

Program EVALPLOT  
(Version 2018-1)

by

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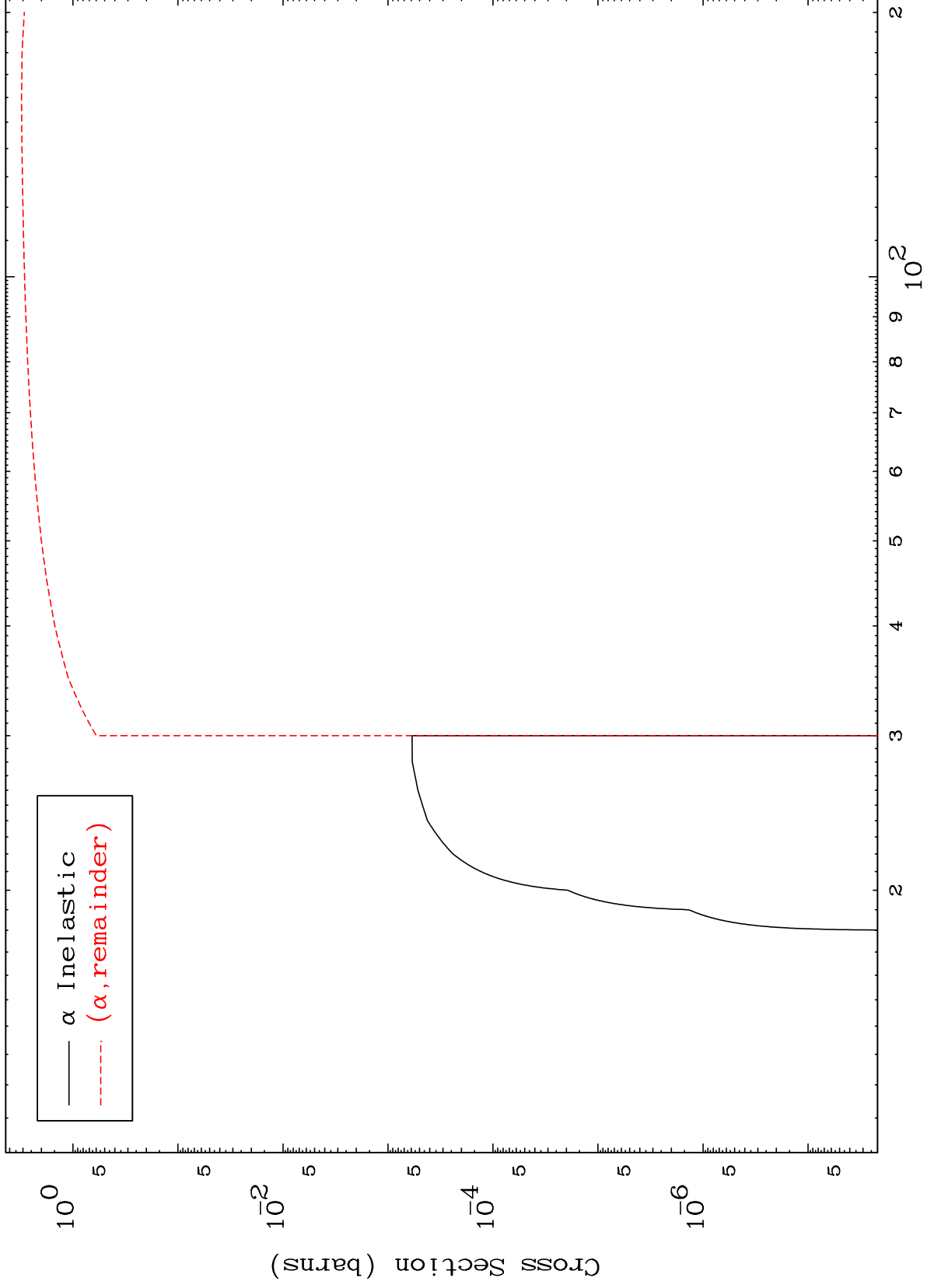
Press Mouse Button to Start

