

Program EVALPLOT  
(Version 2018-1)

by

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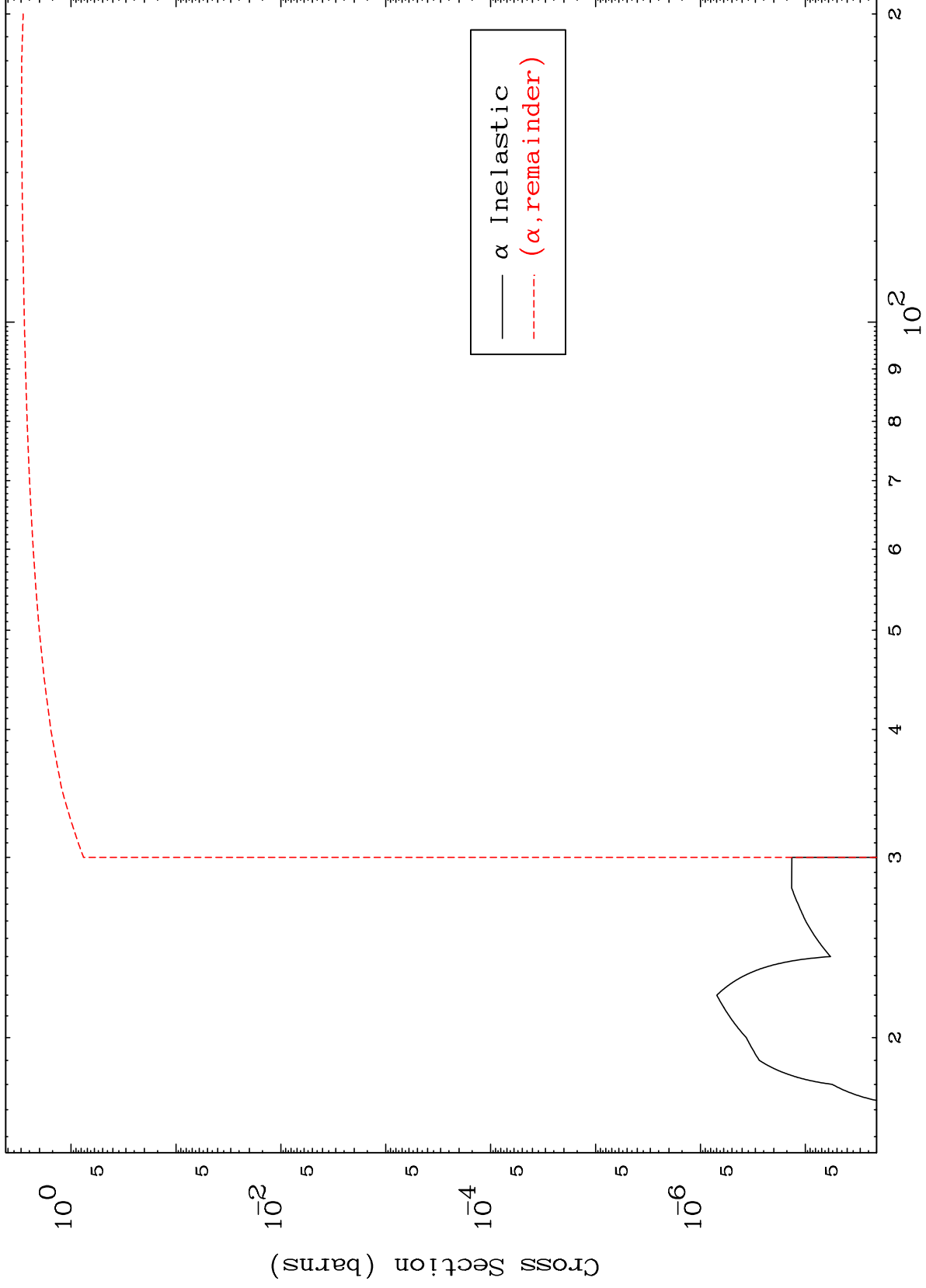
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9713

