

Program EVALPLOT  
(Version 2018-1)

by

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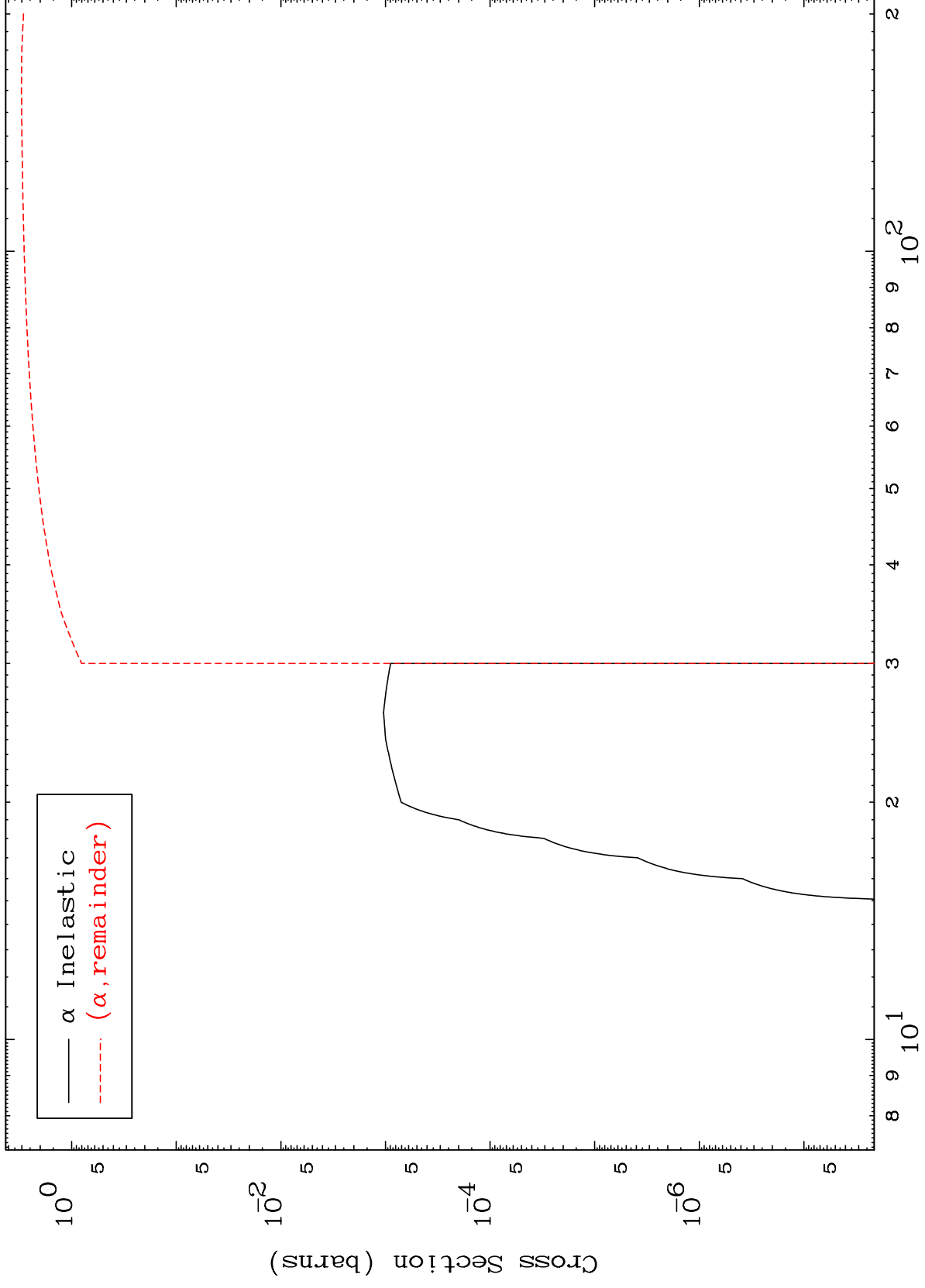
Press Mouse Button to Start

