

Program EVALPLOT  
(Version 2018-1)

by

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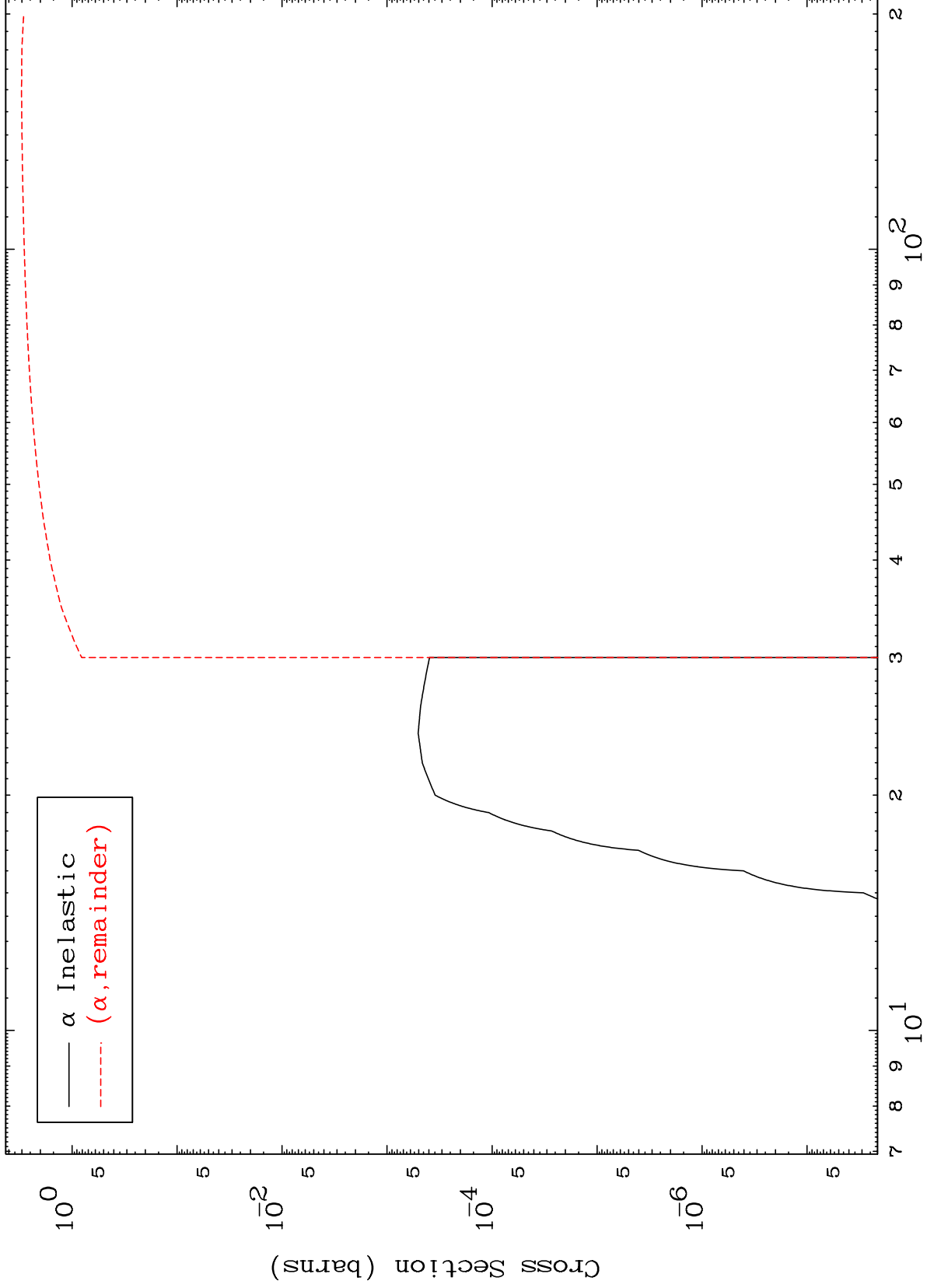
Press Mouse Button to Start