MAT 558

$\alpha$  Major

105-Db-258

0 Kelvin Cross Sections



1

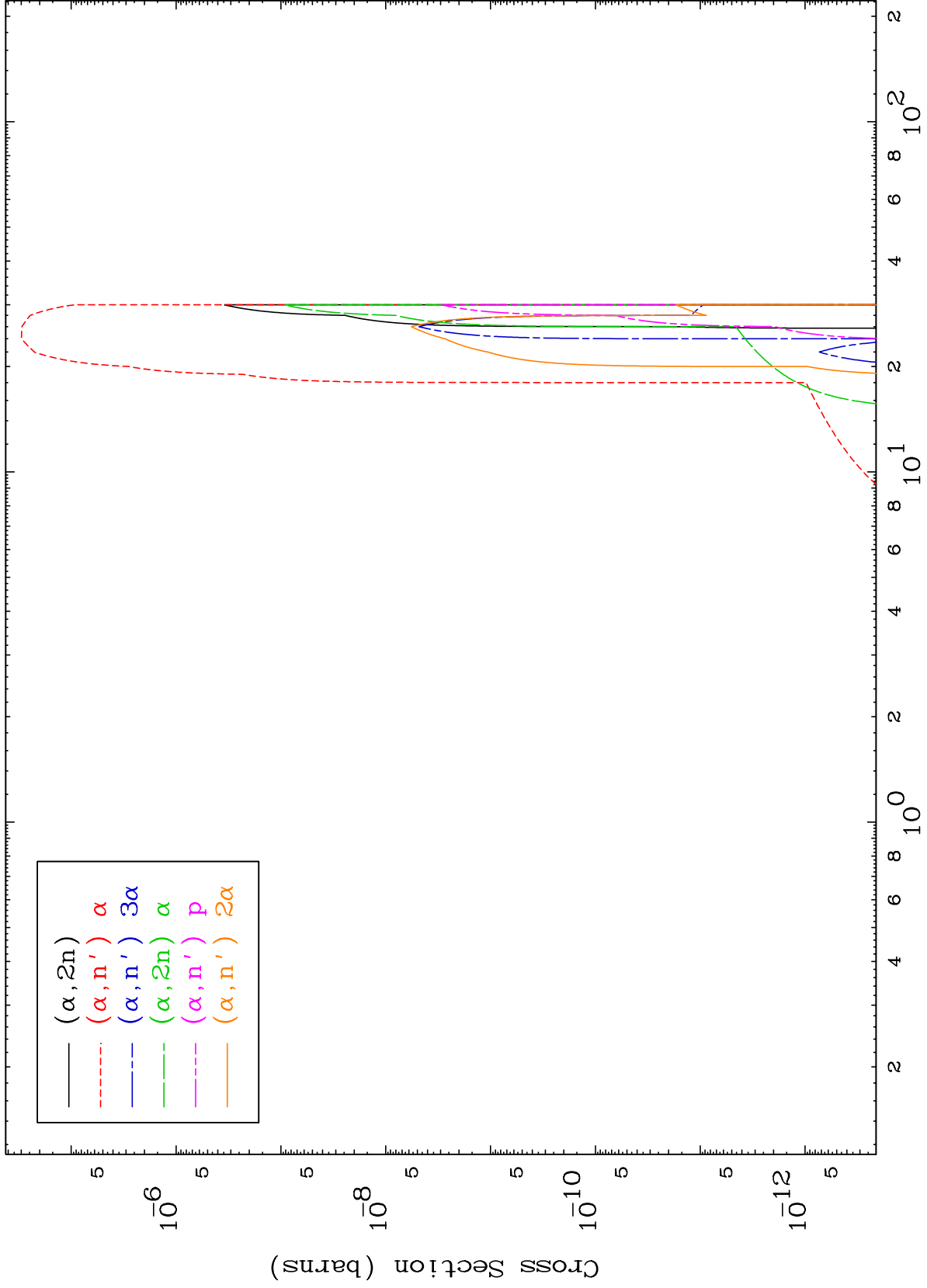
Incident Energy (MeV)

