

Program EVALPLOT  
(Version 2018-1)

by

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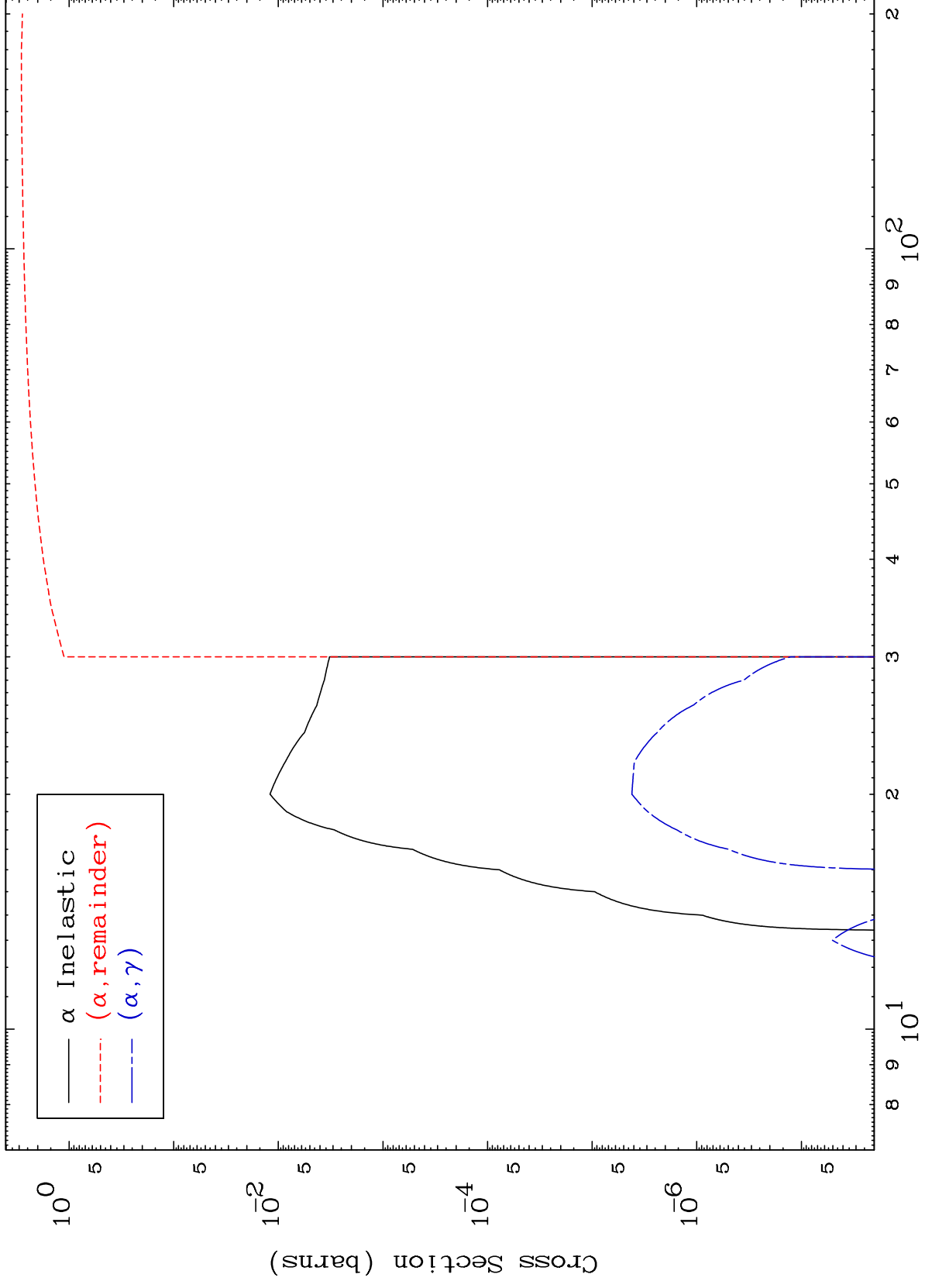
Press Mouse Button to Start

MAT 8337

$\alpha$  Major

83-Bi-213

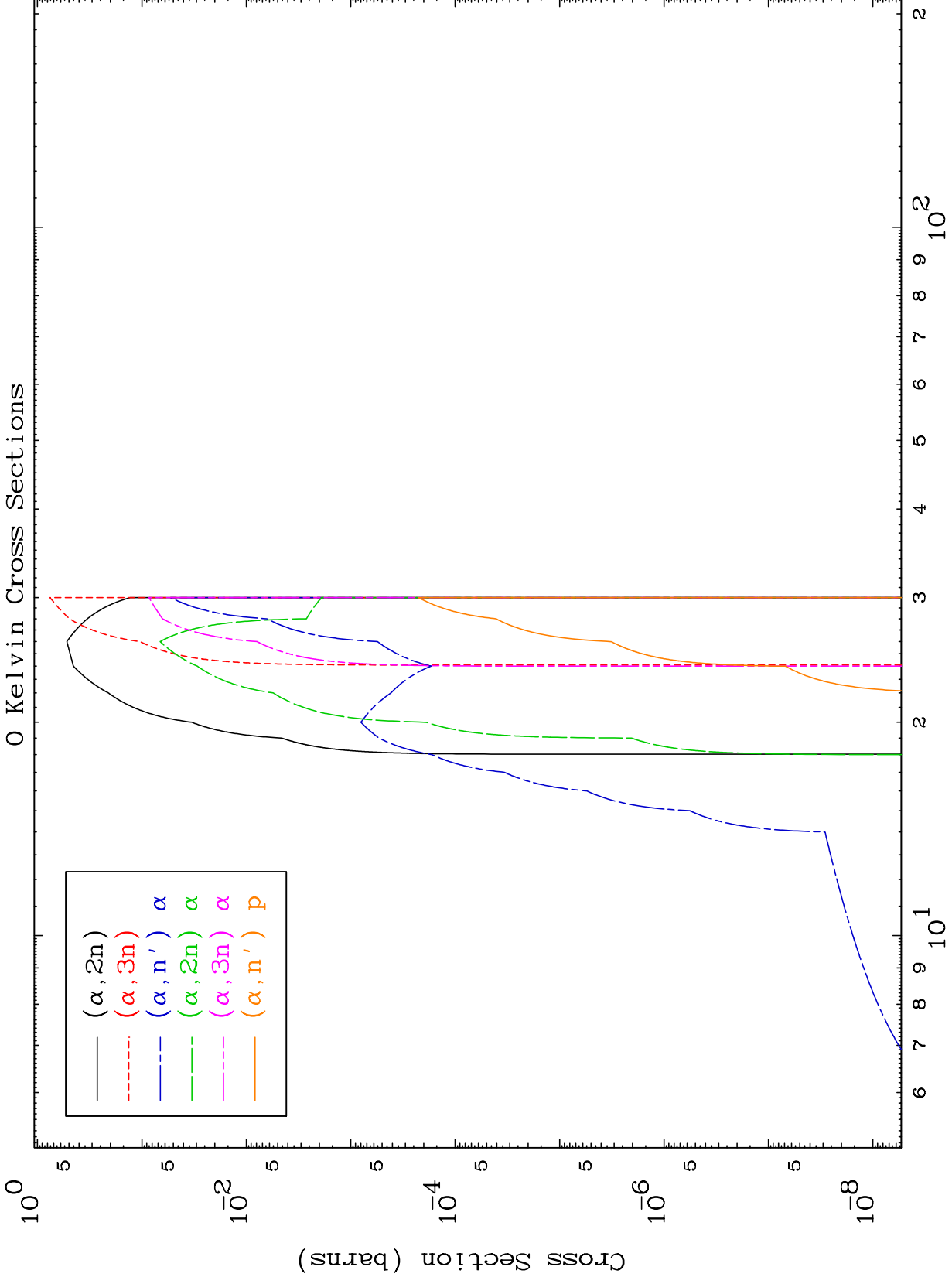
0 Kelvin Cross Sections



1

Incident Energy (MeV)