$\alpha$  Major  
0 Kelvin Cross Sections

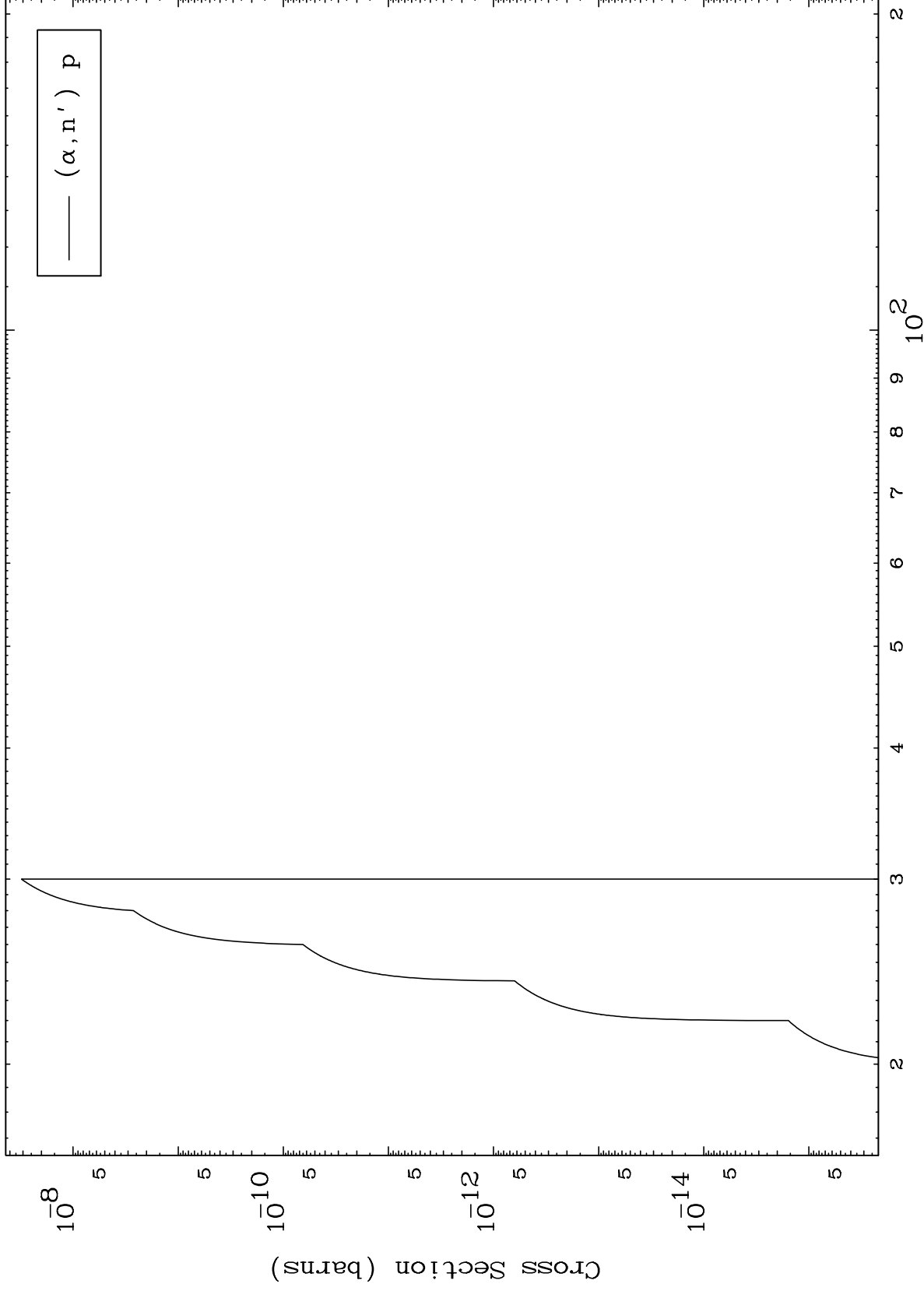
