

Program EVALPLOT  
(Version 2018-1)

by

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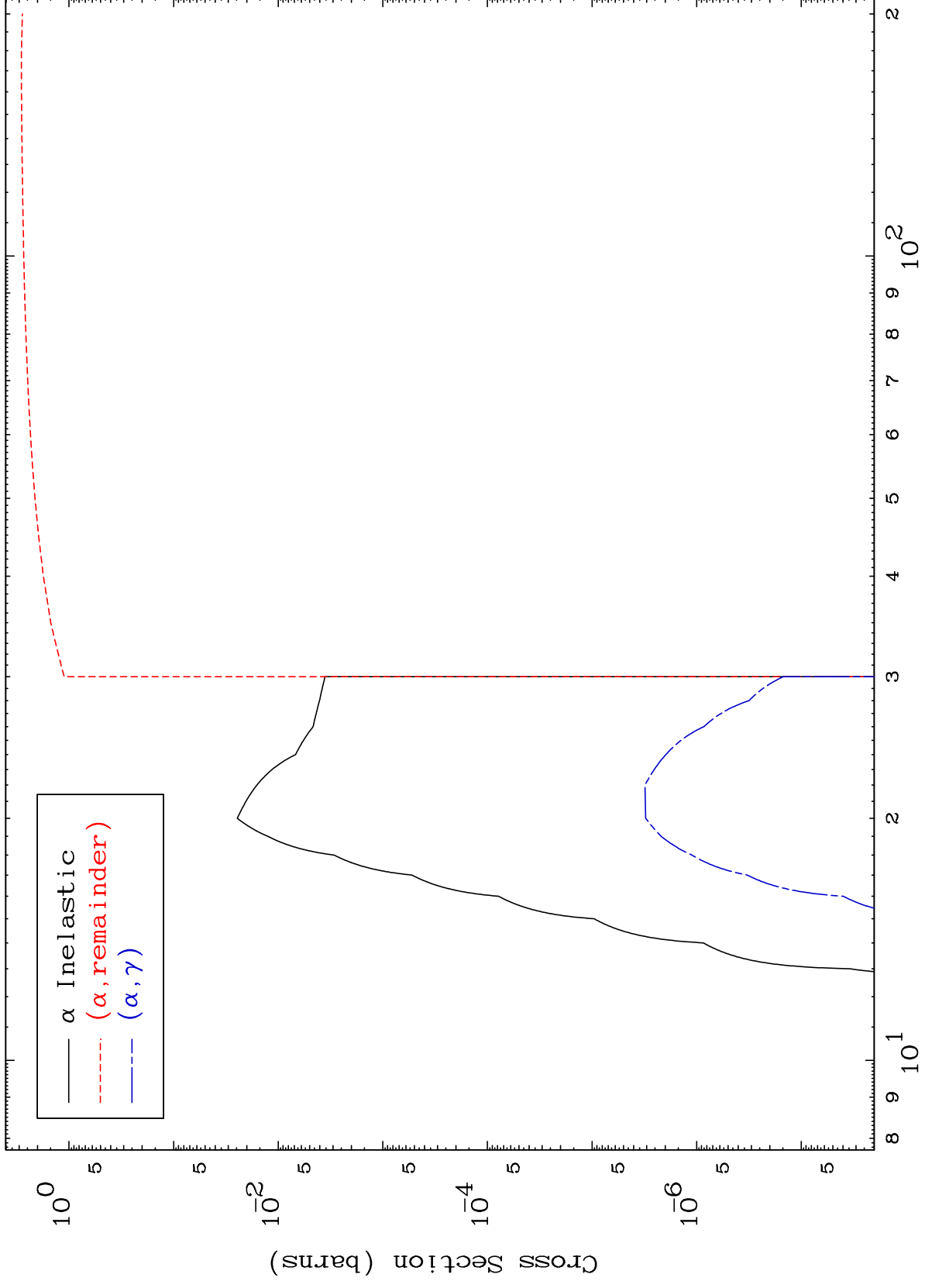
Press Mouse Button to Start

MAT 8334

$\alpha$  Major

83-Bi-212

0 Kelvin Cross Sections



$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

1

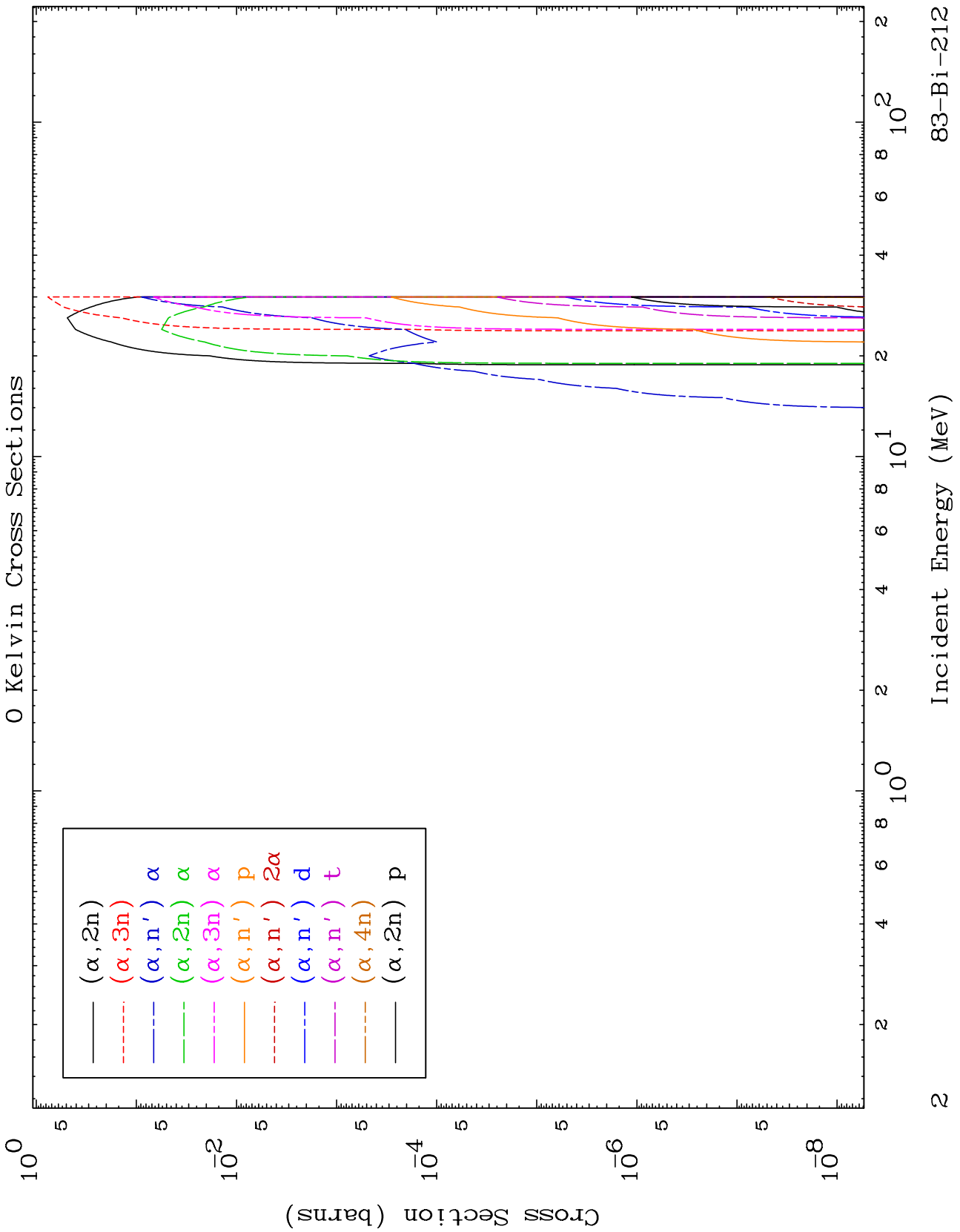
Incident Energy (MeV)