MAT 9734

$\alpha$  Major

97-Bk-243

0 Kelvin Cross Sections



—  $\alpha$  Inelastic  
- - - ( $\alpha$ , remainder)

1

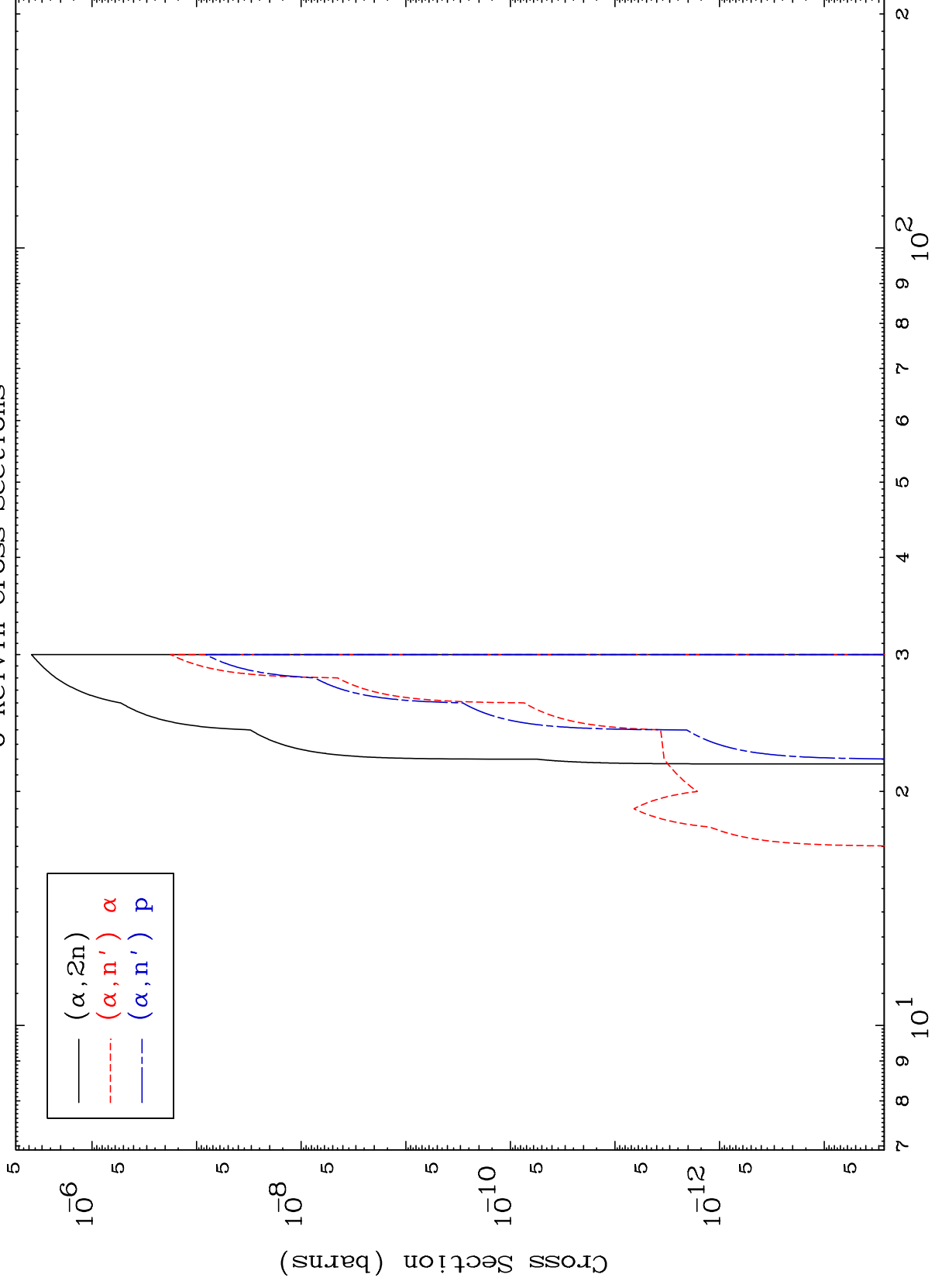
Incident Energy (MeV)