105-Db-258

MAT 558

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

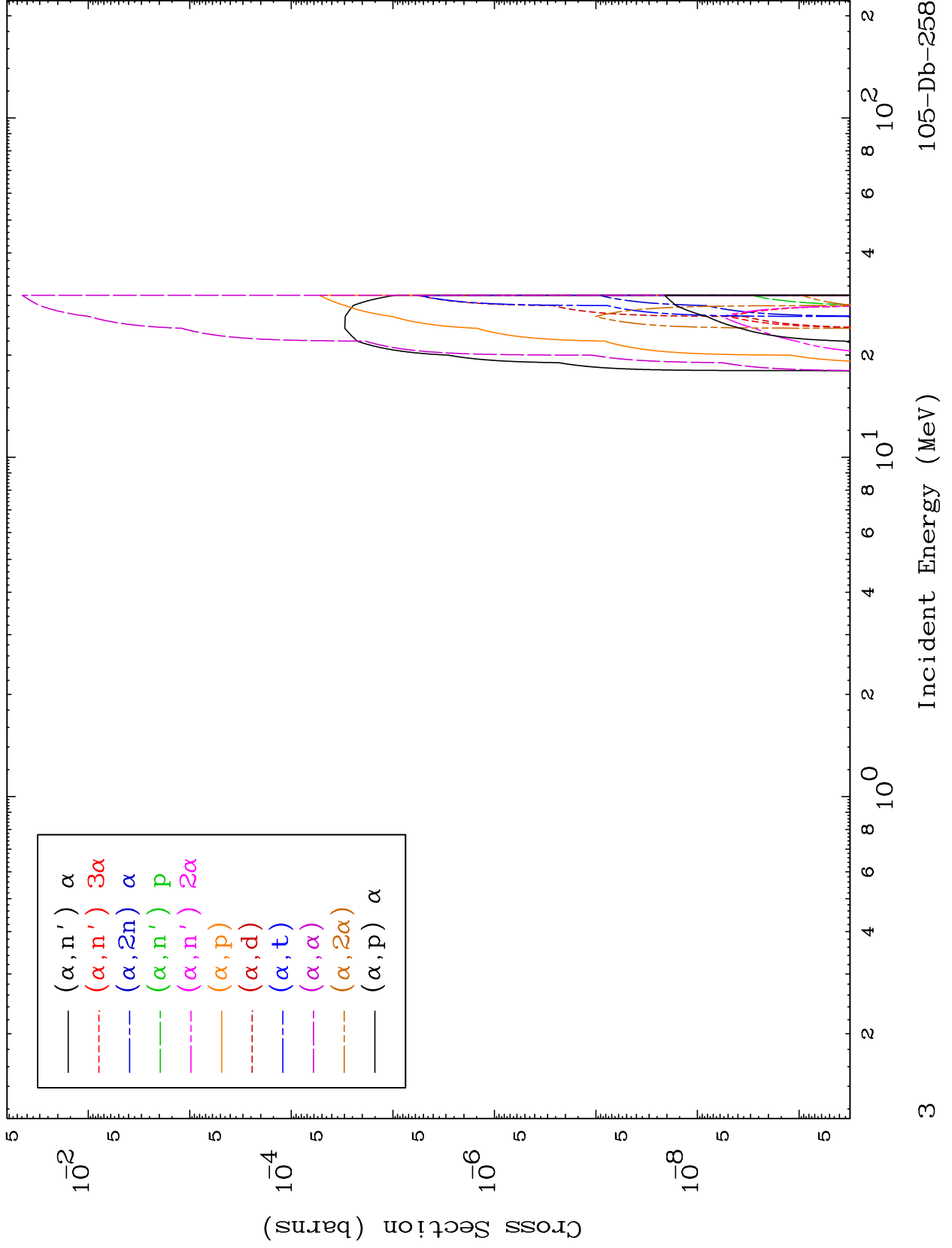
105-Db-258



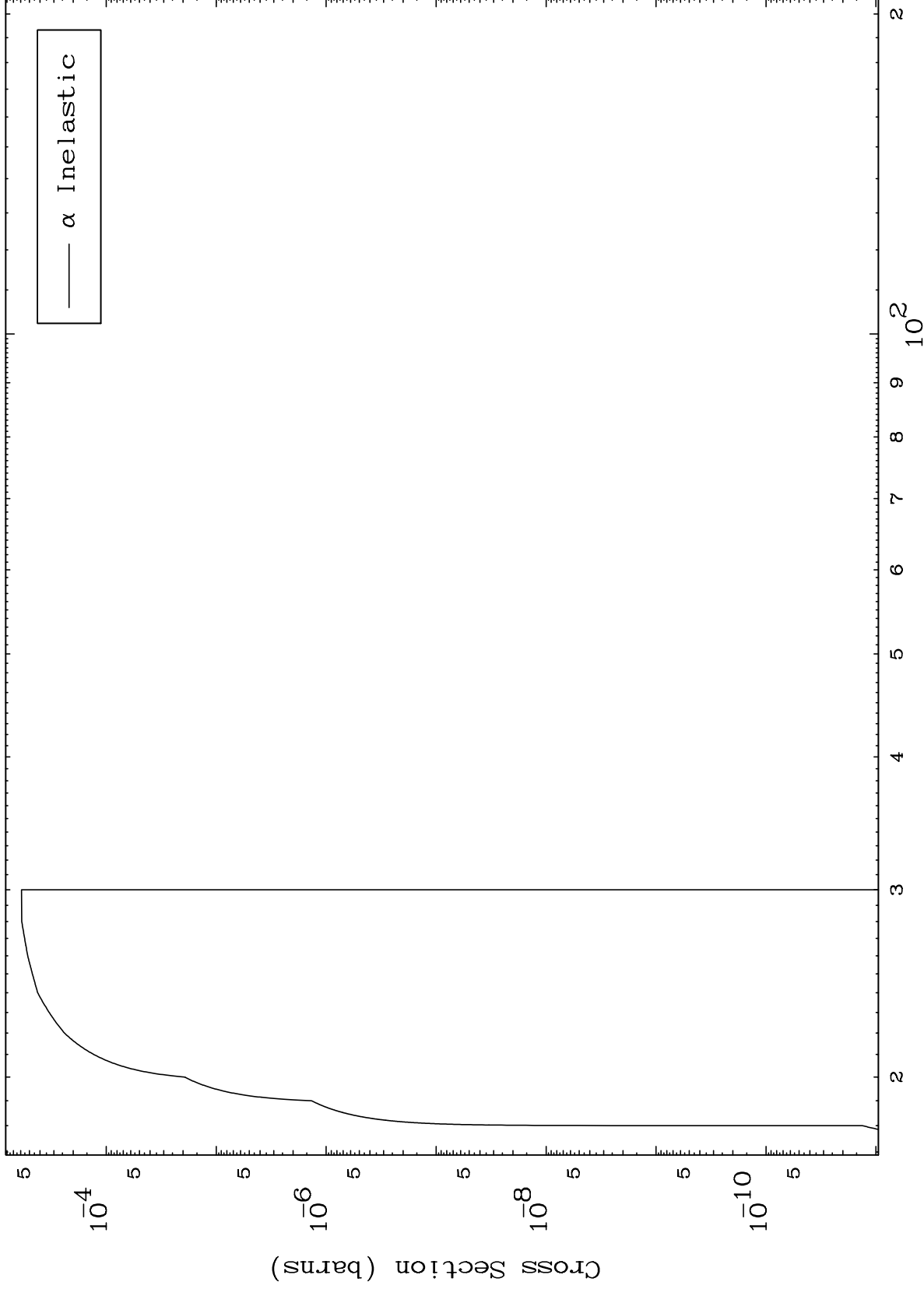
MAT 558

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

