

Program EVALPLOT  
(Version 2018-1)

by

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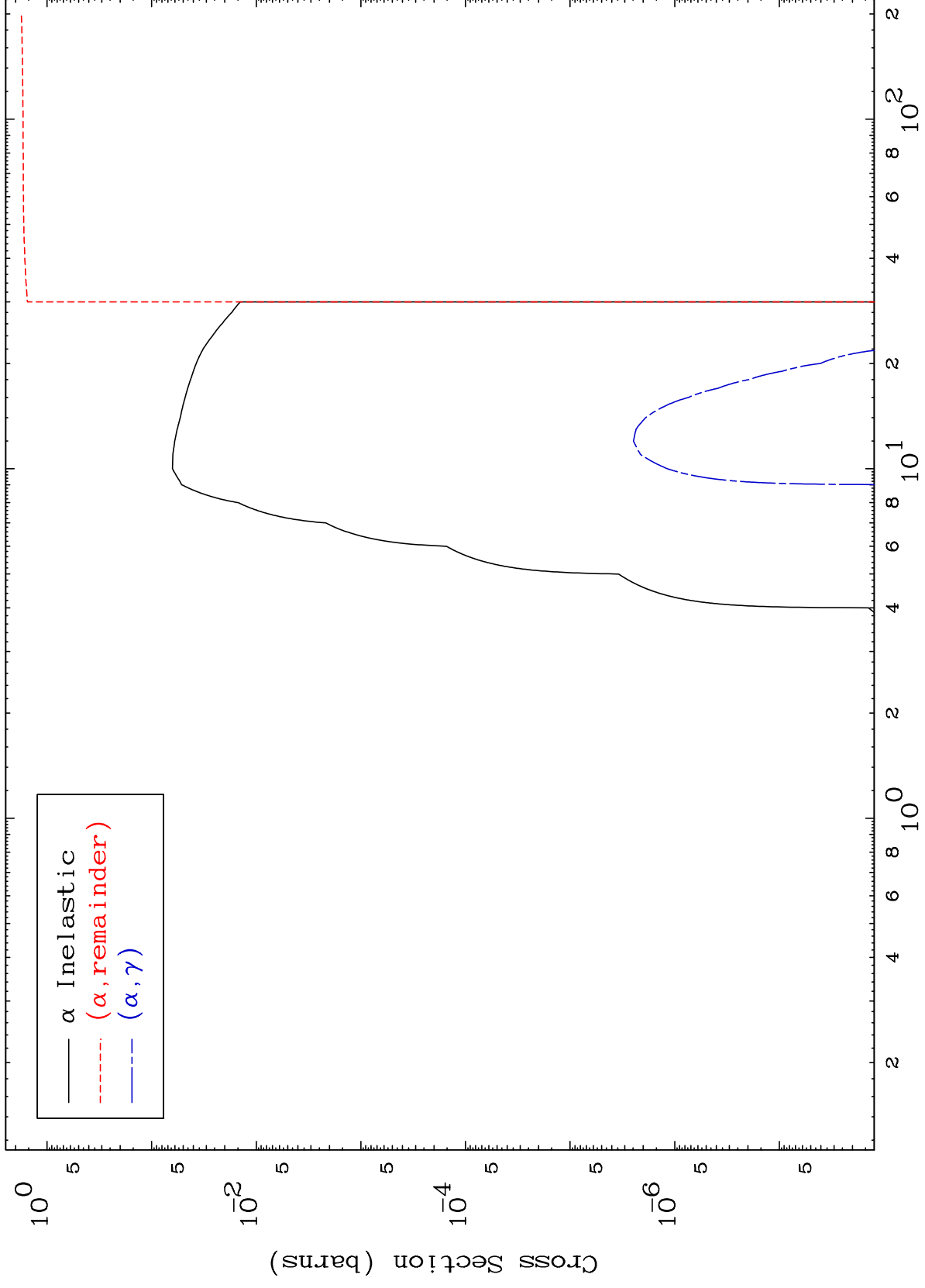
Press Mouse Button to Start