97-Bk-243

MAT 9734

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

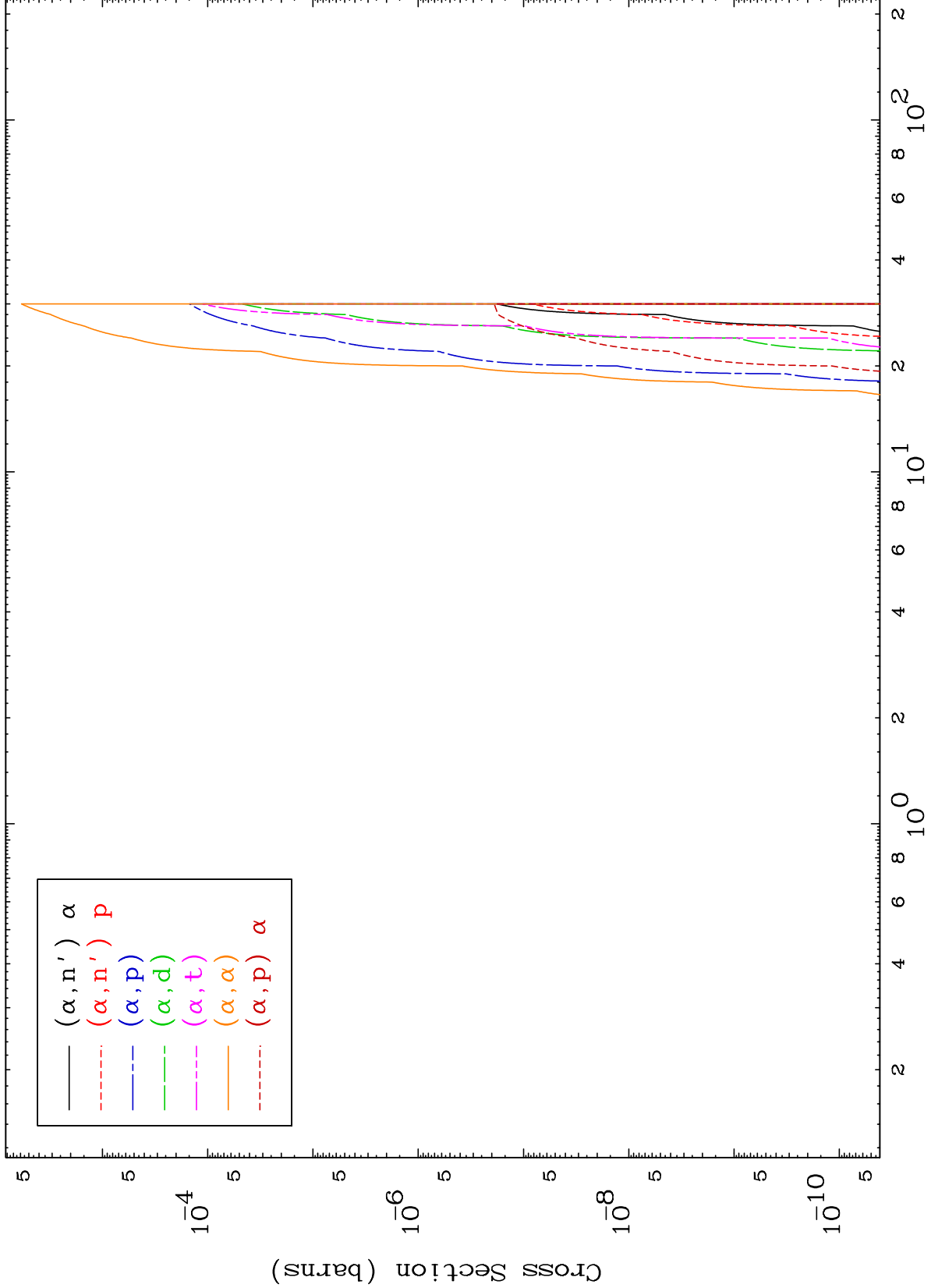
97-Bk-243



2

Incident Energy (MeV)

