

Program EVALPLOT  
(Version 2018-1)

by

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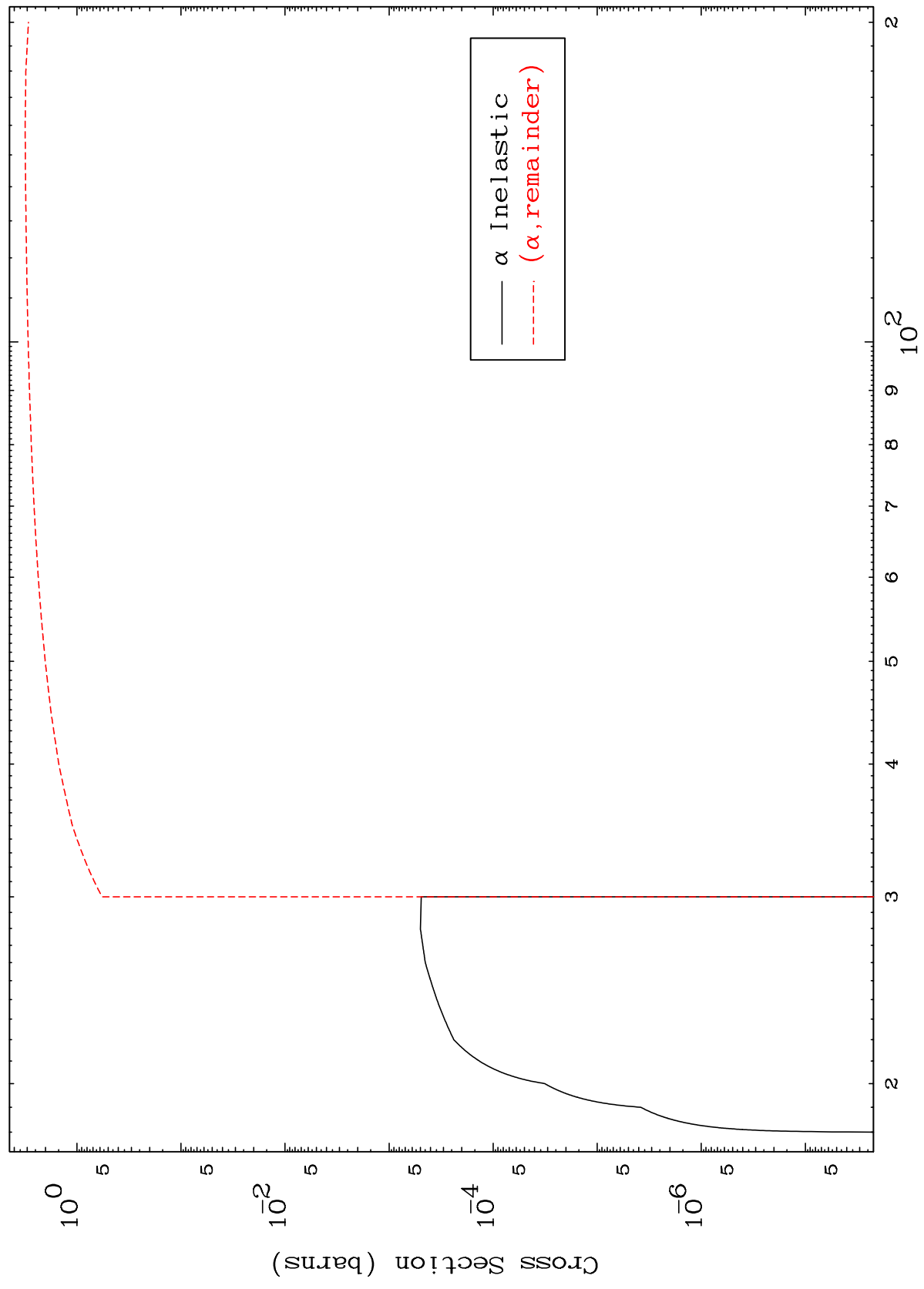
Press Mouse Button to Start