MAT 2955

$\alpha$  Major

0 Kelvin Cross Sections

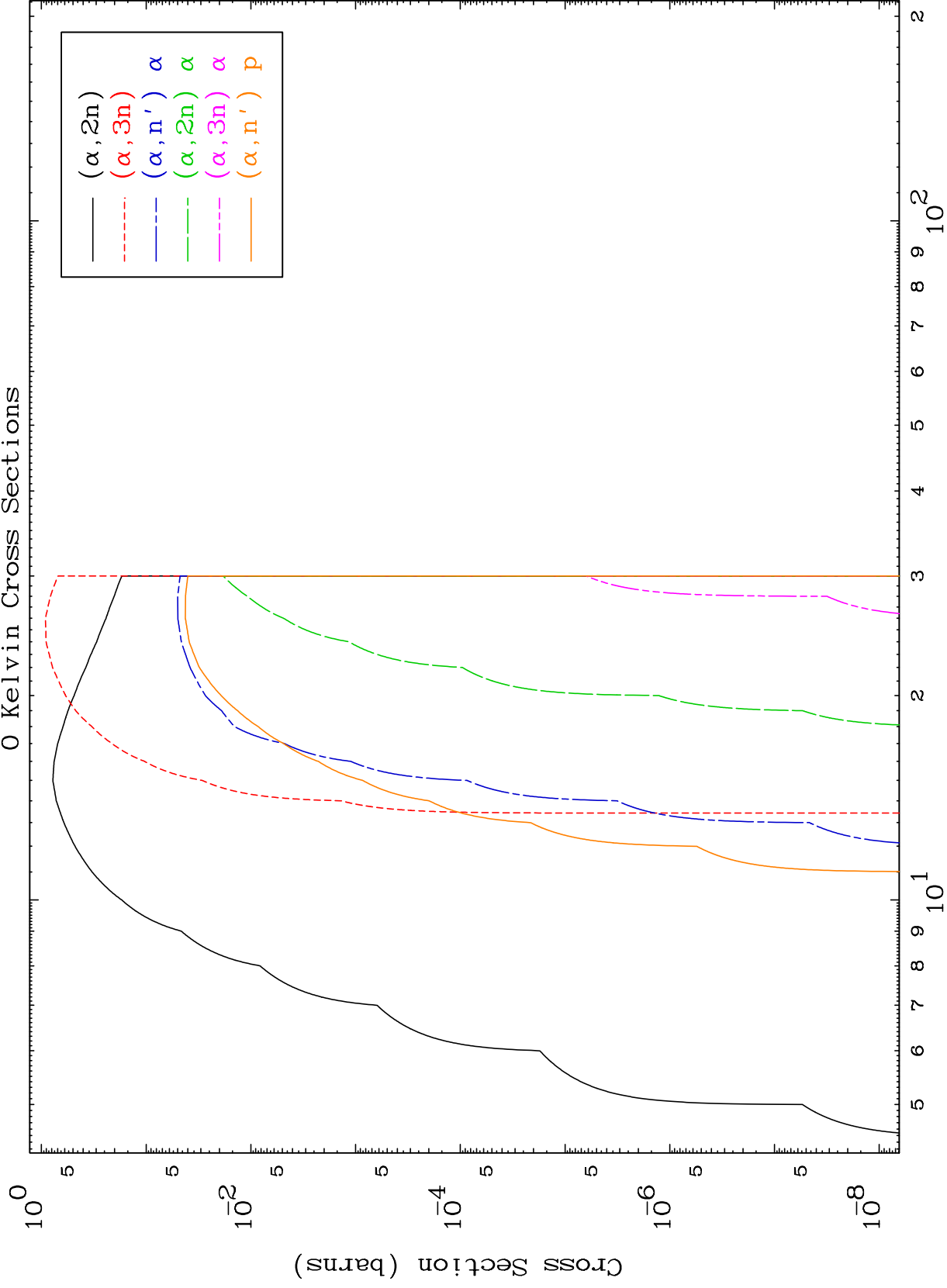
29-Cu-73



MAT 2955

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

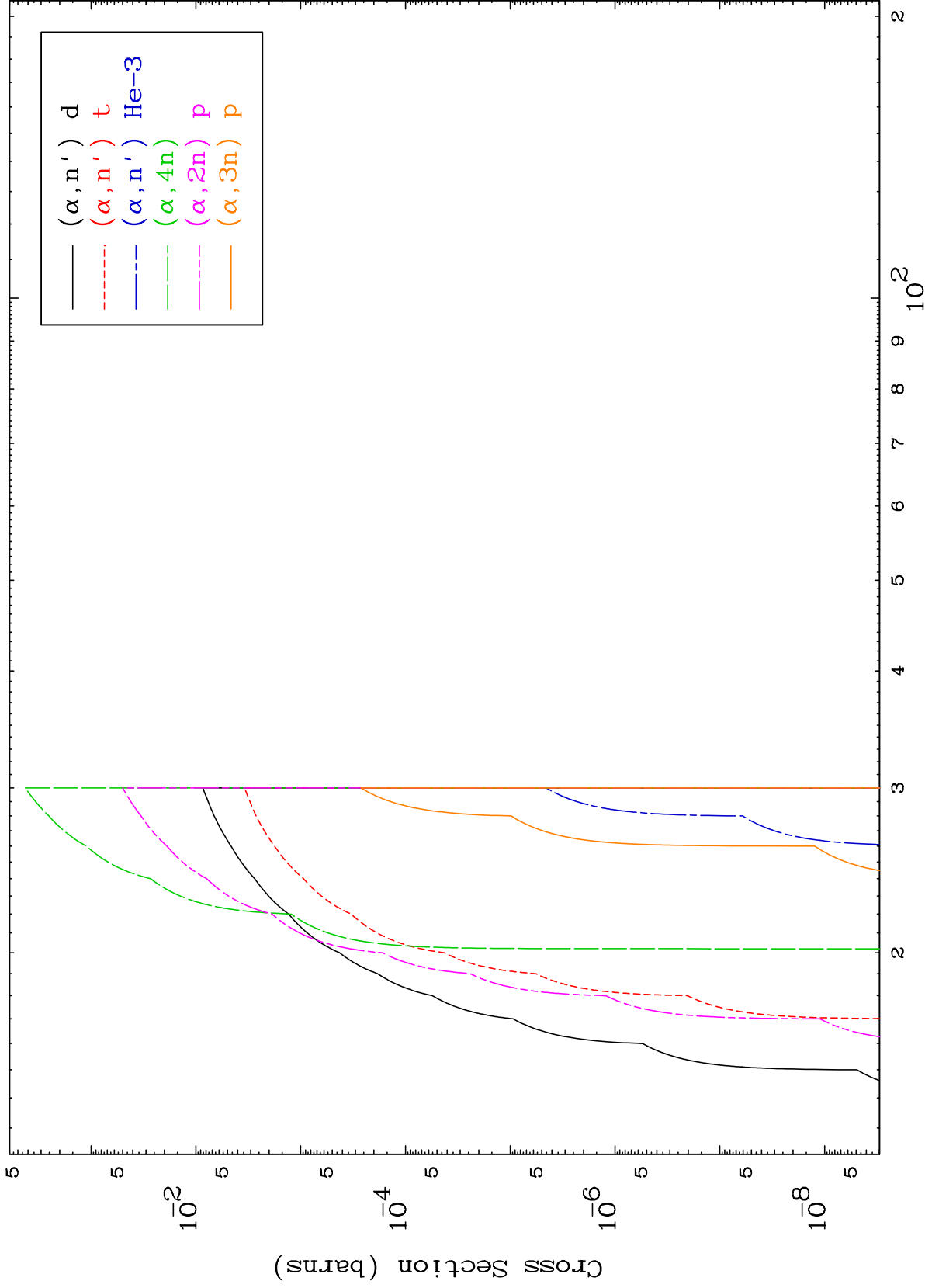
29-Cu-73



2

Incident Energy (MeV)