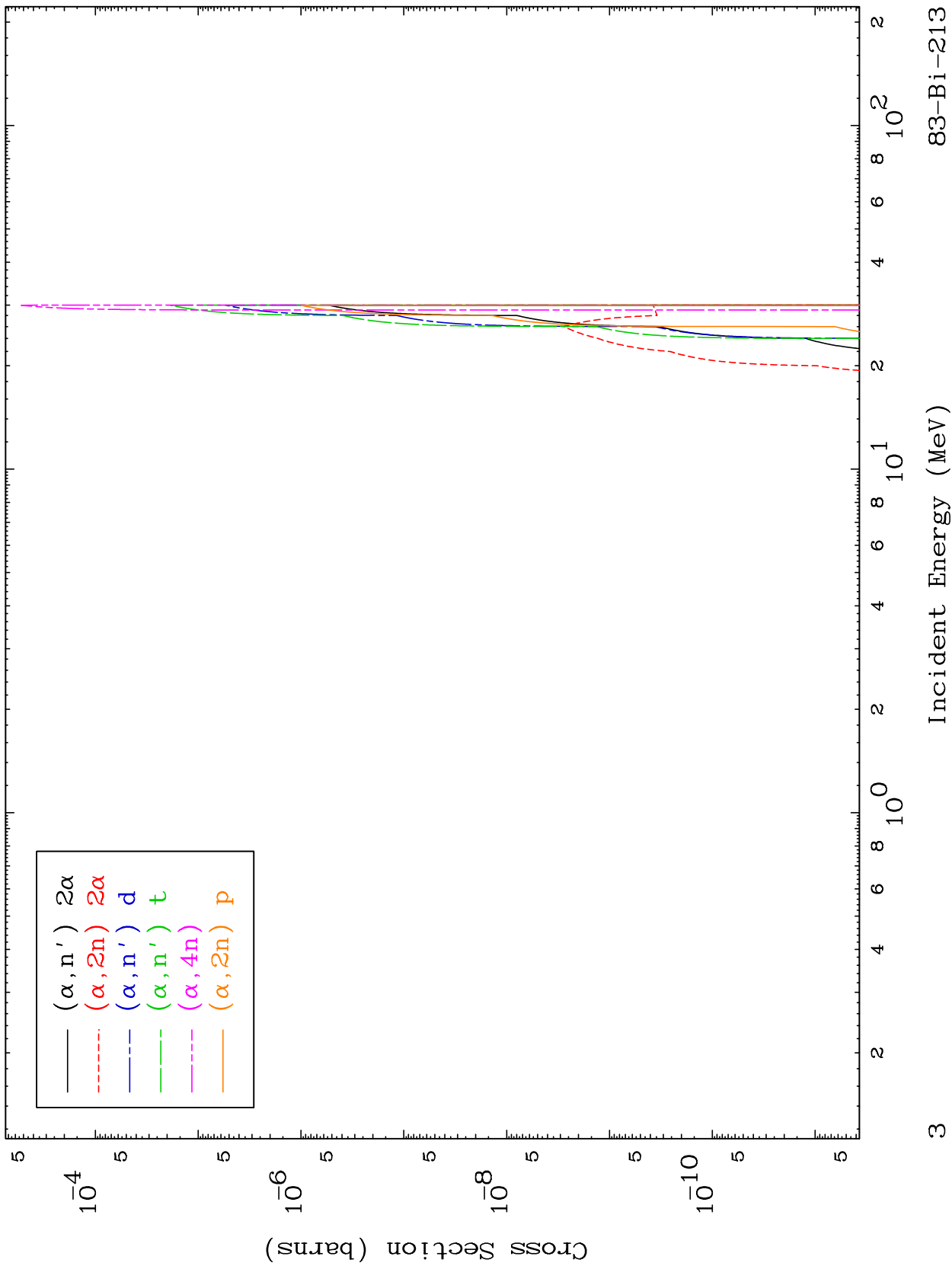
83-Bi-213



MAT 8337

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

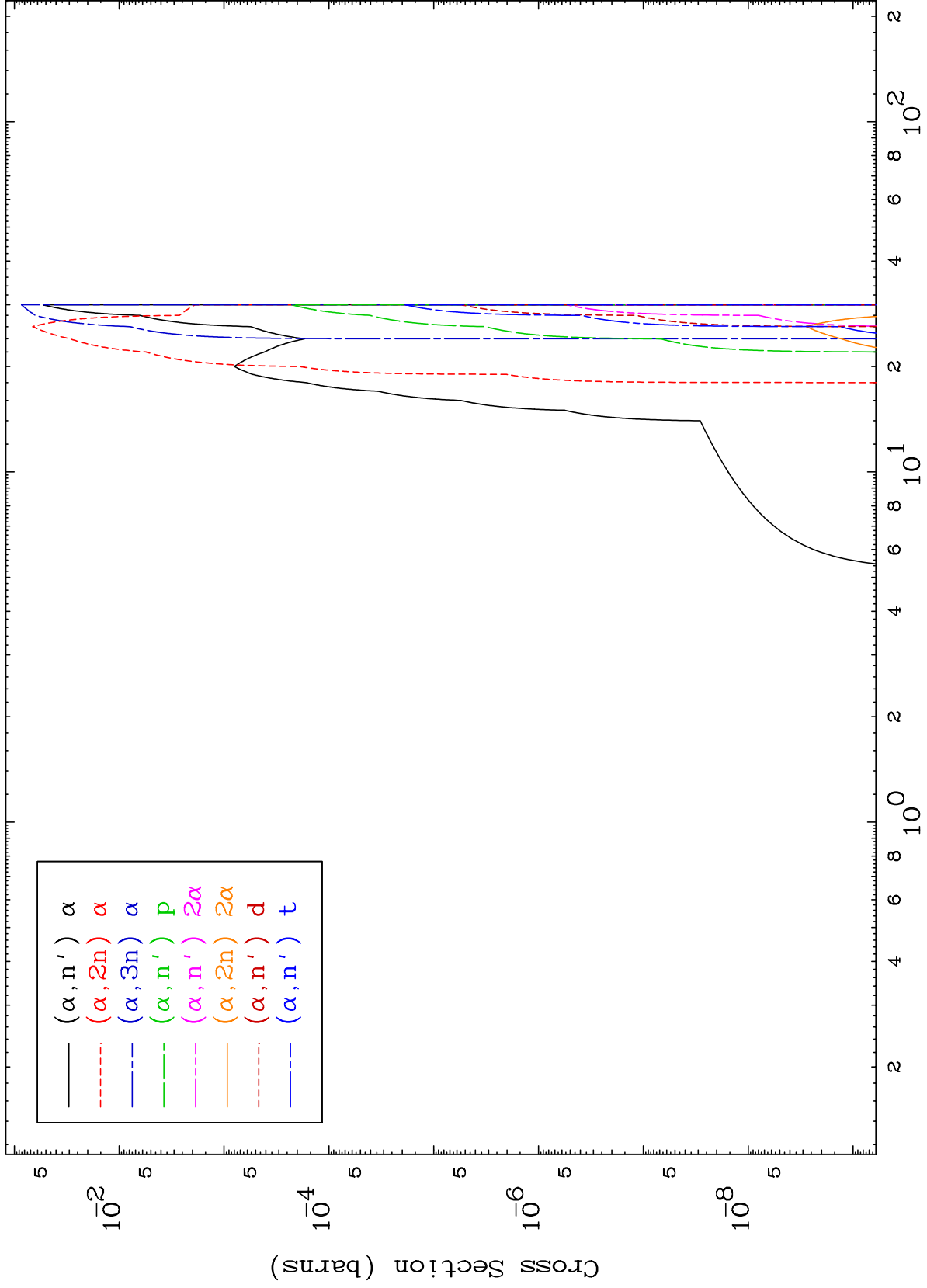
83-Bi-213