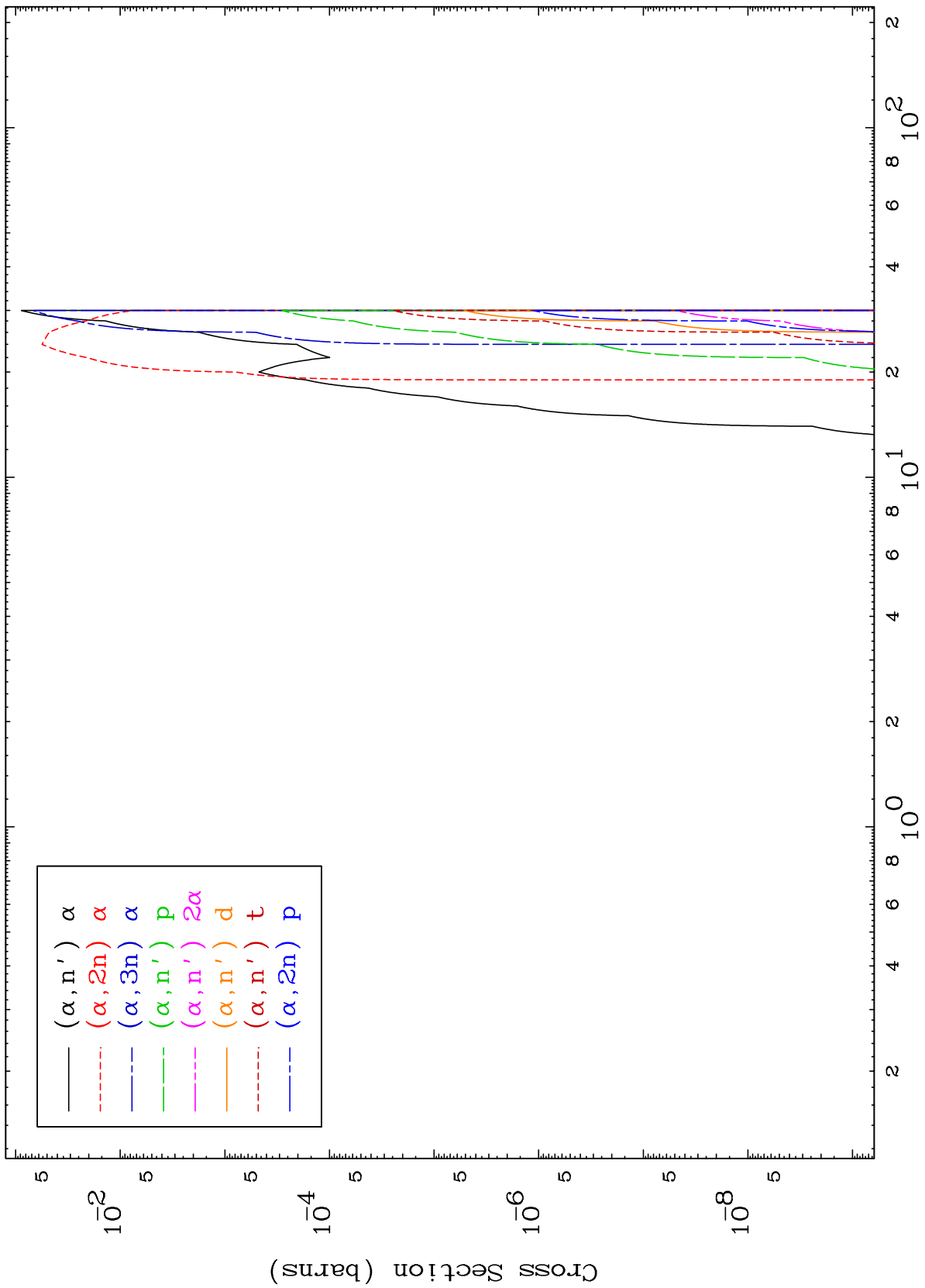
83-Bi-212

MAT 8334

$\alpha$  Neutron Production

83-Bi-212

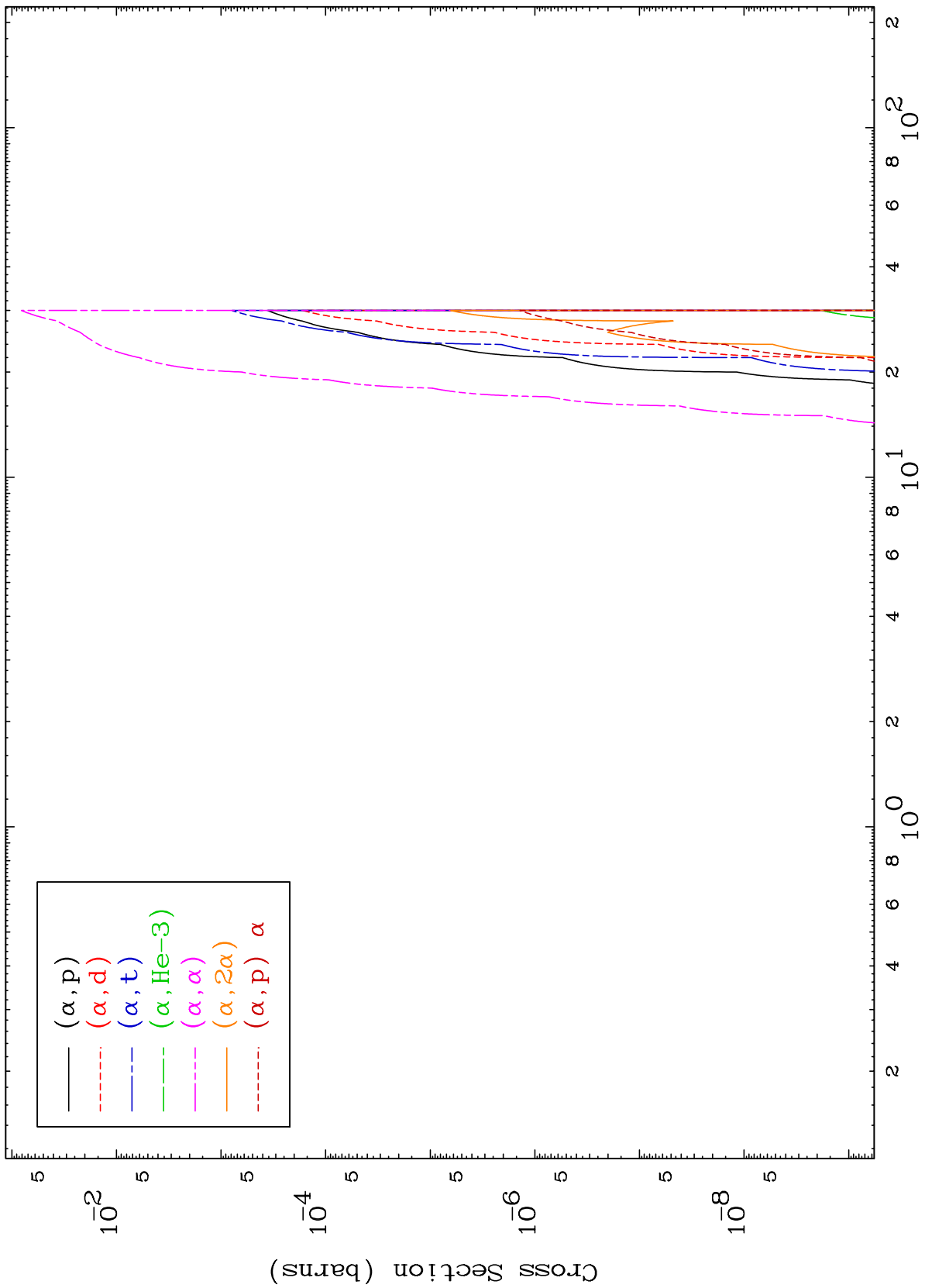




MAT 8334

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-212