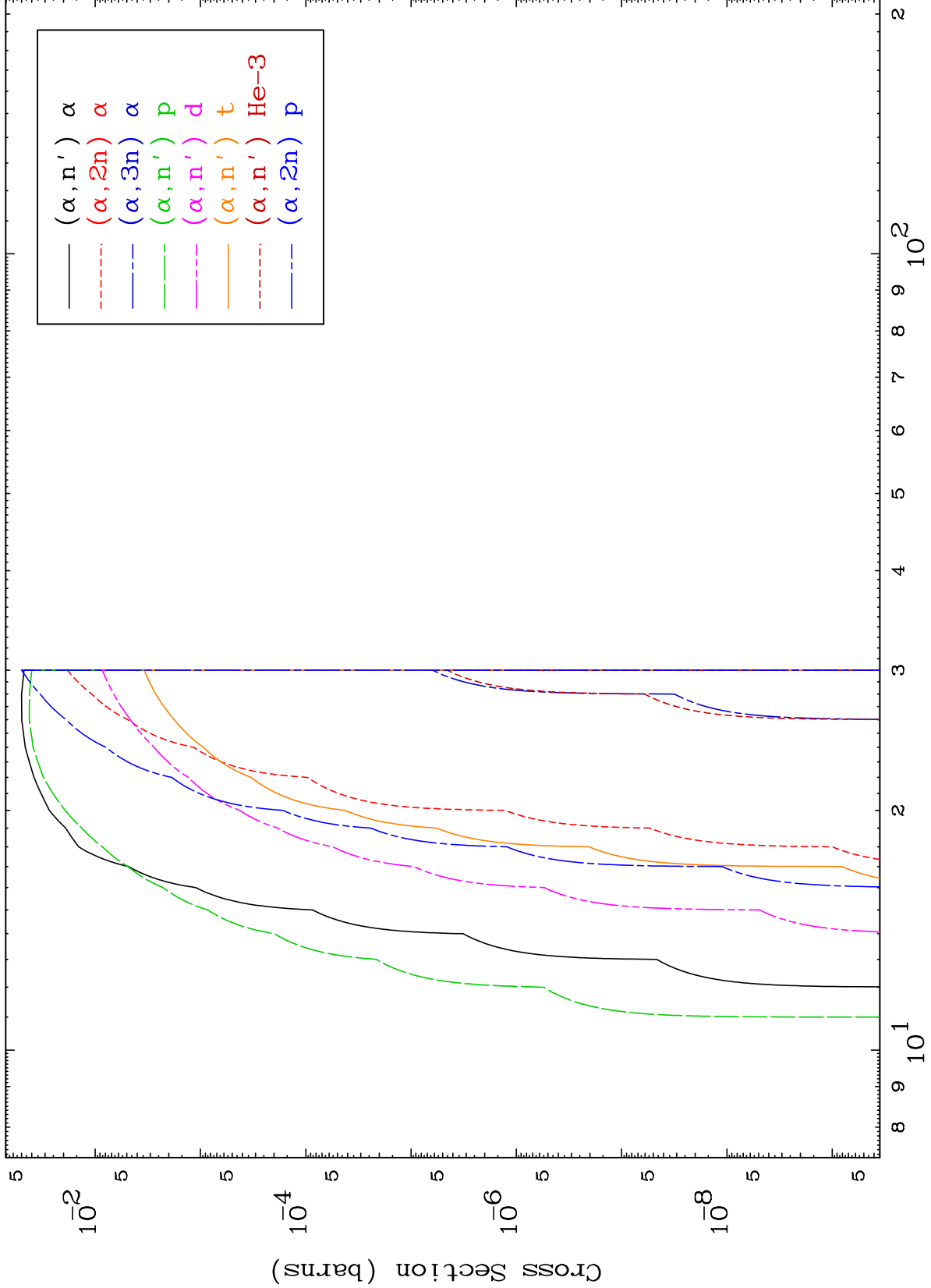
29-Cu-73



MAT 2955

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

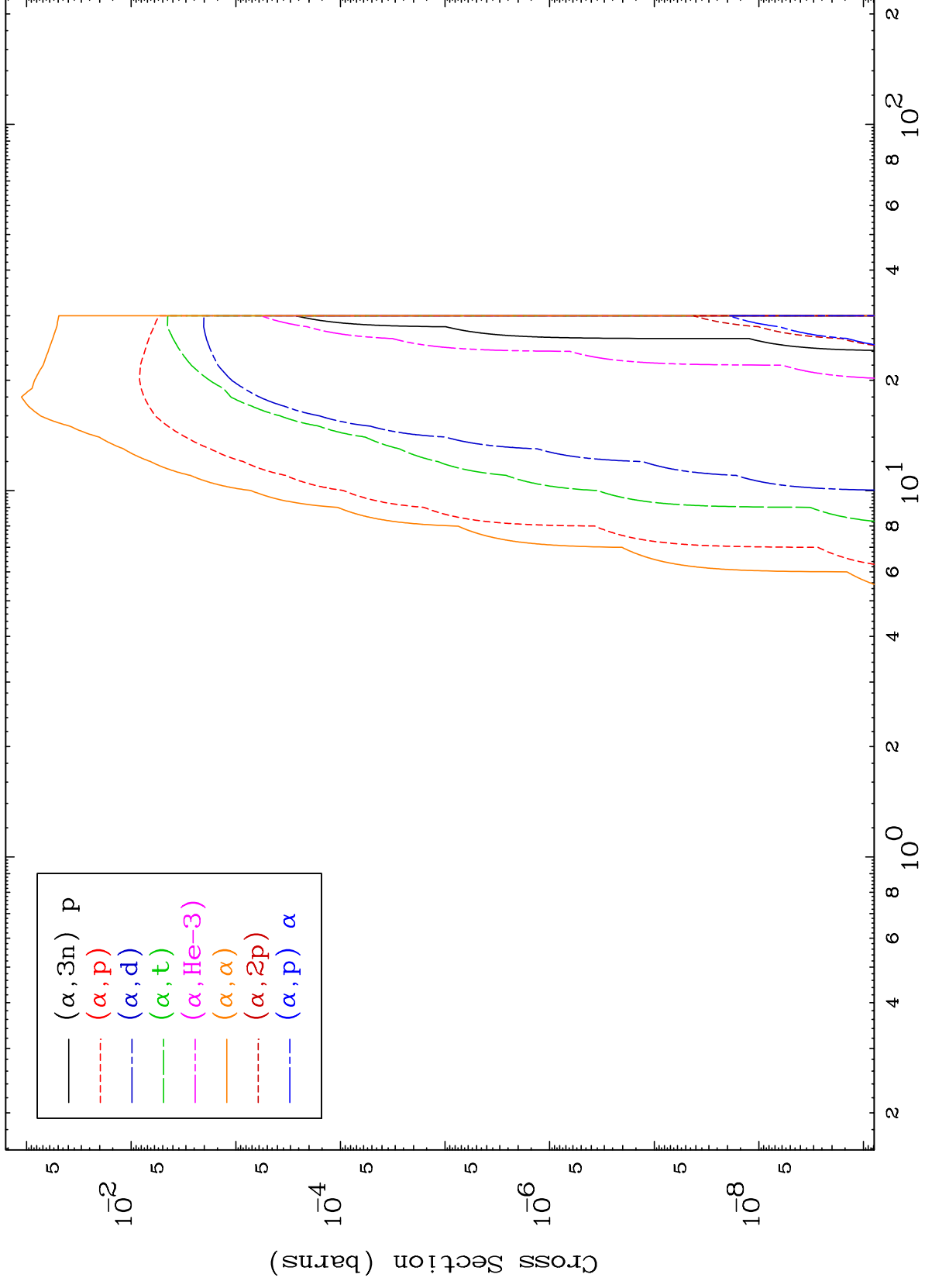
29-Cu-73



4

Incident Energy (MeV)