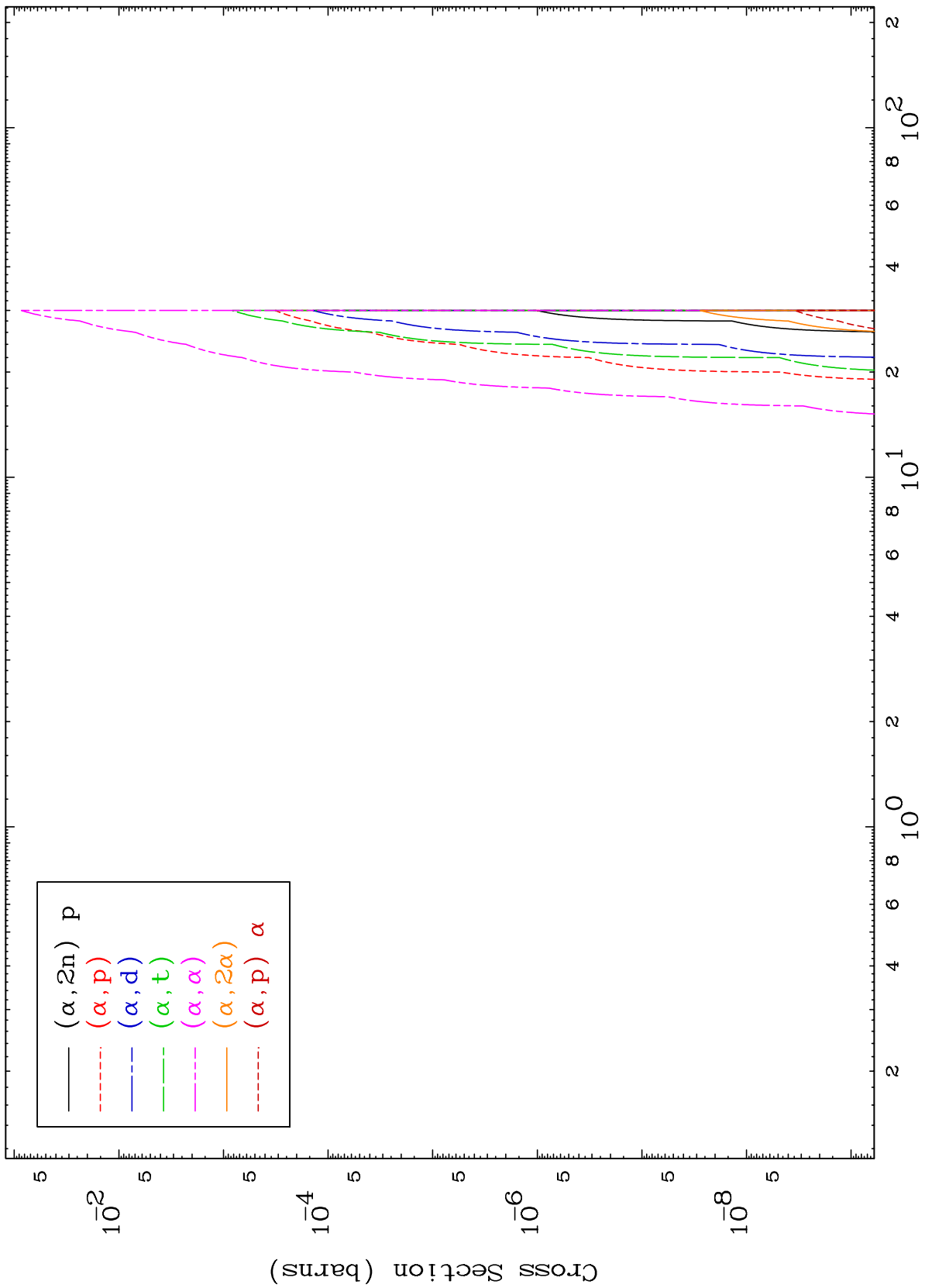


MAT 8337

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-213



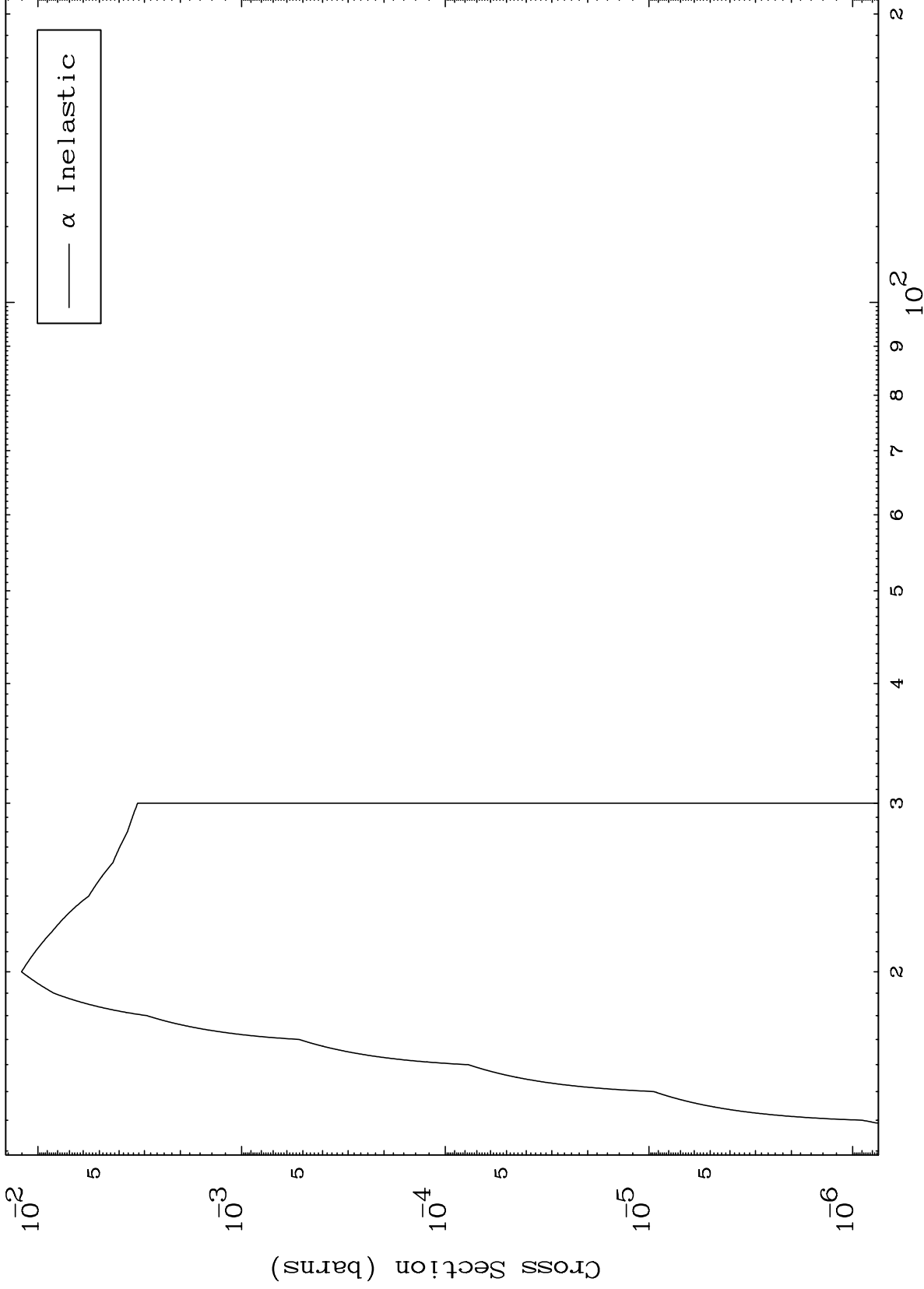