MAT 767

$\alpha$  Major

107-Bh-267

0 Kelvin Cross Sections



Incident Energy (MeV)

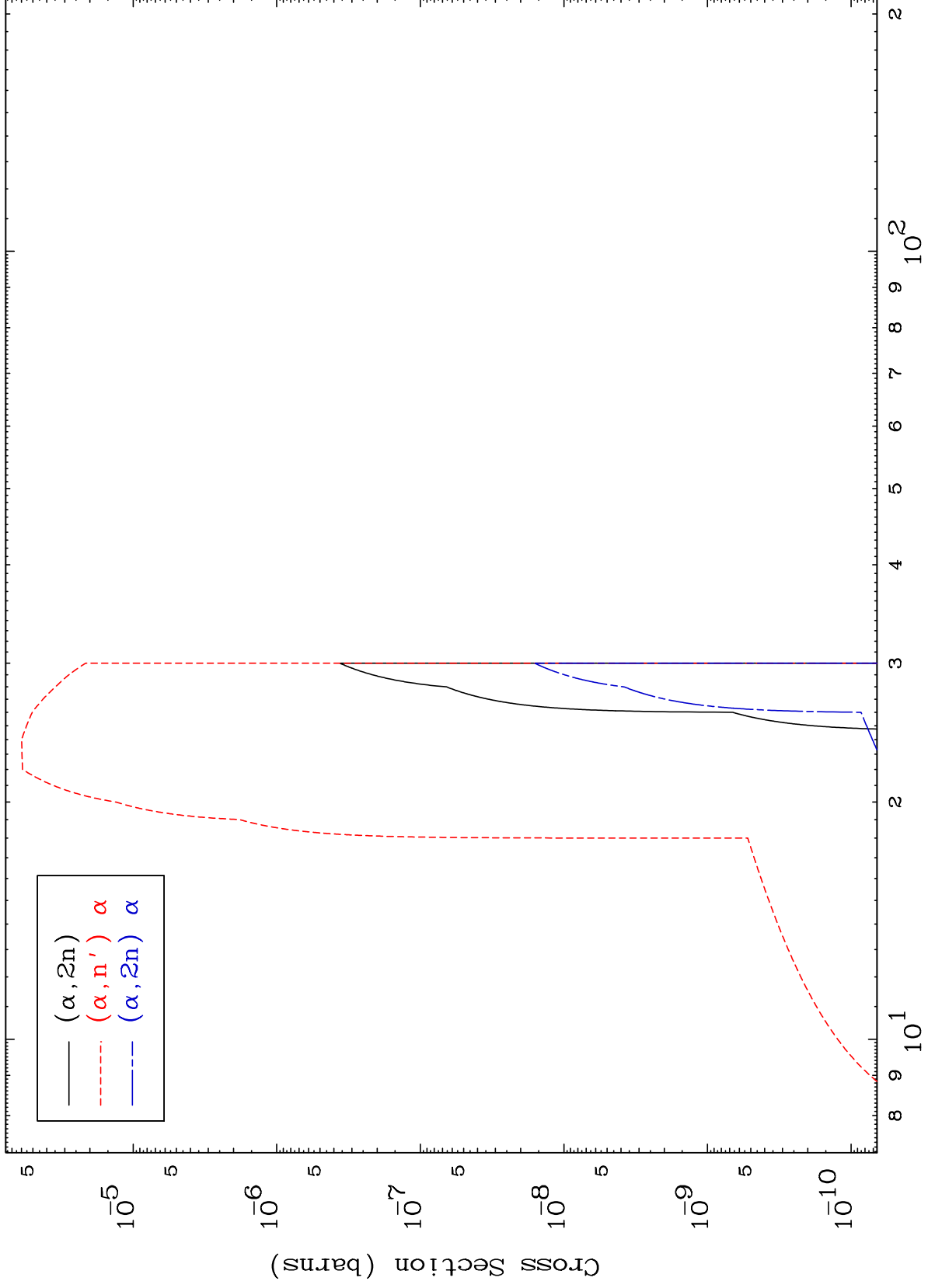
107-Bh-267

1