MAT 9737

0 Kelvin

$\alpha$  Major Cross Sections

97-Bk-244



1

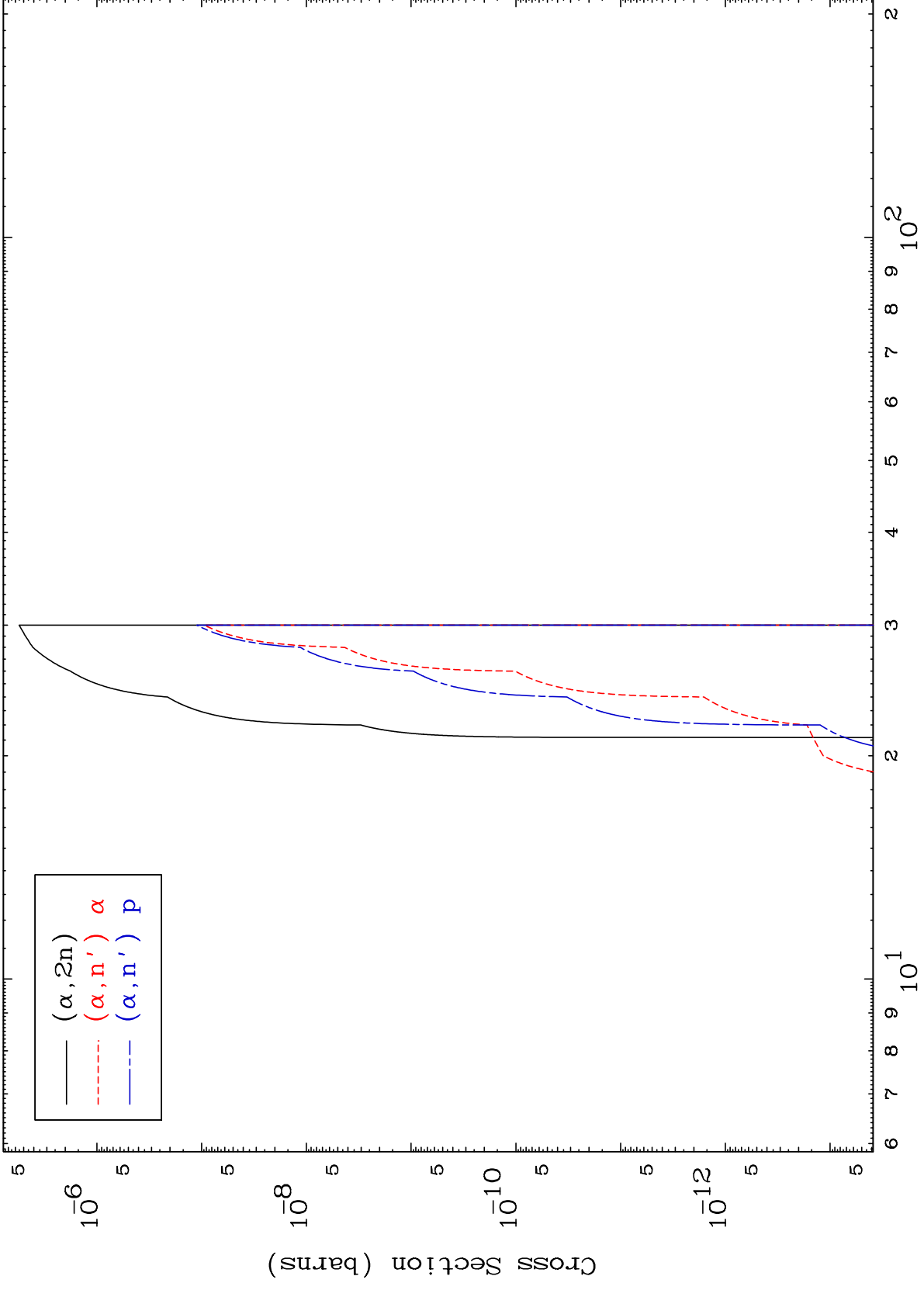
Incident Energy (MeV)