97-Bk-243

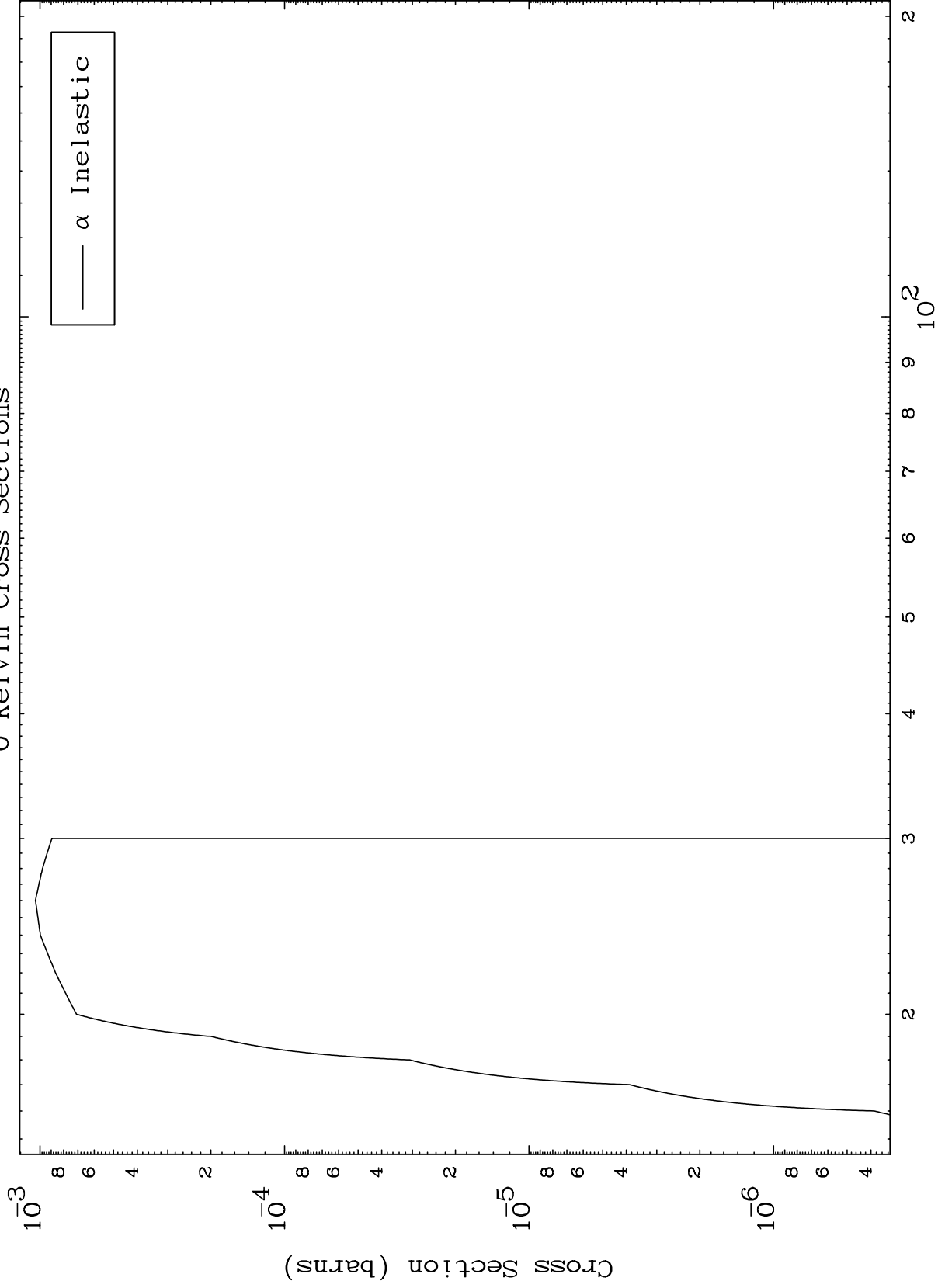


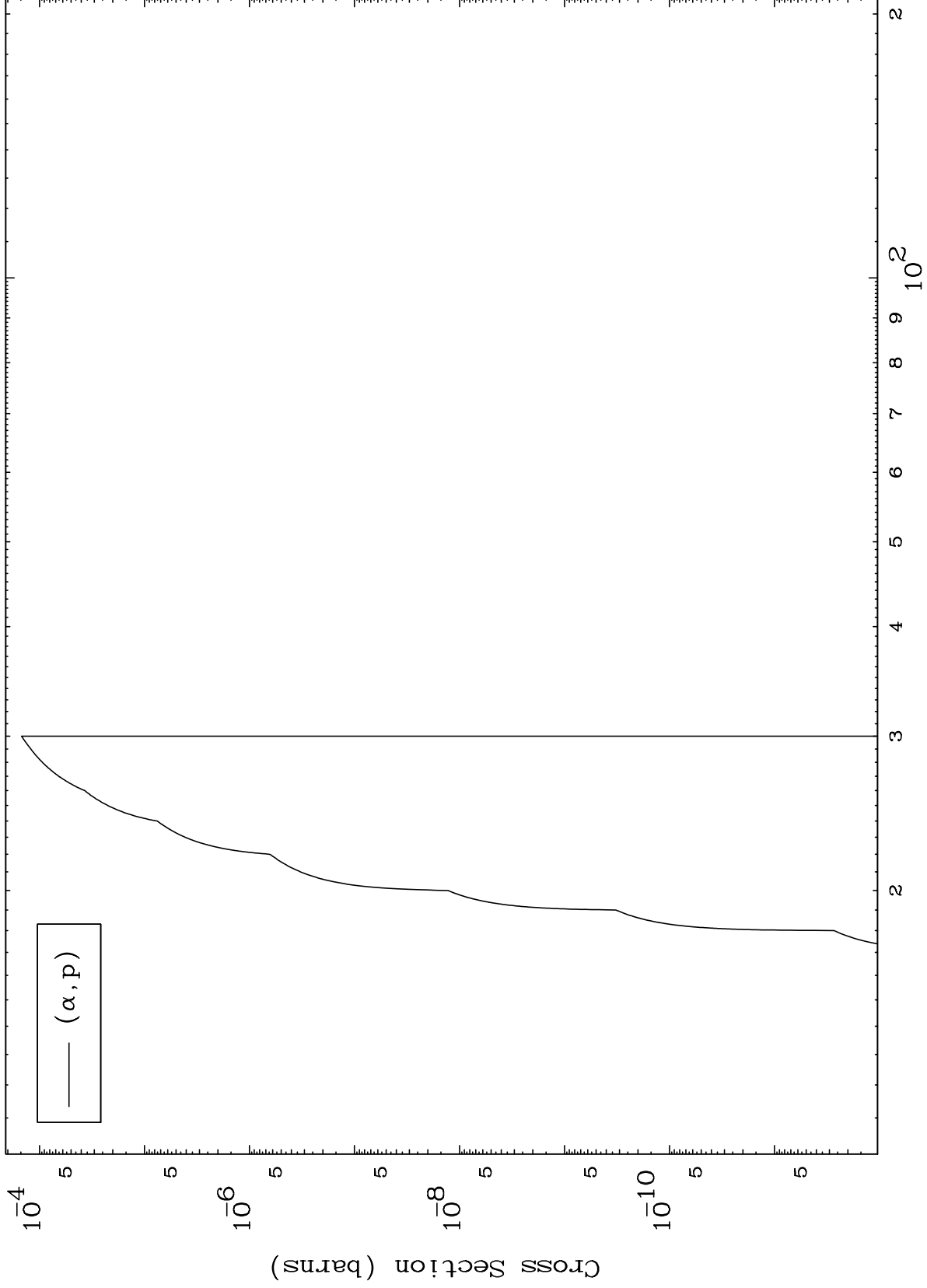
