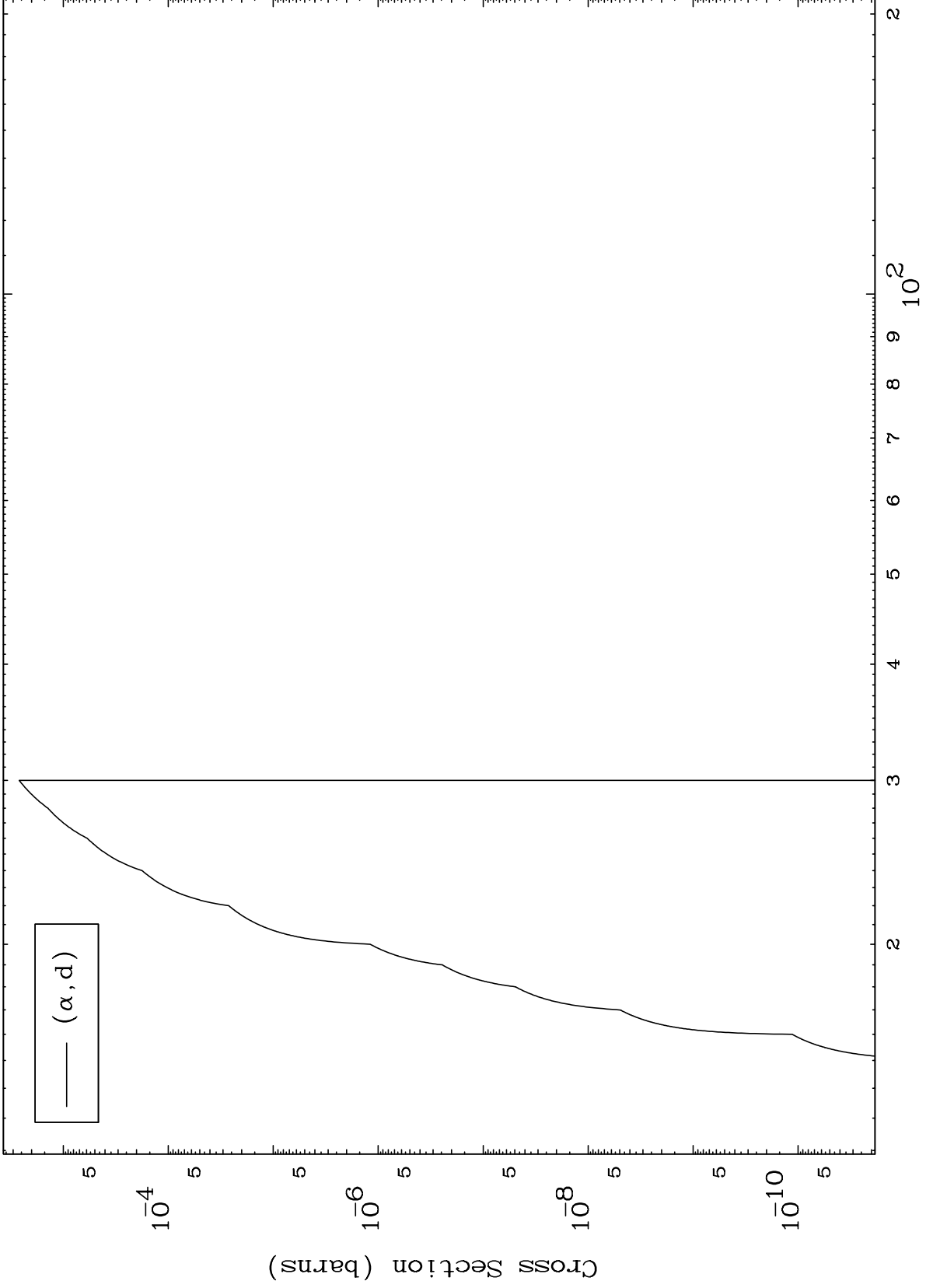


6

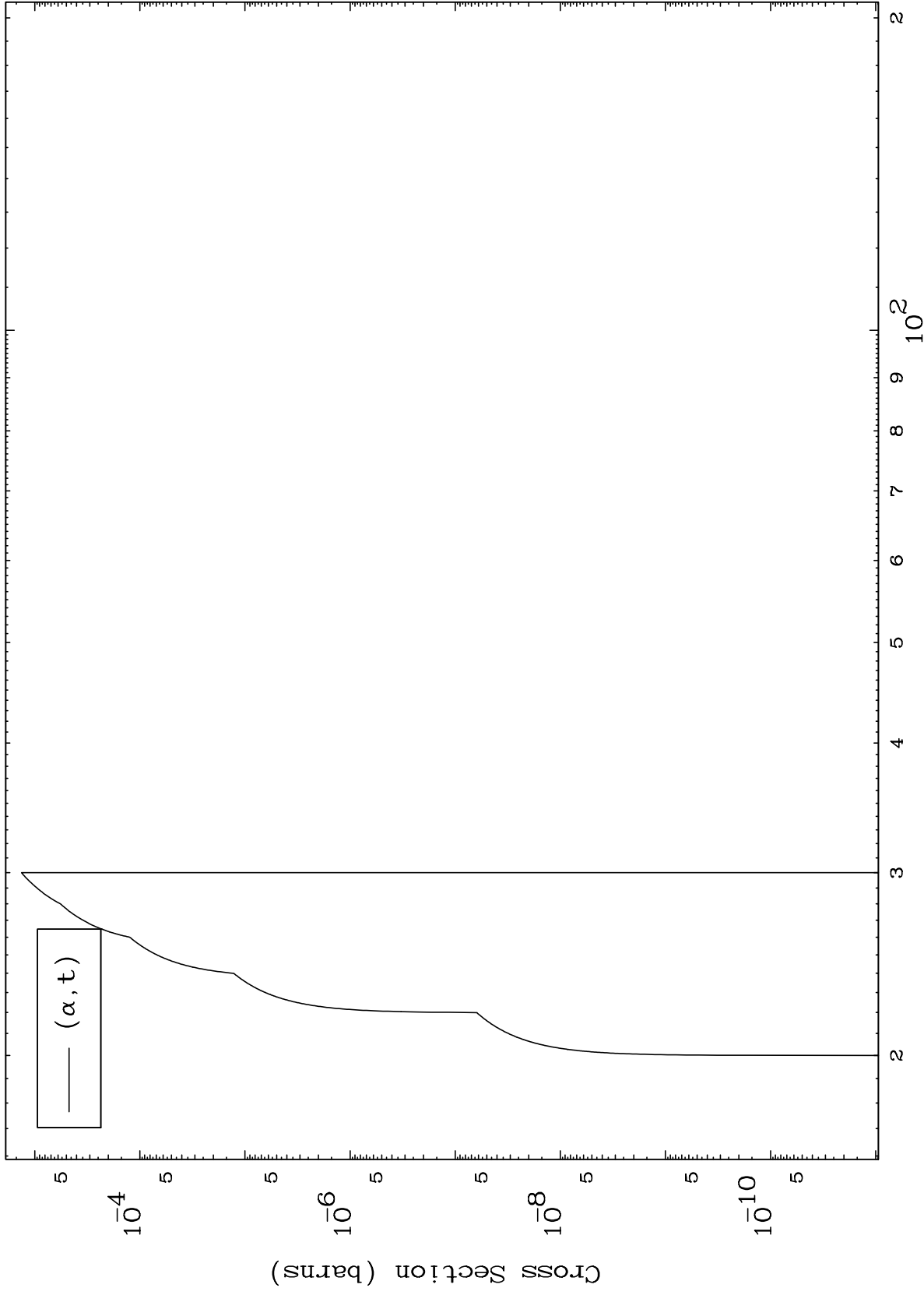
Incident Energy (MeV)