97-Bk-244

MAT 9737

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

97-Bk-244



( $\alpha, 2n$ )  
( $\alpha, n'$ )  $\alpha$   
( $\alpha, n'$ )  $p$

2

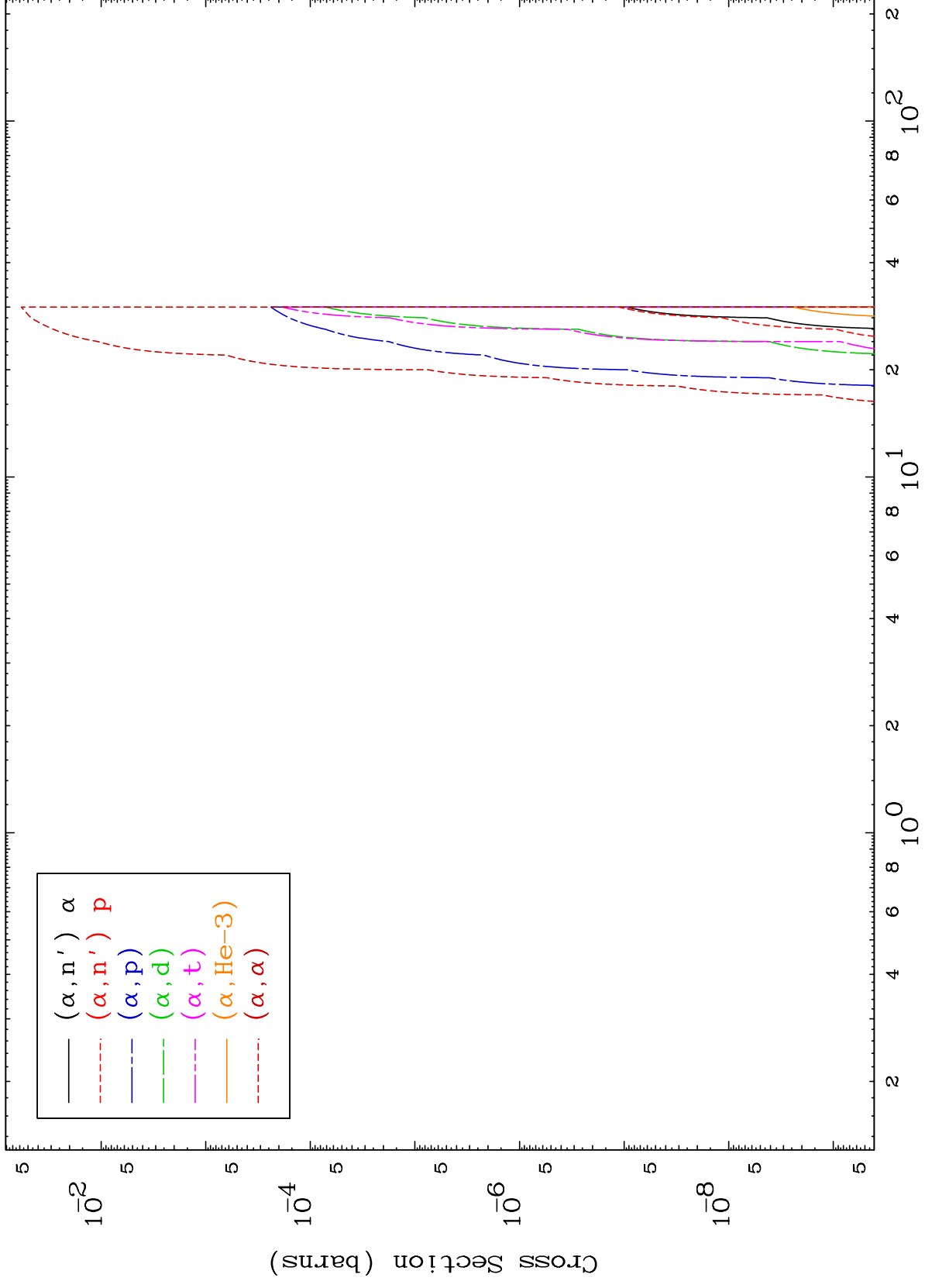
Incident Energy (MeV)

