

Program EVALPLOT  
(Version 2018-1)

by

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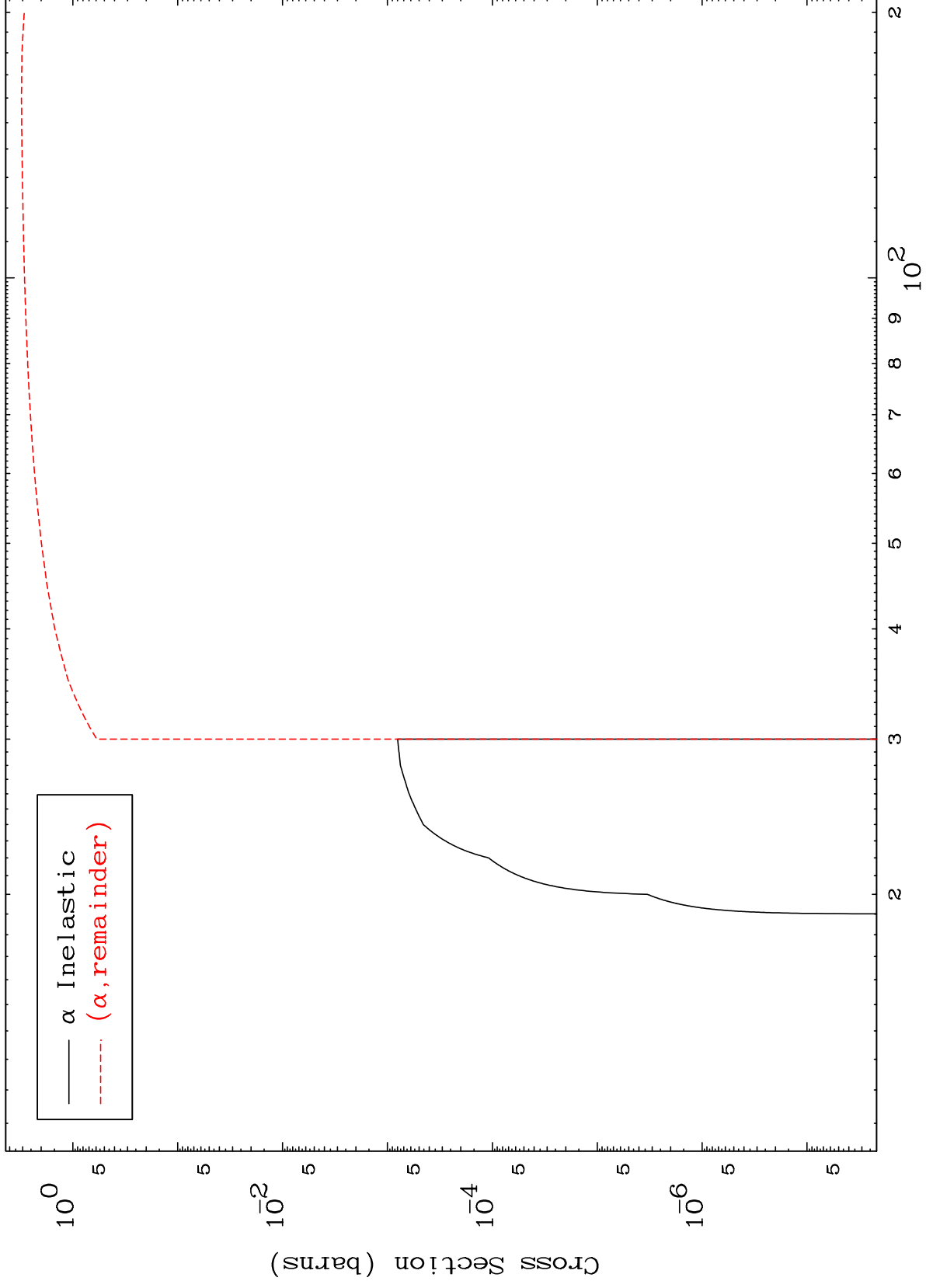
Press Mouse Button to Start