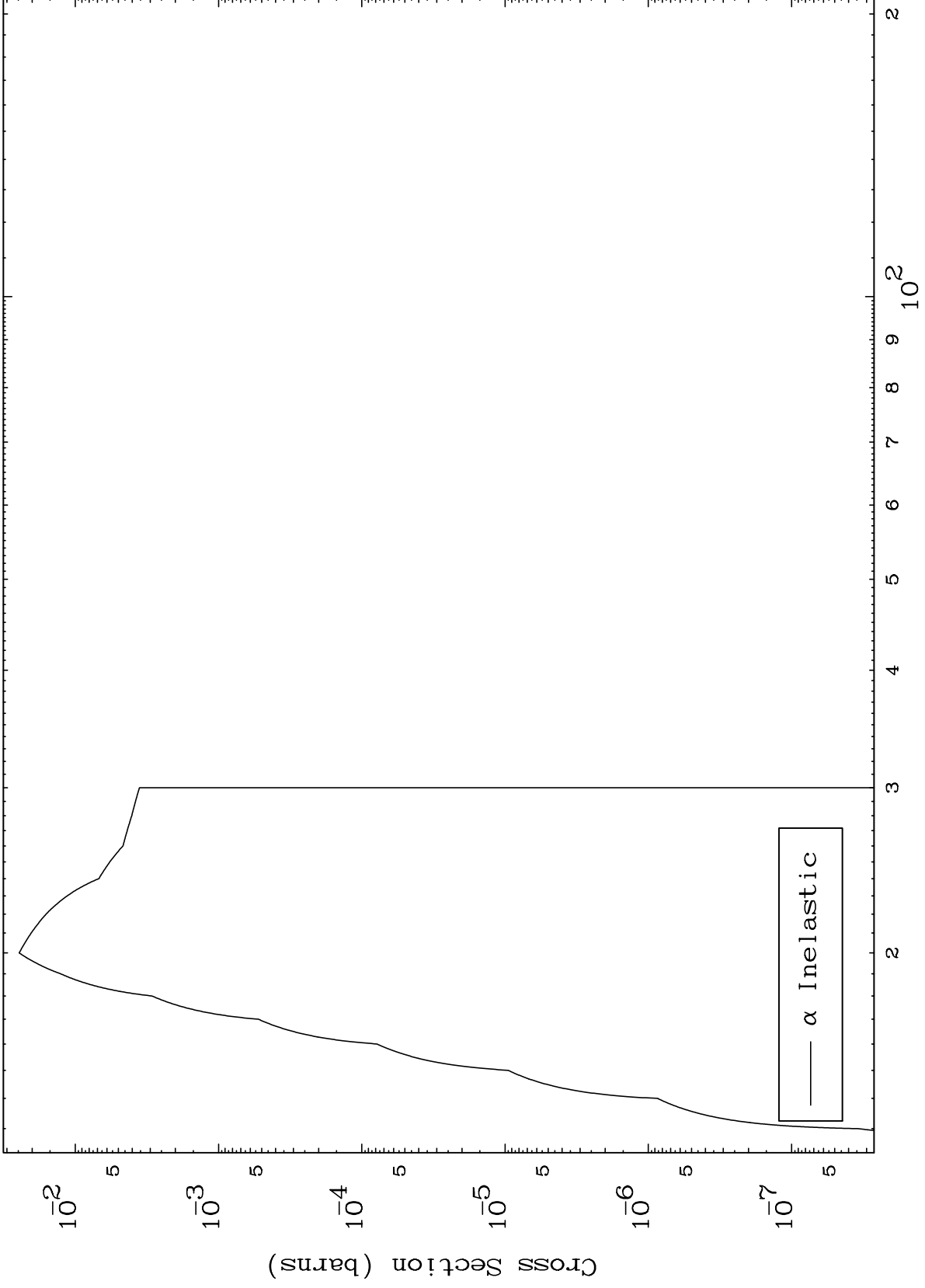


MAT 8334

( $\alpha, n'$ ) Level

83-Bi-212

0 Kelvin Cross Sections



5

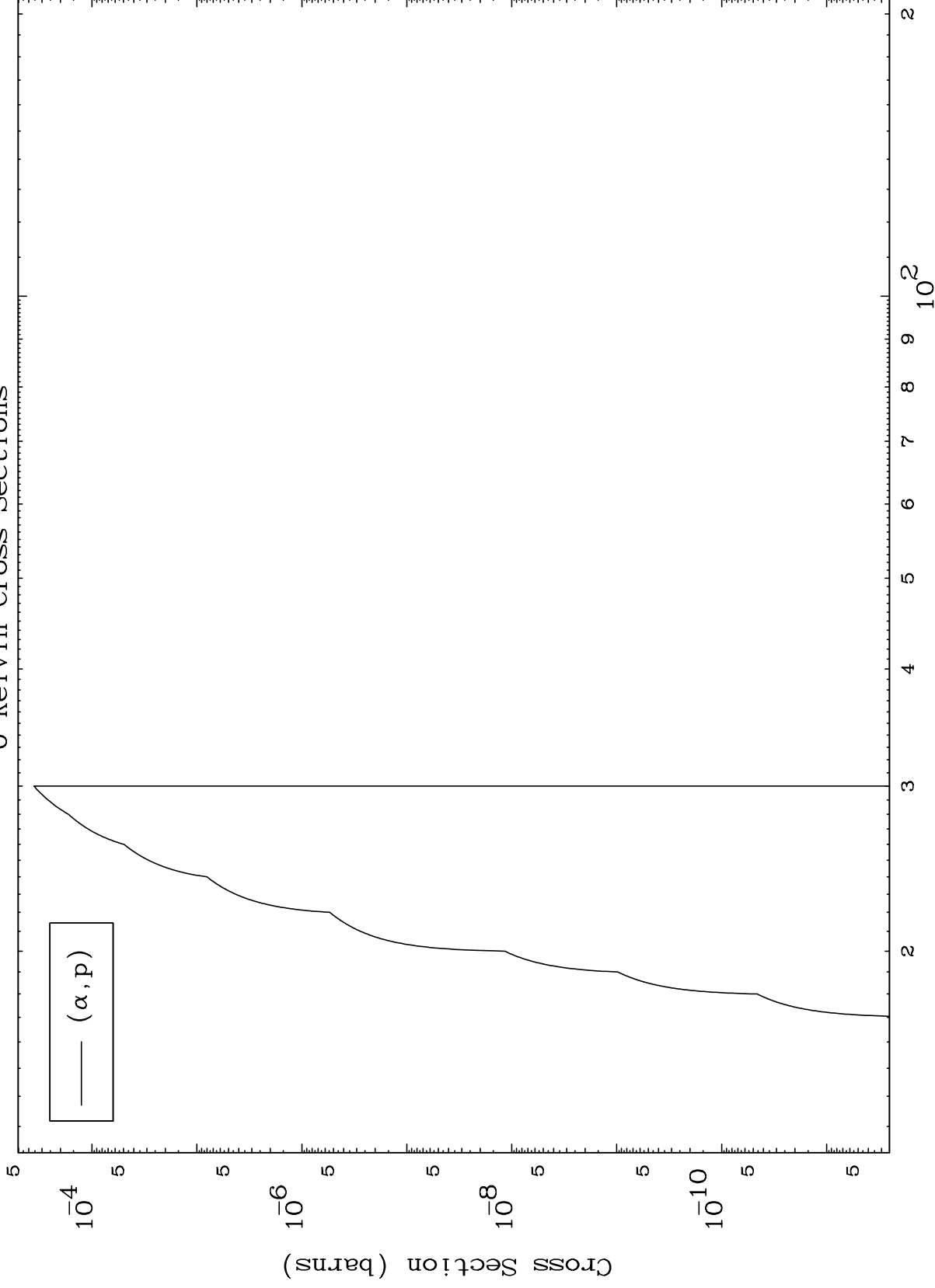
Incident Energy (MeV)