MAT 767

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

107-Bh-267



2

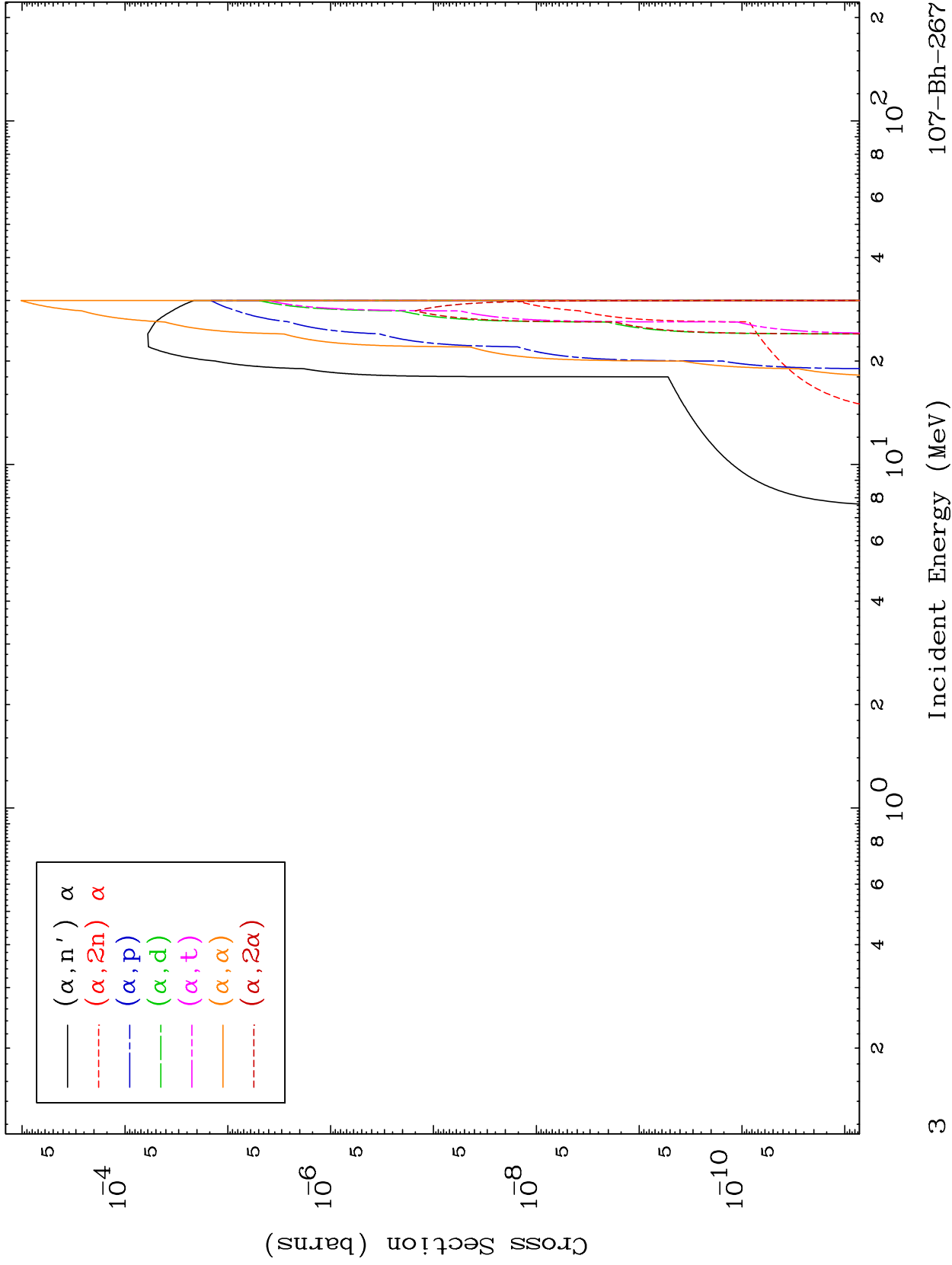
Incident Energy (MeV)