MAT 557

$\alpha$  Major

105-Db-257

0 Kelvin Cross Sections



1

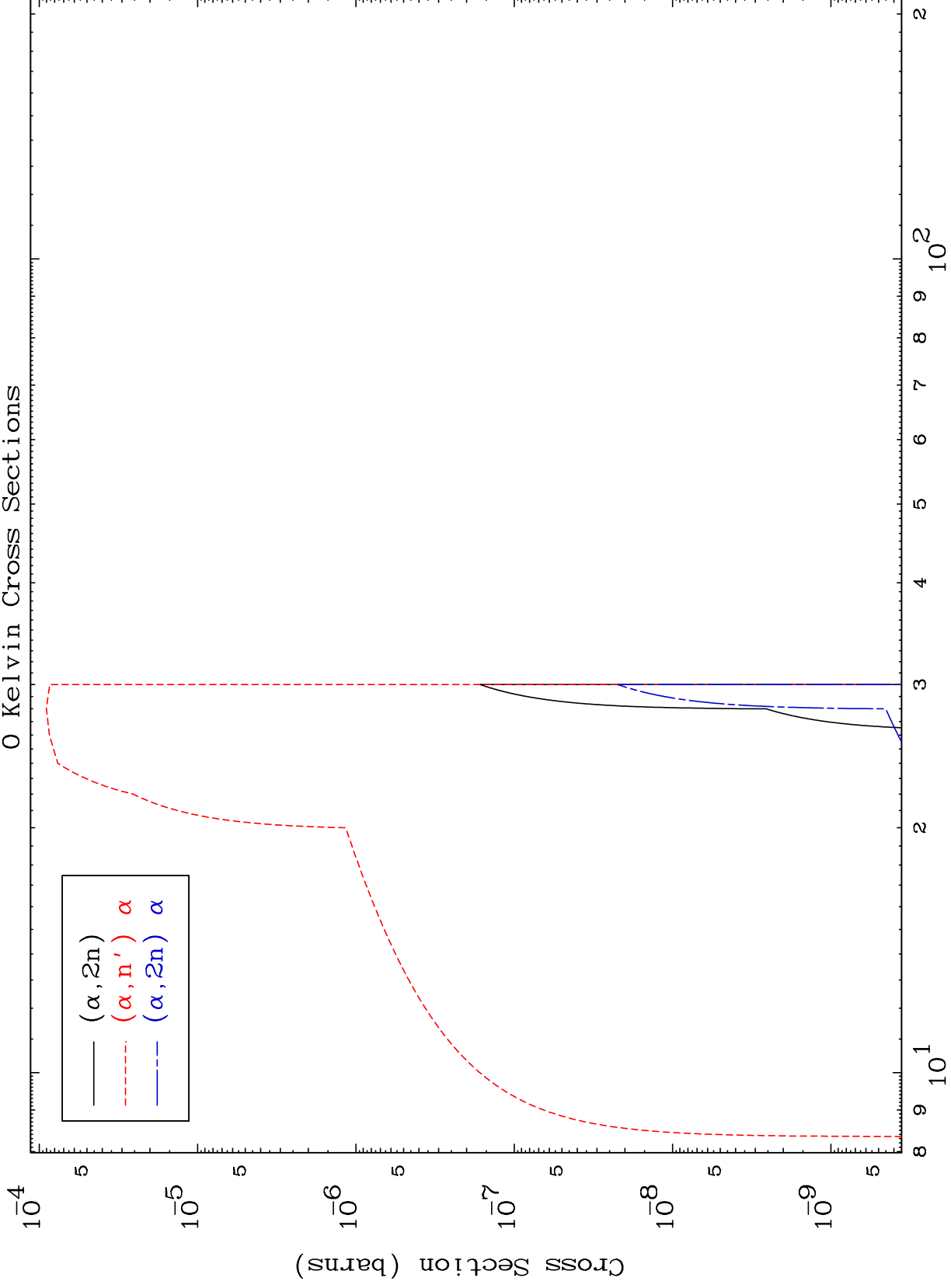
Incident Energy (MeV)

105-Db-257

MAT 557

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

105-Db-257

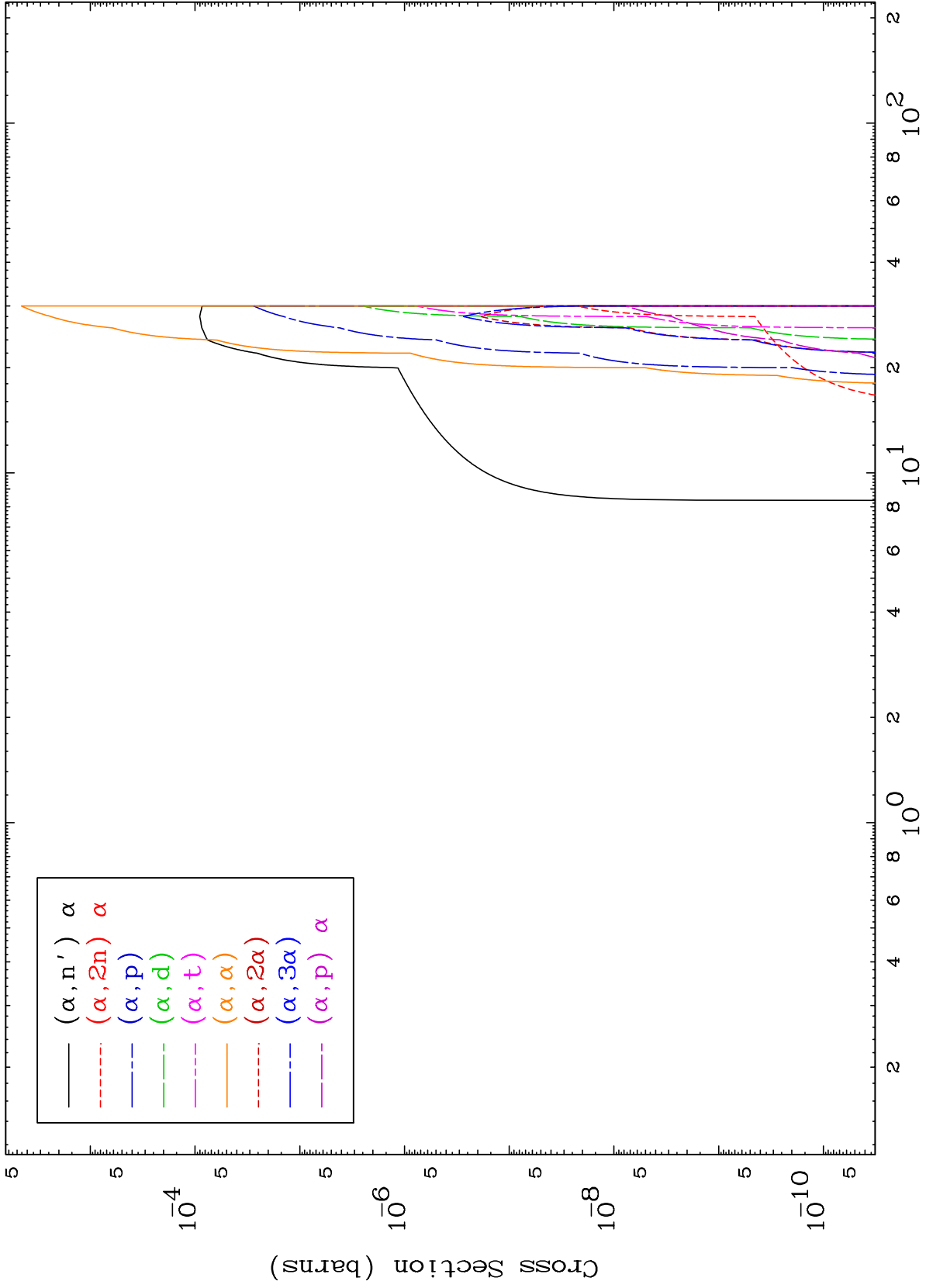


( $\alpha, 2n$ )  
( $\alpha, n'$ )  $\alpha$   
( $\alpha, 2n$ )  $\alpha$

105-Db-257

Incident Energy (MeV)