97-Bk-244

MAT 9737

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

97-Bk-244



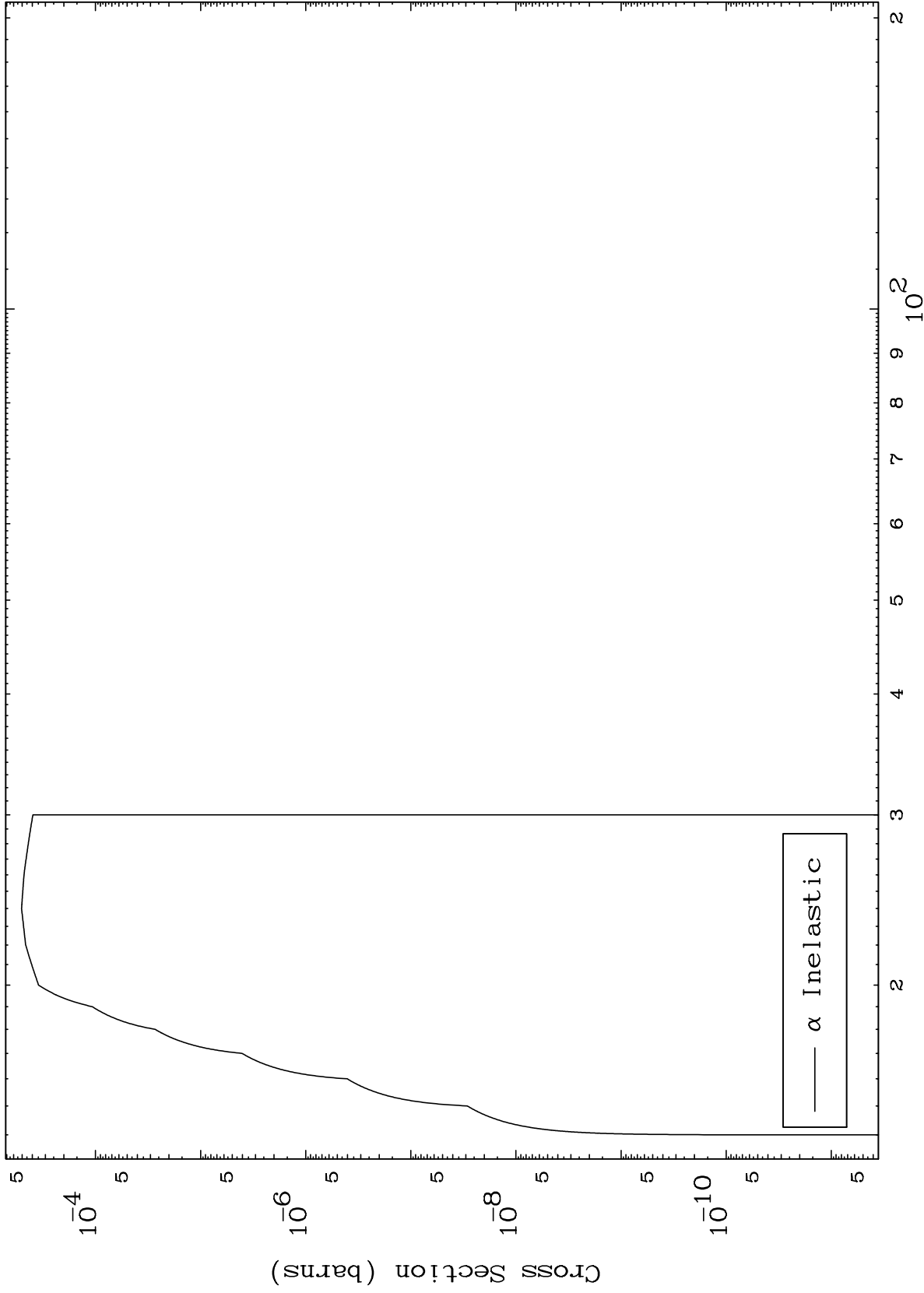
97-Bk-244

Incident Energy (MeV)