MAT 9734

( $\alpha, n'$ ) Level

97-Bk-243

0 Kelvin Cross Sections

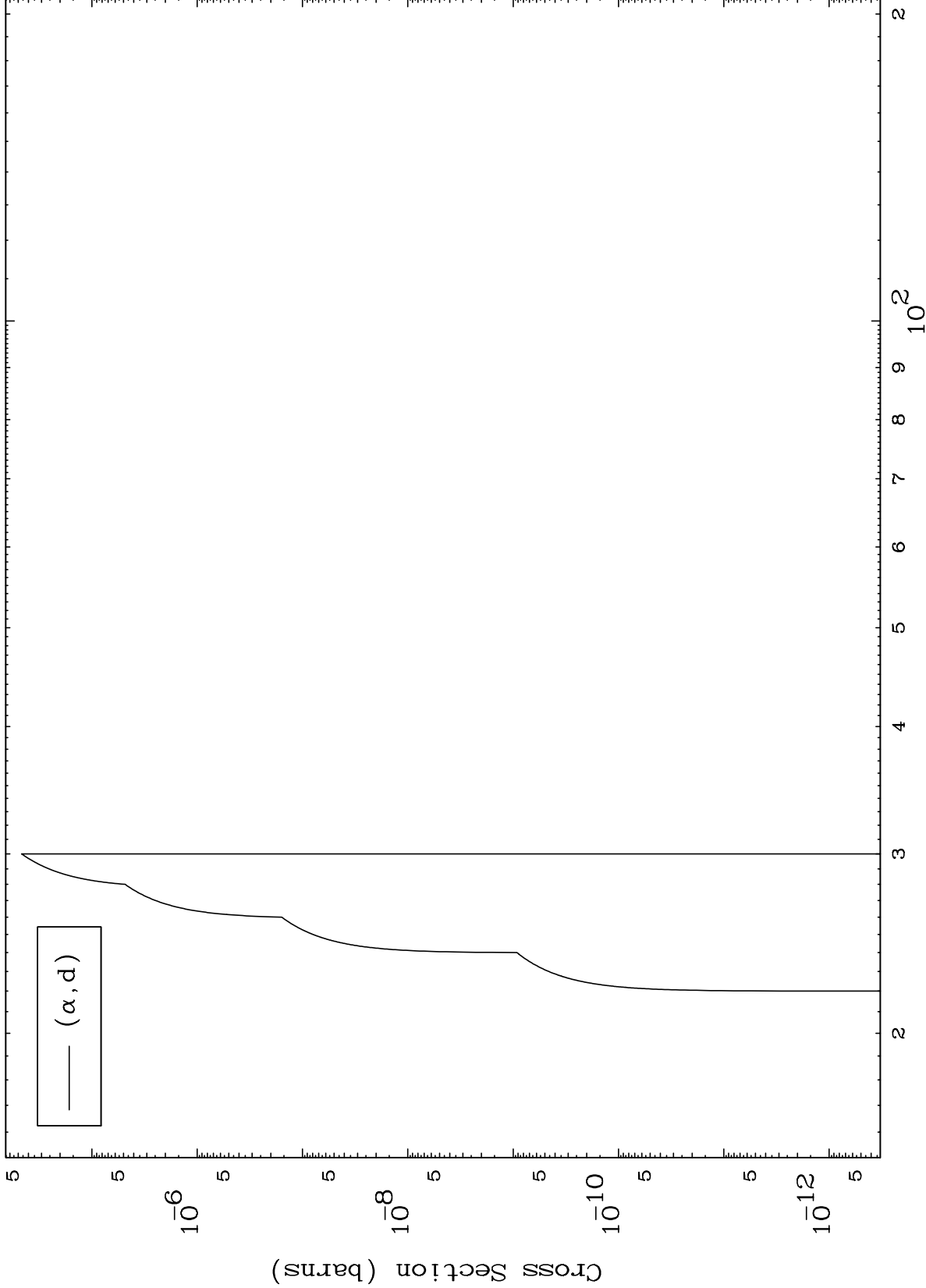




MAT 9734

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