29-Cu-73

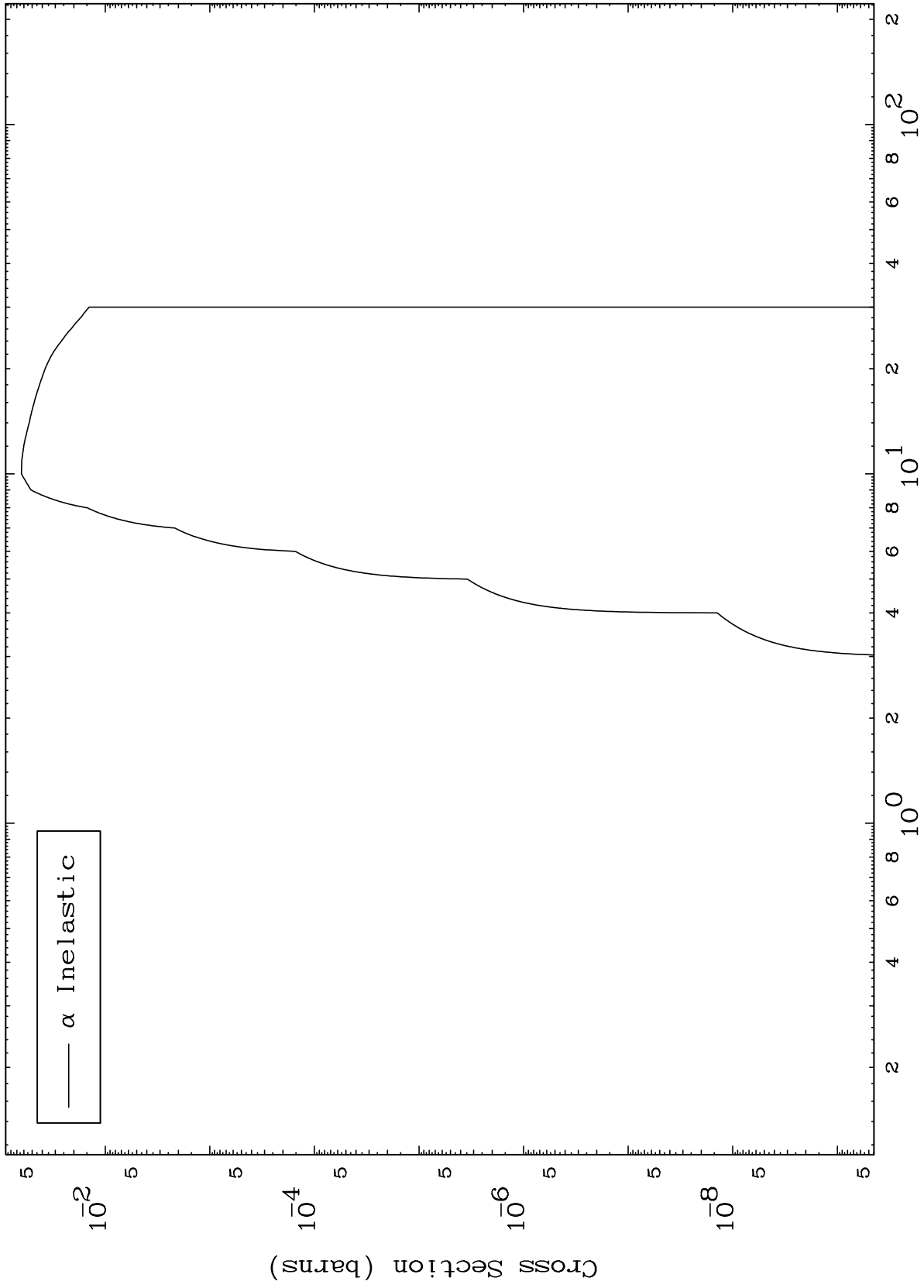


MAT 2955

( $\alpha, n'$ ) Level

29-Cu-73

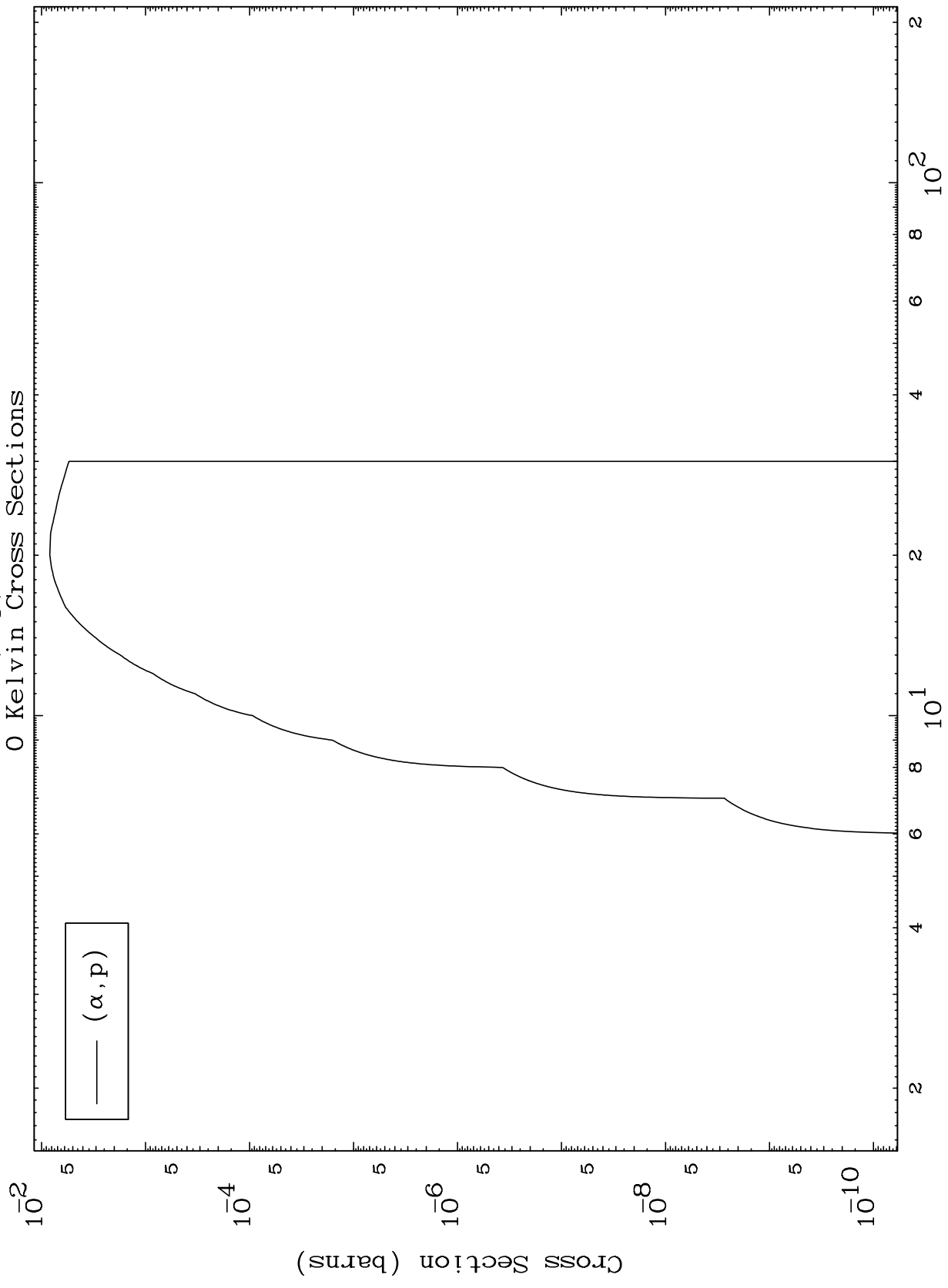
0 Kelvin Cross Sections



MAT 2955

( $\alpha, p$ ) Levels

29-Cu-73

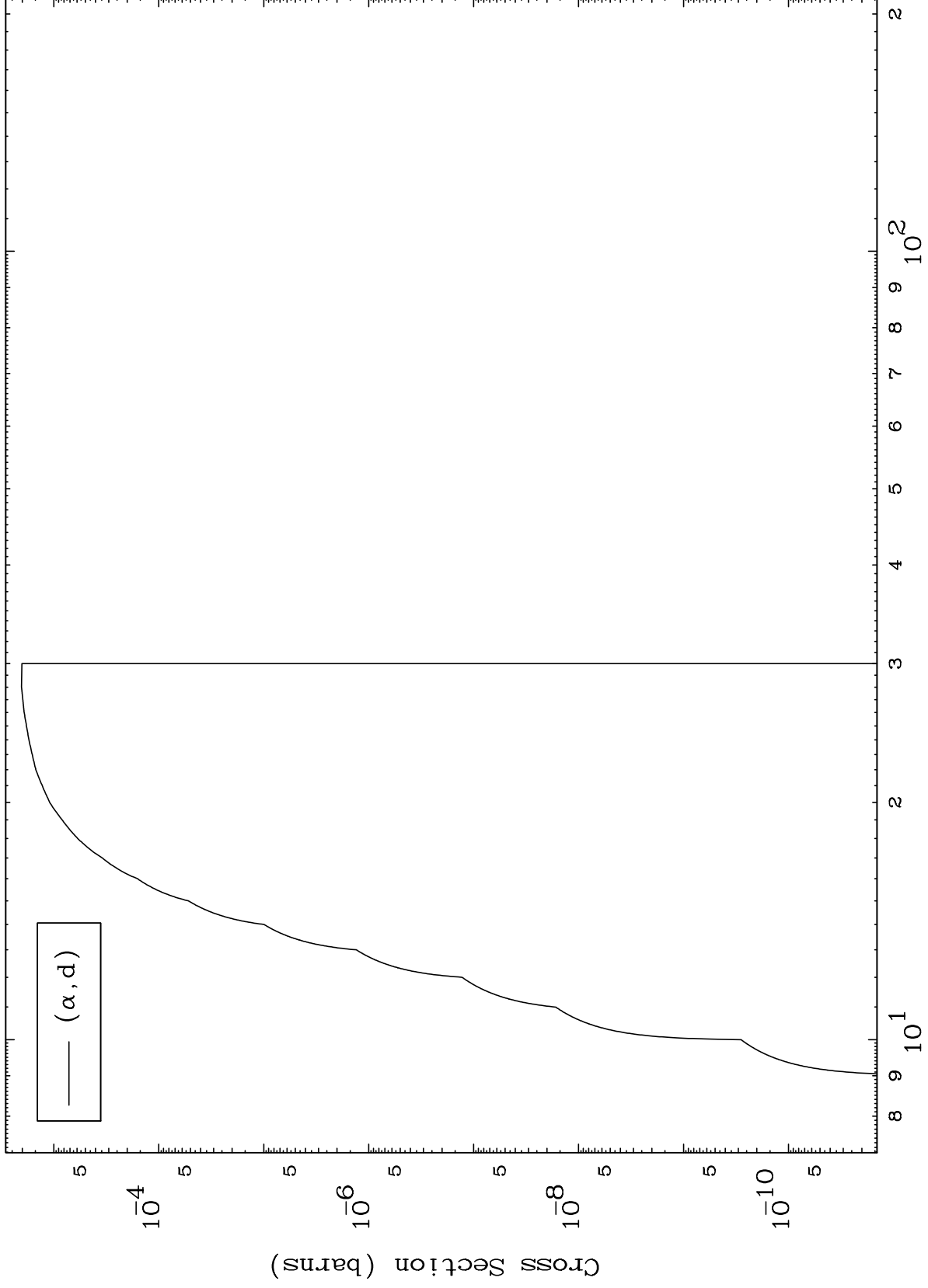




MAT 2955

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

29-Cu-73



8