MAT 9737

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

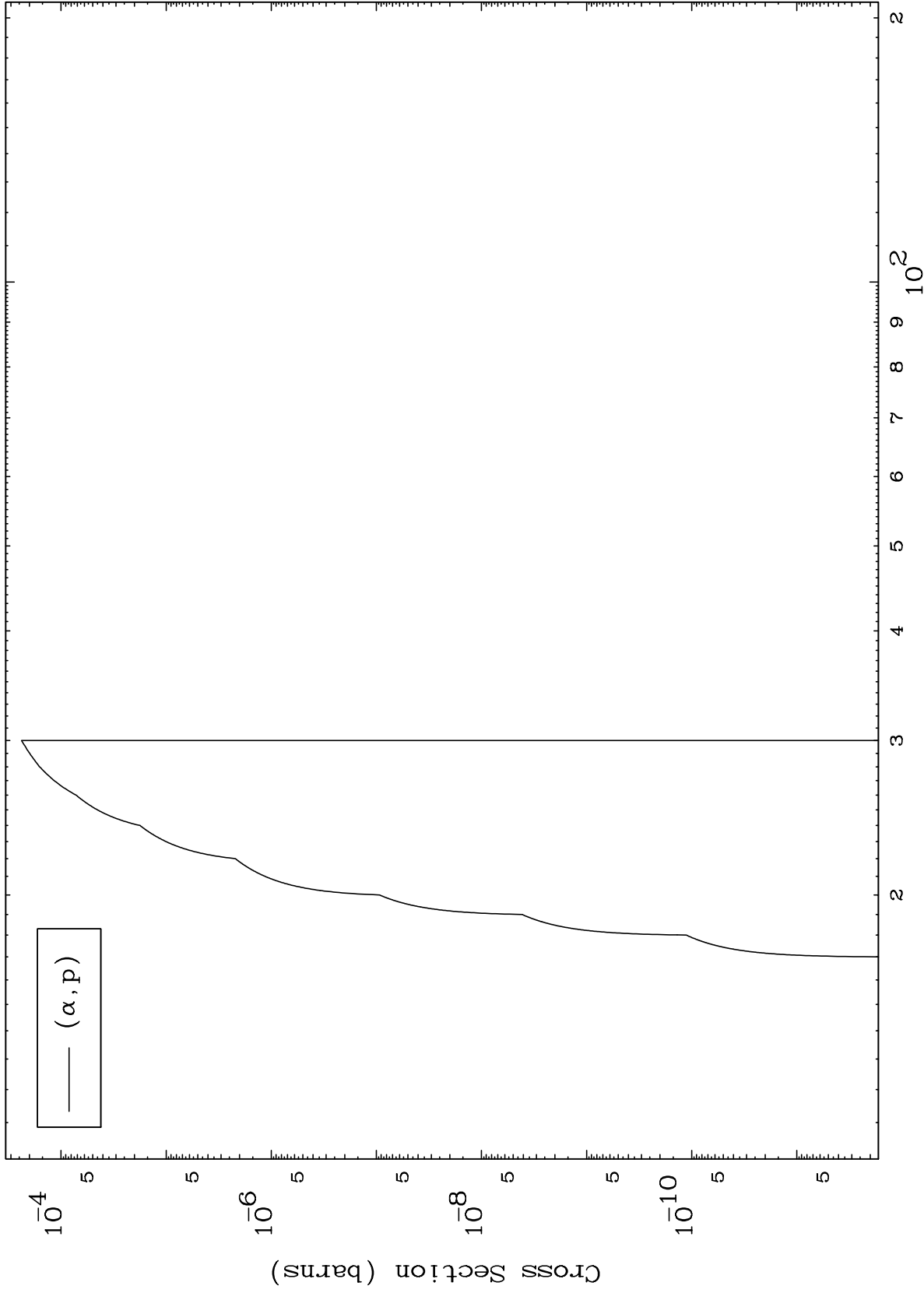
97-Bk-244