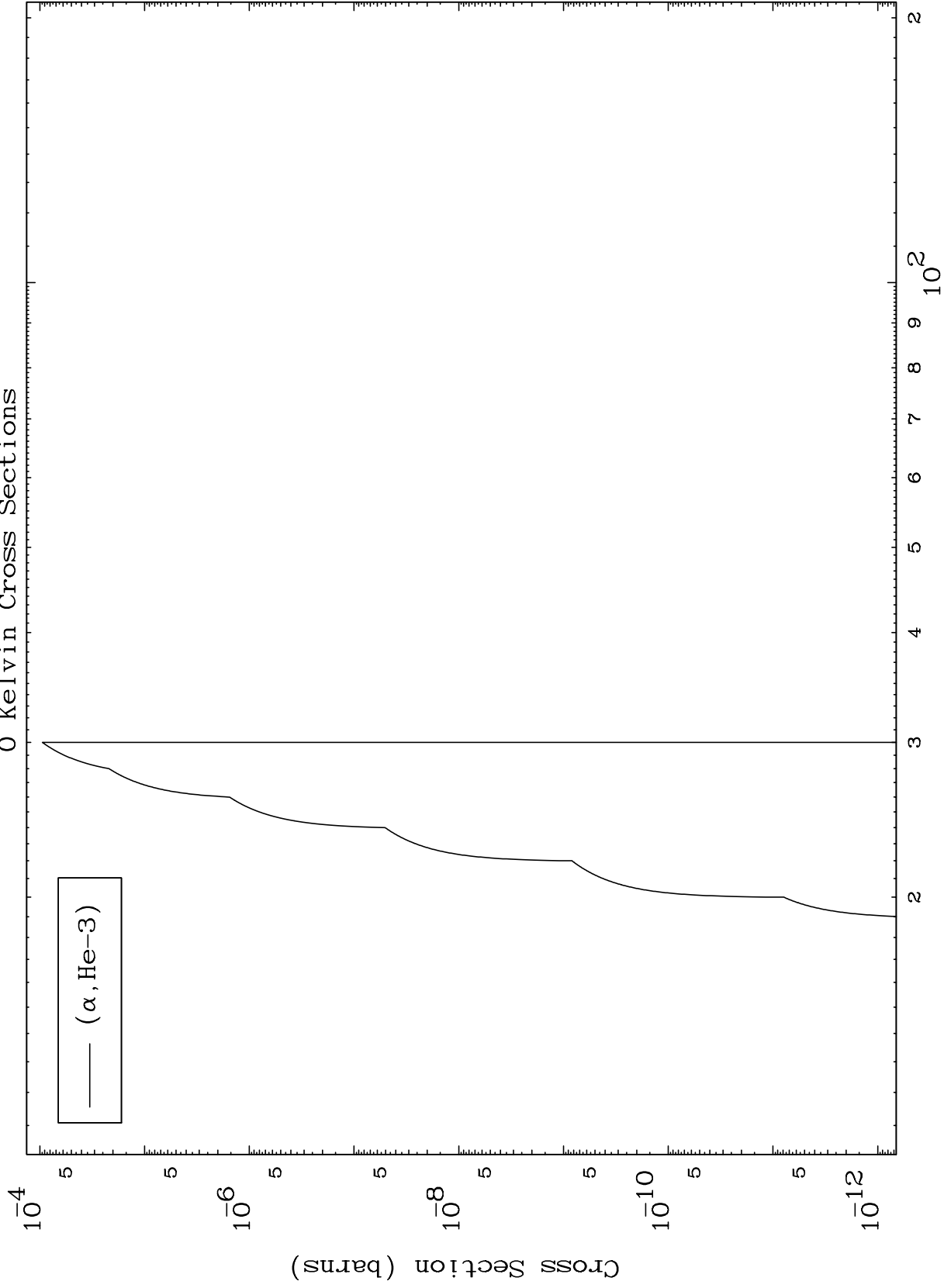
56-Ba-129

0 Kelvin Cross Sections







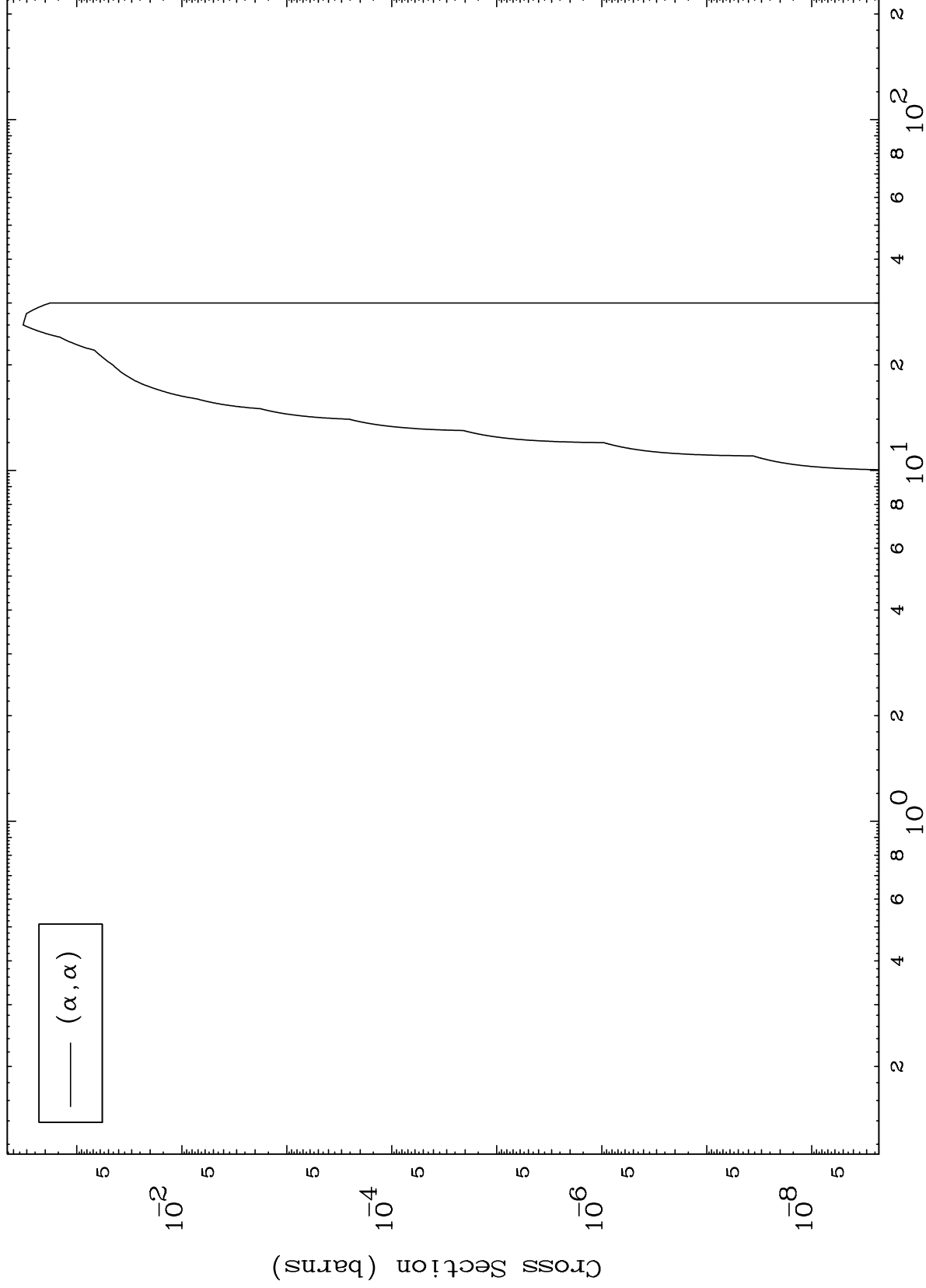


MAT 5622

( $\alpha, \alpha$ ) Levels