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( $\alpha, n'$ ) Level

83-Bi-213

0 Kelvin Cross Sections



6

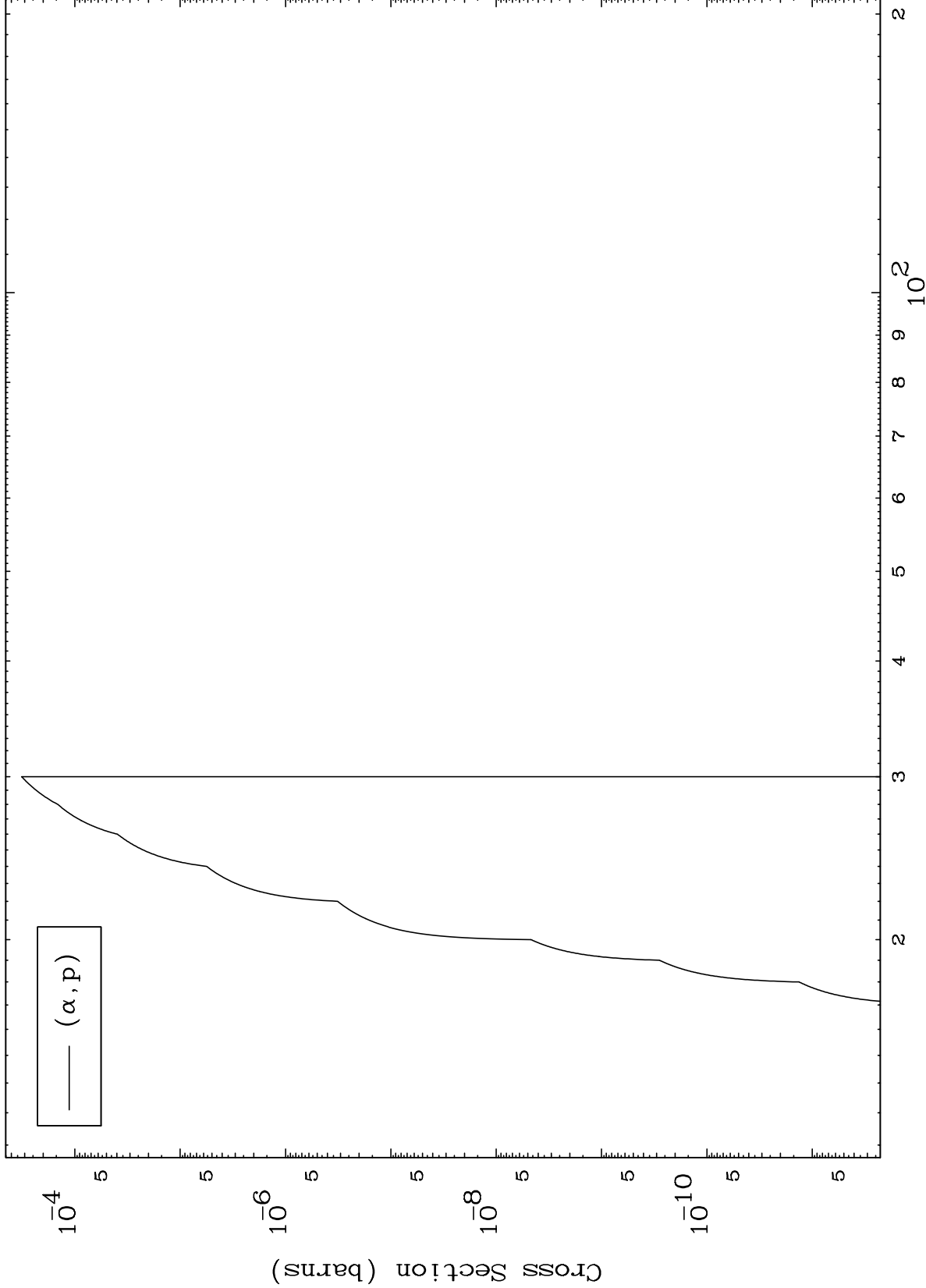
Incident Energy (MeV)