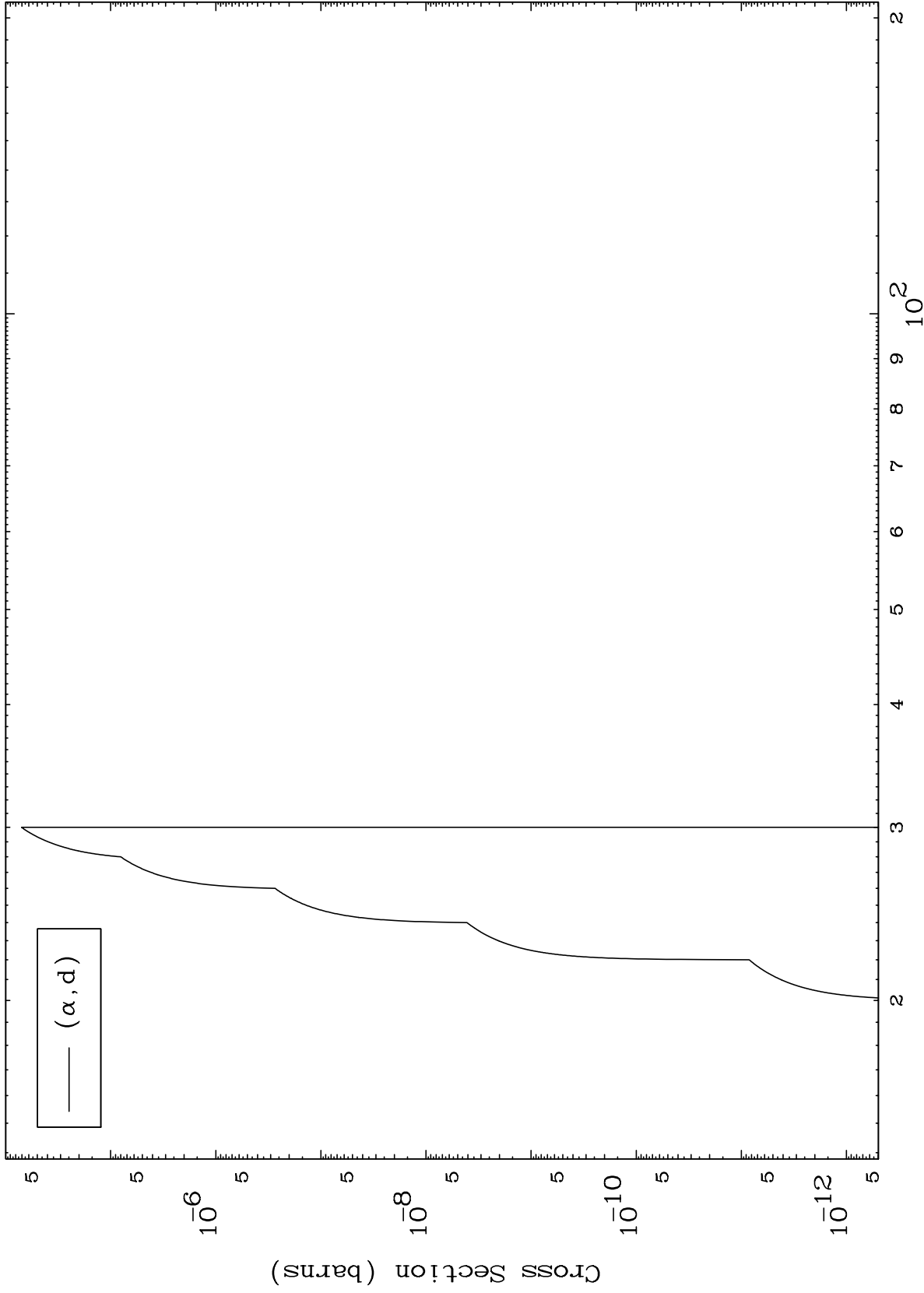


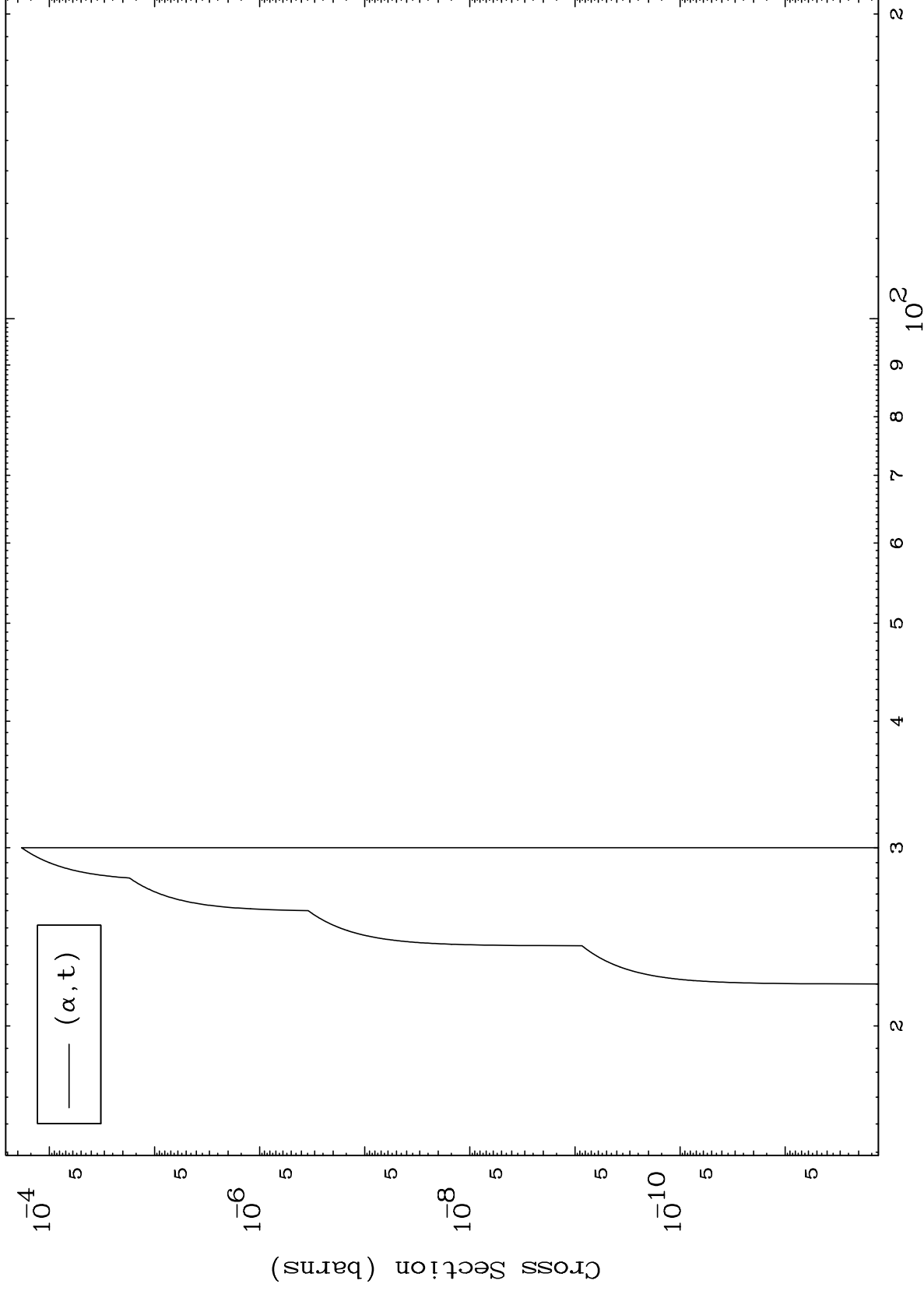
4

Incident Energy (MeV)