56-Ba-129

0 Kelvin Cross Sections



10

Incident Energy (MeV)

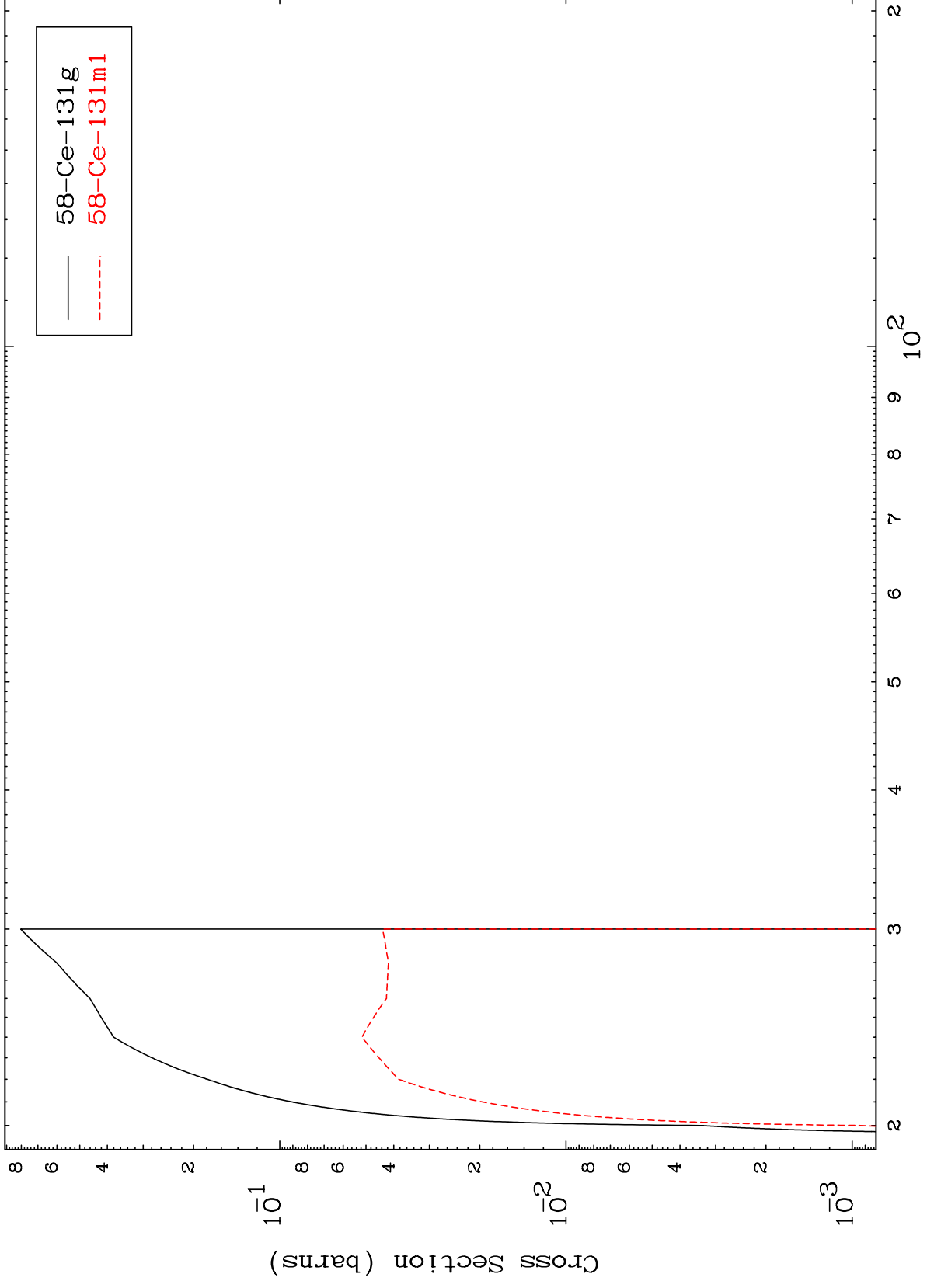
56-Ba-129

MAT 5622

( $\alpha, 2n$ )