105-Db-258



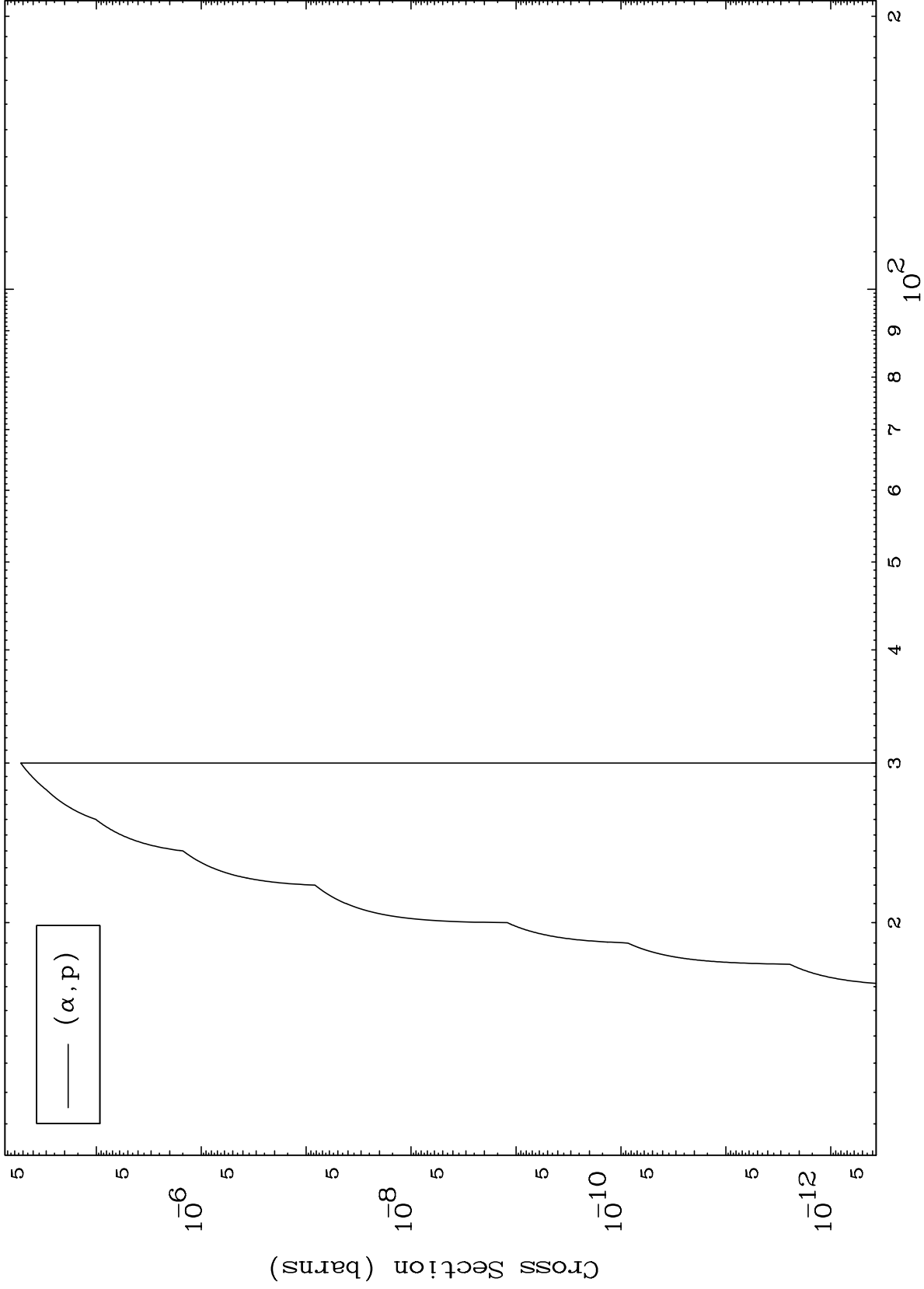
0 Kelvin Cross Sections



MAT 558

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

105-Db-258



5

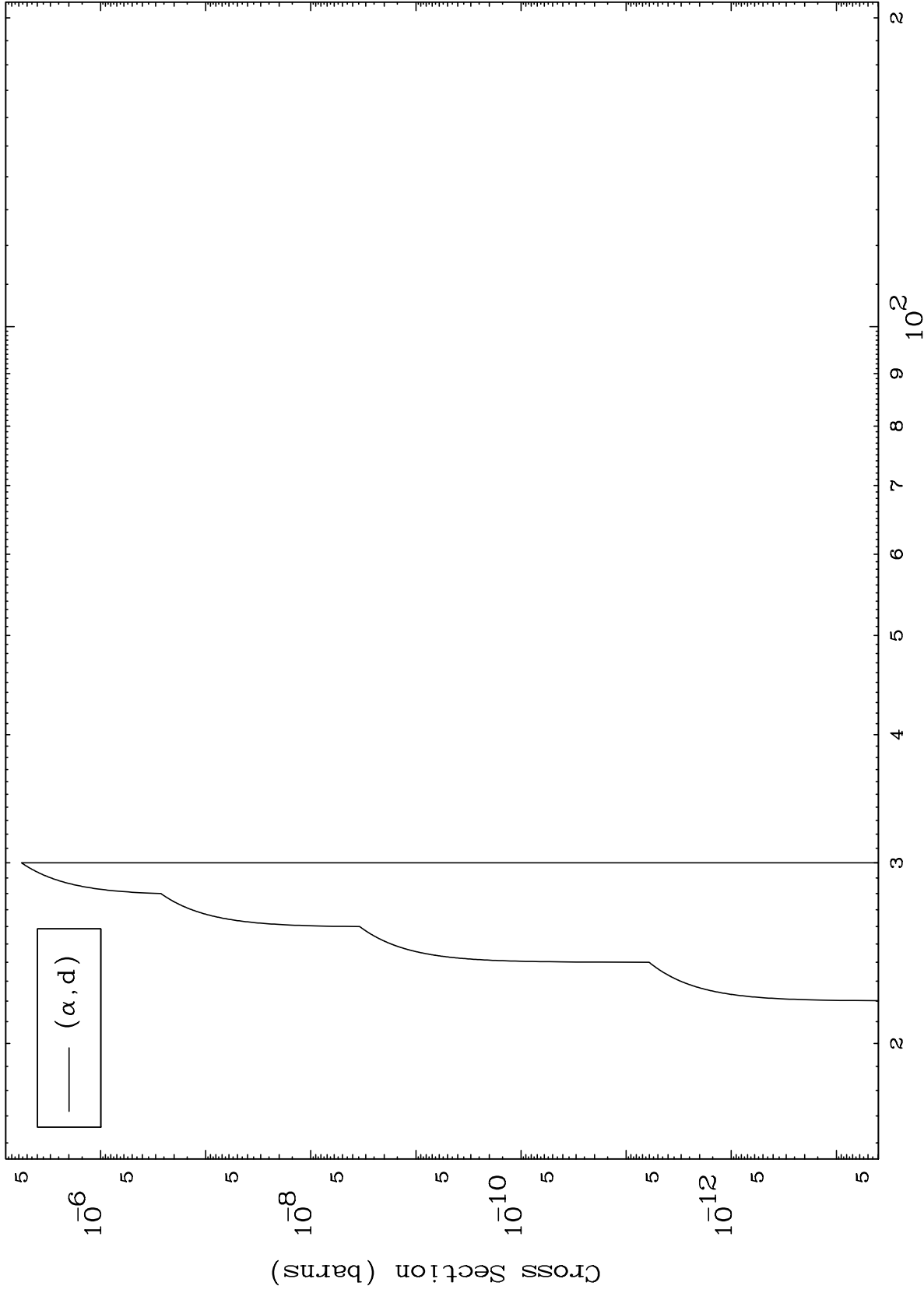
Incident Energy (MeV)