2

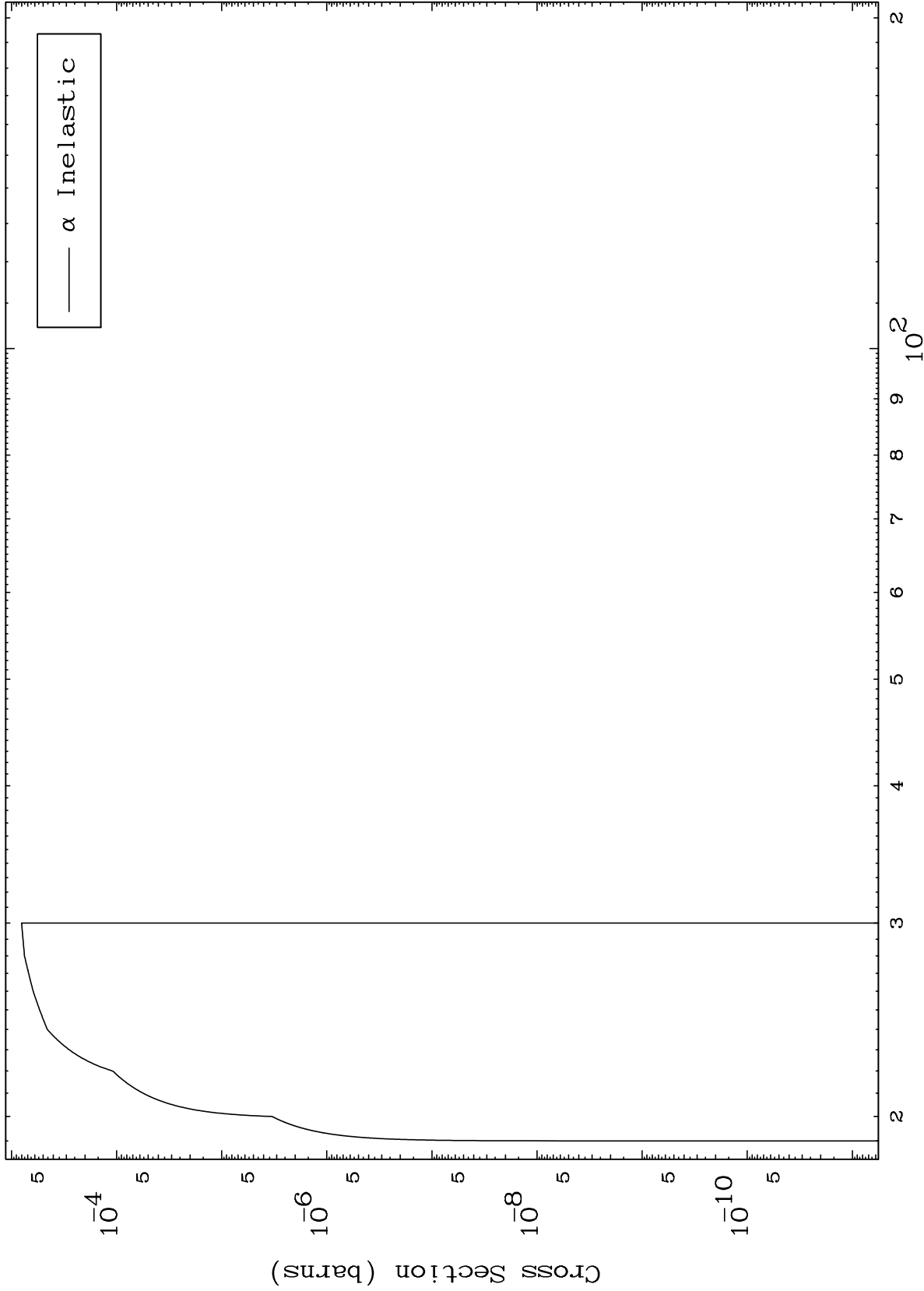


MAT 557

( $\alpha, n'$ ) Level

105-Db-257

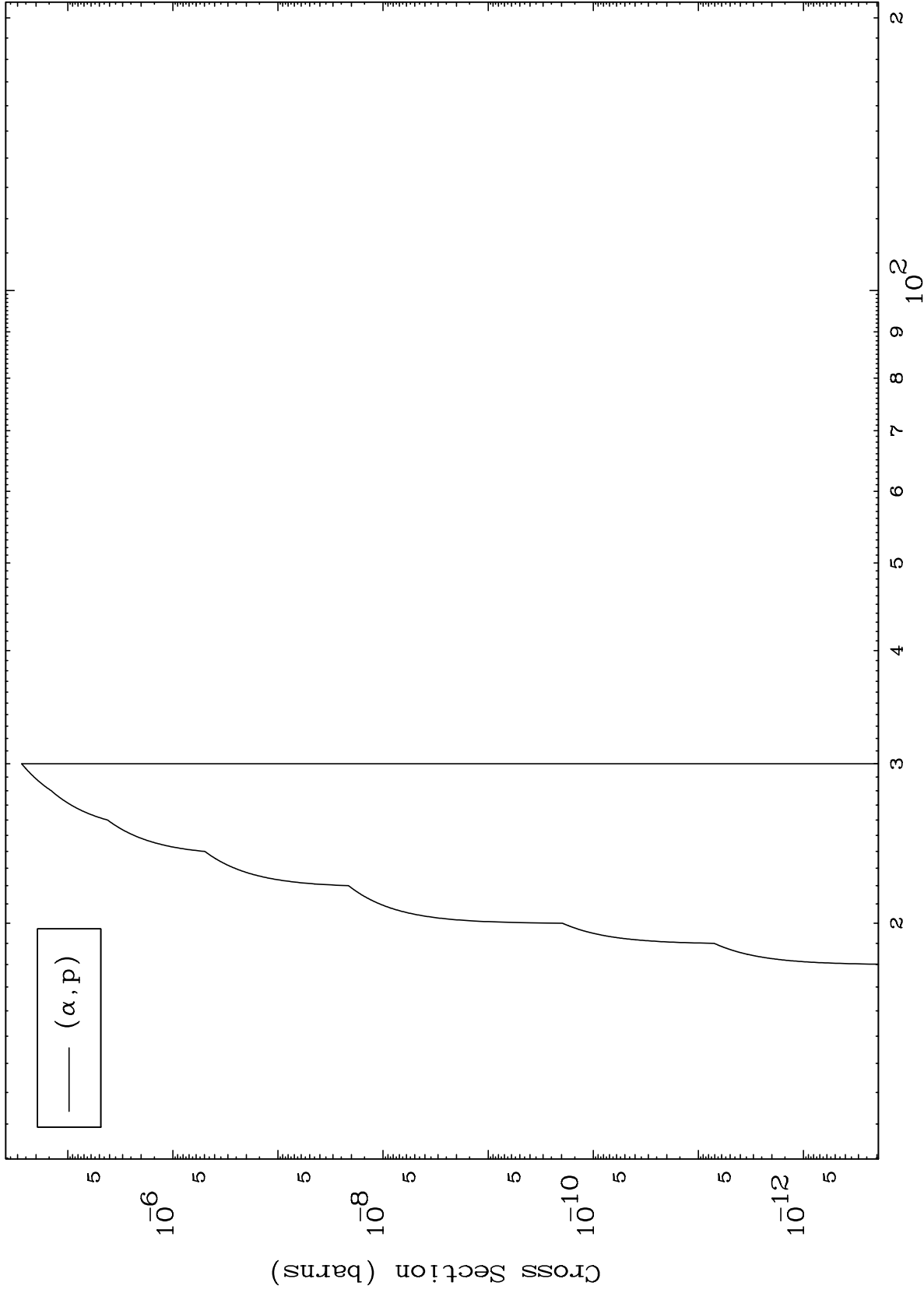