56-Ba-129

Radionuclide Production Cross Section



11

Incident Energy (MeV)

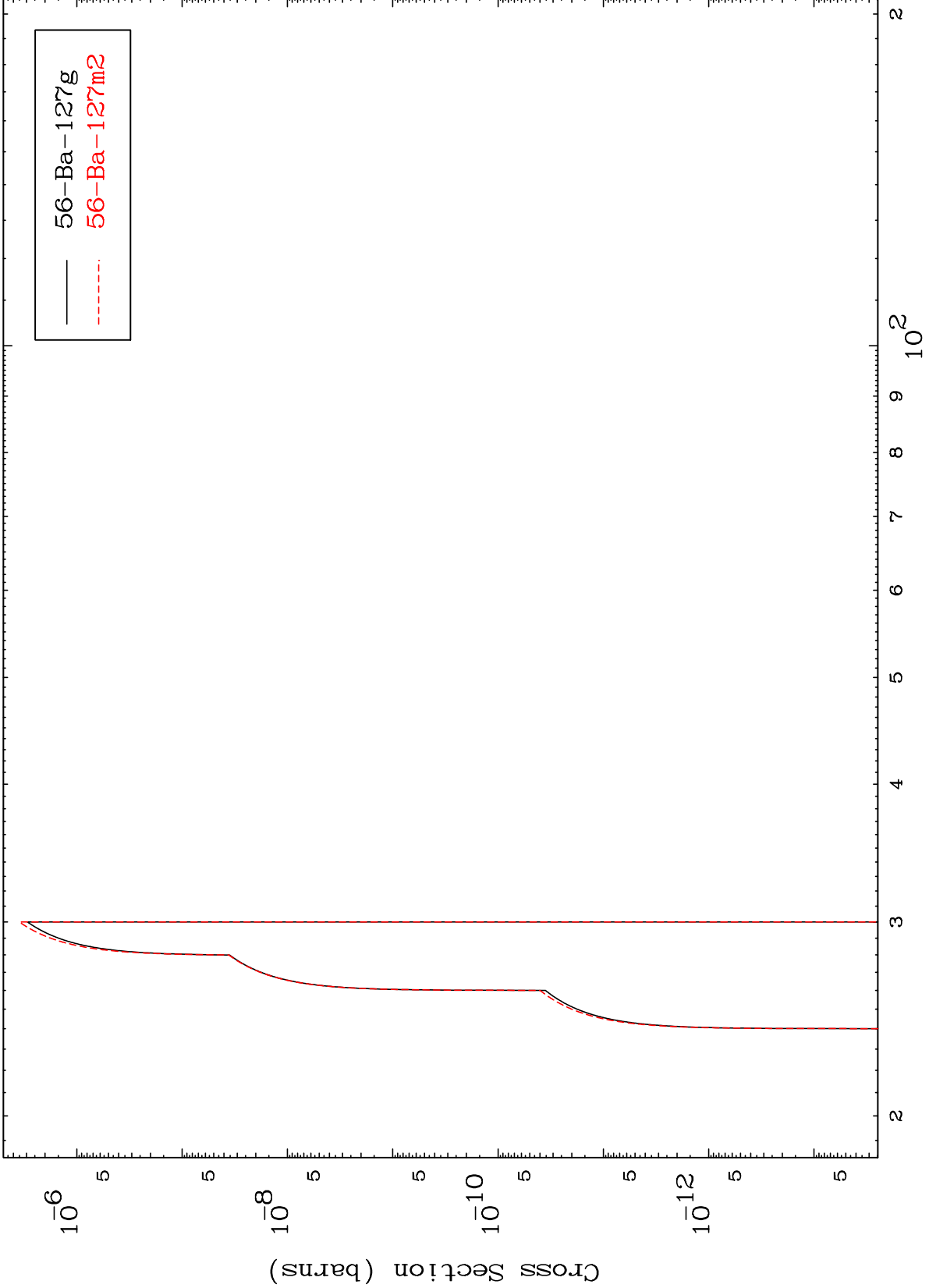
56-Ba-129

MAT 5622

56-Ba-129

$(\alpha, 2n)$   $\alpha$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