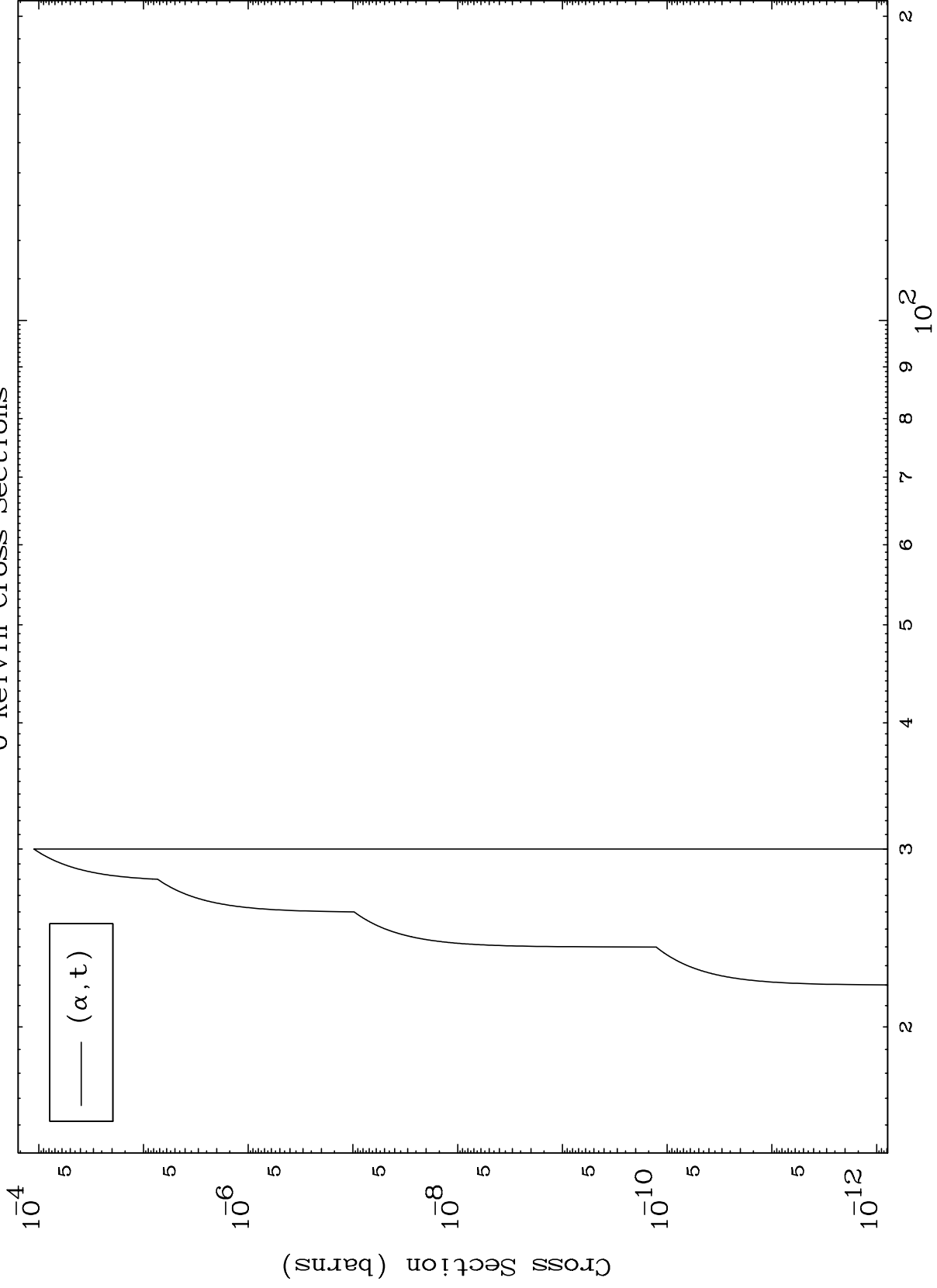
97-Bk-243



6

Incident Energy (MeV)