0 Kelvin Cross Sections

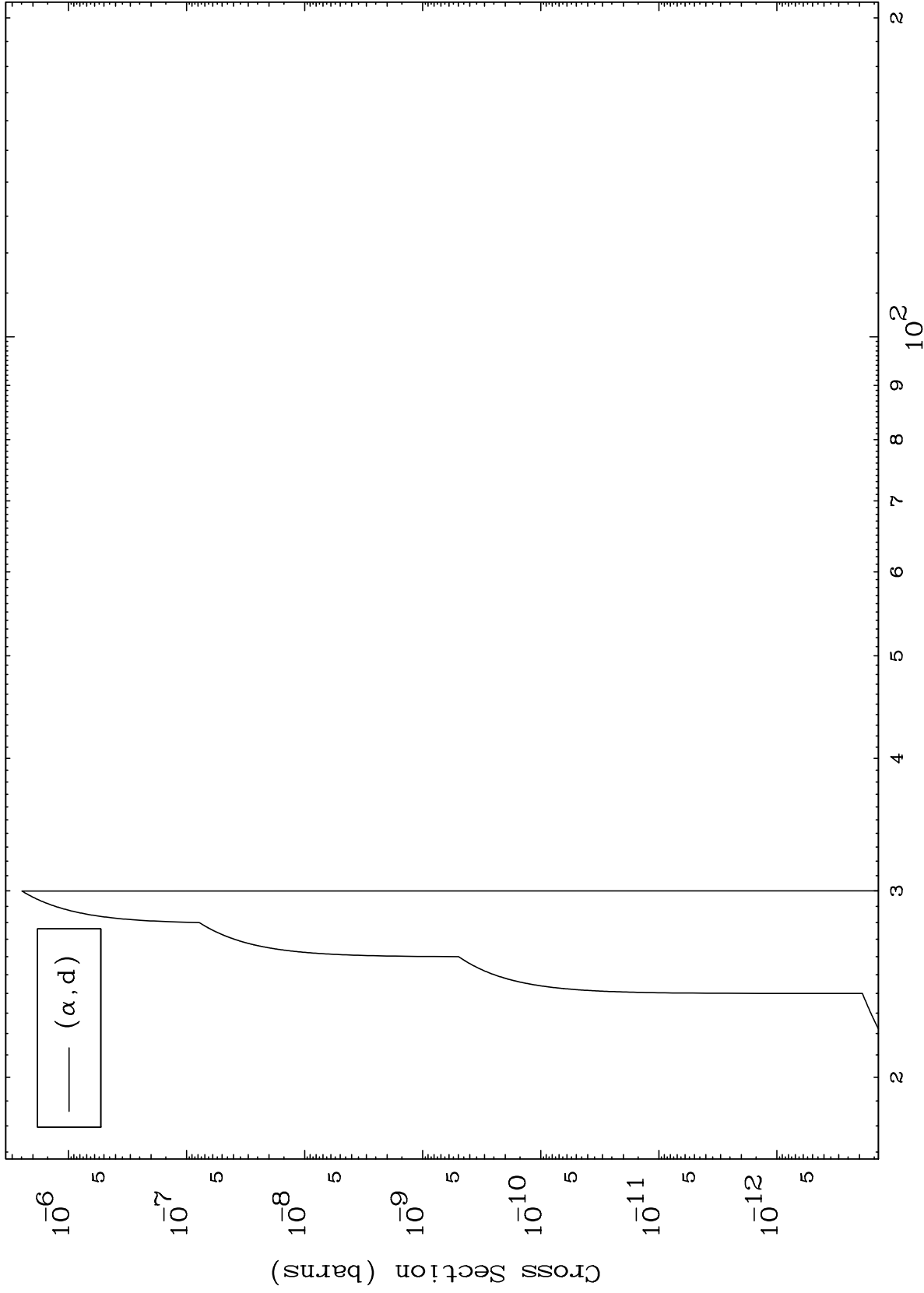


4

Incident Energy (MeV)