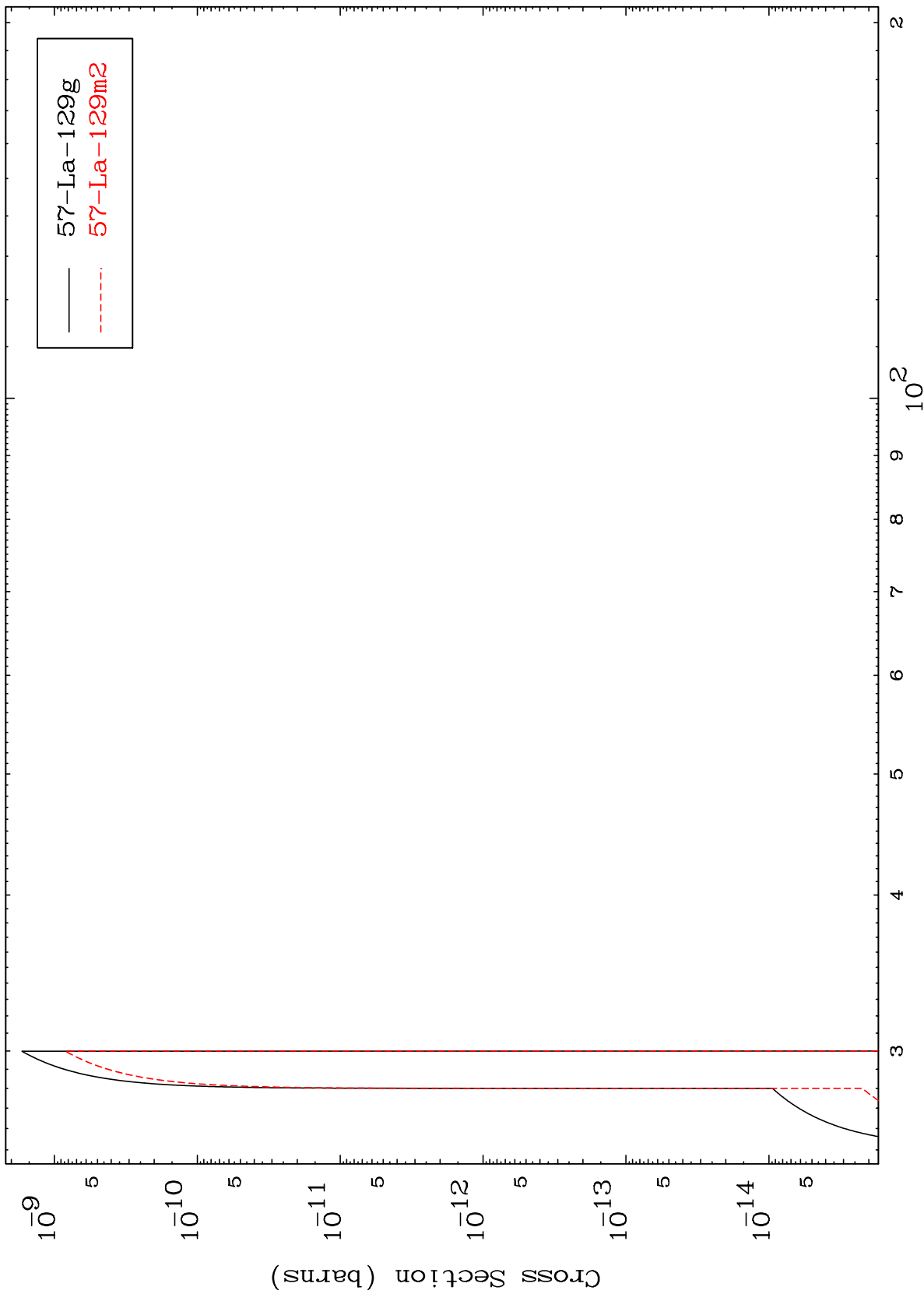
56-Ba-129

MAT 5622

$(\alpha, n')$  t

56-Ba-129

Radionuclide Production Cross Section



13

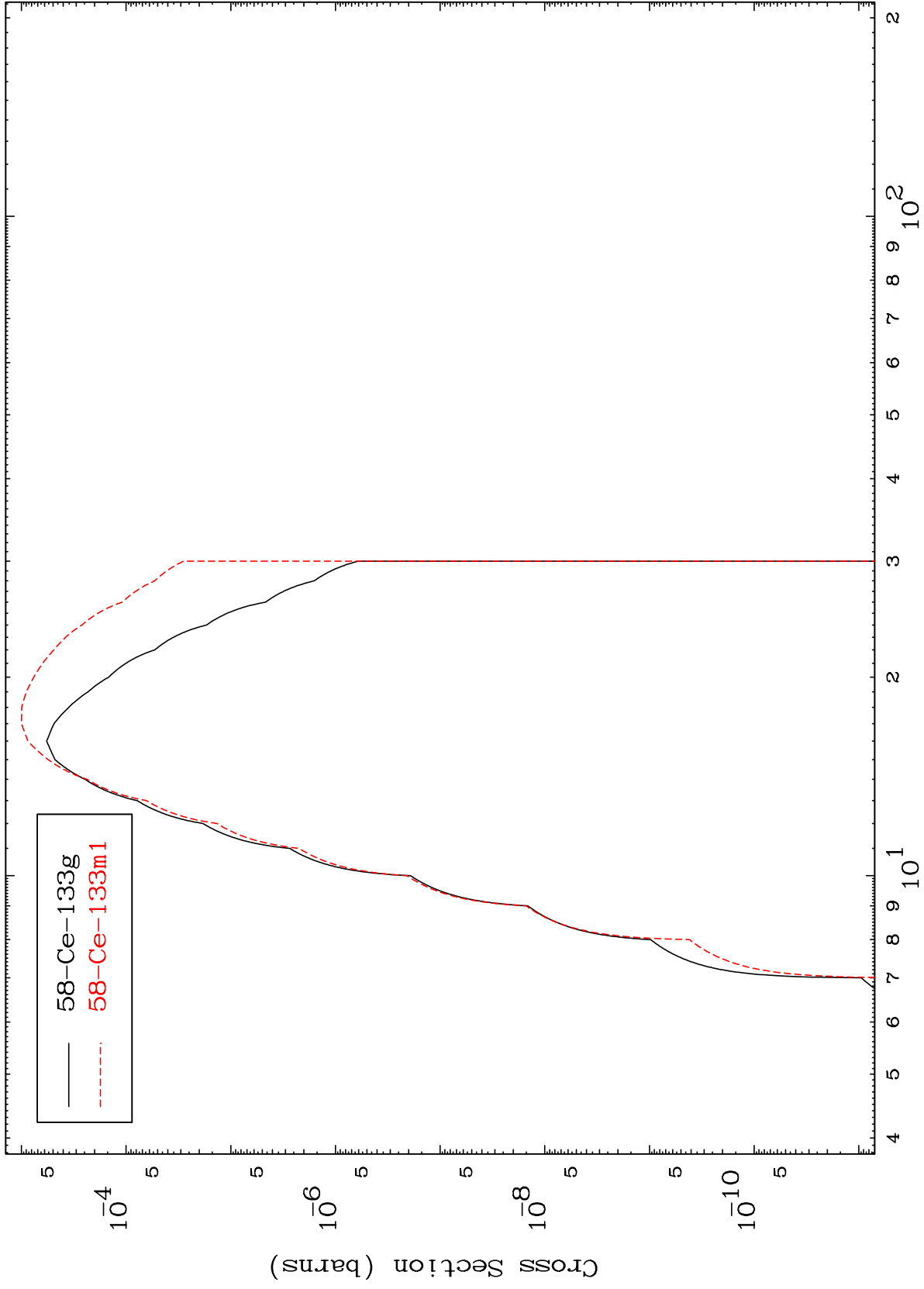
Incident Energy (MeV)