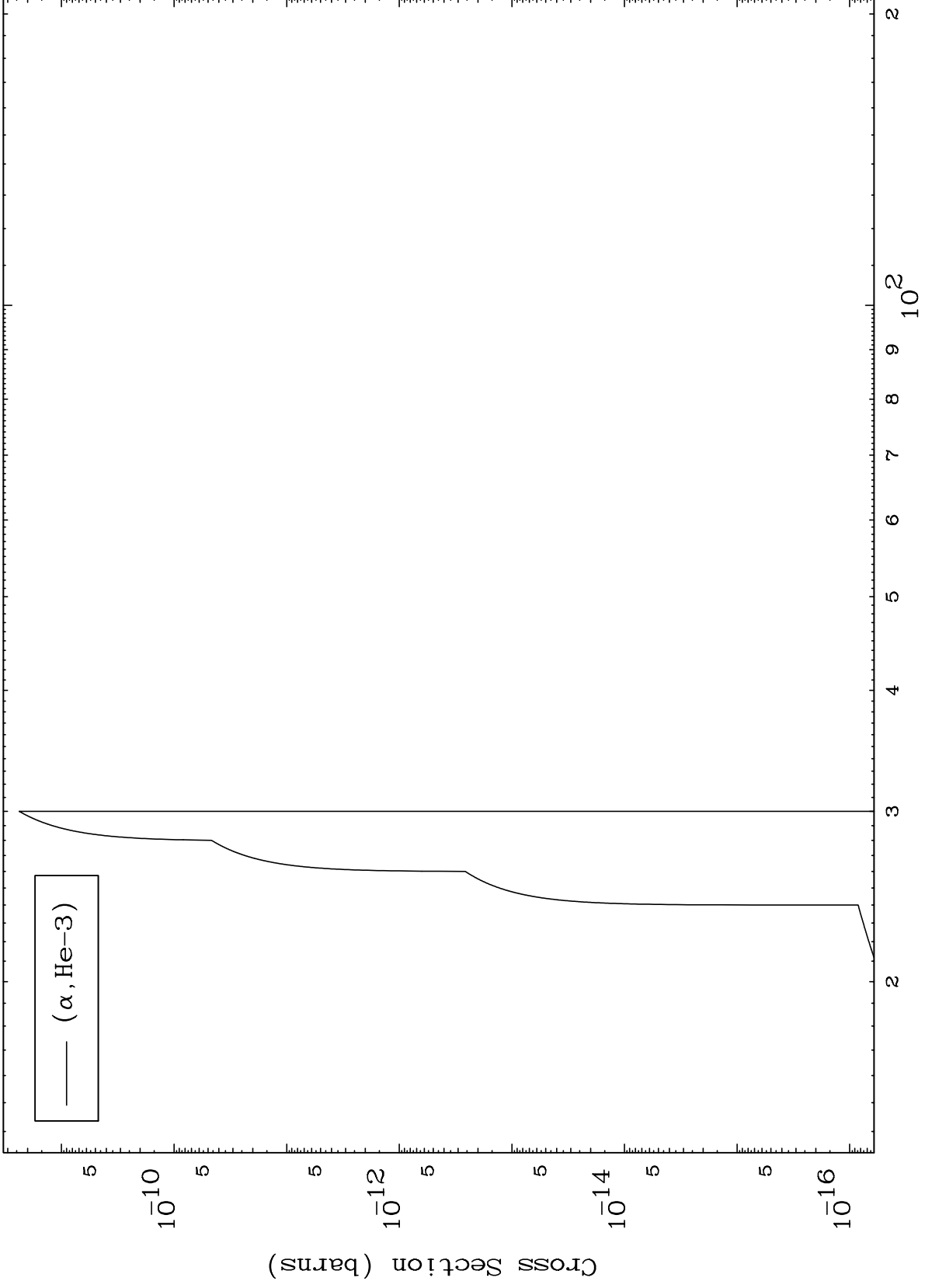
97-Bk-244









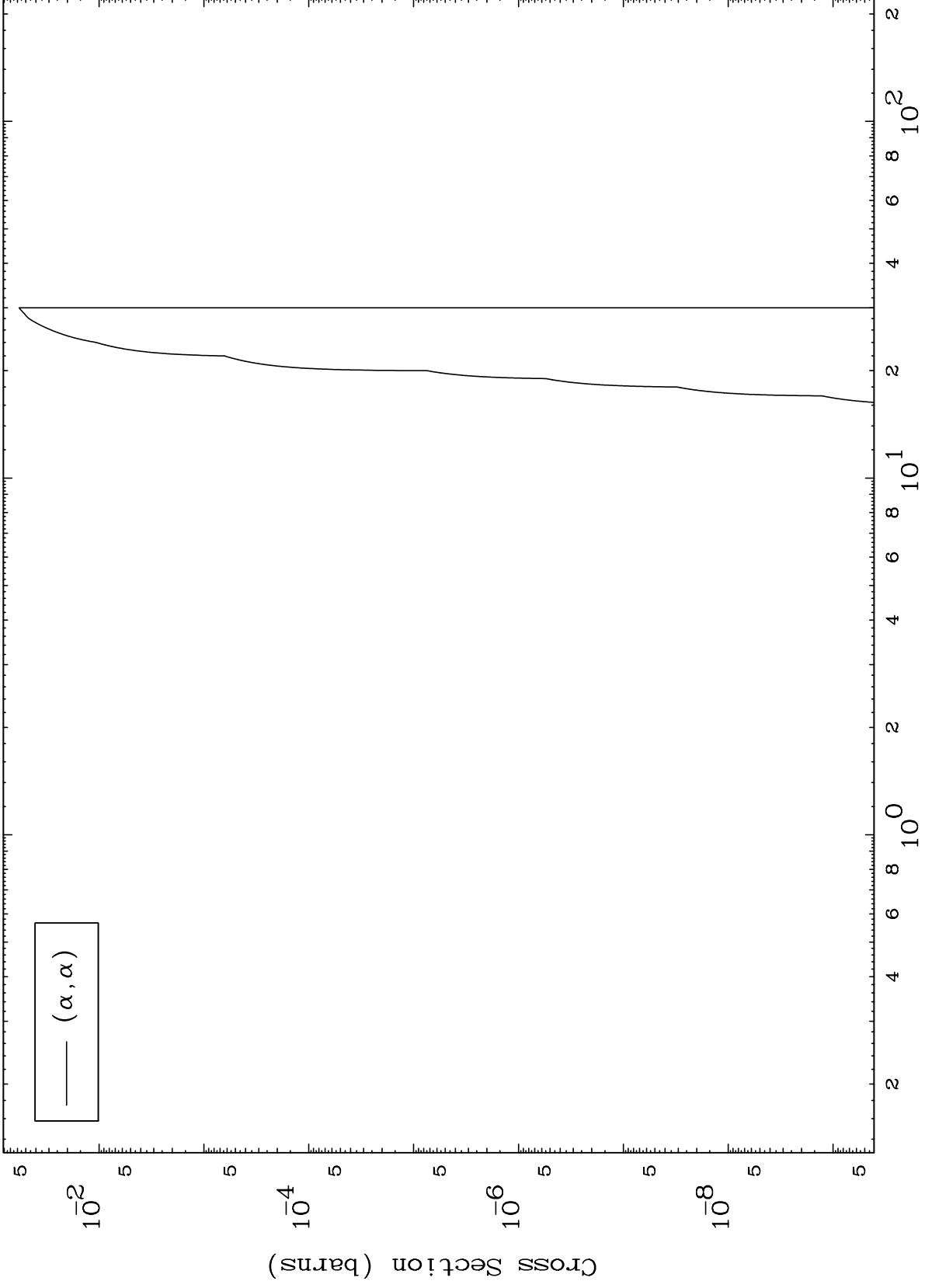


MAT 9737

( $\alpha, \alpha$ ) Levels

97-Bk-244