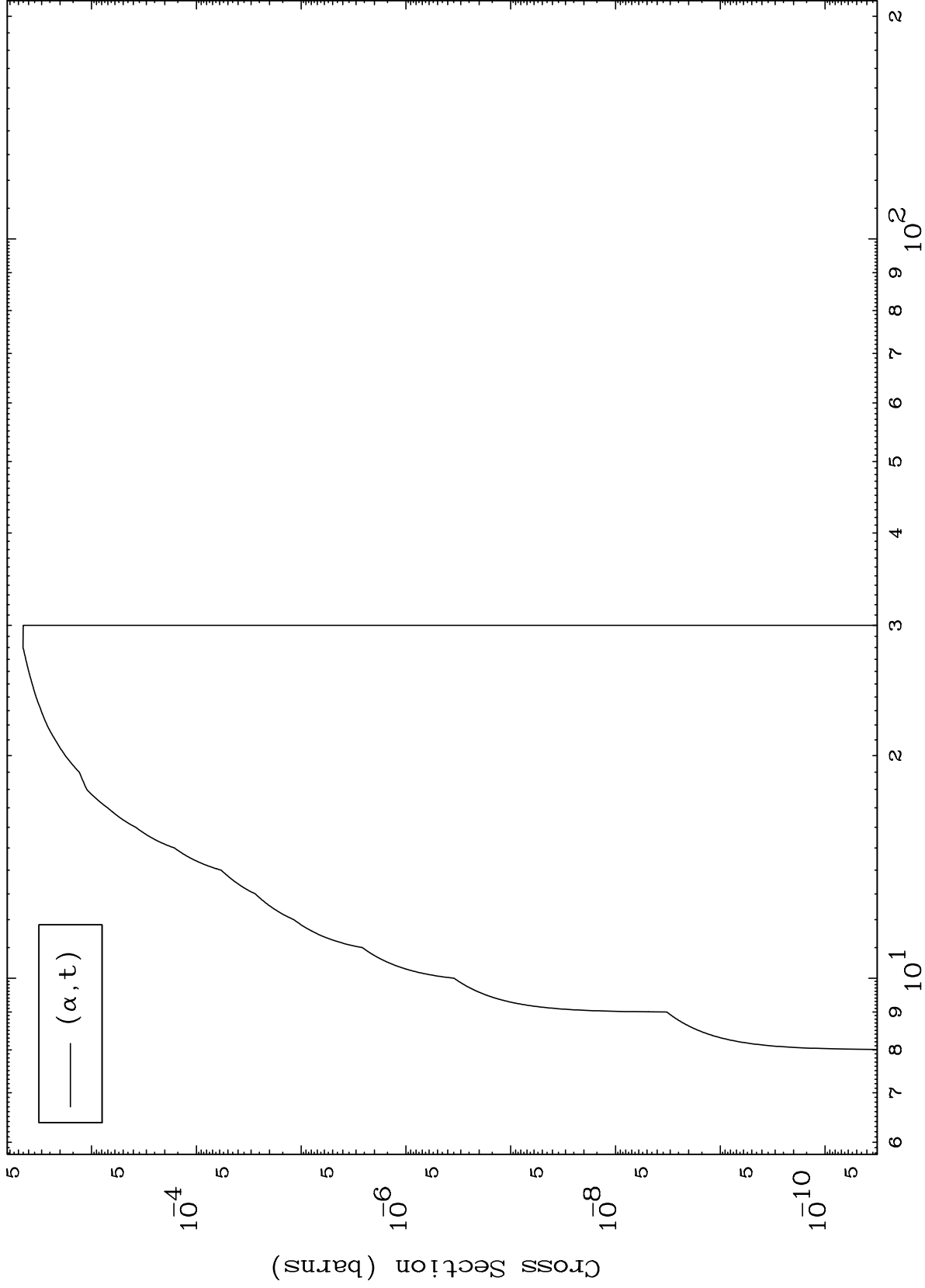
Incident Energy (MeV)

29-Cu-73

MAT 2955

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

29-Cu-73



9

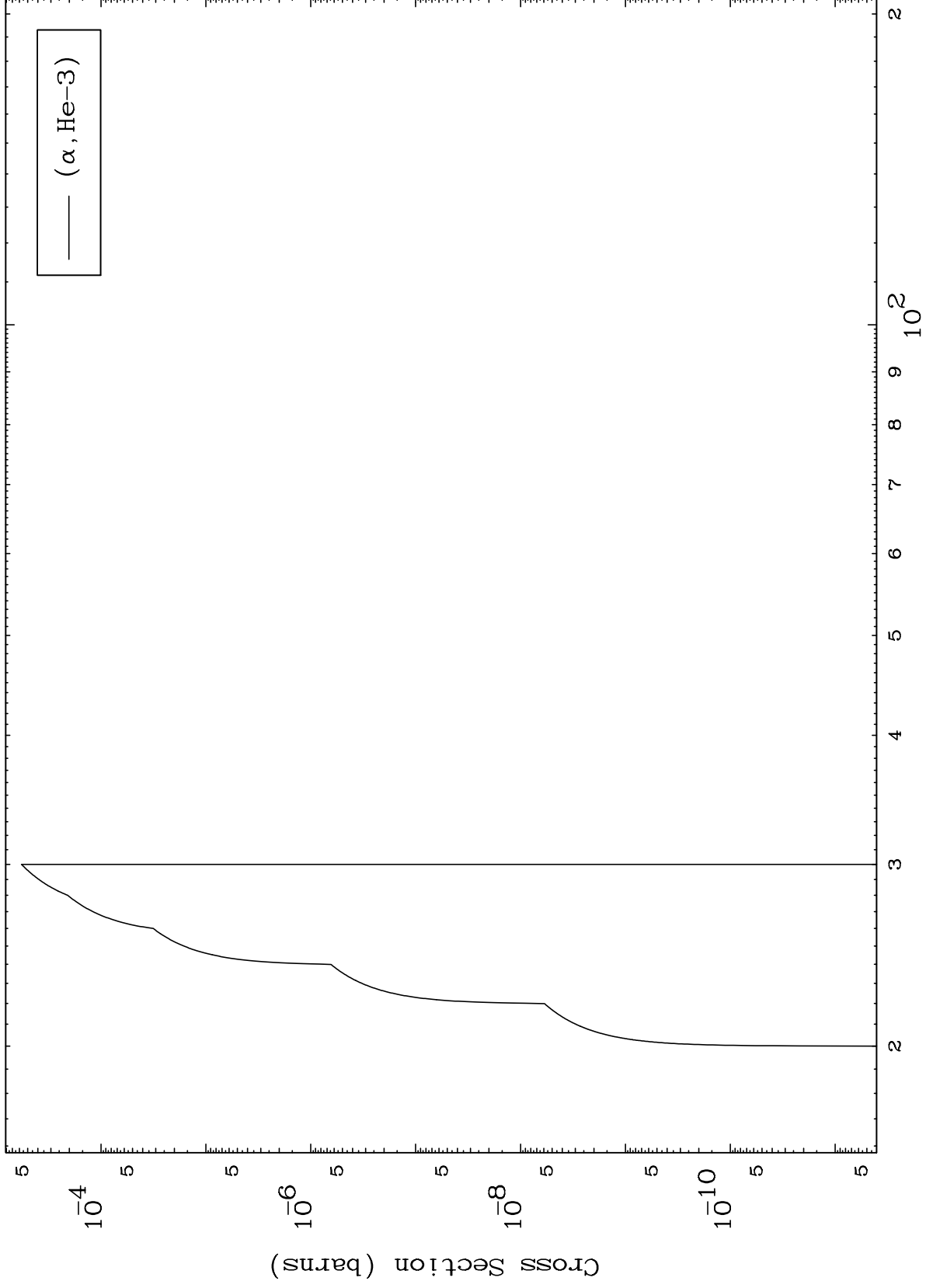
Incident Energy (MeV)

29-Cu-73

MAT 2955

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

29-Cu-73



10

Incident Energy (MeV)