83-Bi-212

MAT 8334

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-212



6

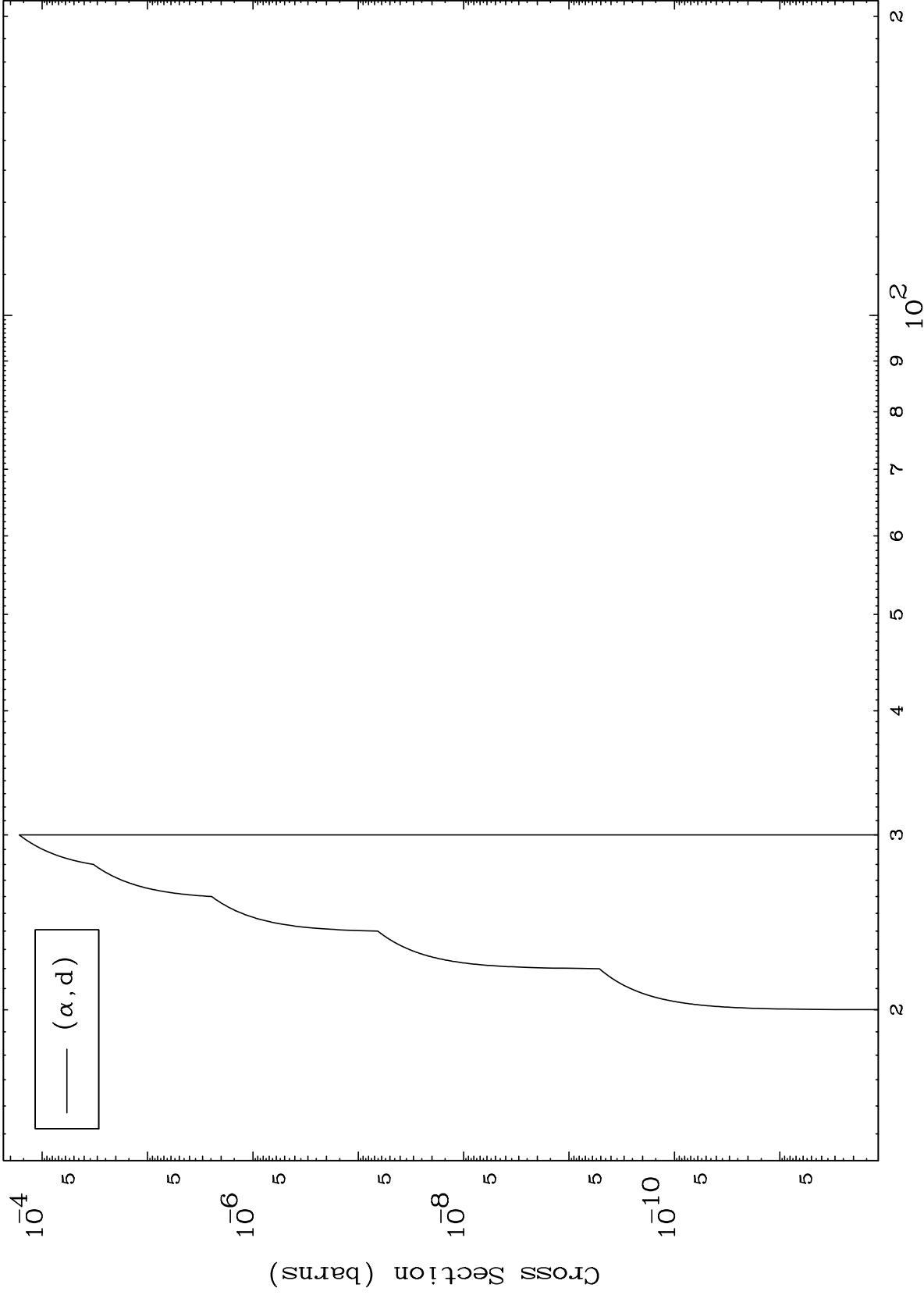
Incident Energy (MeV)