107-Bh-267

MAT 767

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

107-Bh-267

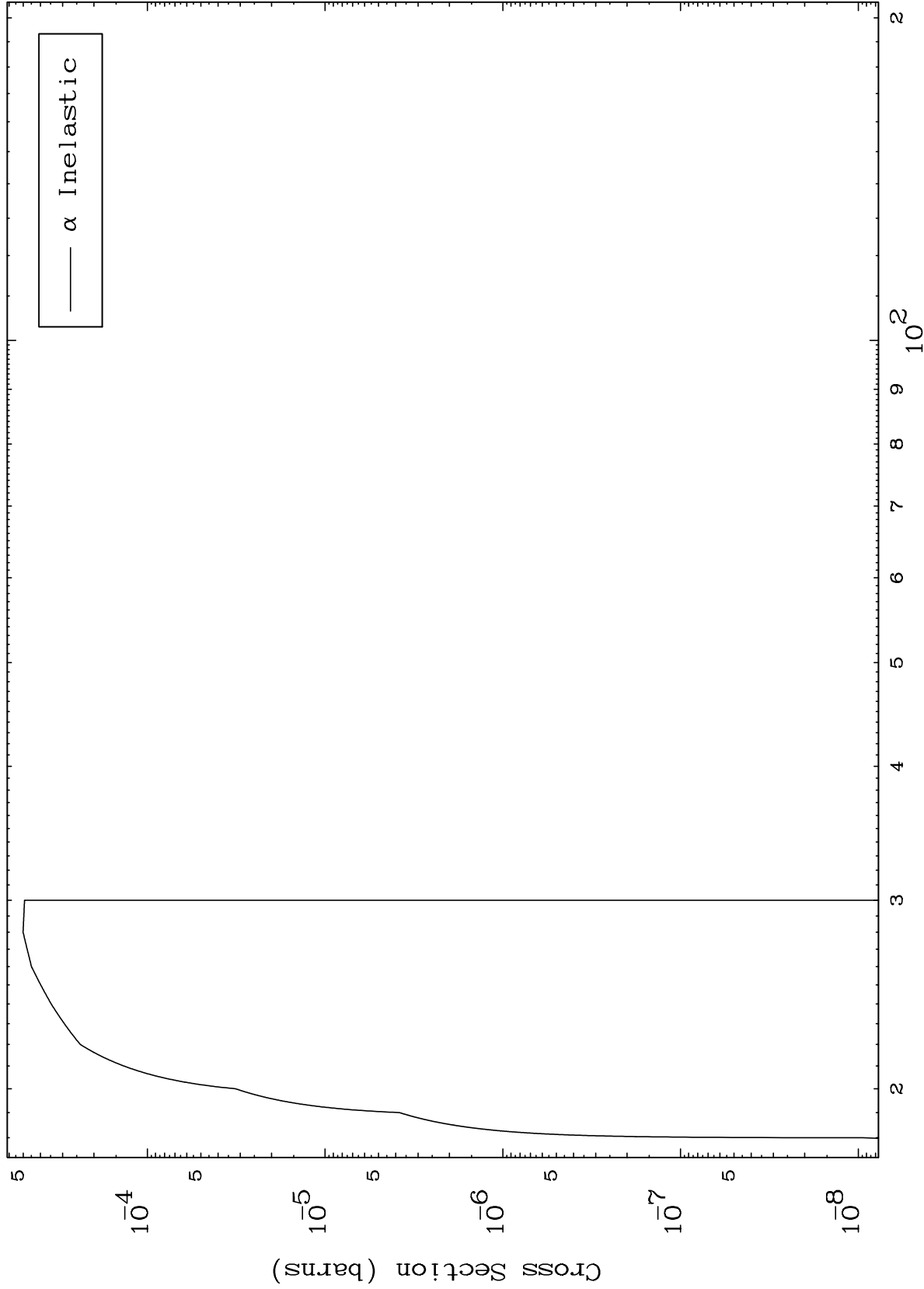