105-Db-257

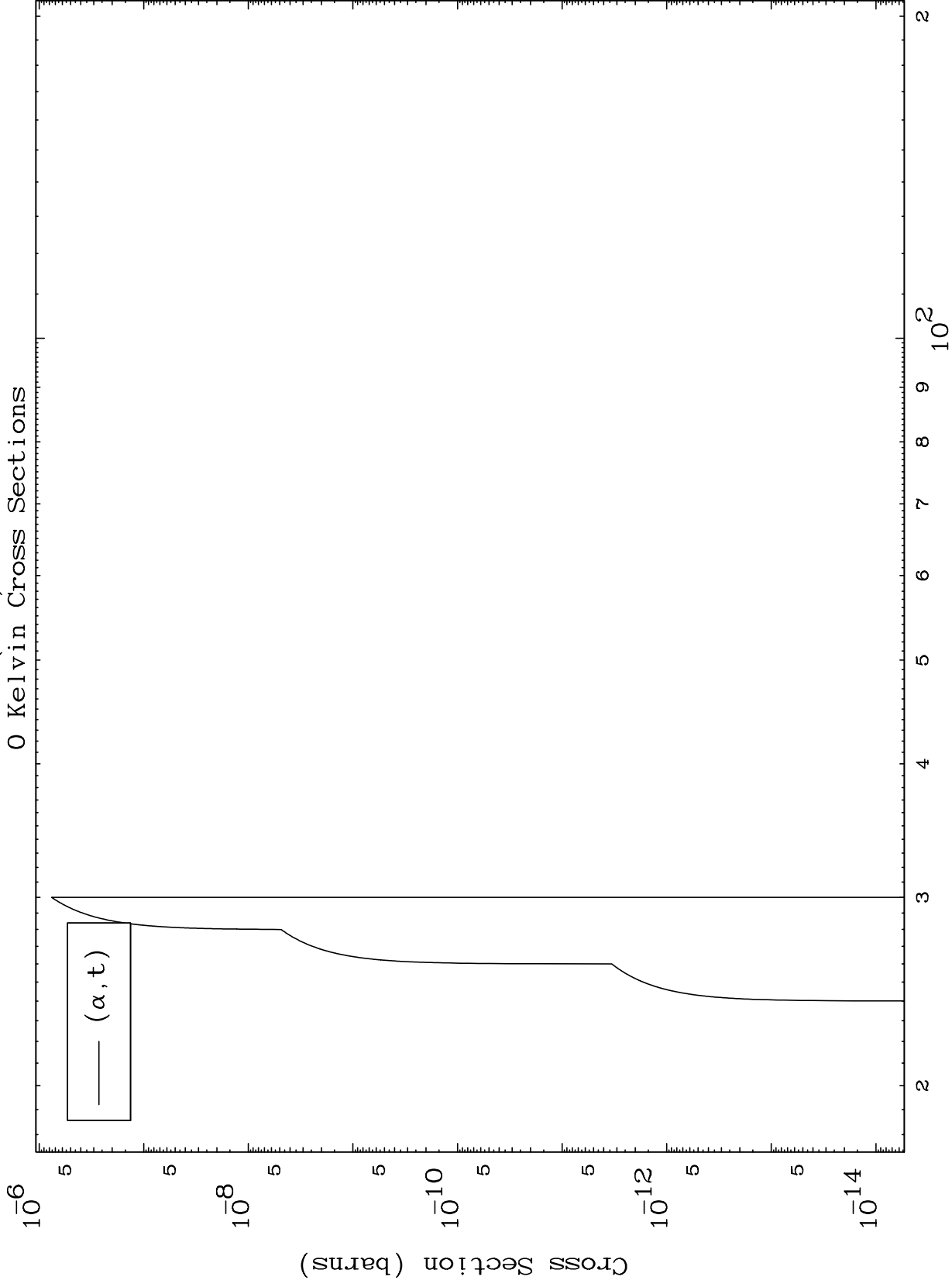




MAT 557

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

105-Db-257



7

Incident Energy (MeV)