83-Bi-212

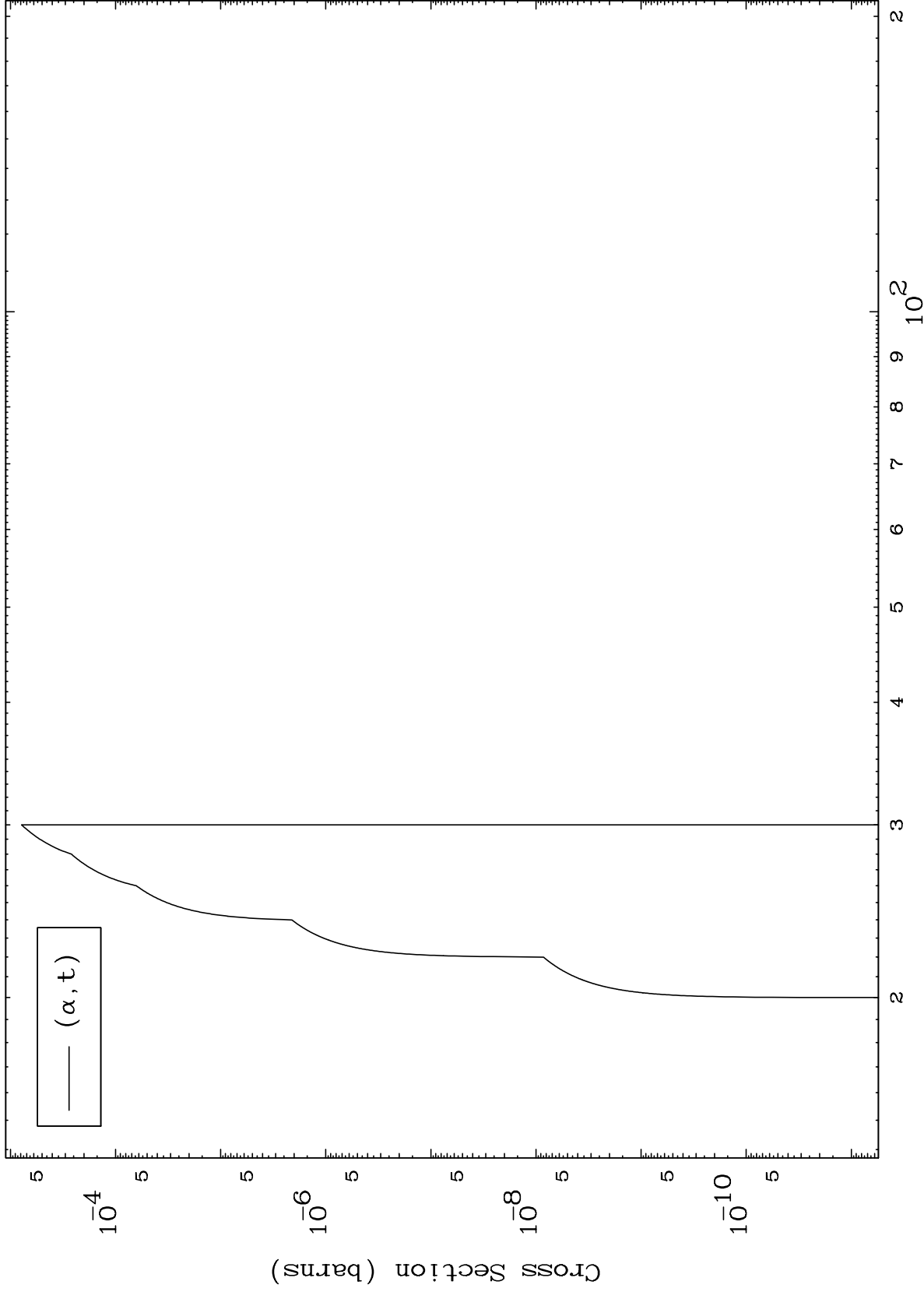
MAT 8334

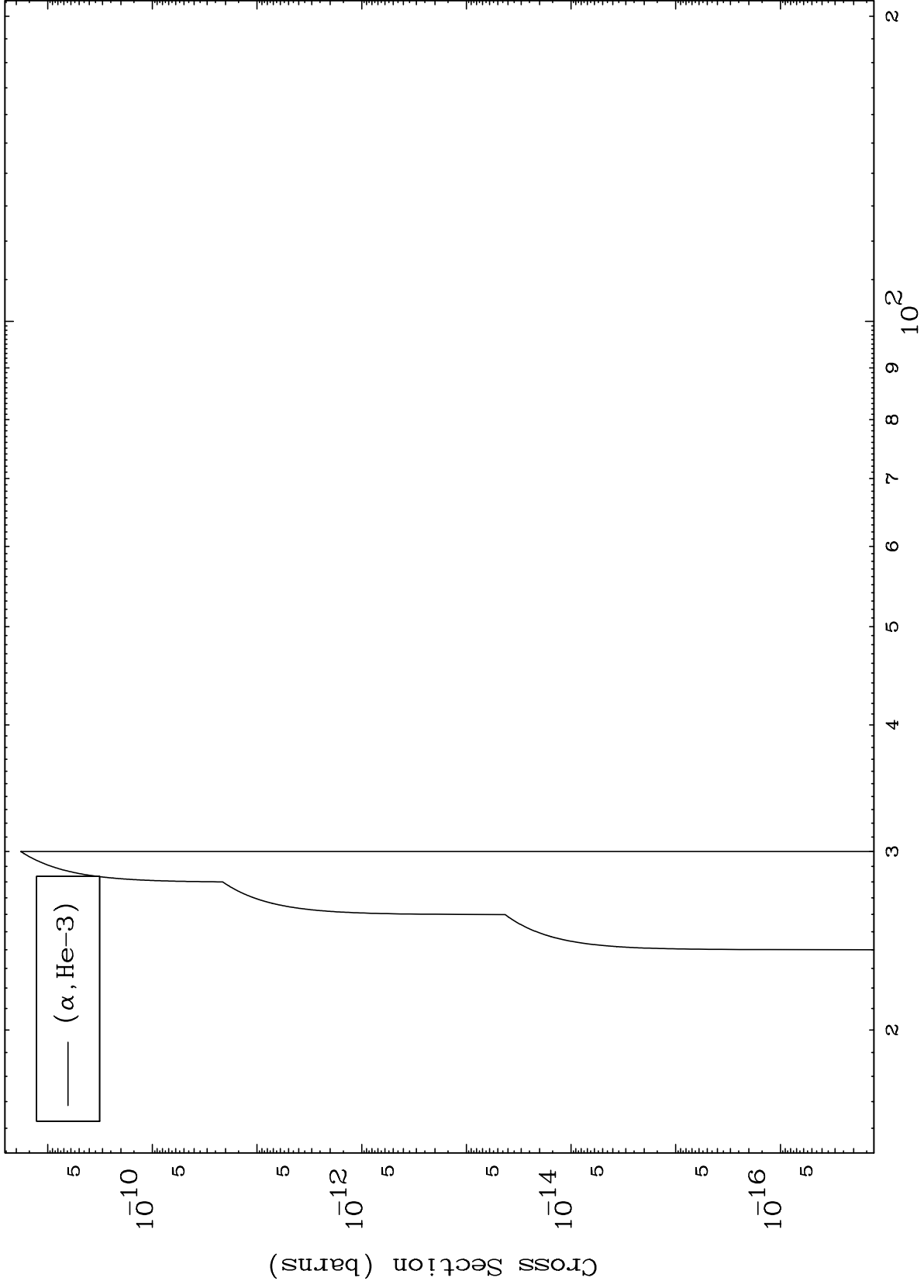
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