MAT 767

( $\alpha, n'$ ) Level

107-Bh-267

0 Kelvin Cross Sections



4

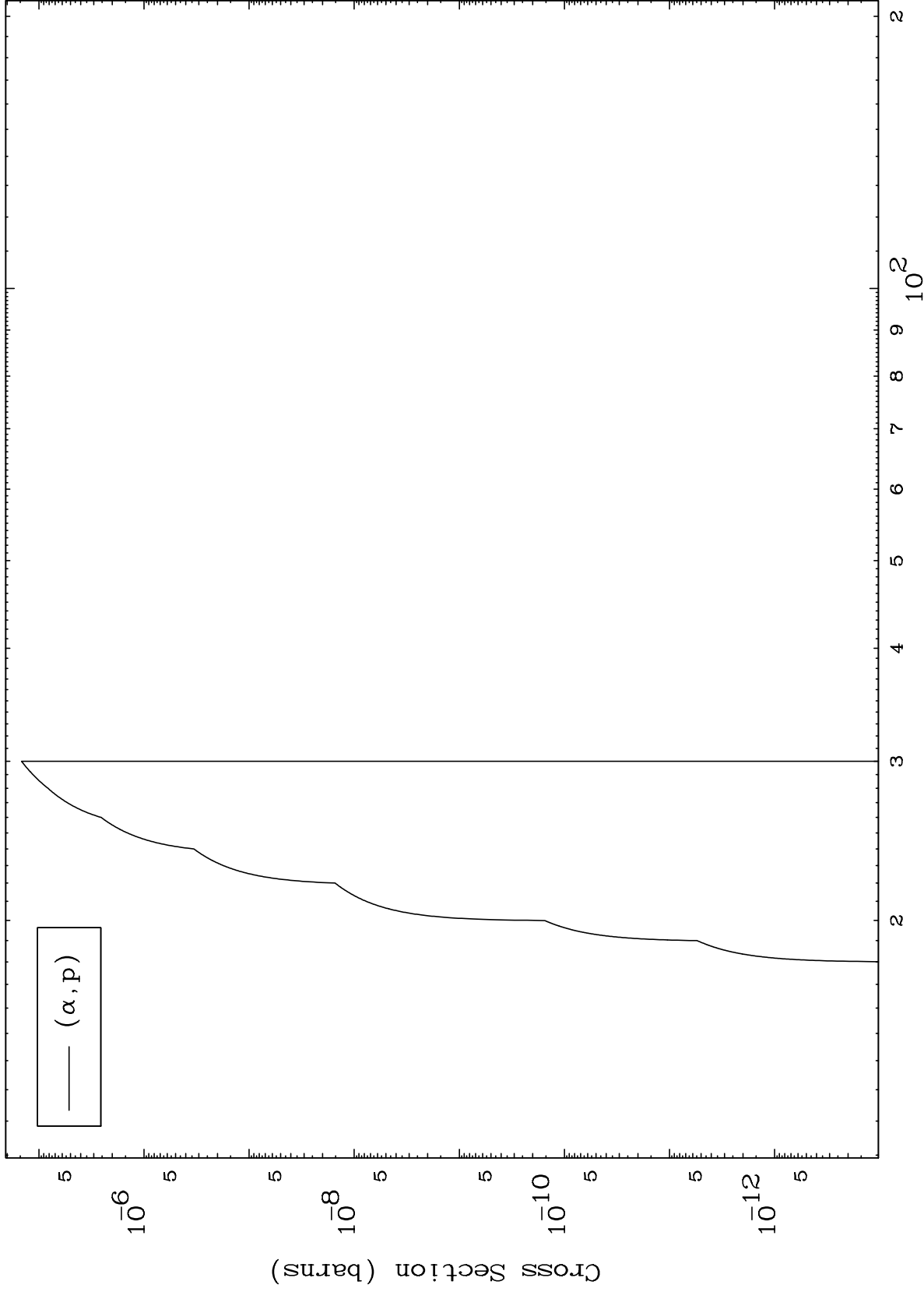
Incident Energy (MeV)