83-Bi-213

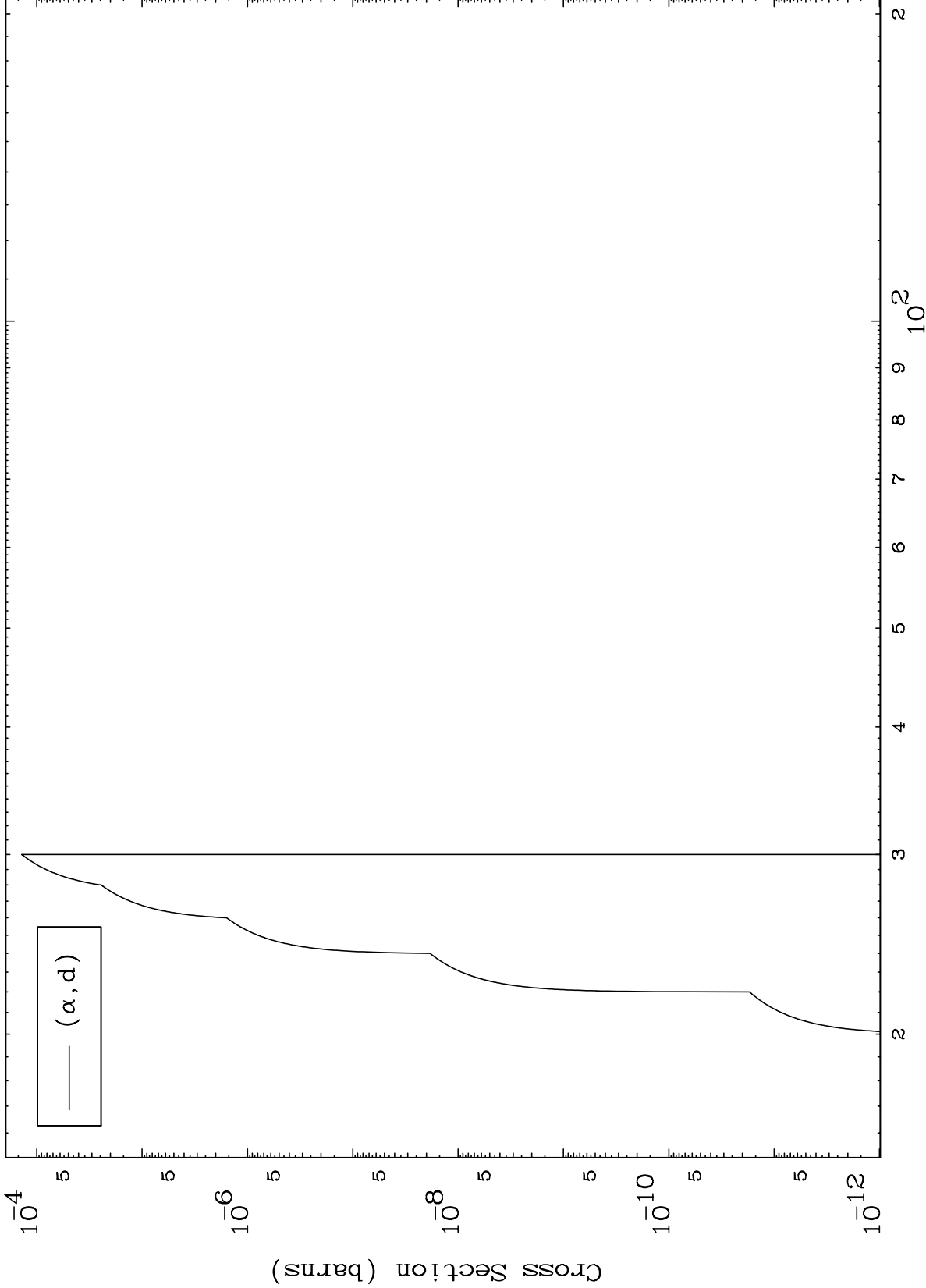
MAT 8337

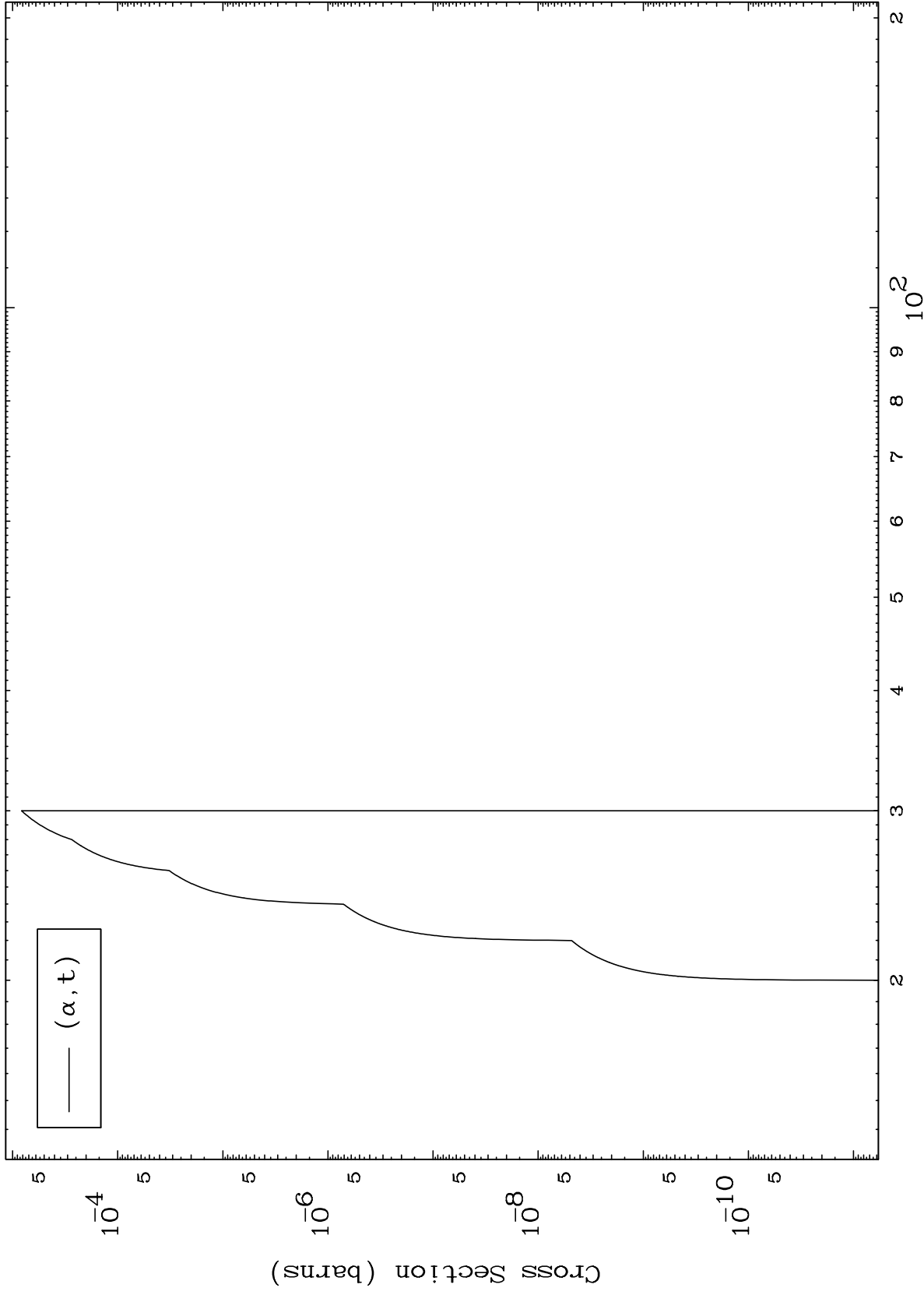
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-213