56-Ba-129

MAT 5622

56-Ba-129

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



56-Ba-129

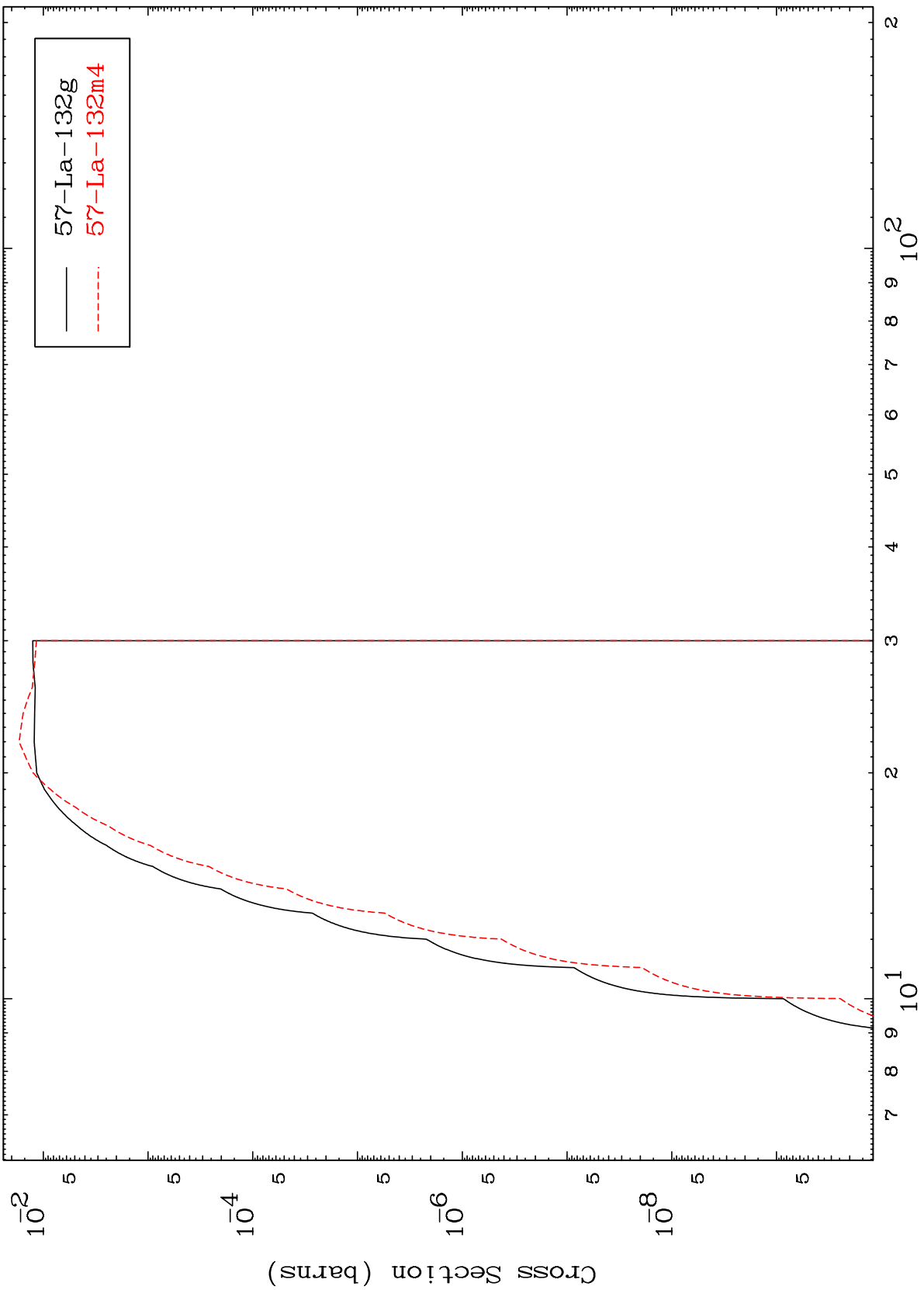
Incident Energy (MeV)

14

MAT 5622

56-Ba-129

( $\alpha, p$ )  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

56-Ba-129

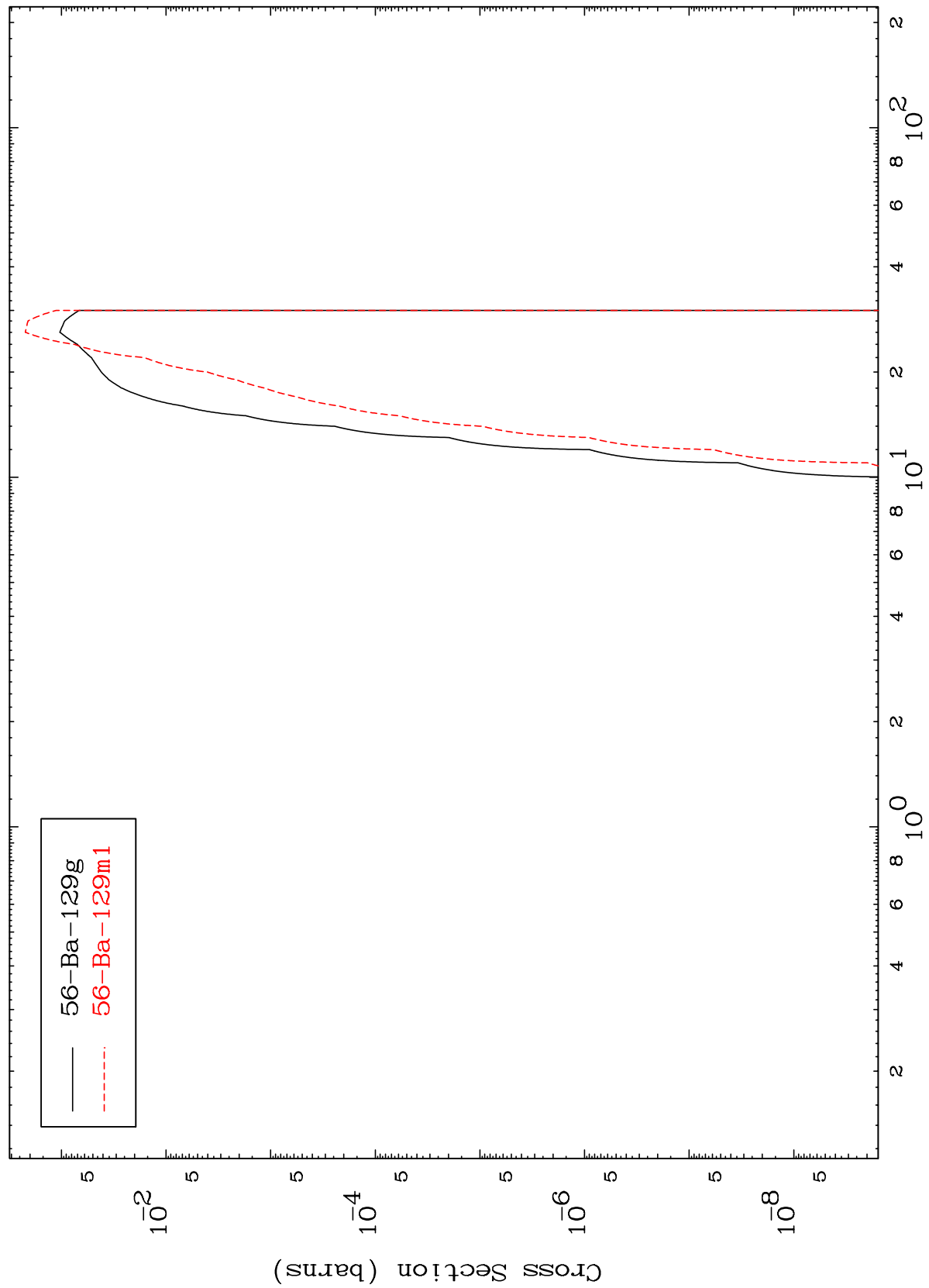


MAT 5622

56-Ba-129

( $\alpha, \alpha$ )