107-Bh-267

MAT 767

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

107-Bh-267



5

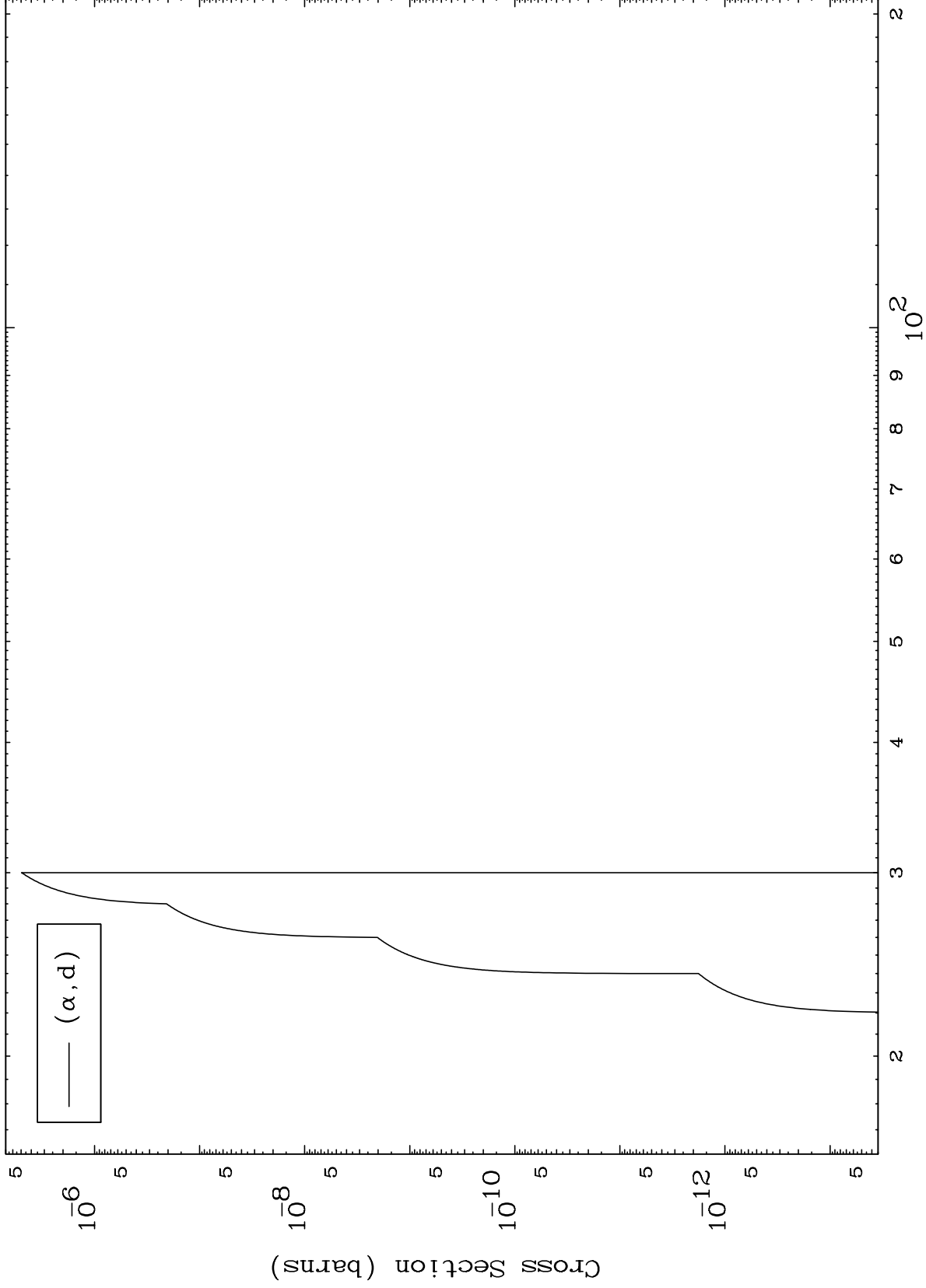
Incident Energy (MeV)