97-Bk-236

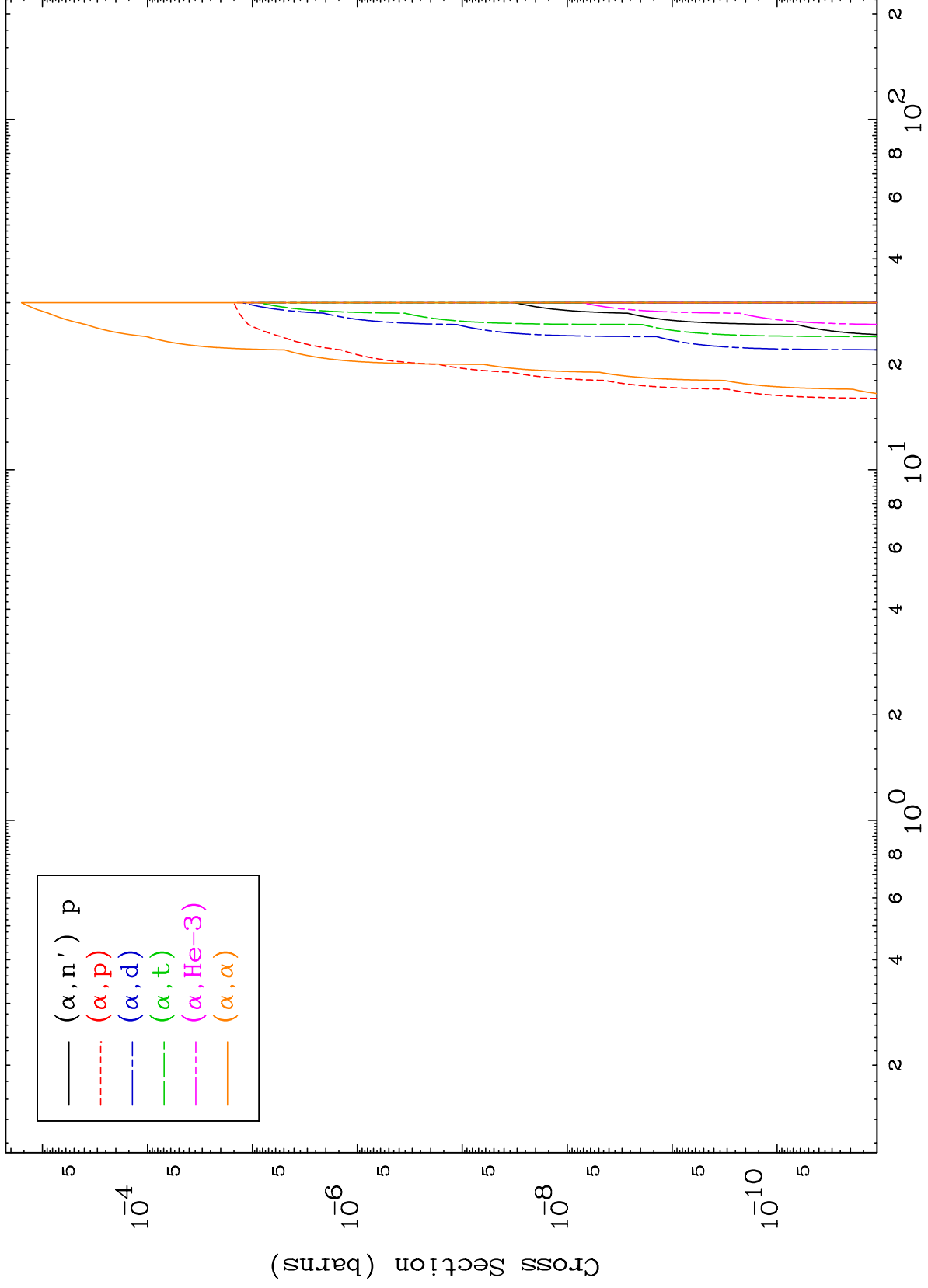


1

Incident Energy (MeV)