Radionuclide Production Cross Section



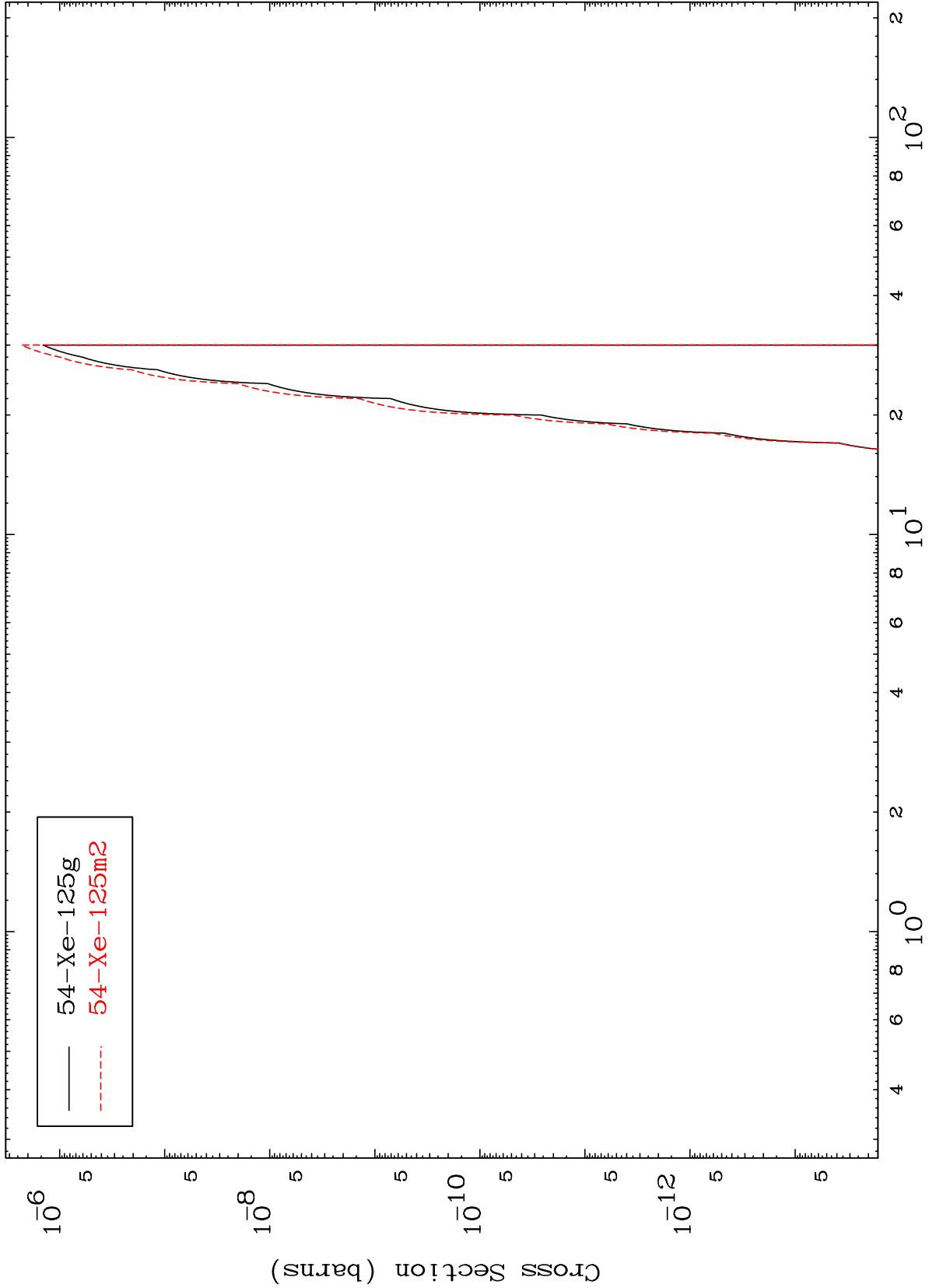
56-Ba-129g  
56-Ba-129m1

MAT 5622

( $\alpha, 2\alpha$ )