83-Bi-212







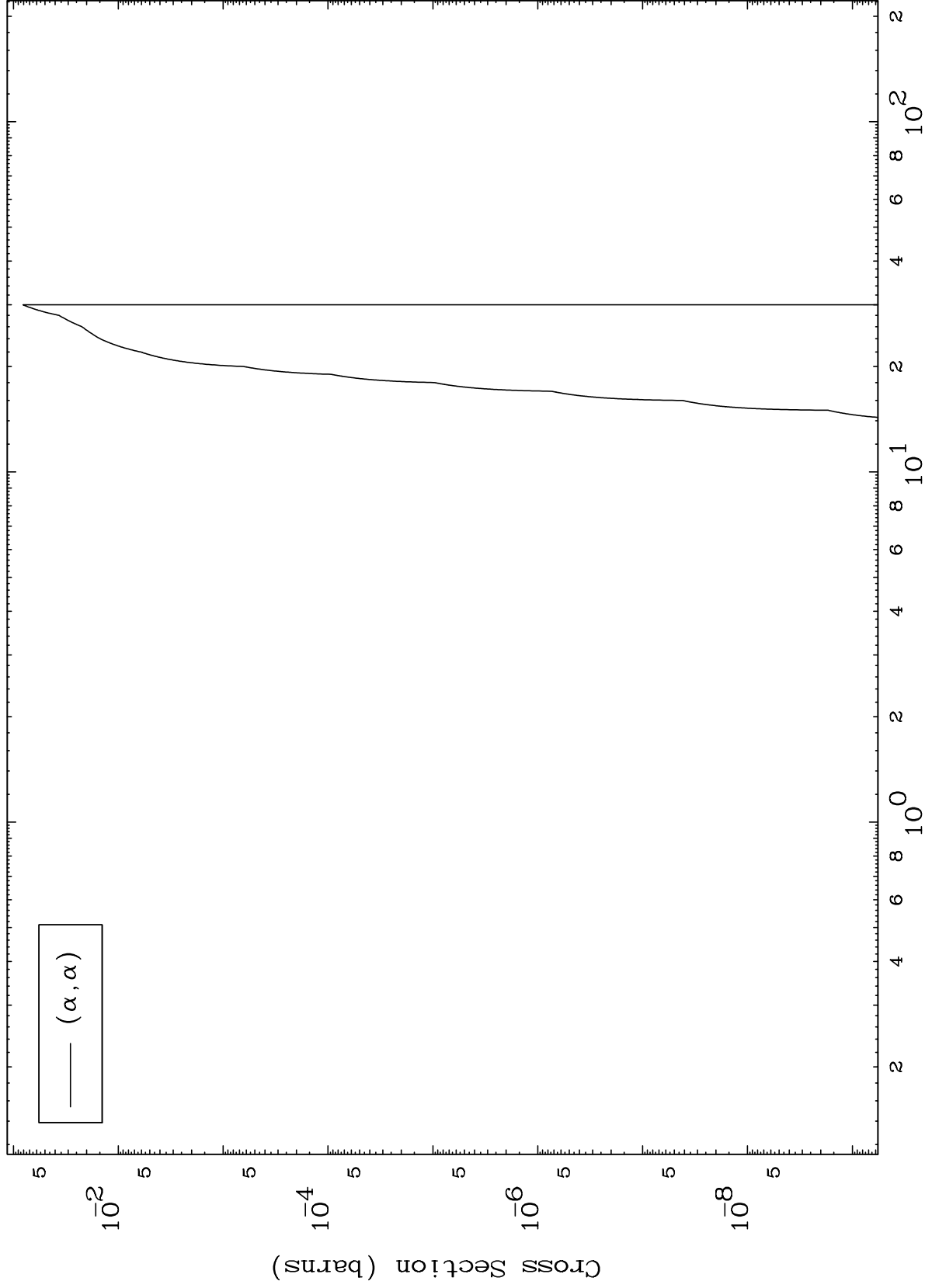


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( $\alpha, \alpha$ ) Levels

83-Bi-212

0 Kelvin Cross Sections



10

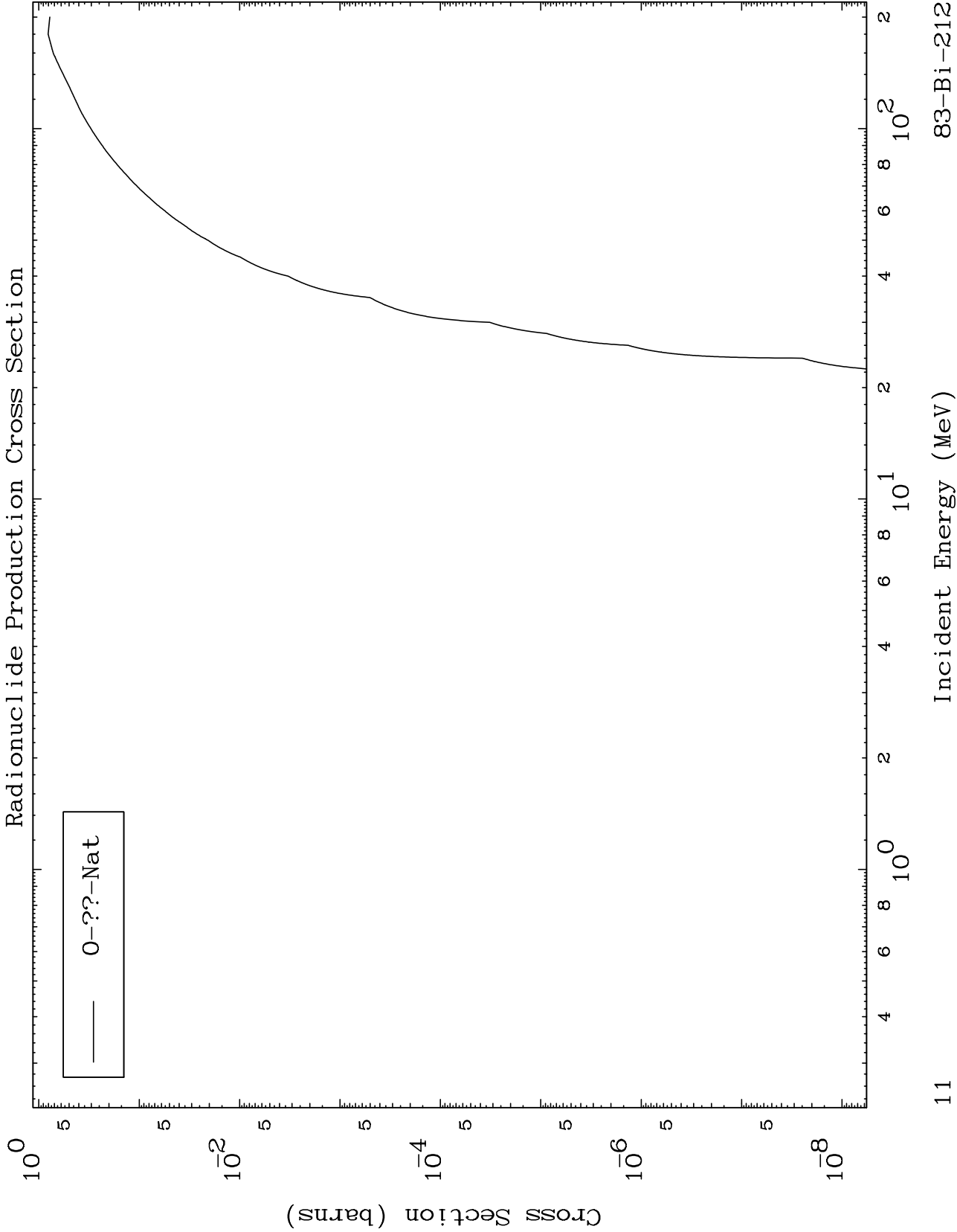
Incident Energy (MeV)