97-Bk-236



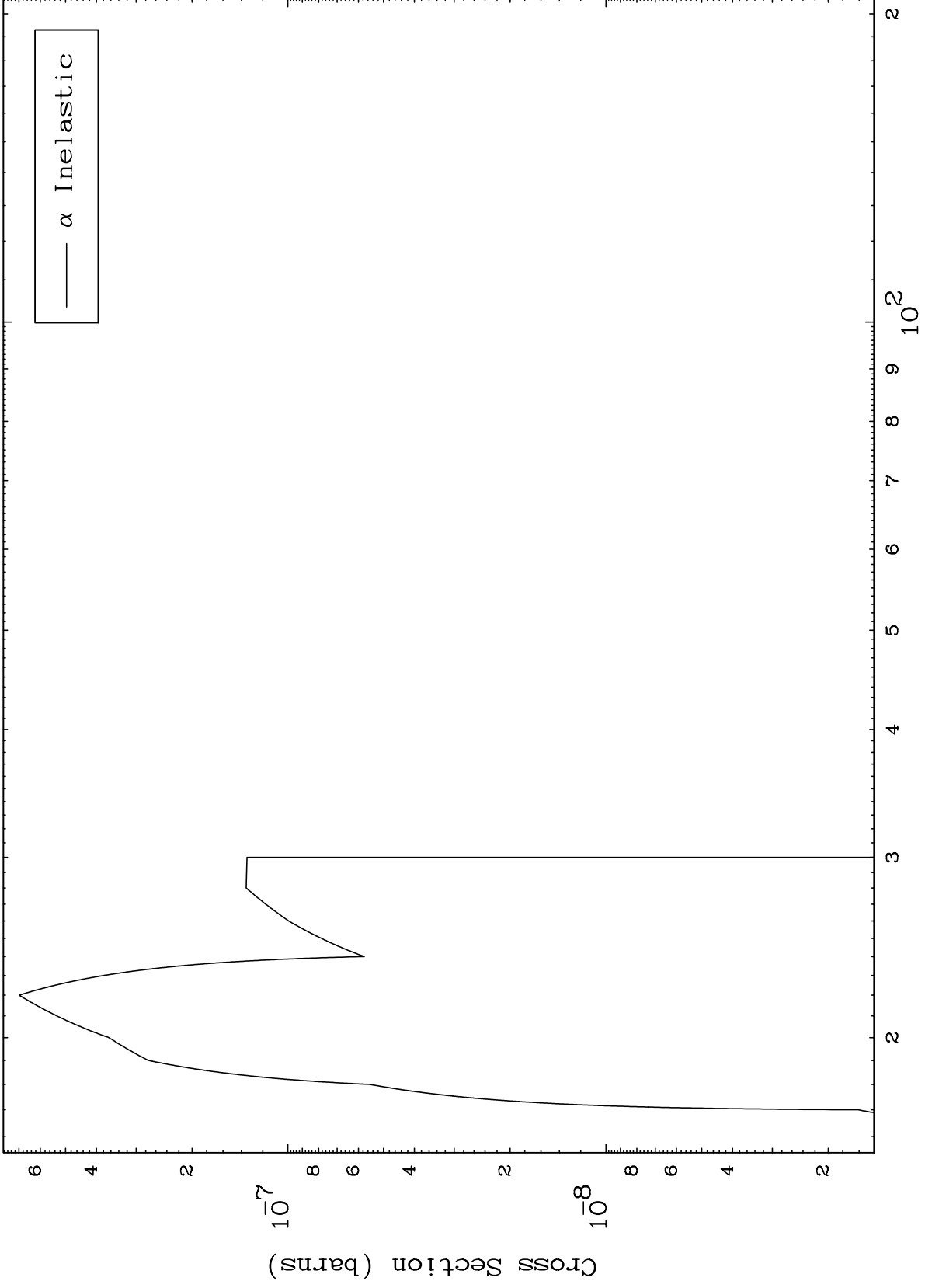


MAT 9713

( $\alpha, n'$ ) Level

97-Bk-236

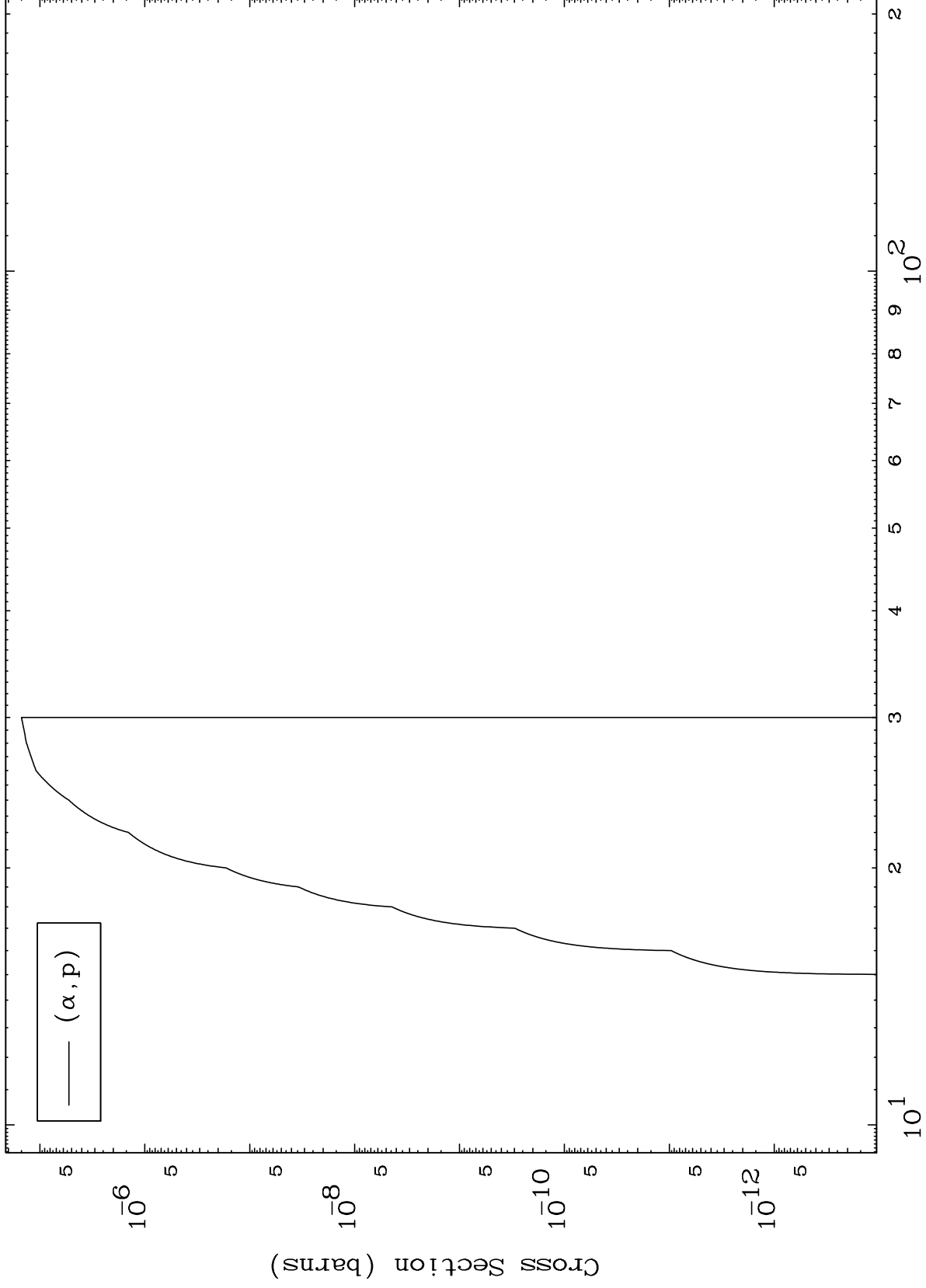