97-Bk-243



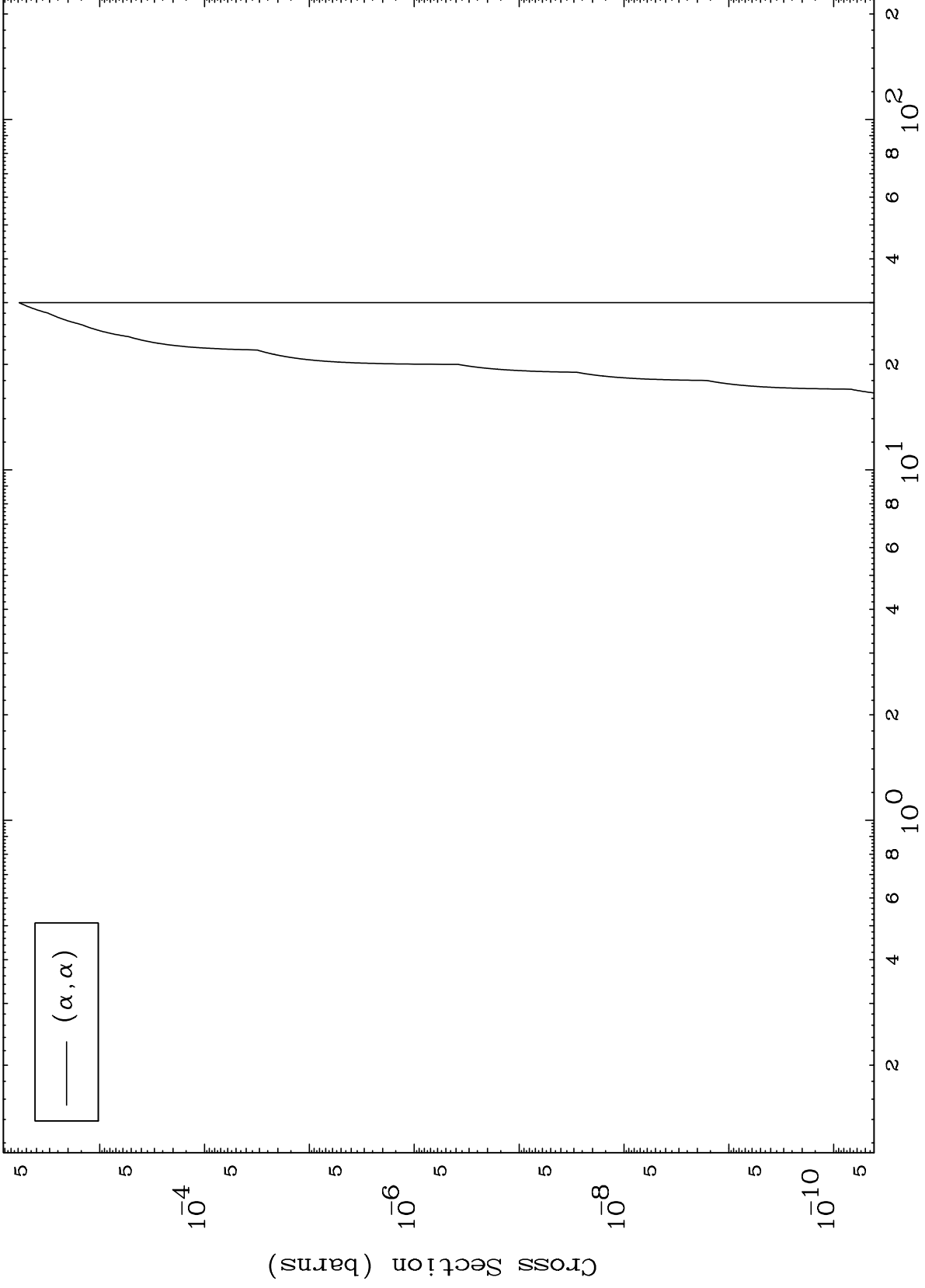


MAT 9734

( $\alpha, \alpha$ ) Levels

97-Bk-243

0 Kelvin Cross Sections



MAT 9734

97-Bk-243

$\alpha$  Fission  
Radionuclide Production Cross Section