105-Db-257

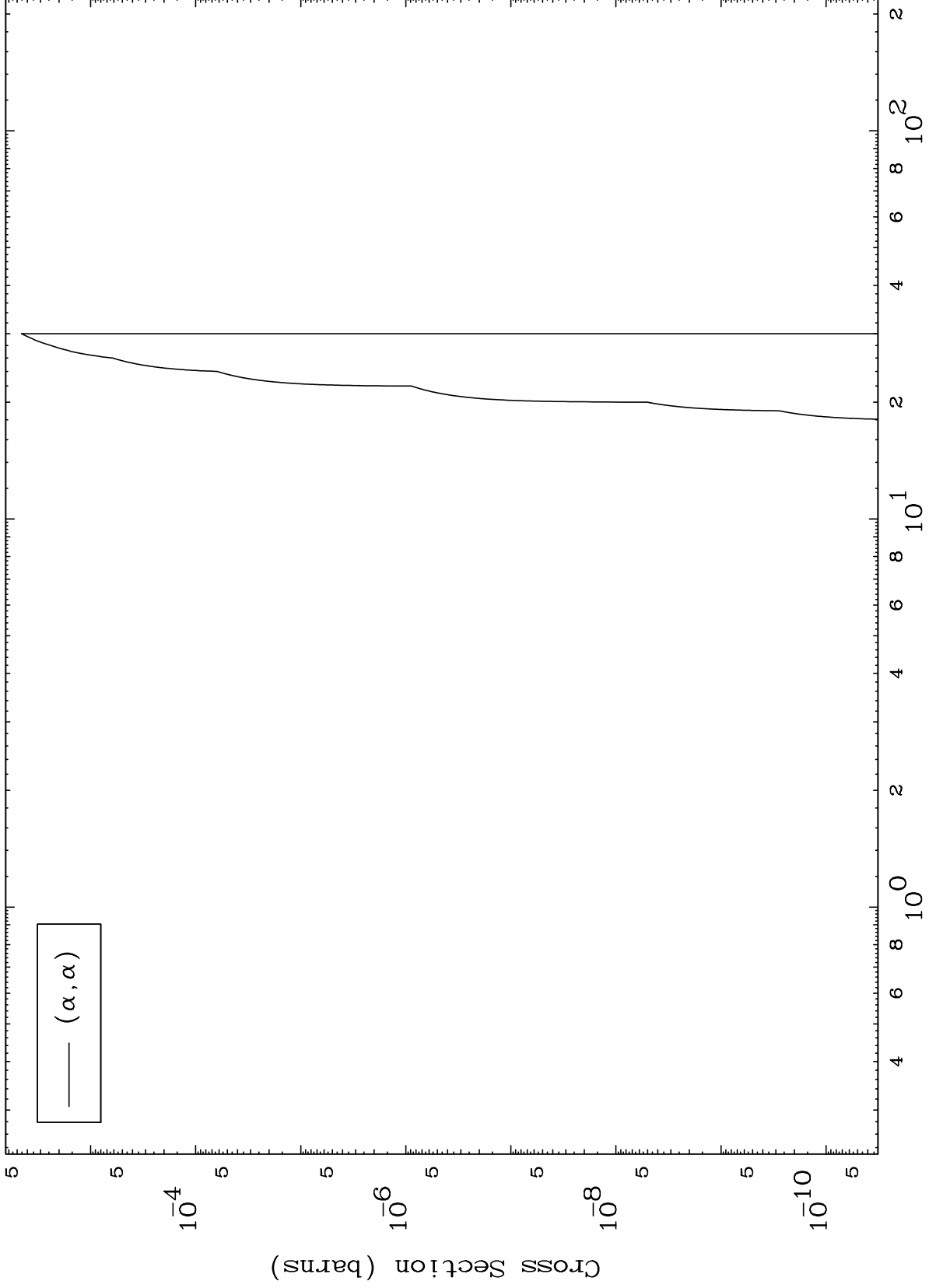


MAT 557

( $\alpha, \alpha$ ) Levels

105-Db-257

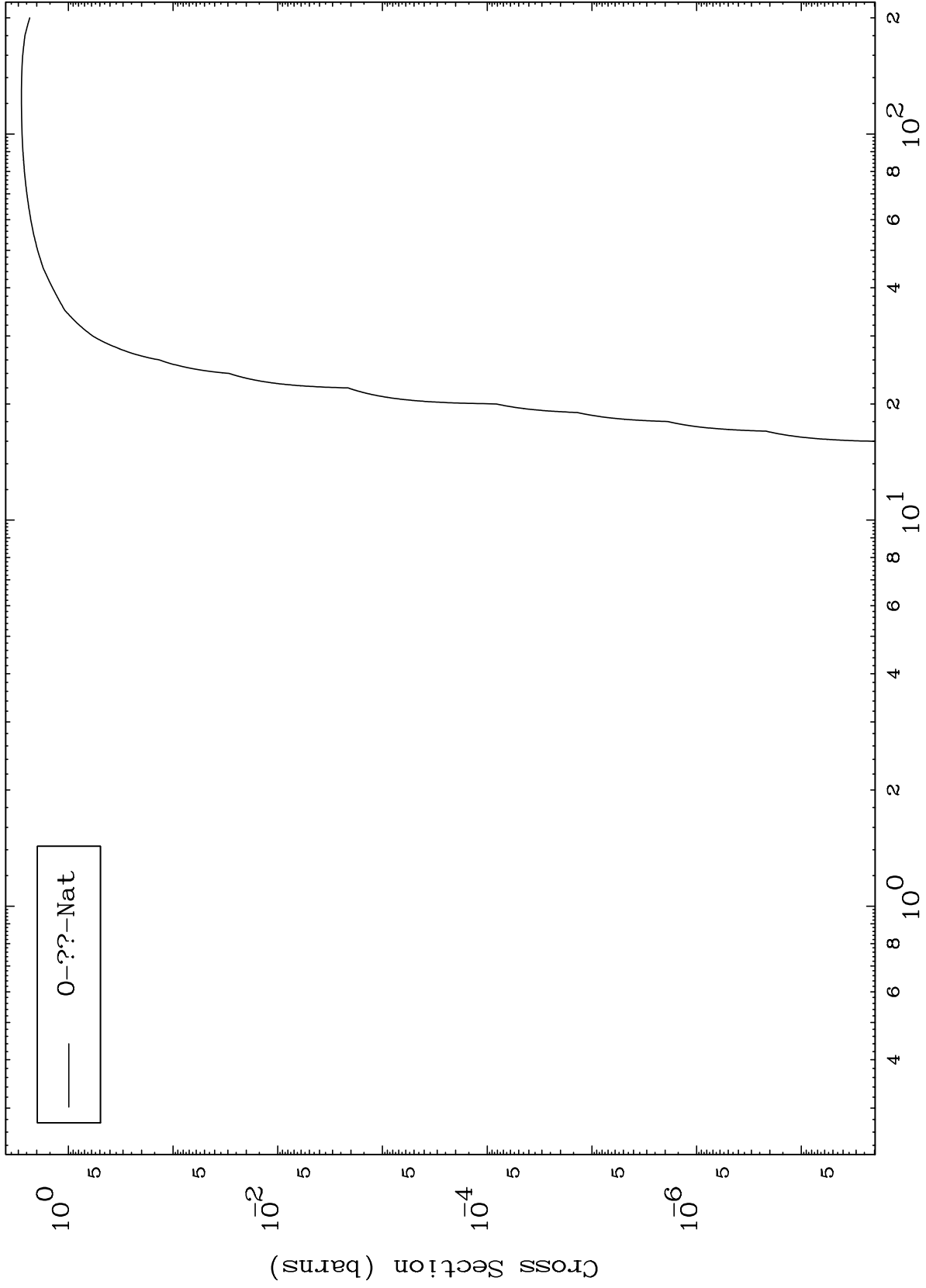
0 Kelvin Cross Sections



MAT 557

105-Db-257

$\alpha$  Fission  