0 Kelvin Cross Sections



MAT 9713

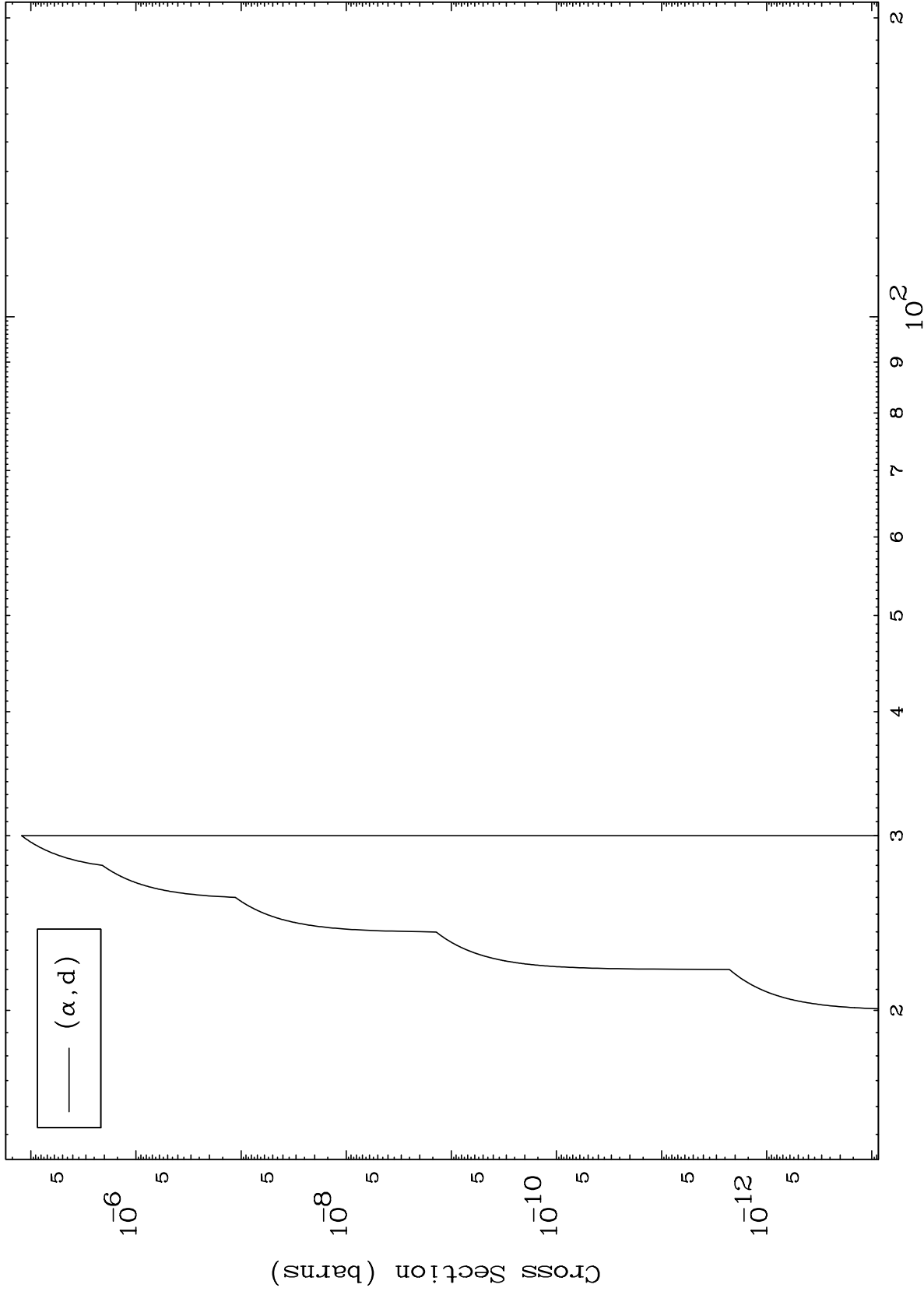
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