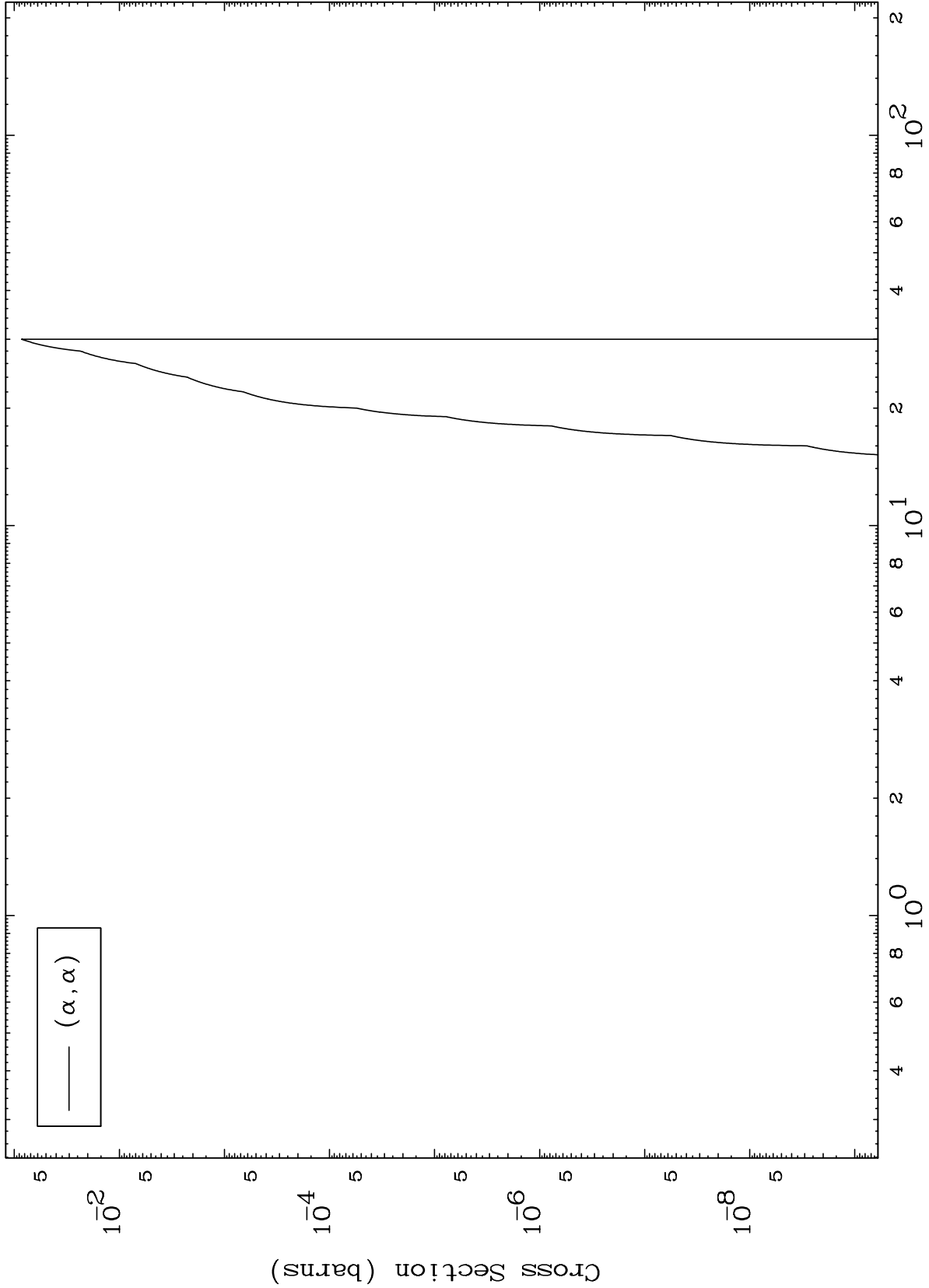


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( $\alpha, \alpha$ ) Levels

83-Bi-213

0 Kelvin Cross Sections



10

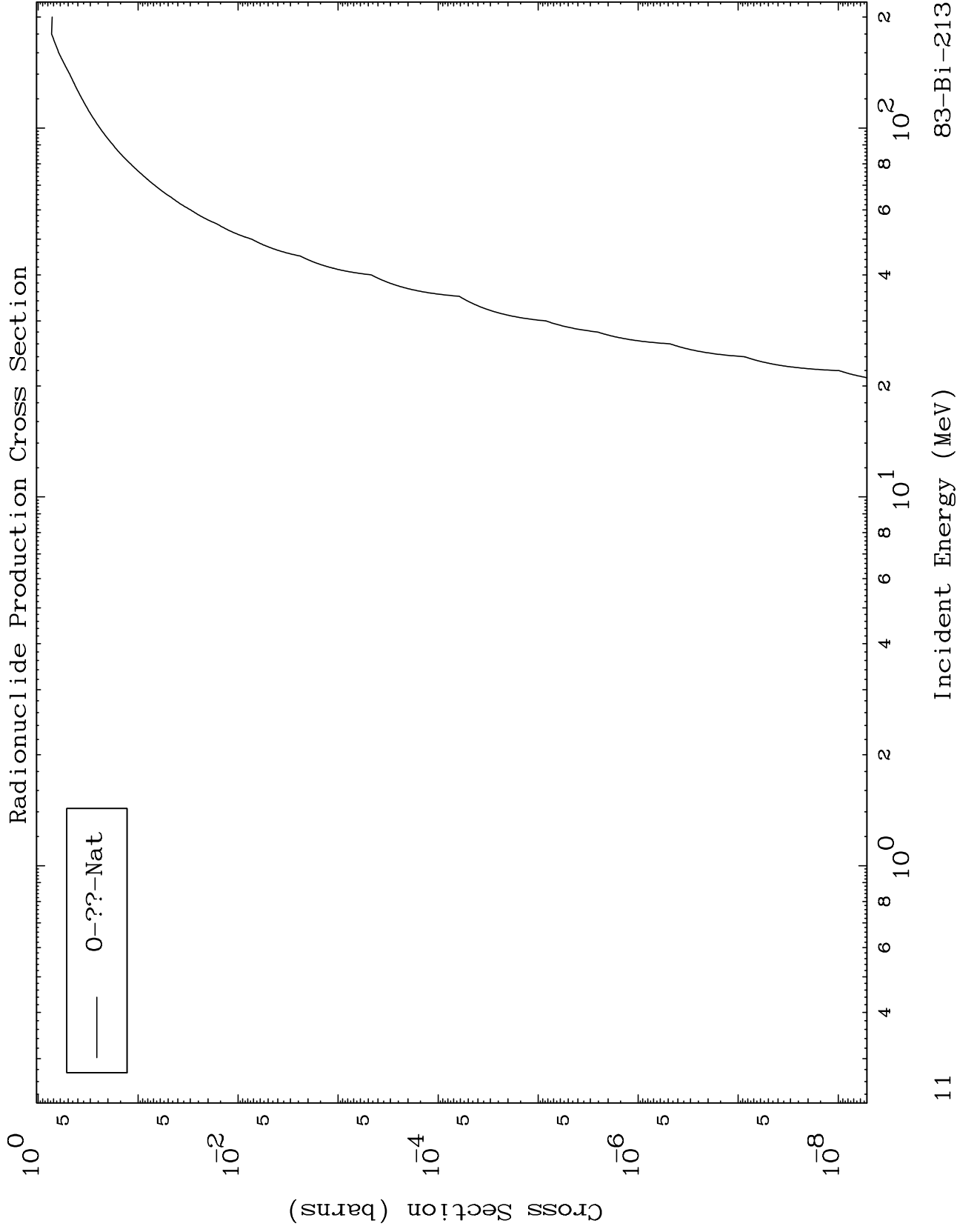
Incident Energy (MeV)