Radionuclide Production Cross Section

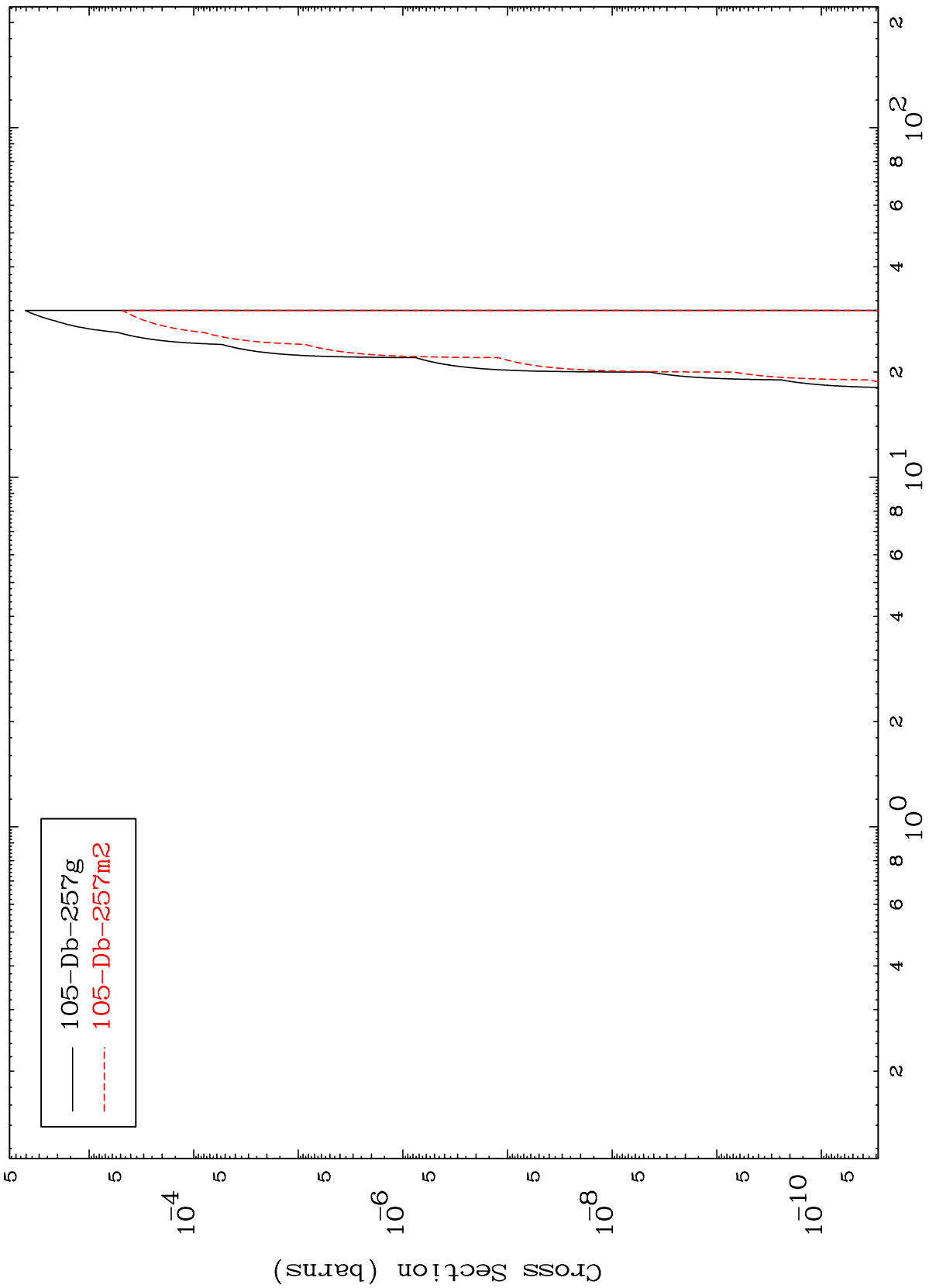


MAT 557

( $\alpha, \alpha$ )

105-Db-257

Radionuclide Production Cross Section



— 105-Db-257g  
- - - 105-Db-257m2

10

Incident Energy (MeV)