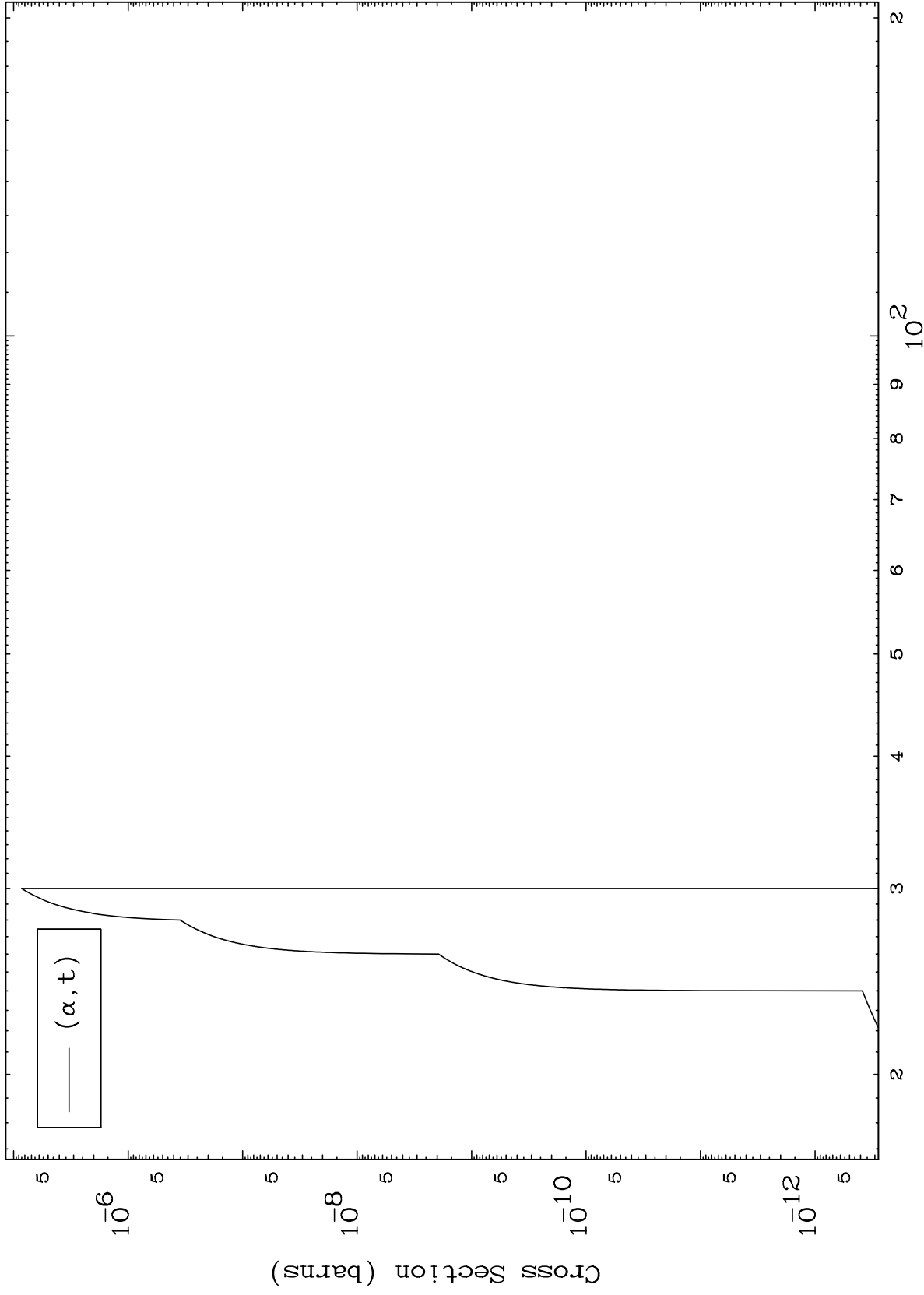
97-Bk-236



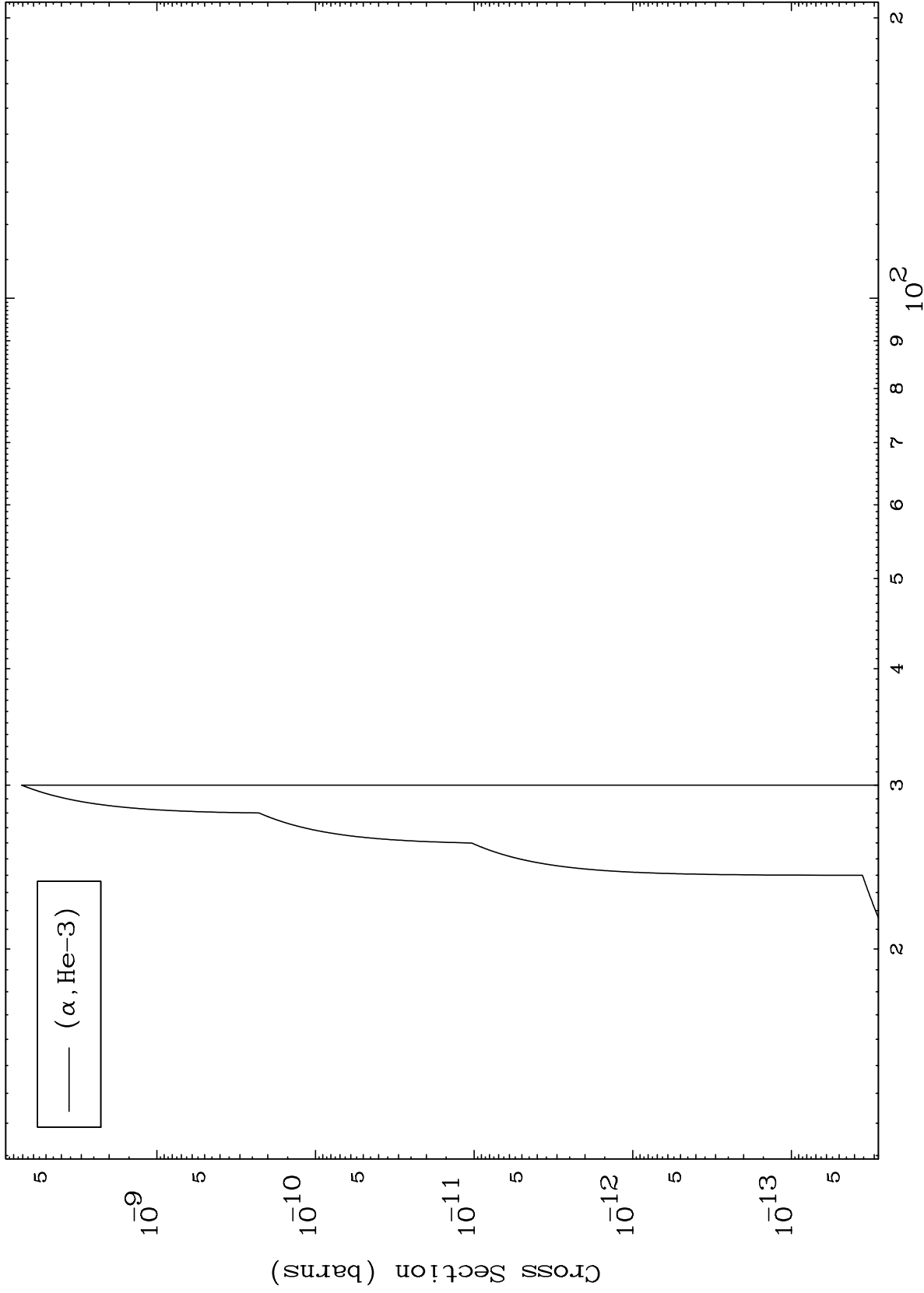
Incident Energy (MeV)