83-Bi-213

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$\alpha$  Fission

83-Bi-213

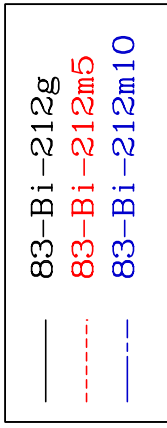
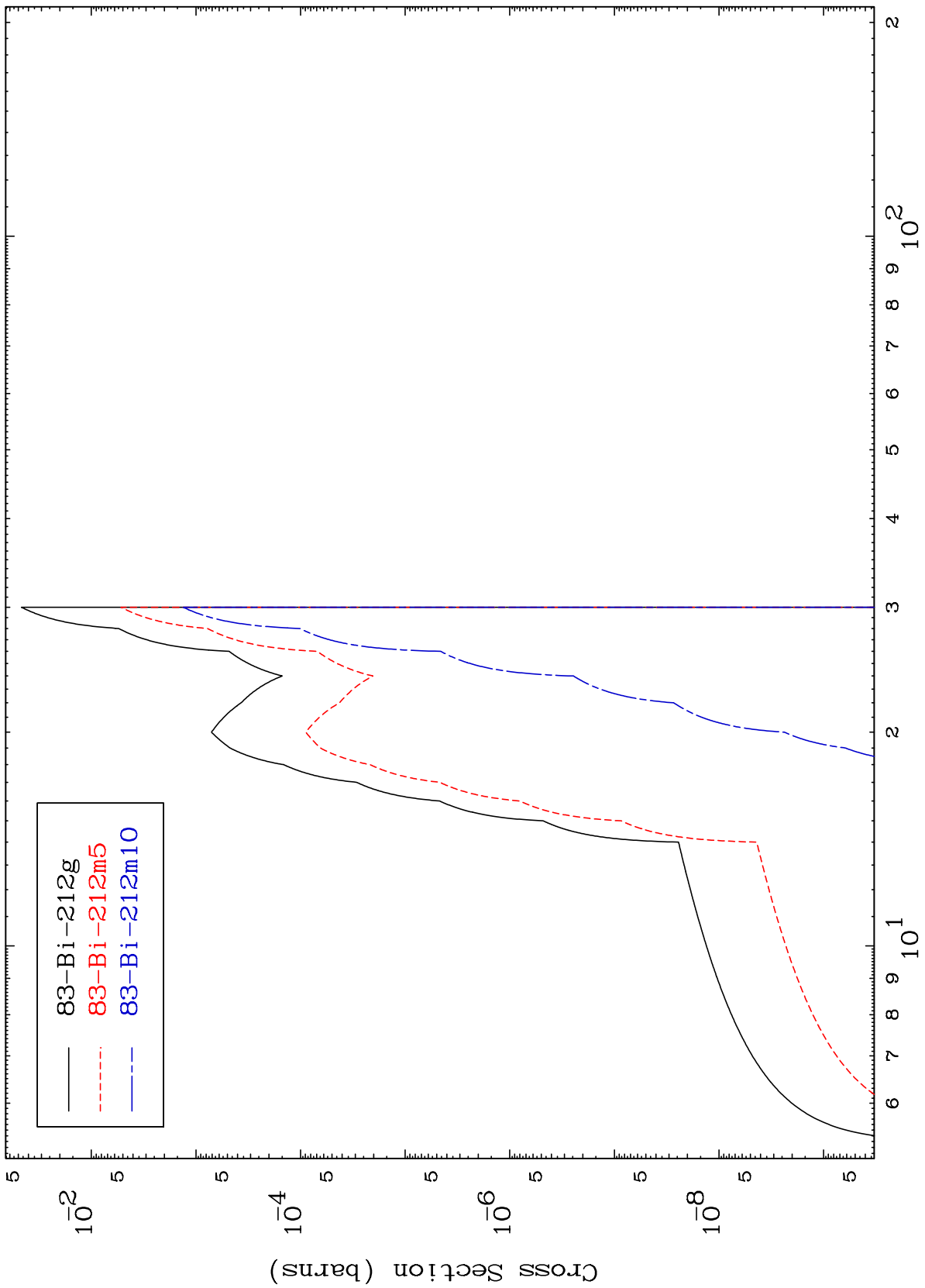


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$(\alpha, n')$   $\alpha$

83-Bi-213

Radionuclide Production Cross Section



12

Incident Energy (MeV)