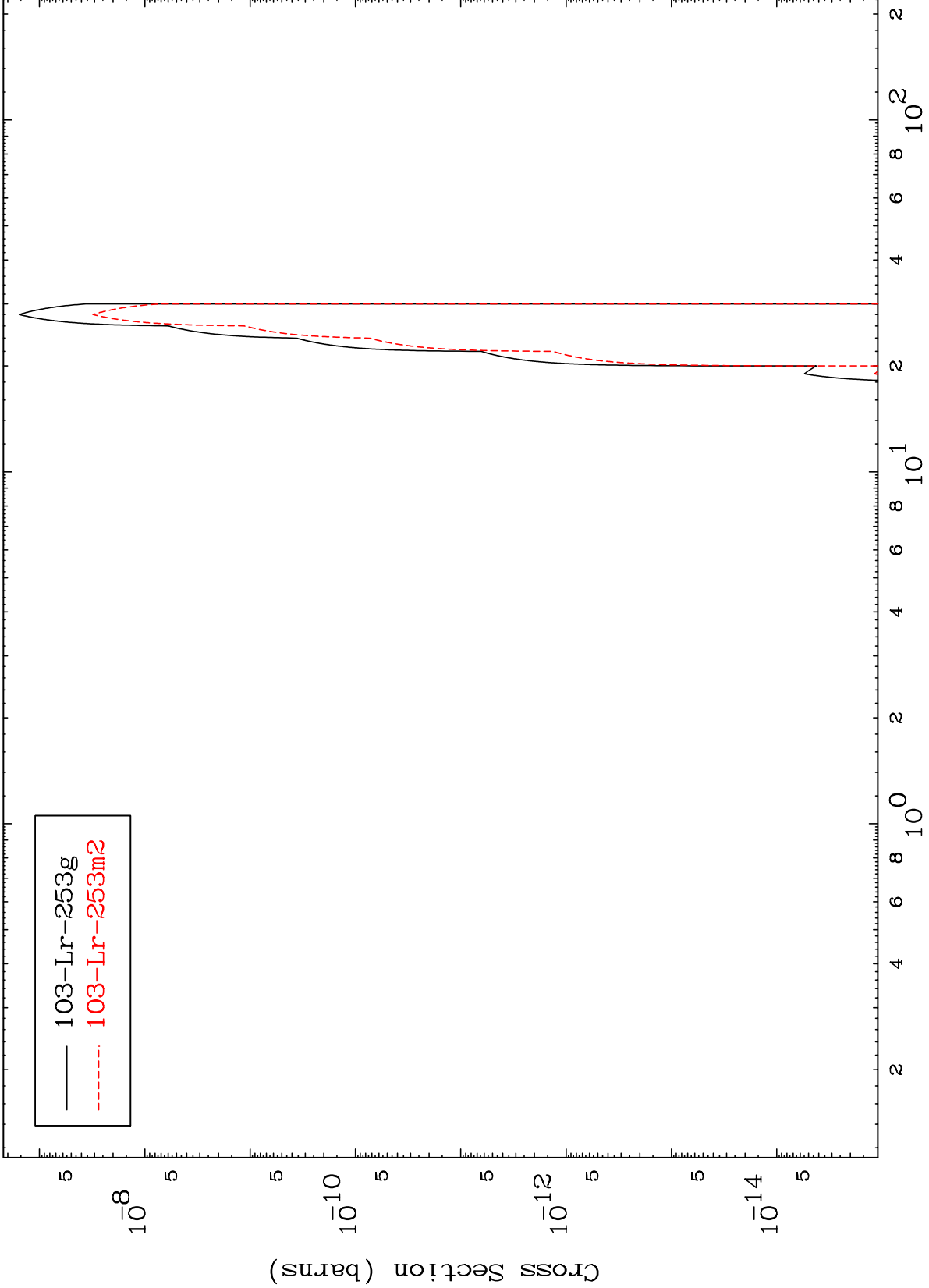
105-Db-257

MAT 557

( $\alpha, 2\alpha$ )

105-Db-257

Radionuclide Production Cross Section



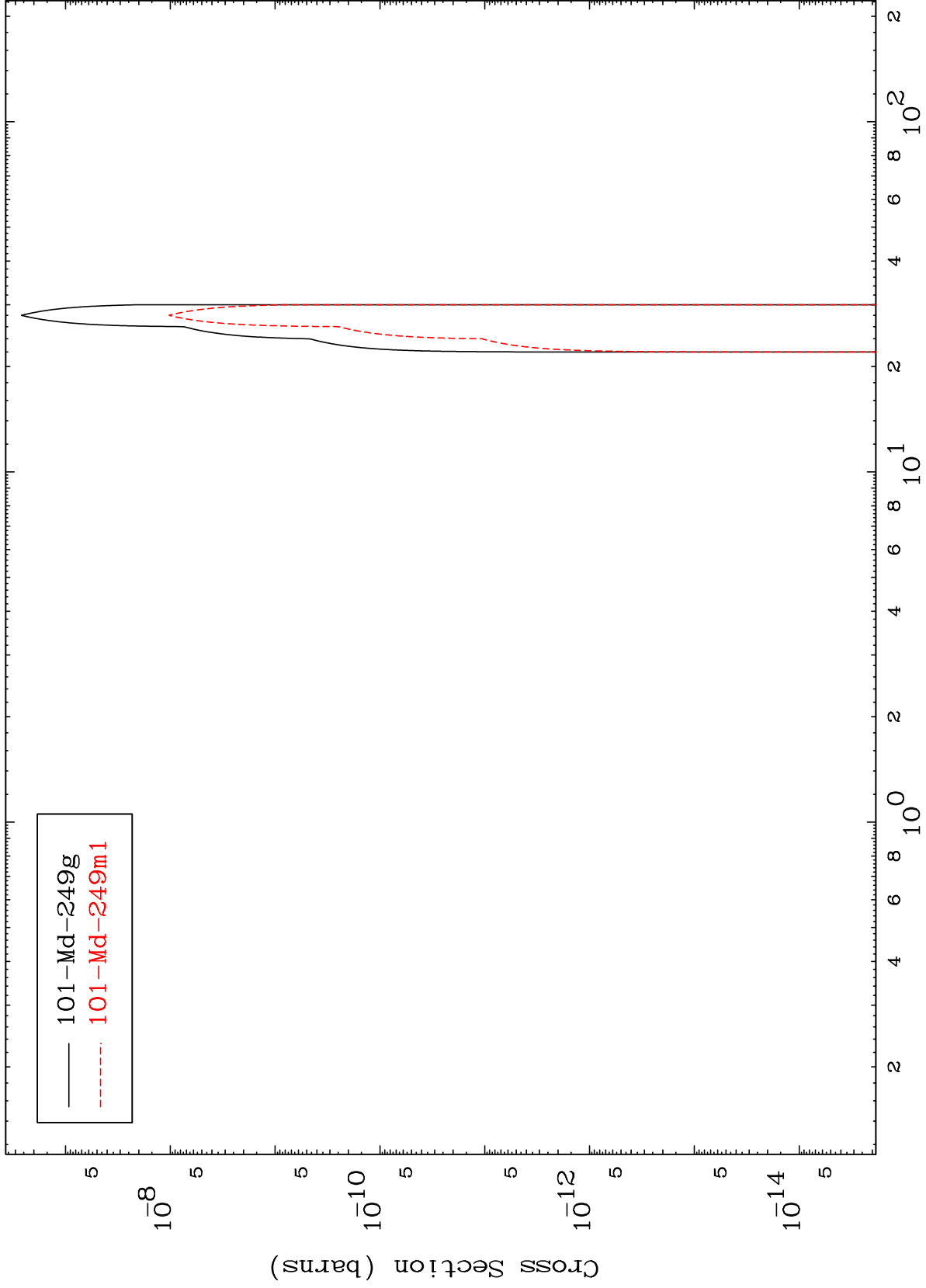
— 103-Lr-253g  
- - - 103-Lr-253m2

MAT 557

( $\alpha, 3\alpha$ )

105-Db-257

Radionuclide Production Cross Section



12

Incident Energy (MeV)

105-Db-257