105-Db-258

MAT 558

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

105-Db-258



6

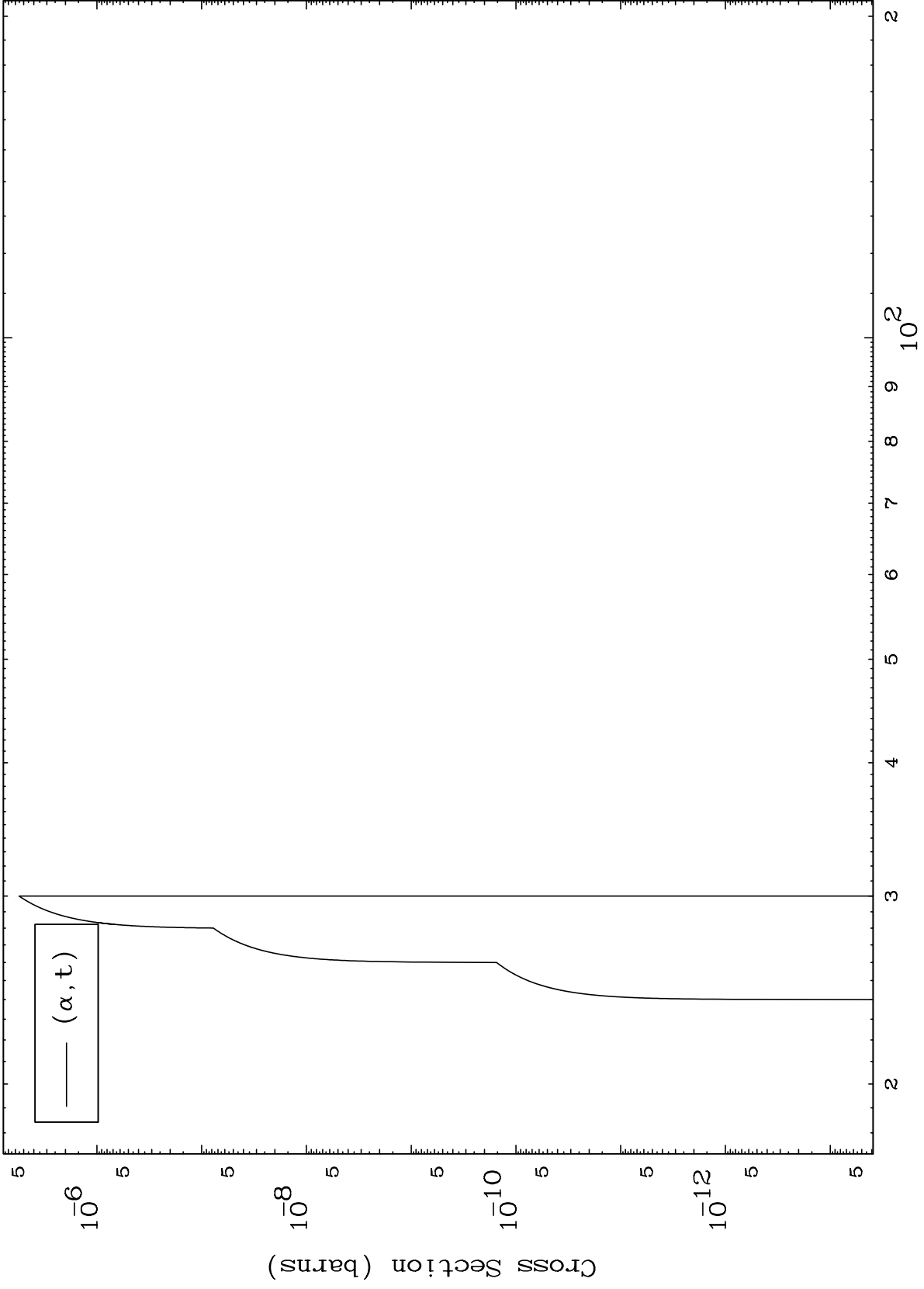
Incident Energy (MeV)

105-Db-258

MAT 558

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

105-Db-258



7

Incident Energy (MeV)