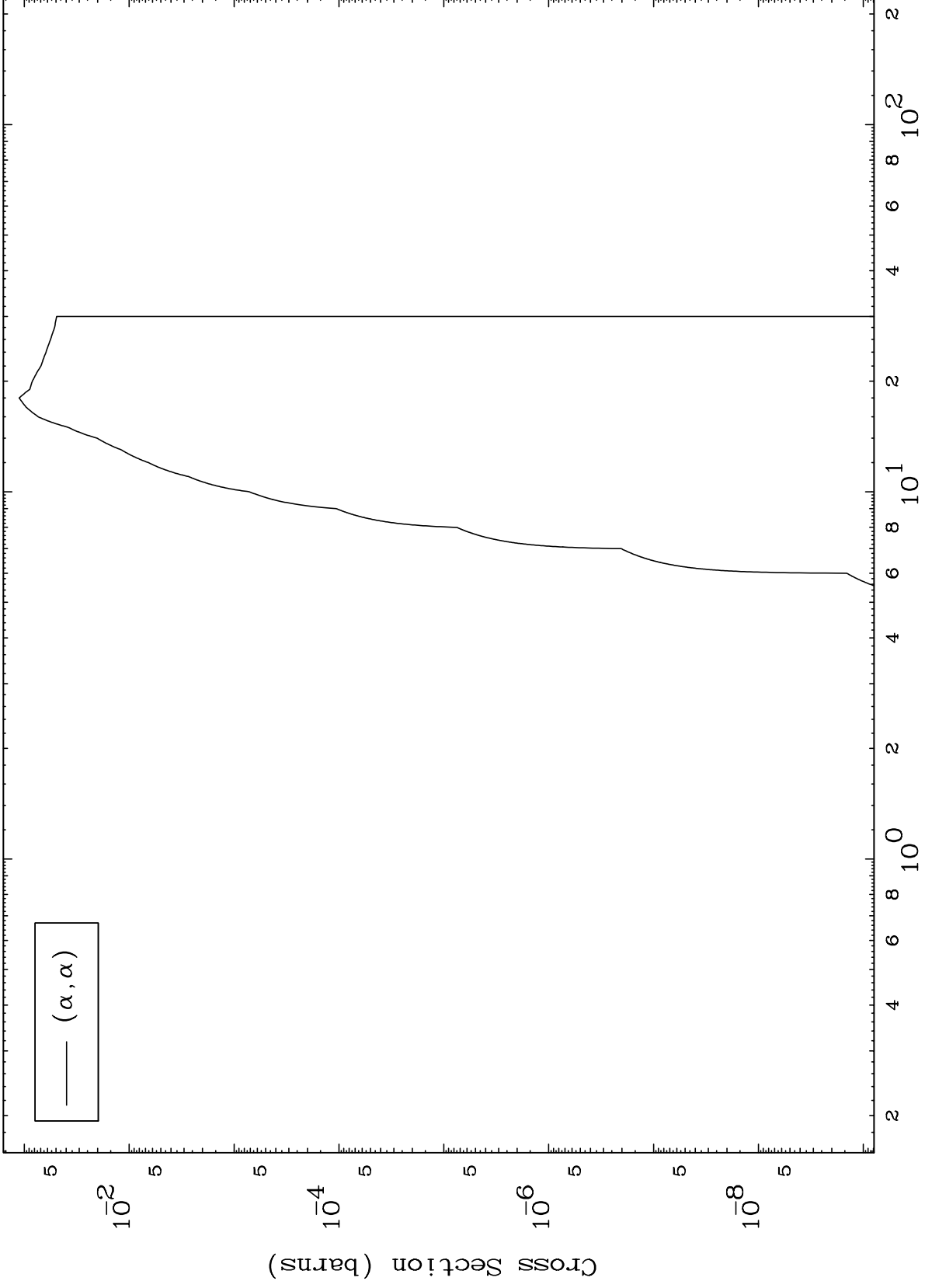
29-Cu-73

MAT 2955

( $\alpha, \alpha$ ) Levels

29-Cu-73

0 Kelvin Cross Sections

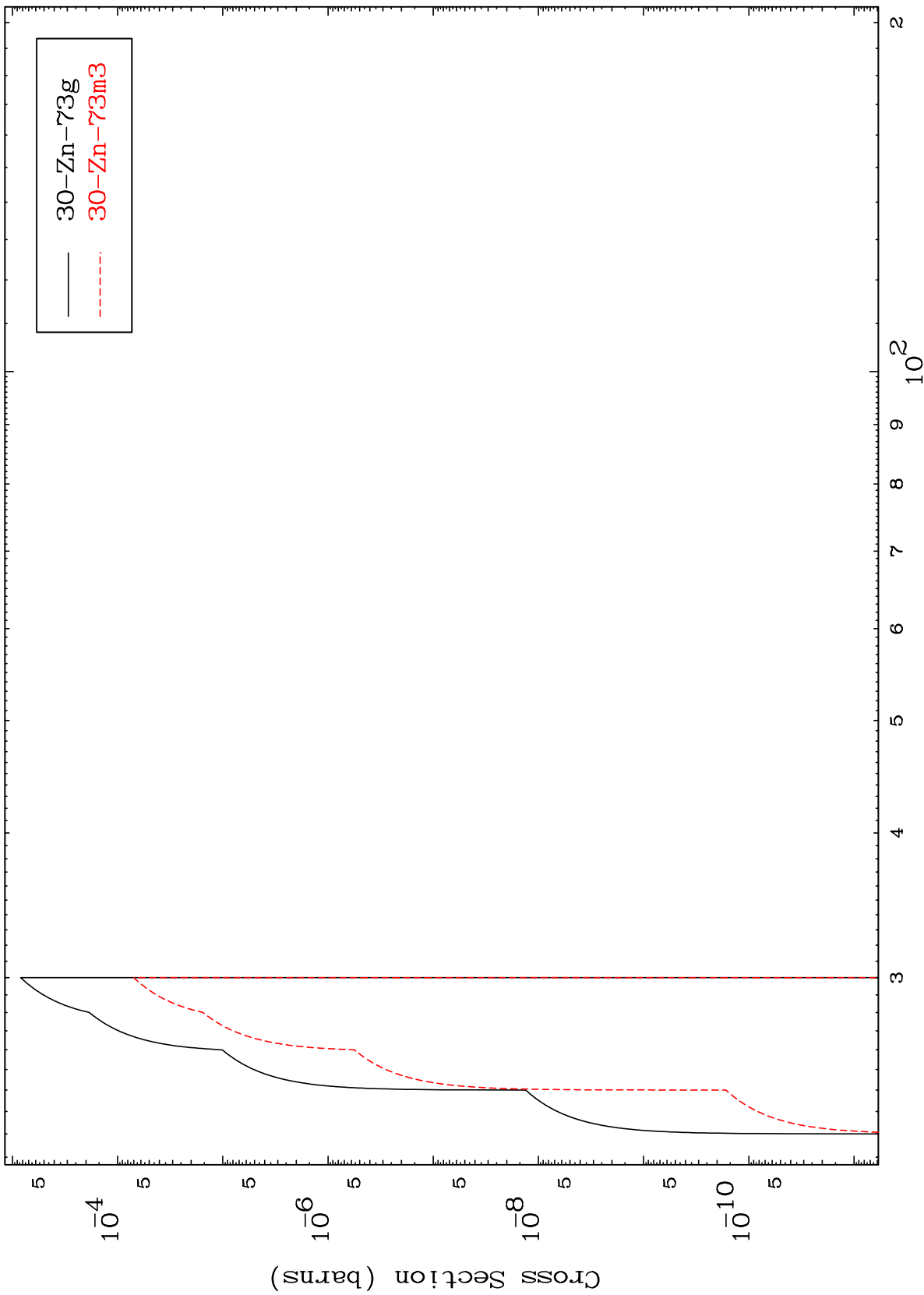


MAT 2955

( $\alpha, 2n$ ) d

29-Cu-73

Radionuclide Production Cross Section



12

Incident Energy (MeV)