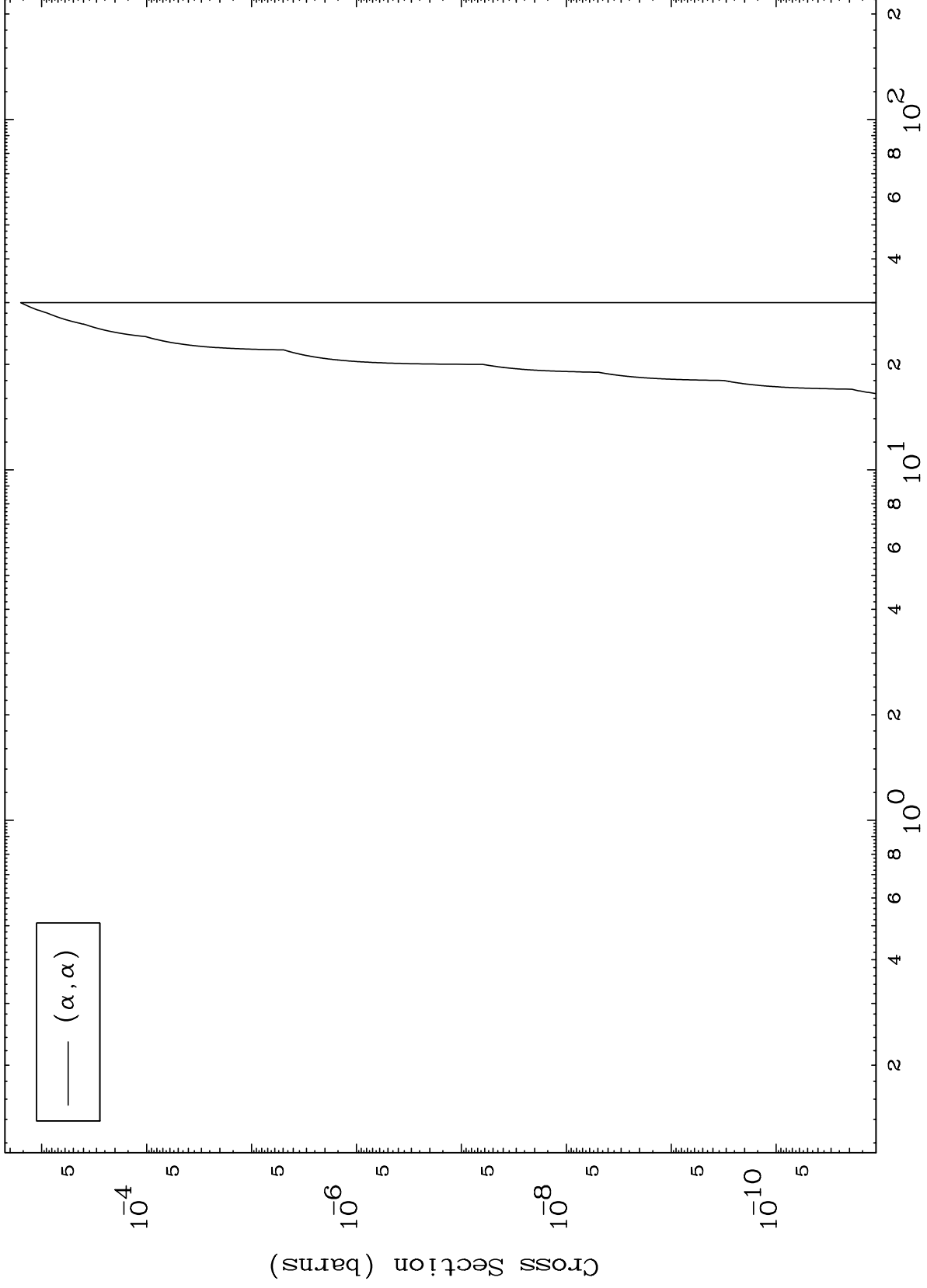
97-Bk-236







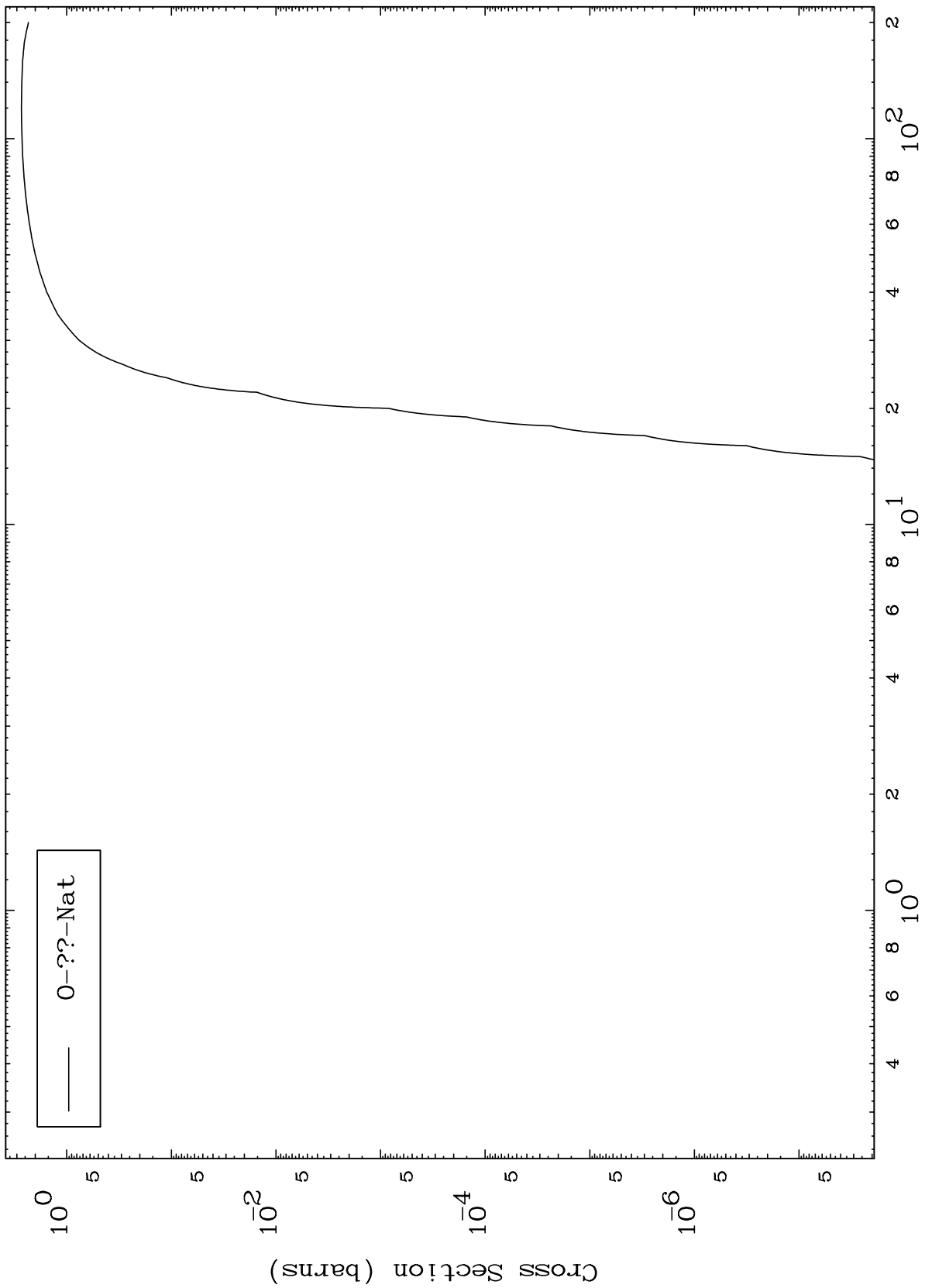




MAT 9713

97-Bk-236

$\alpha$  Fission  
Radionuclide Production Cross Section



0-??-Nat

10

97-Bk-236

Incident Energy (MeV)