107-Bh-267

MAT 767

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

107-Bh-267



6

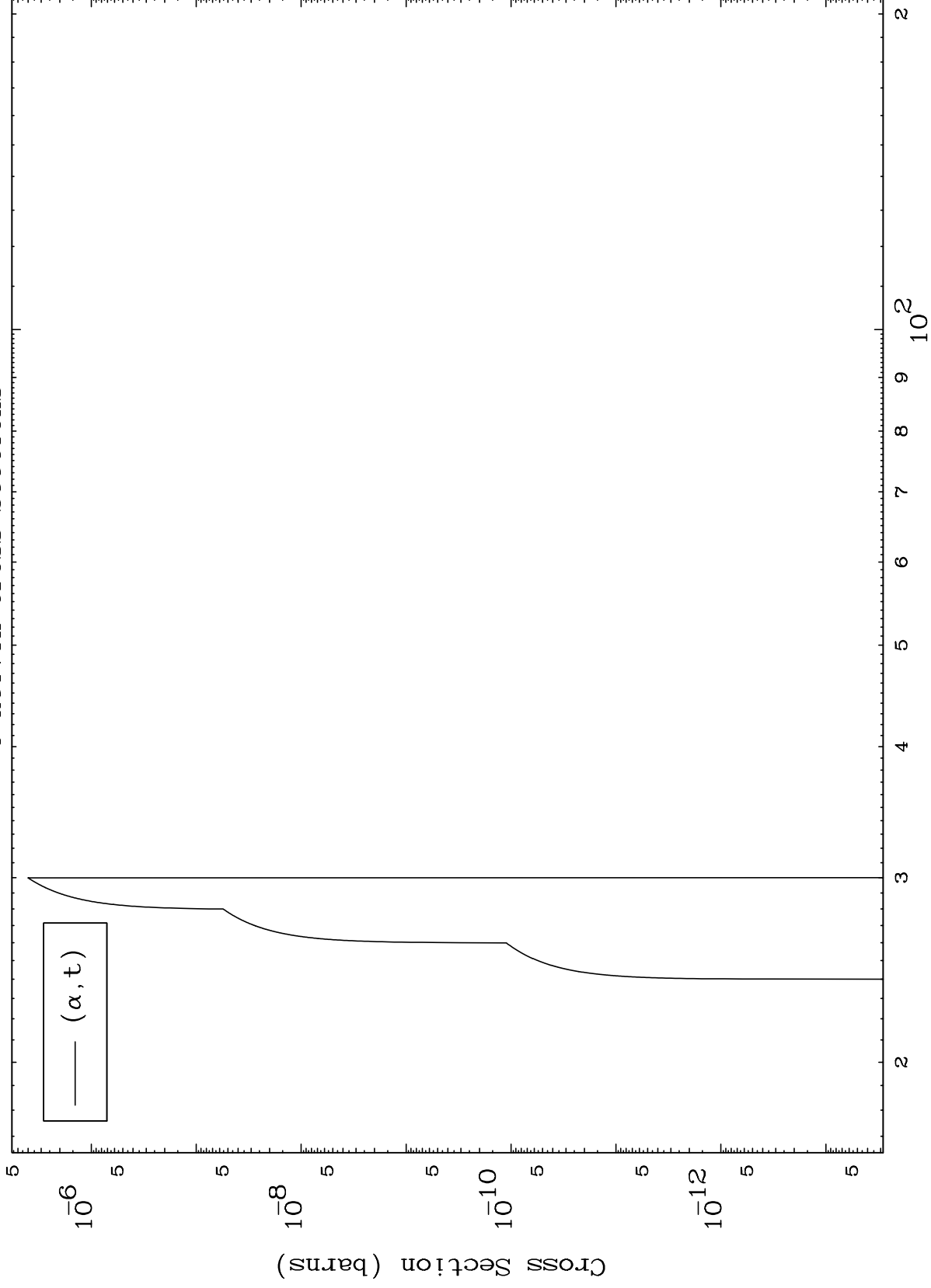
Incident Energy (MeV)

107-Bh-267

MAT 767

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

107-Bh-267



7

Incident Energy (MeV)