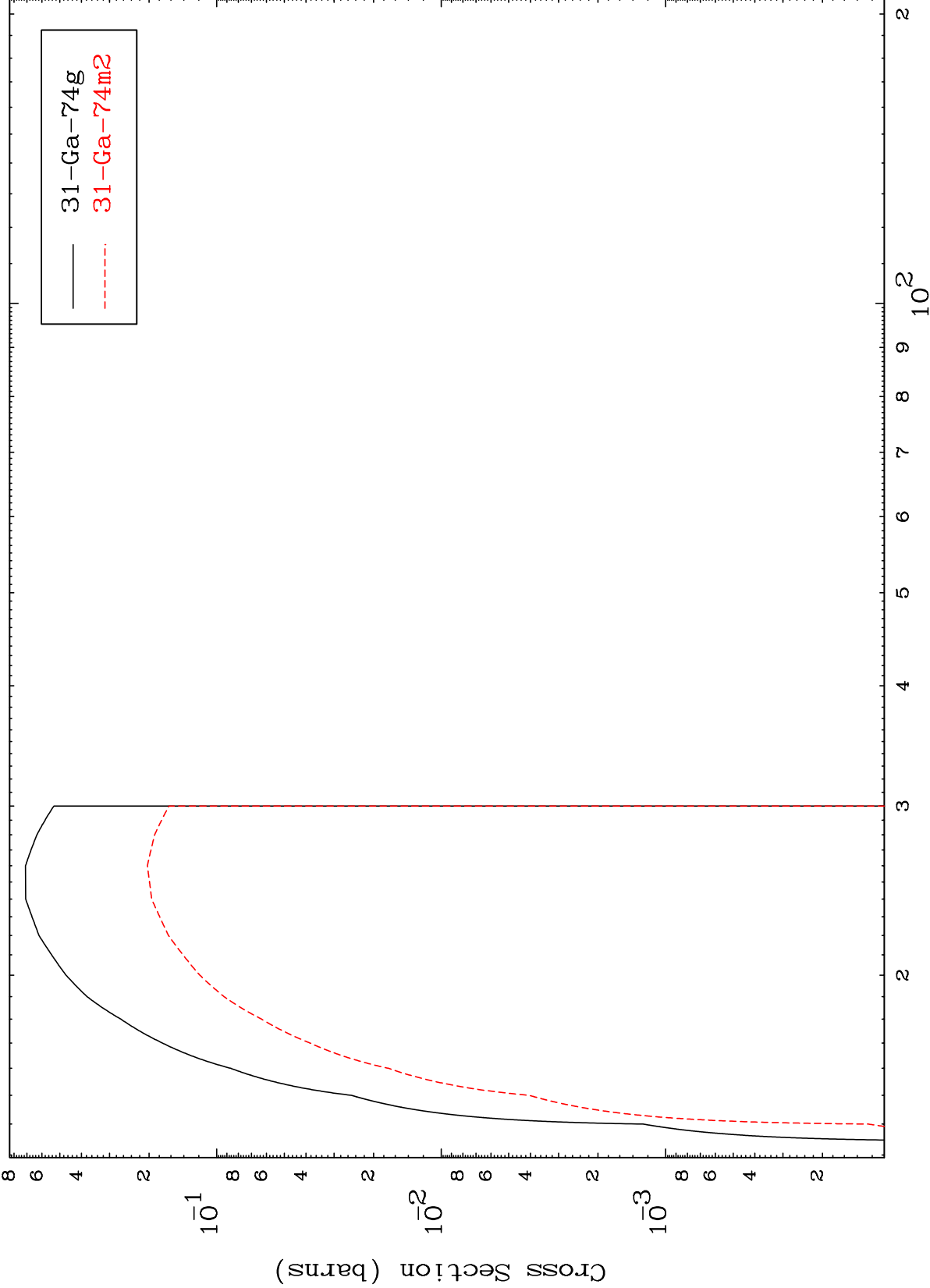
29-Cu-73

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( $\alpha, 3n$ )

29-Cu-73

Radionuclide Production Cross Section



13

Incident Energy (MeV)