56-Ba-129

Radionuclide Production Cross Section



17

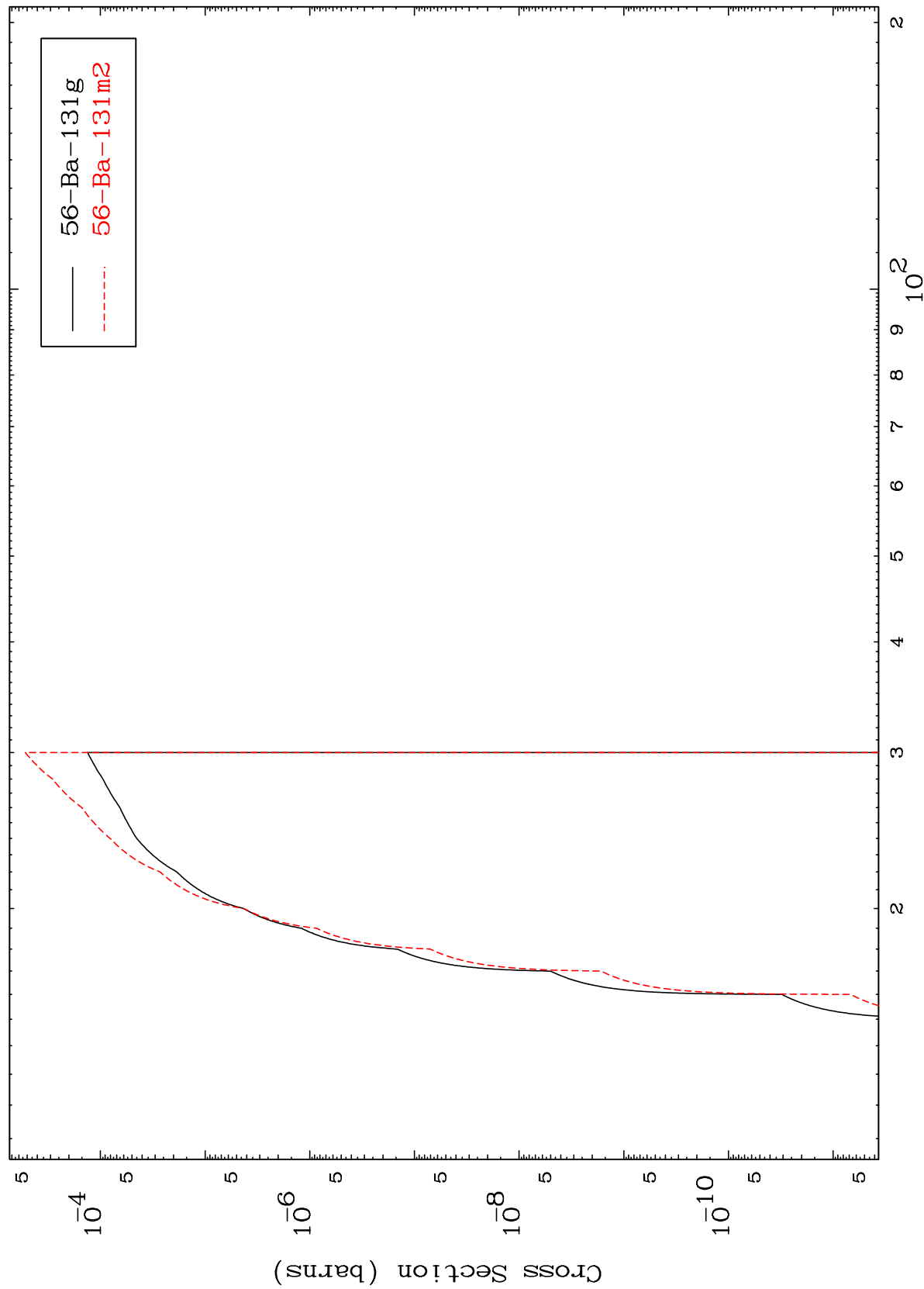
Incident Energy (MeV)

56-Ba-129

MAT 5622

56-Ba-129

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

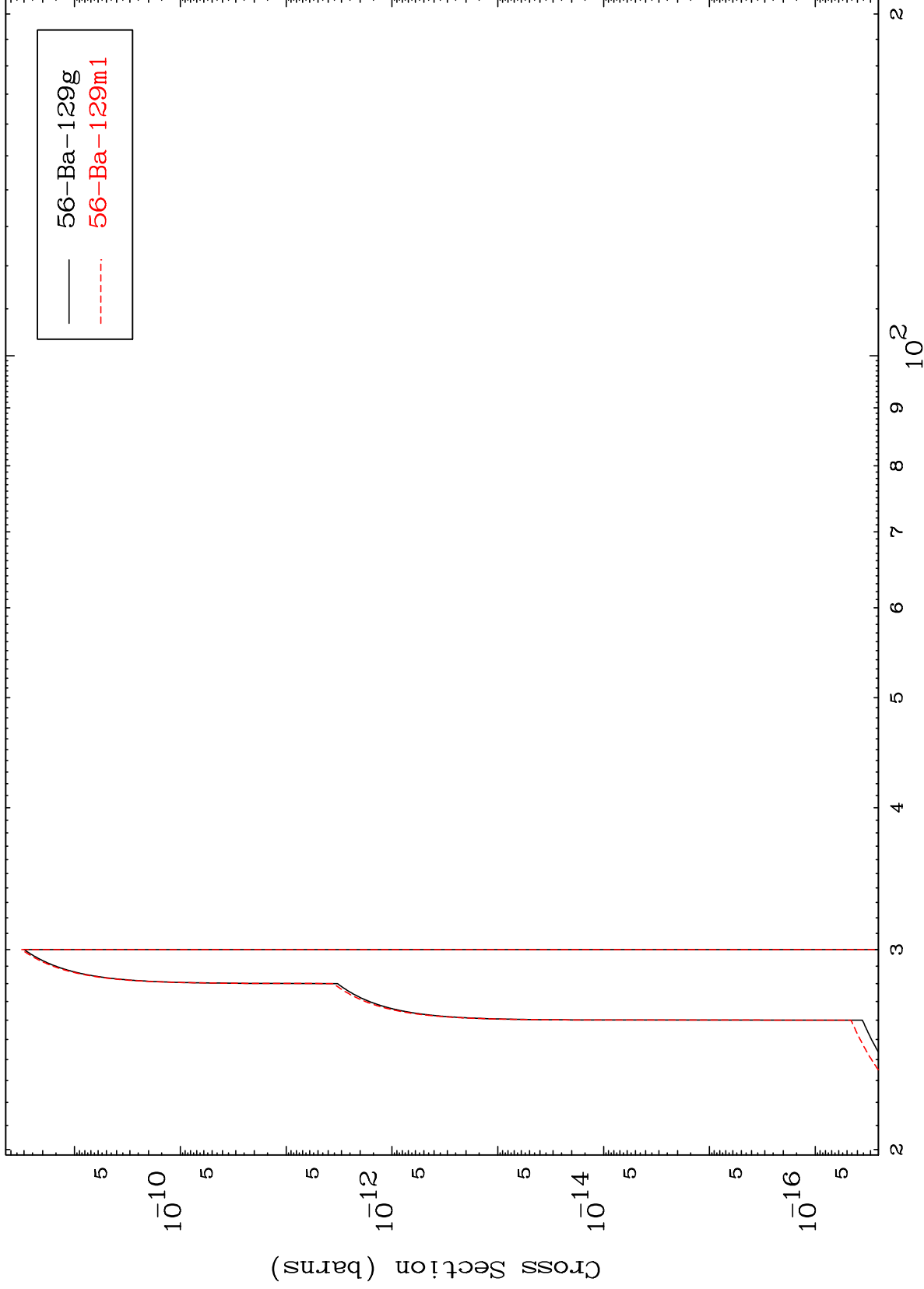
56-Ba-129

MAT 5622

( $\alpha, p$ ) t

56-Ba-129

Radionuclide Production Cross Section



19

Incident Energy (MeV)

56-Ba-129