83-Bi-212

MAT 8334

$\alpha$  Fission

83-Bi-212

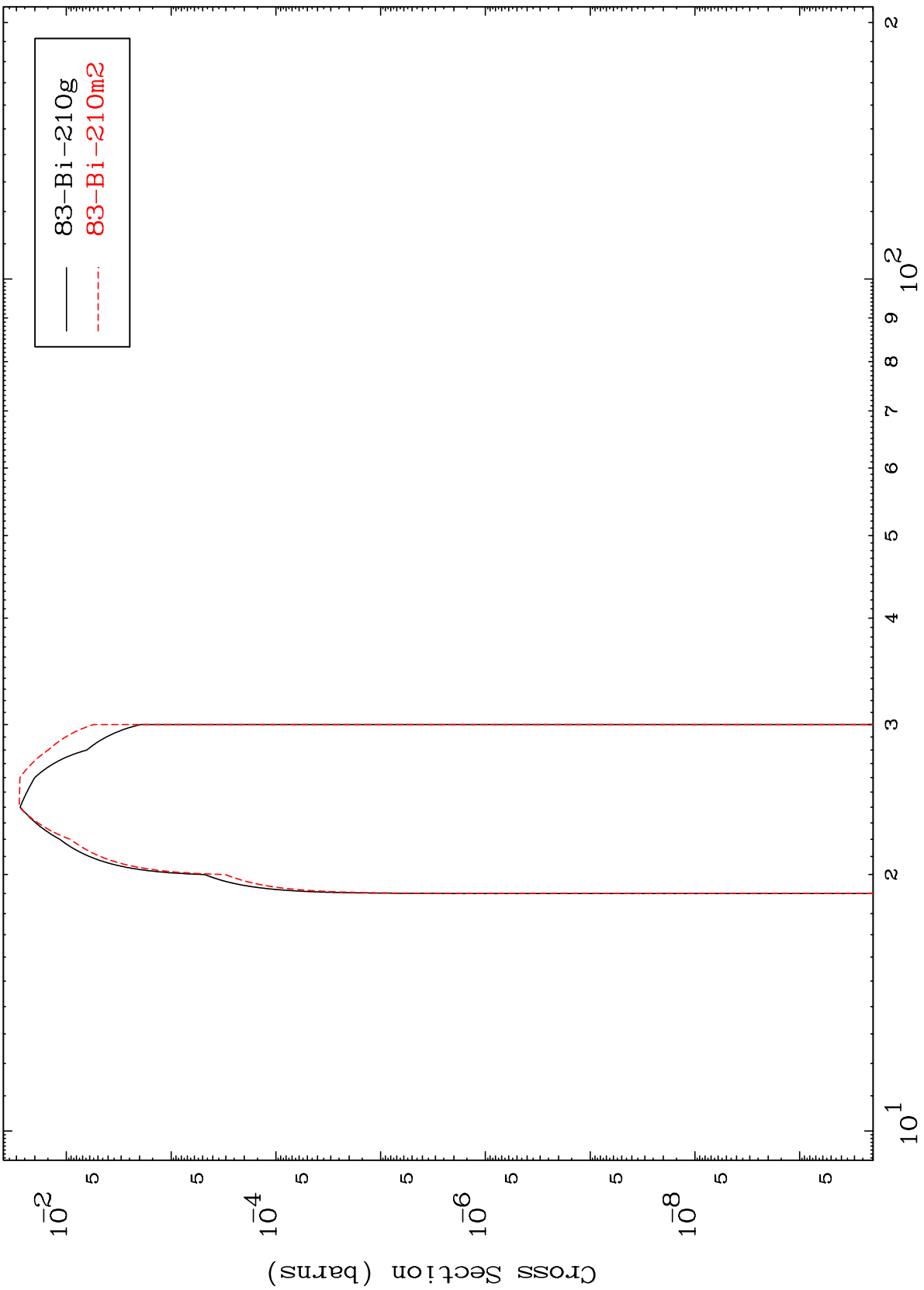


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83-Bi-212

$(\alpha, 2n)$   $\alpha$

Radionuclide Production Cross Section



83-Bi-212

Incident Energy (MeV)