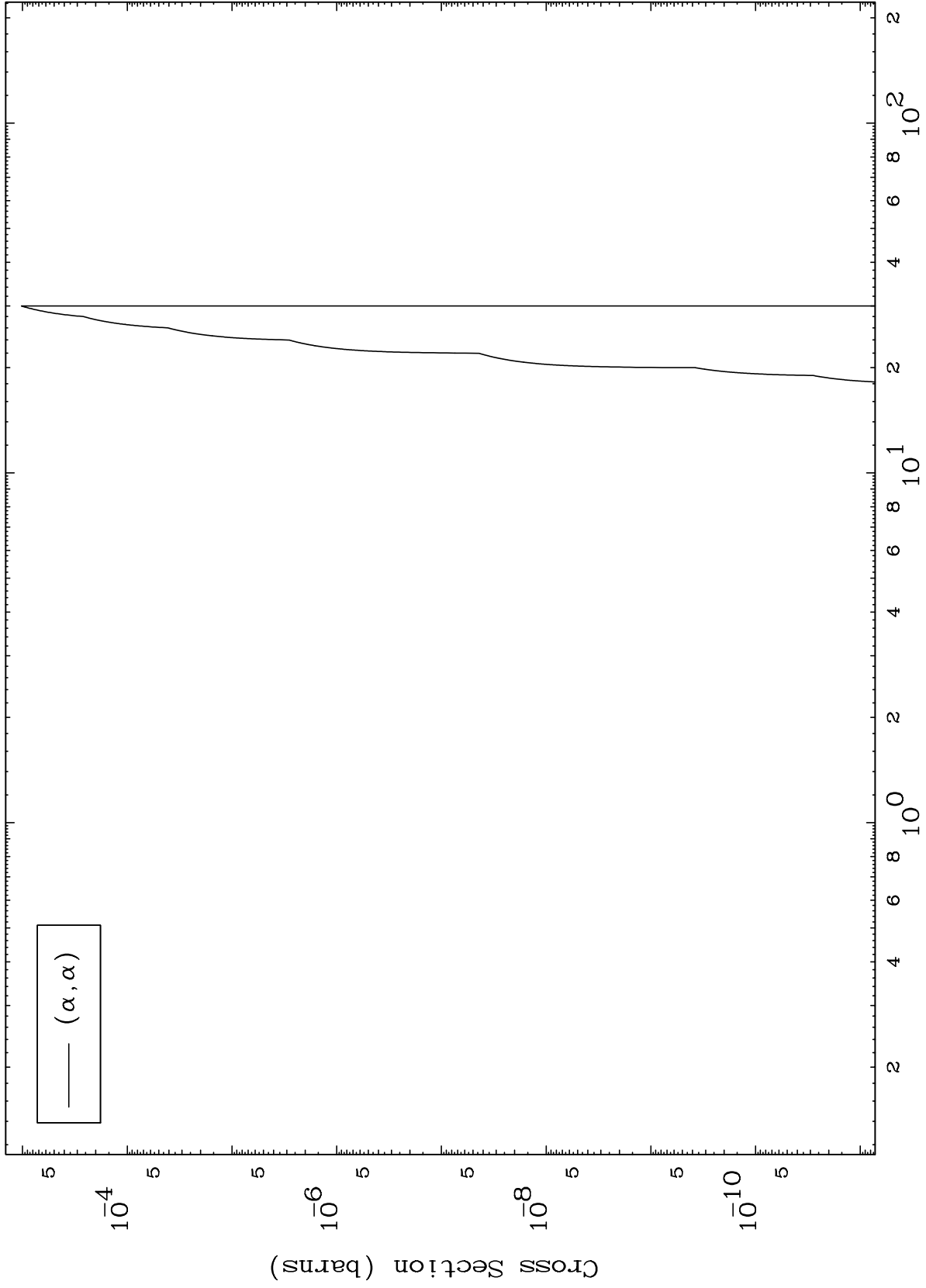
107-Bh-267



MAT 767

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

107-Bh-267



MAT 767

107-Bh-267

$\alpha$  Fission  
Radionuclide Production Cross Section