105-Db-258

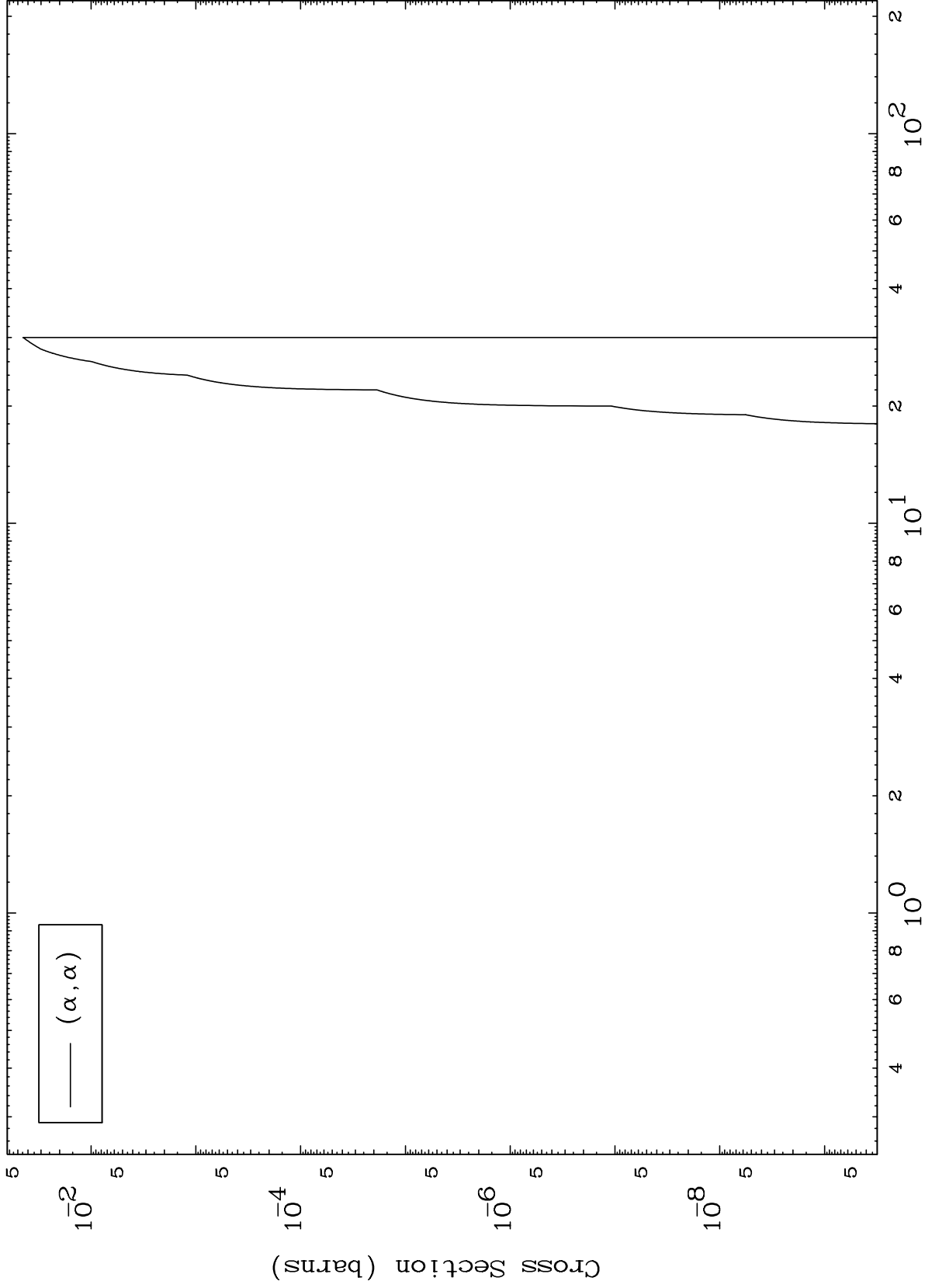


MAT 558

( $\alpha, \alpha$ ) Levels

105-Db-258

0 Kelvin Cross Sections



8

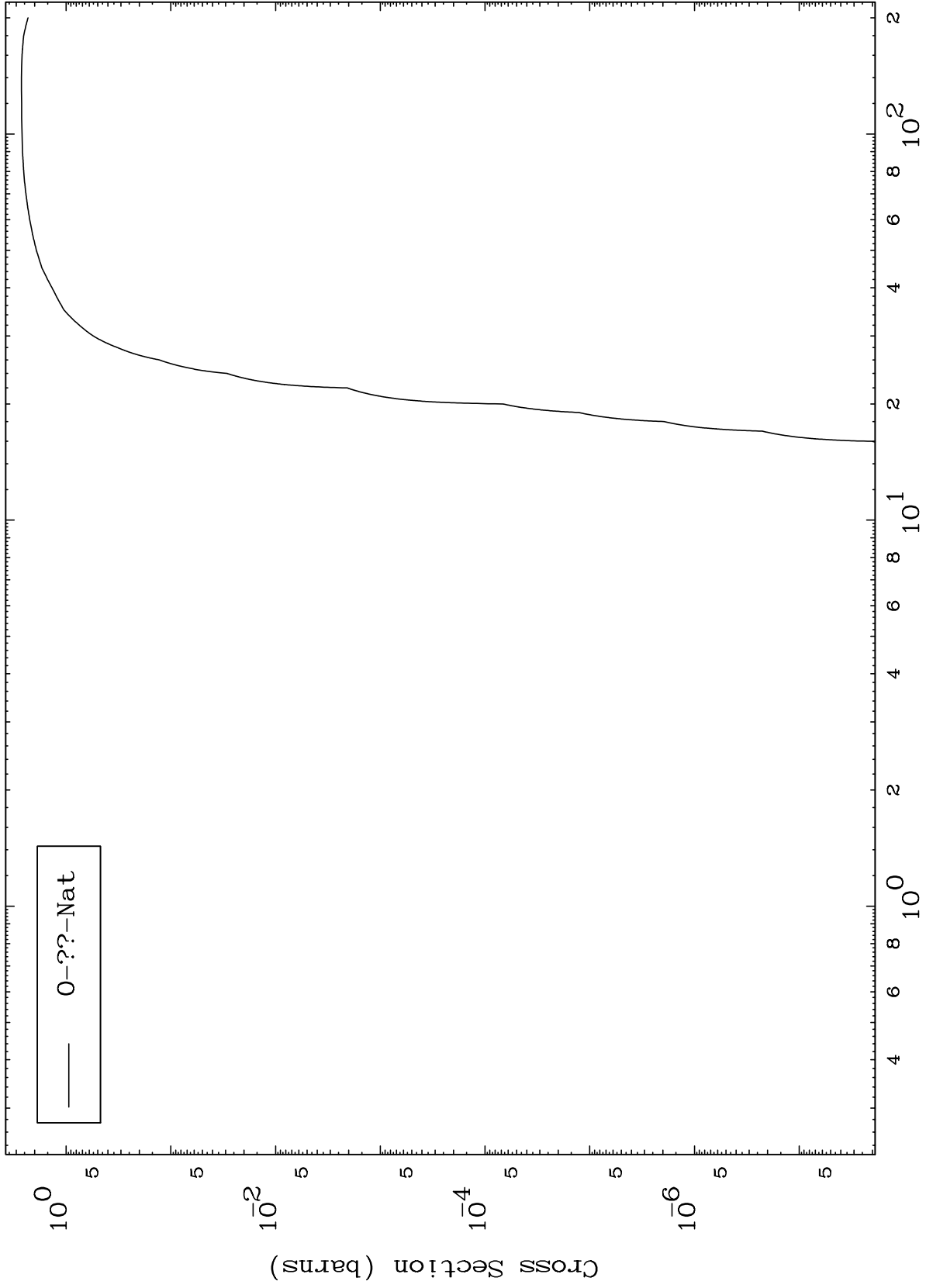
Incident Energy (MeV)