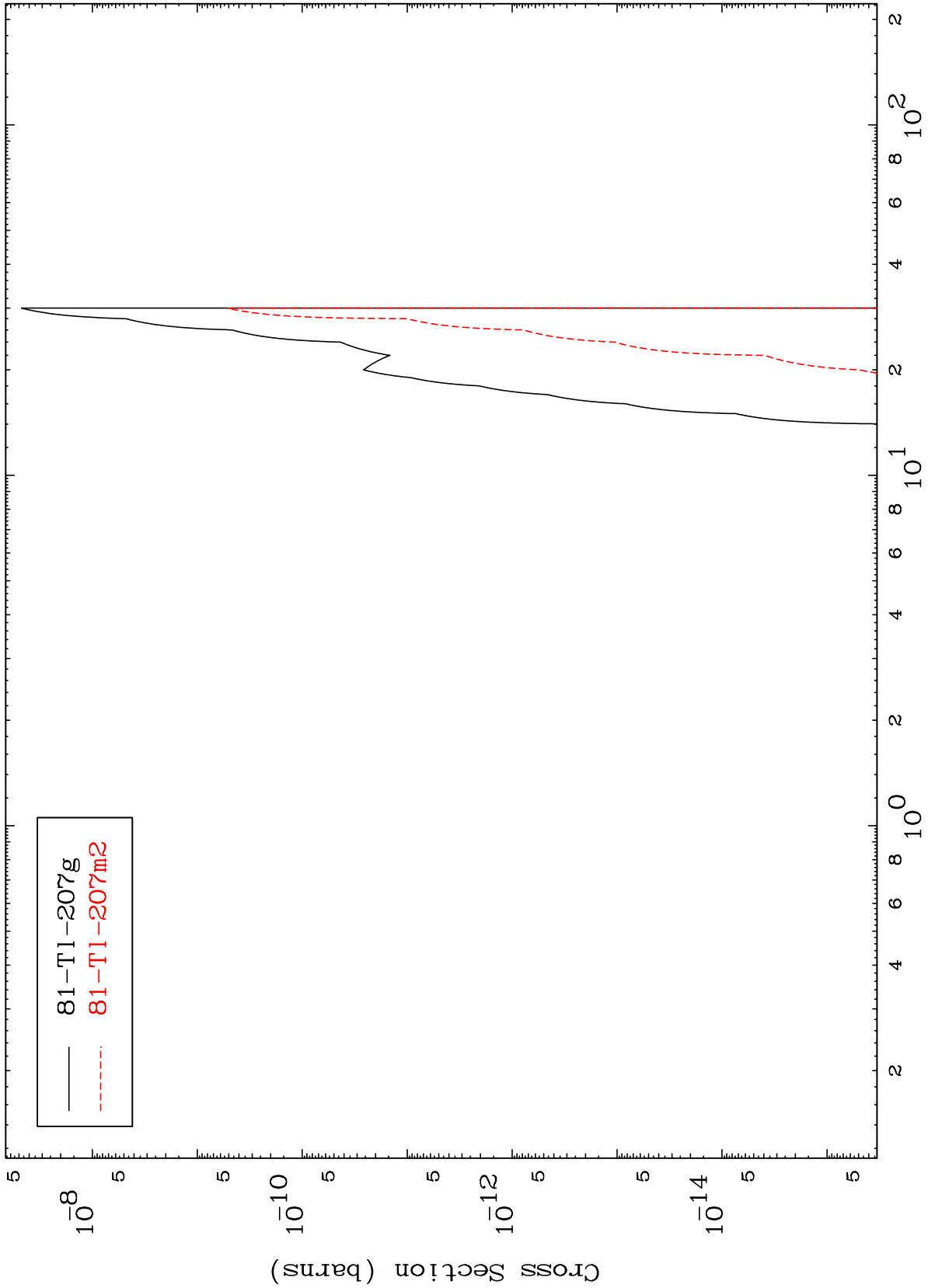
12

MAT 8334

( $\alpha, n'$ )  $2\alpha$

83-Bi-212

Radionuclide Production Cross Section

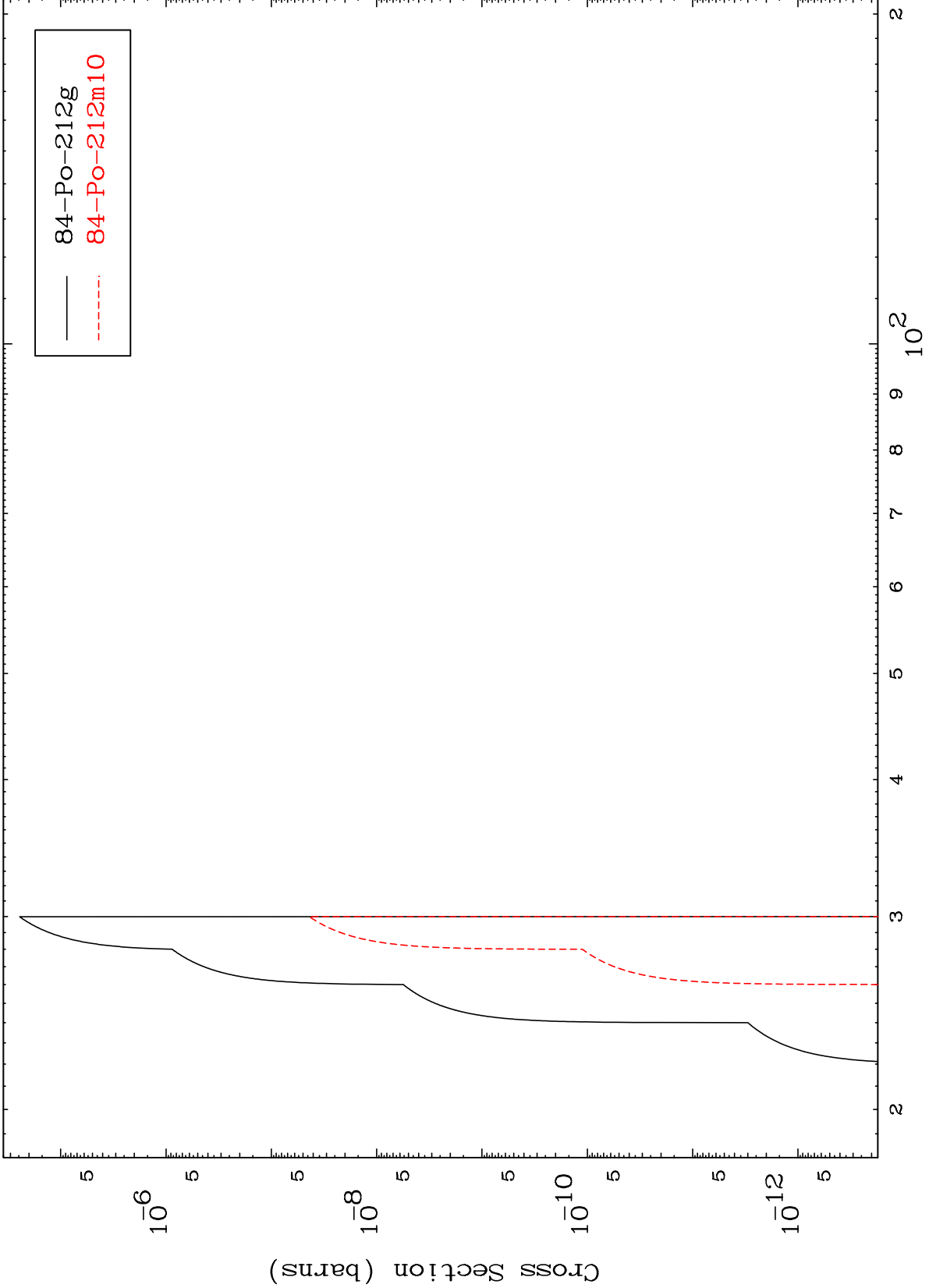


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( $\alpha, n'$ ) t

83-Bi-212

Radionuclide Production Cross Section



14

Incident Energy (MeV)

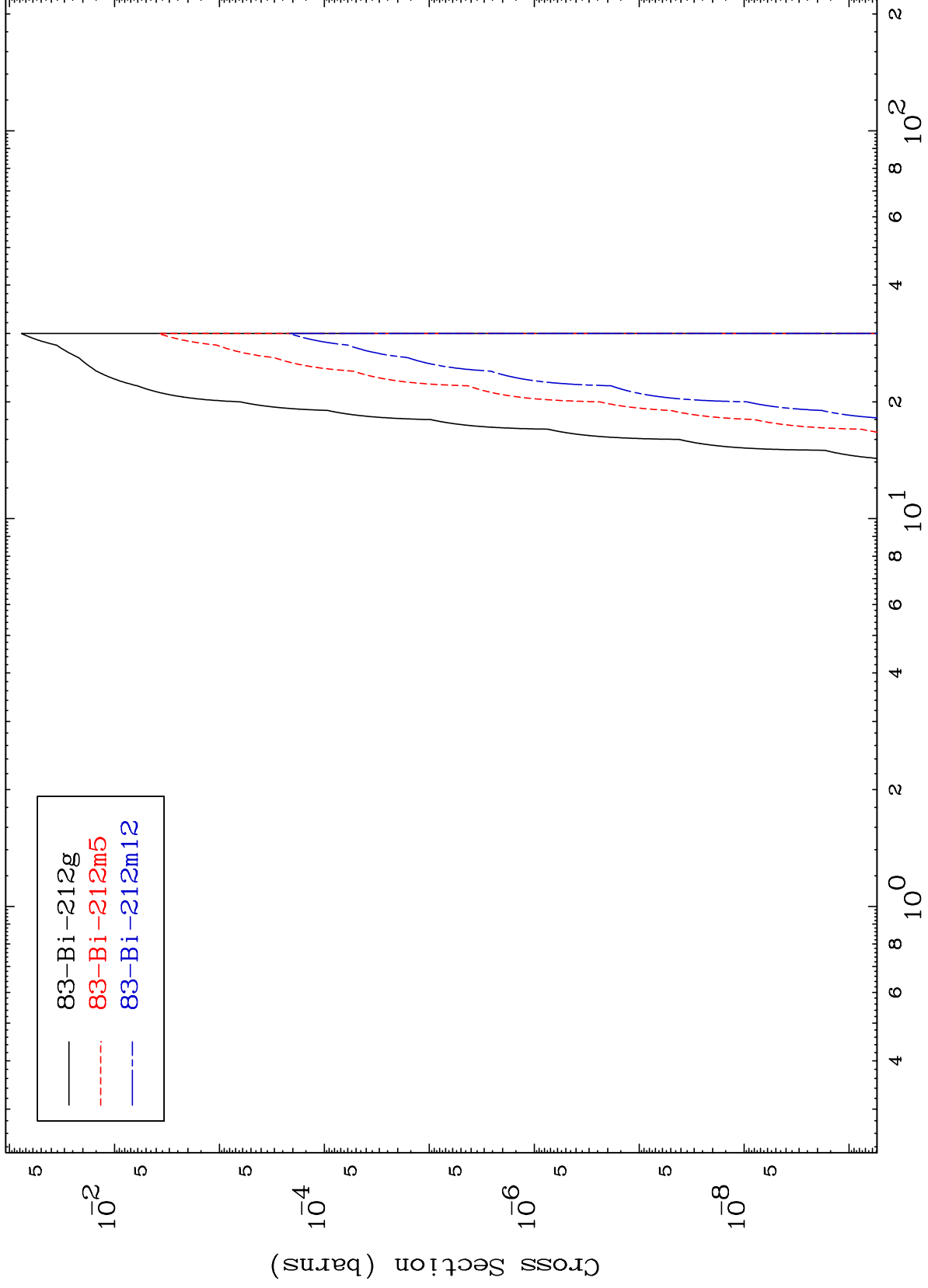
83-Bi-212

MAT 8334

( $\alpha, \alpha$ )

83-Bi-212

Radionuclide Production Cross Section



15

Incident Energy (MeV)

83-Bi-212