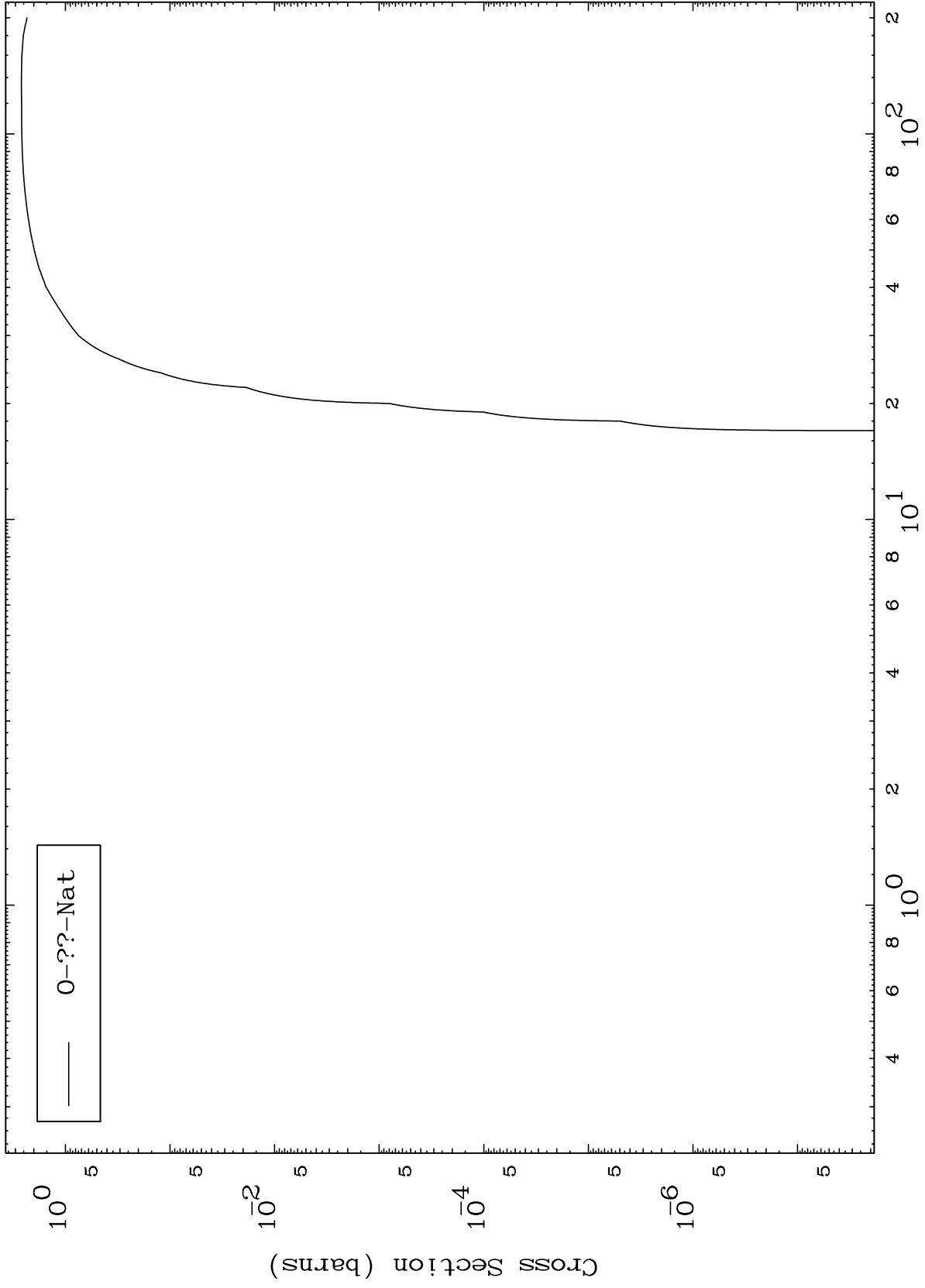
0 Kelvin Cross Sections



MAT 9737

97-Bk-244

$\alpha$  Fission  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

97-Bk-244