105-Db-258

MAT 558

105-Db-258

$\alpha$  Fission  
Radionuclide Production Cross Section

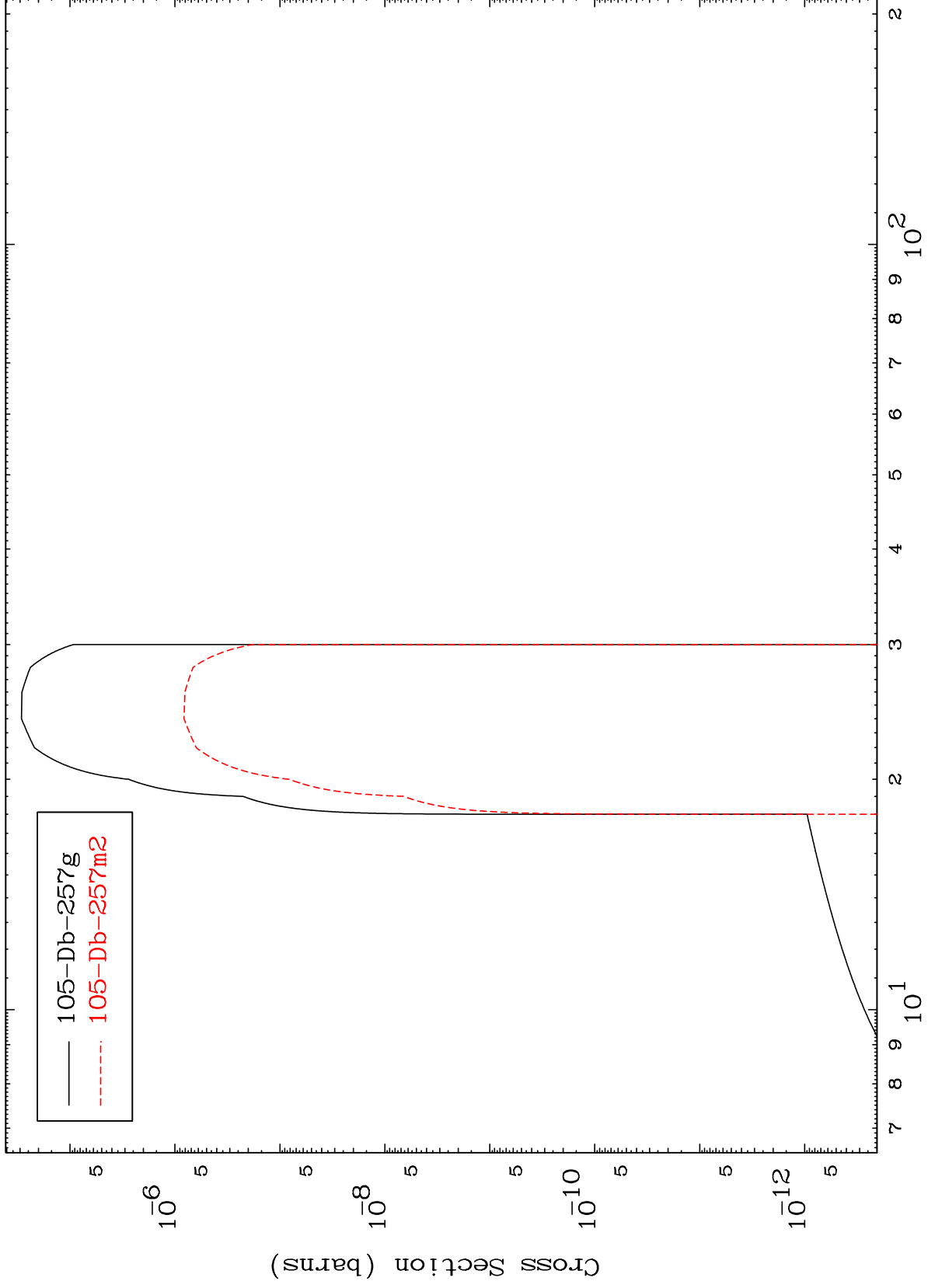


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$(\alpha, n')$   $\alpha$

105-Db-258

Radionuclide Production Cross Section



10

Incident Energy (MeV)