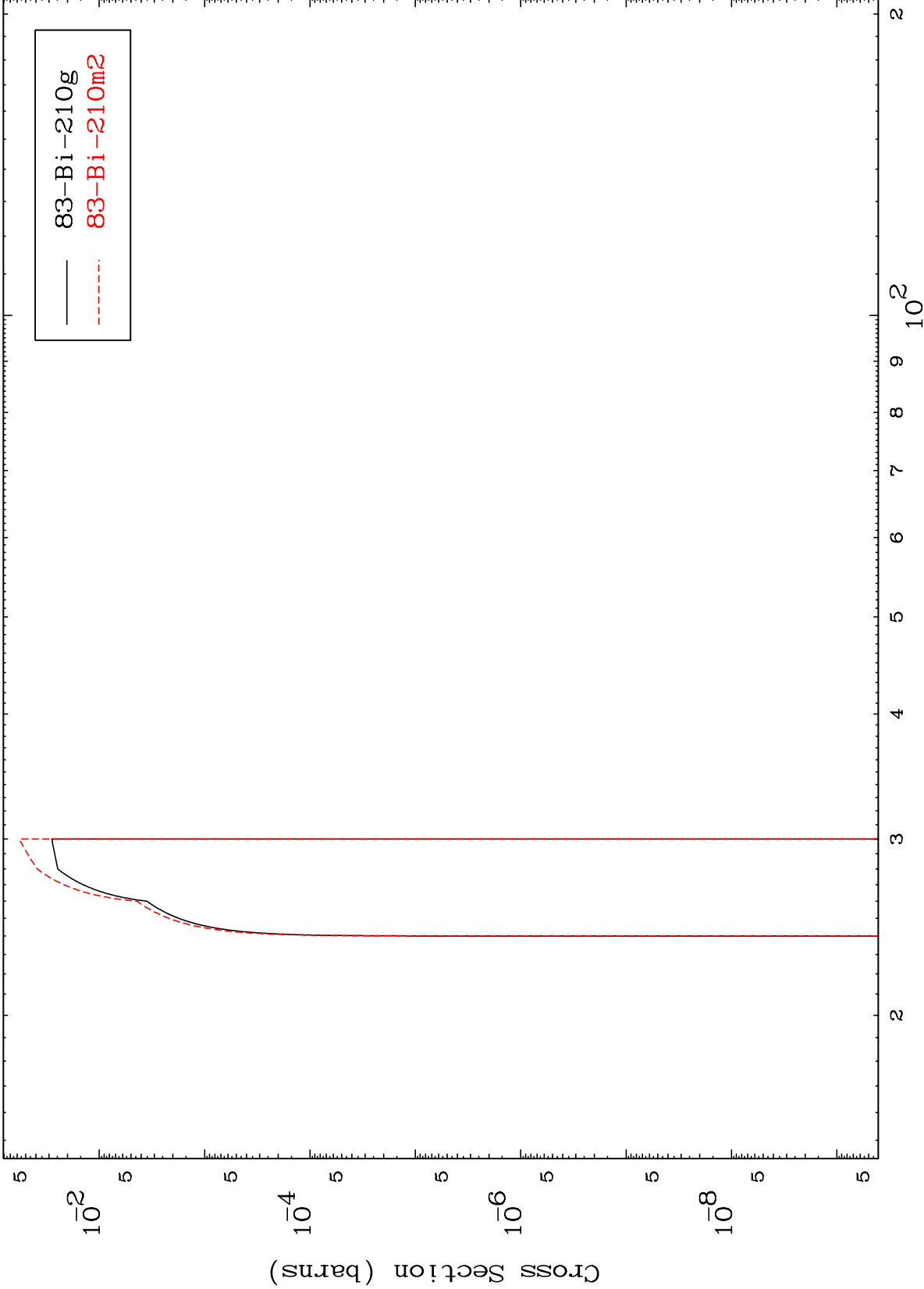
83-Bi-213

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$(\alpha, 3n)$   $\alpha$

83-Bi-213

Radionuclide Production Cross Section



83-Bi-210g  
83-Bi-210m2

13

Incident Energy (MeV)

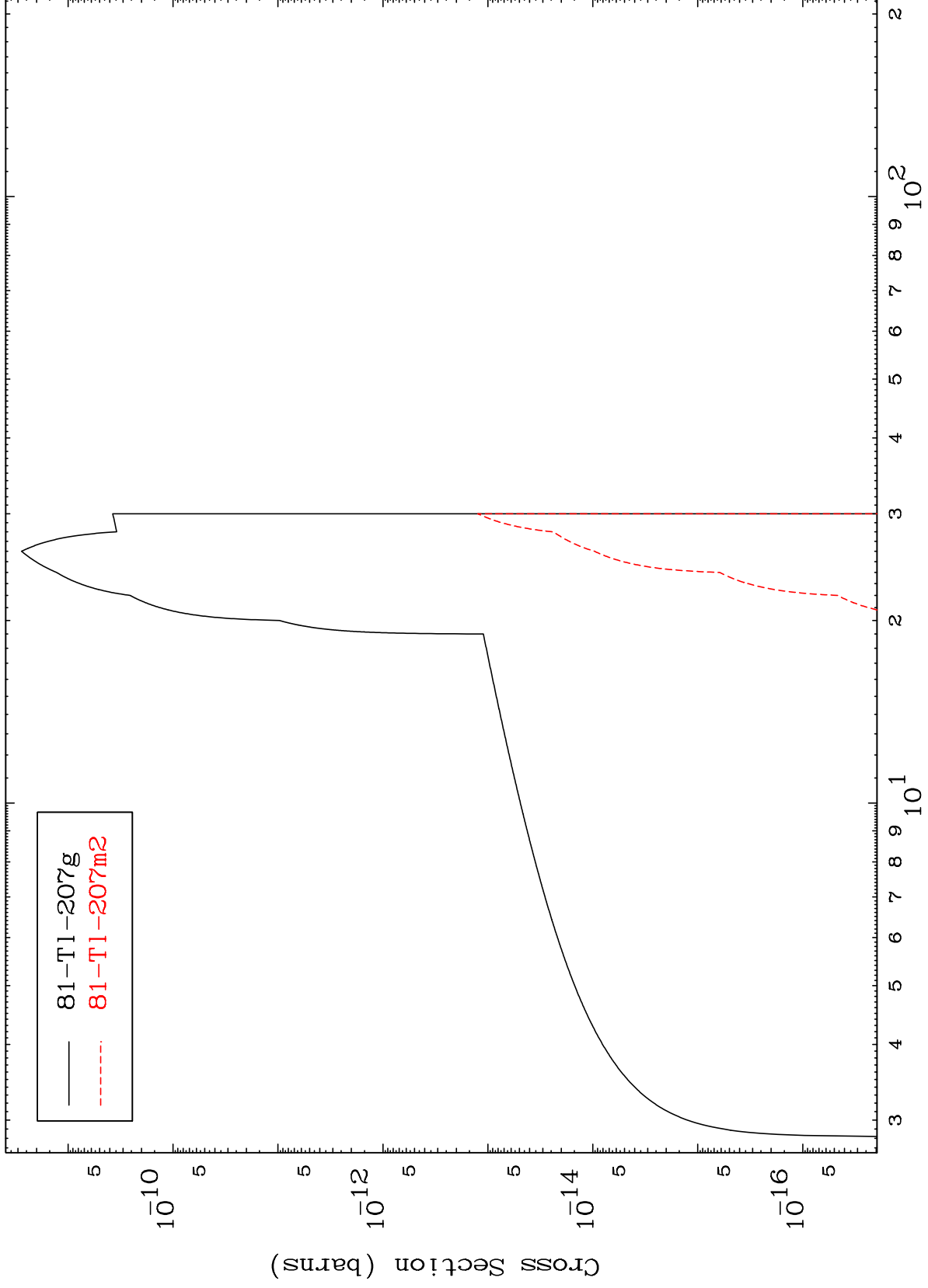
83-Bi-213

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$(\alpha, 2n) 2\alpha$

83-Bi-213

Radionuclide Production Cross Section



81-Tl-207g  
81-Tl-207m2

14

Incident Energy (MeV)

83-Bi-213