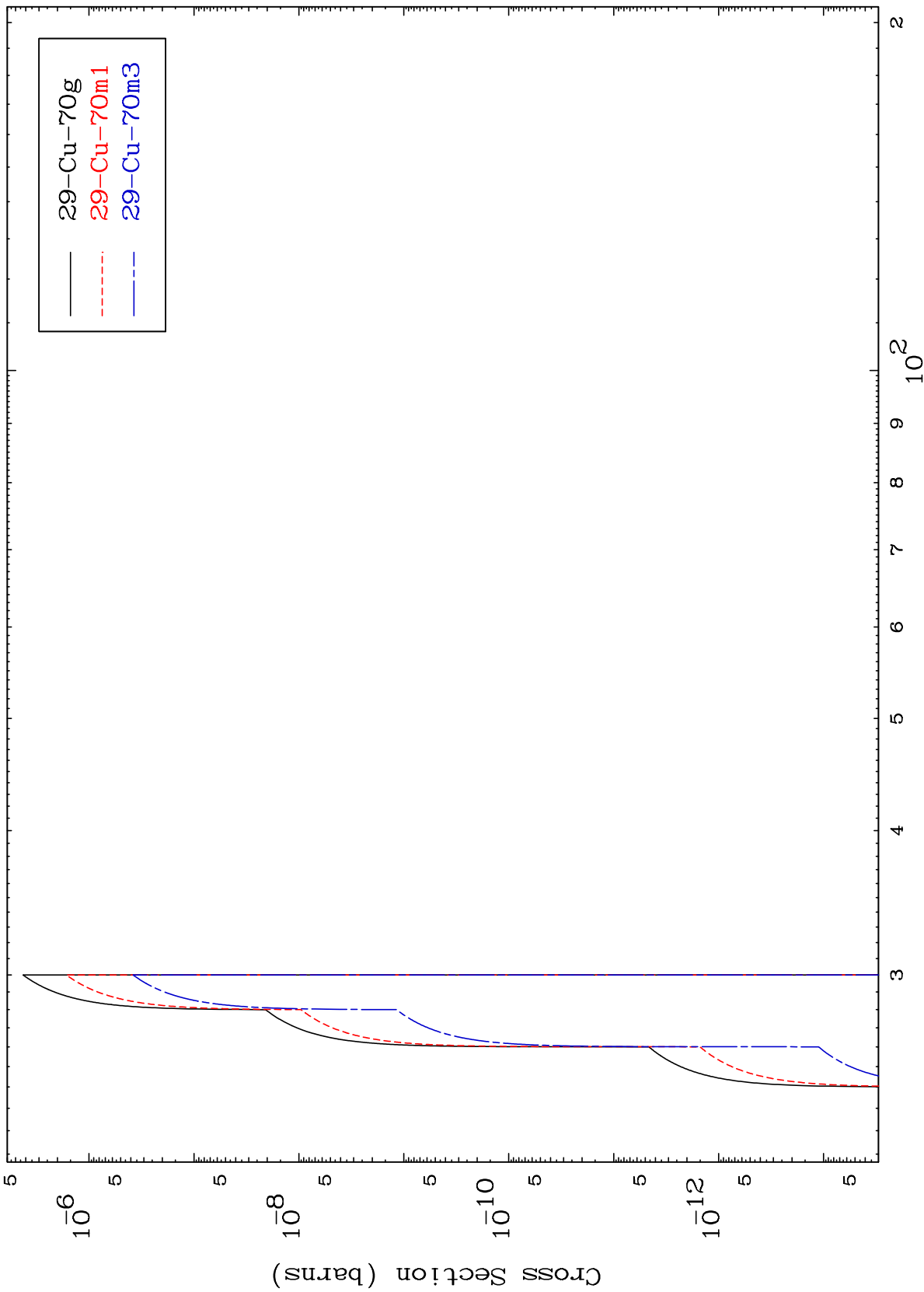
29-Cu-73

MAT 2955

$(\alpha, 3n)$   $\alpha$

$^{29}\text{Cu-73}$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

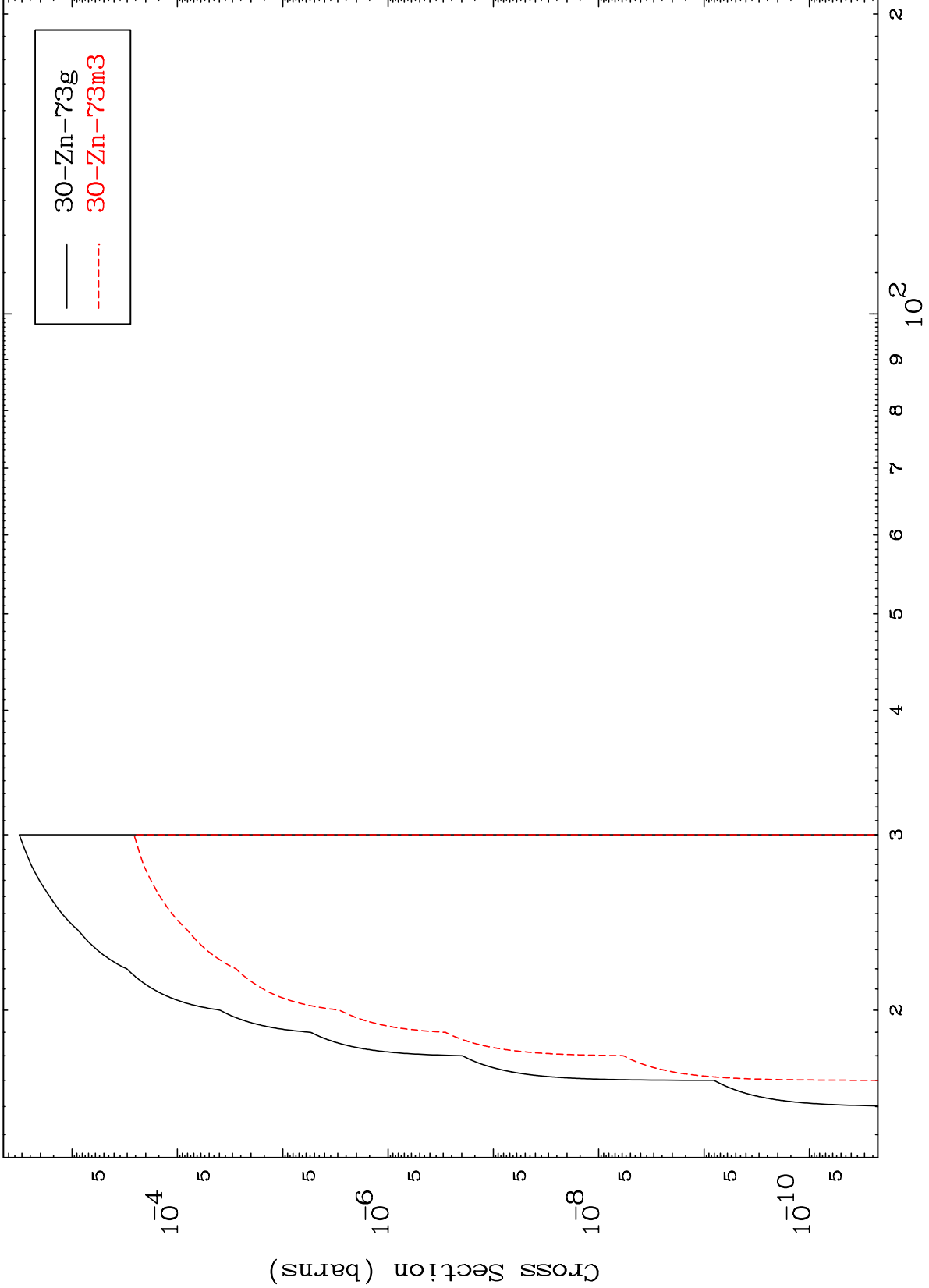
$^{29}\text{Cu-73}$

MAT 2955

( $\alpha, n'$ ) t

29-Cu-73

Radionuclide Production Cross Section



15

Incident Energy (MeV)

29-Cu-73

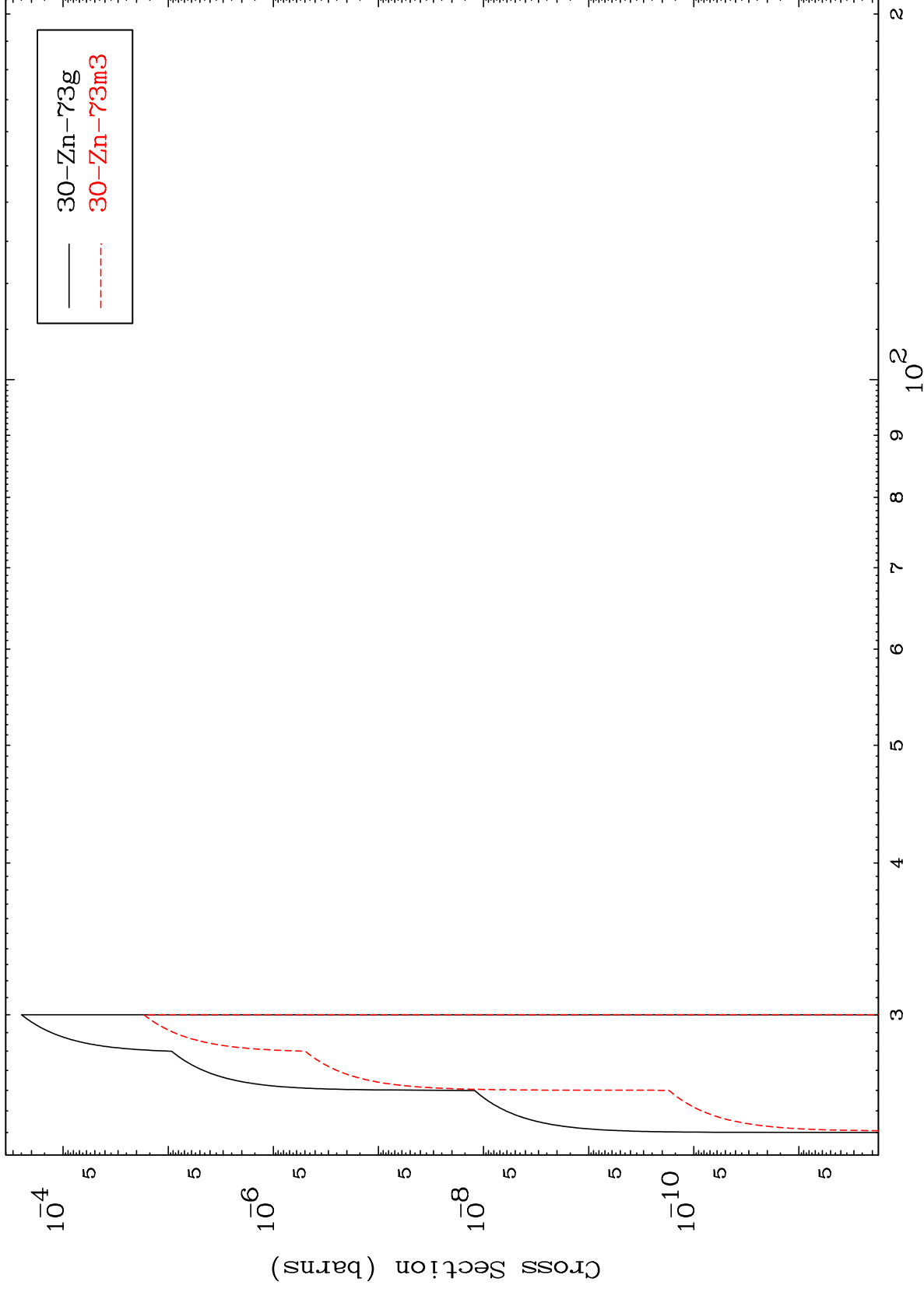


MAT 2955

$(\alpha, 3n)$  p

29-Cu-73

Radionuclide Production Cross Section



16

Incident Energy (MeV)

29-Cu-73