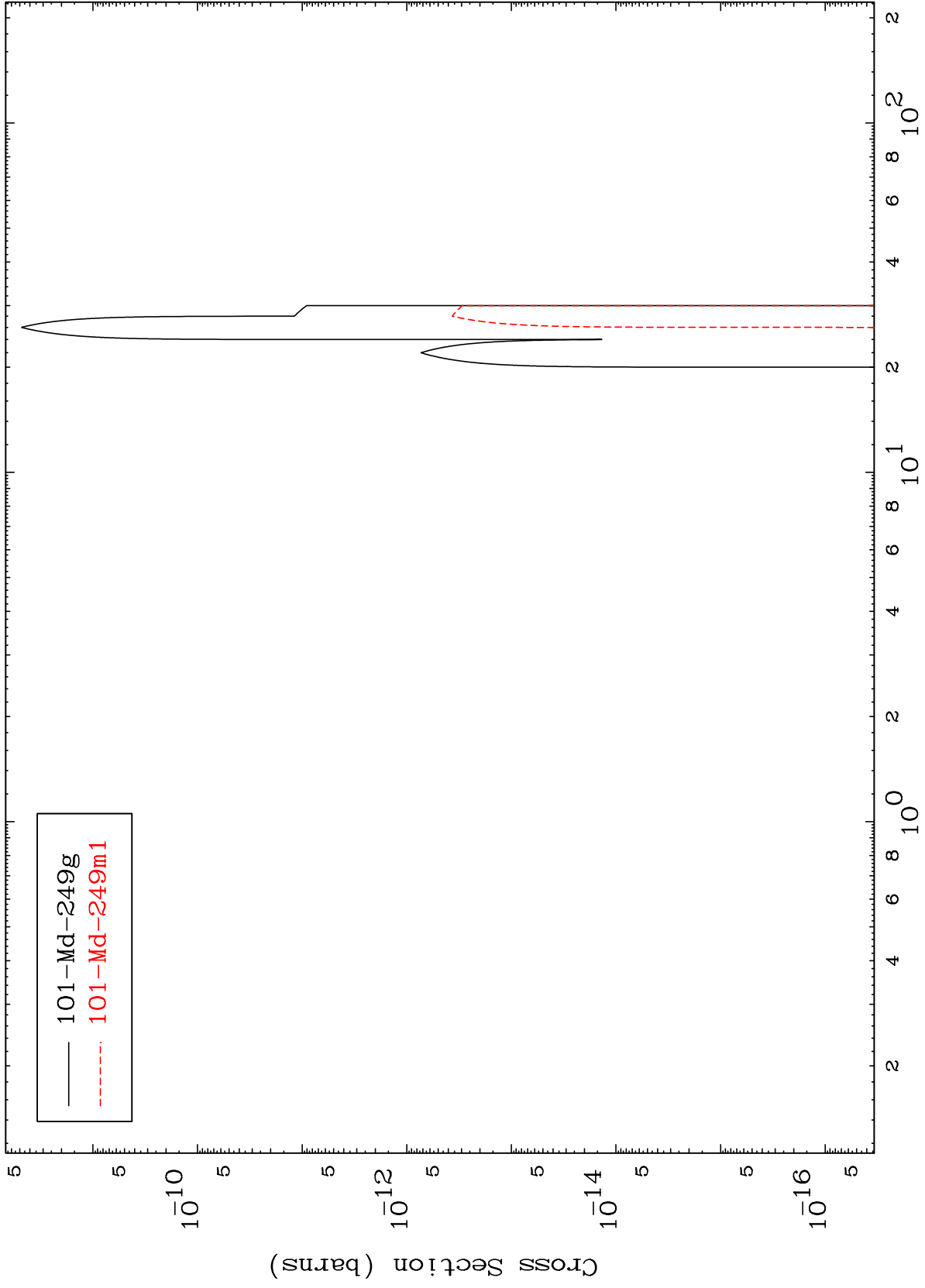
105-Db-258

MAT 558

( $\alpha, n'$ )  $3\alpha$

105-Db-258

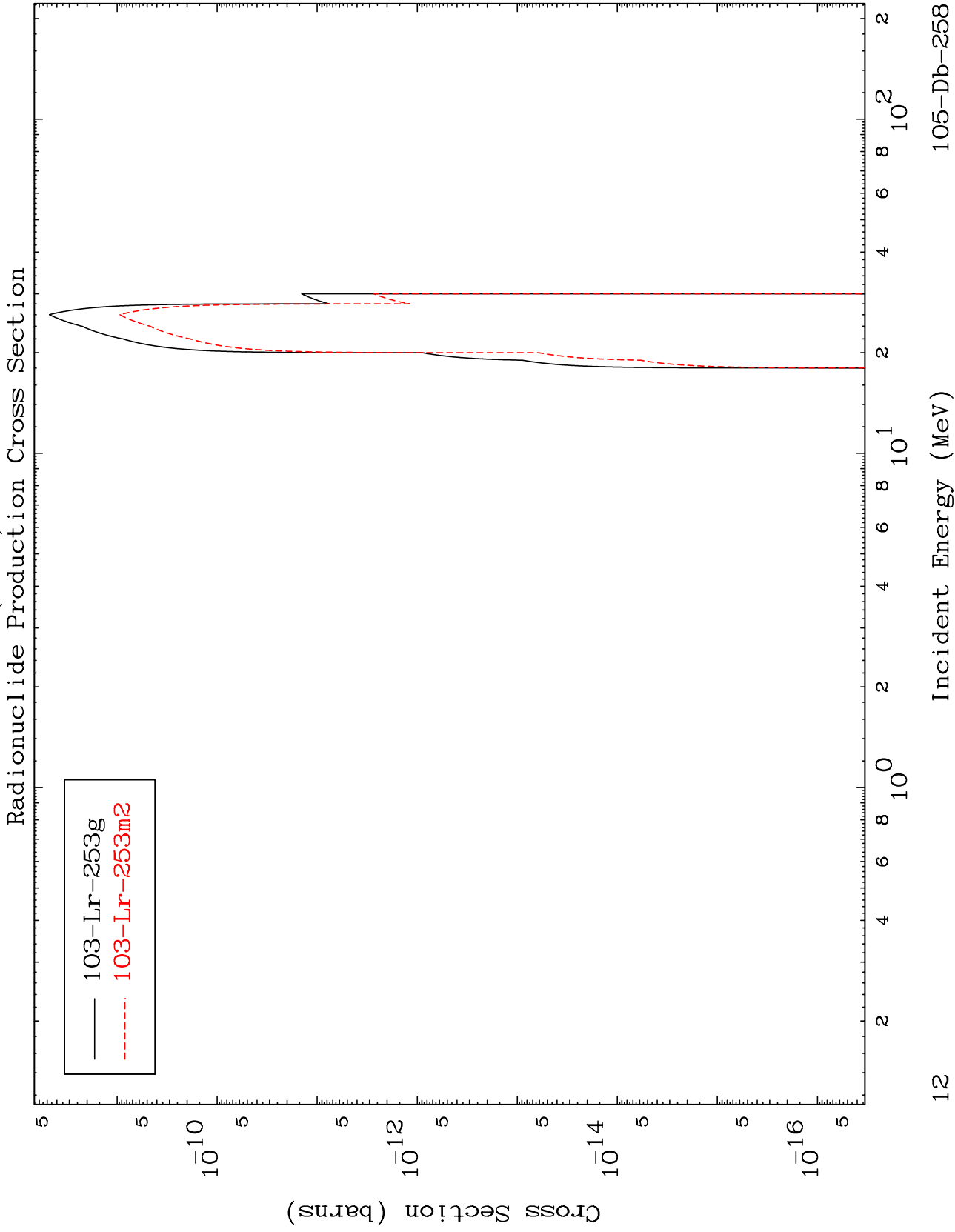
Radionuclide Production Cross Section



MAT 558

( $\alpha, n'$ )  $2\alpha$

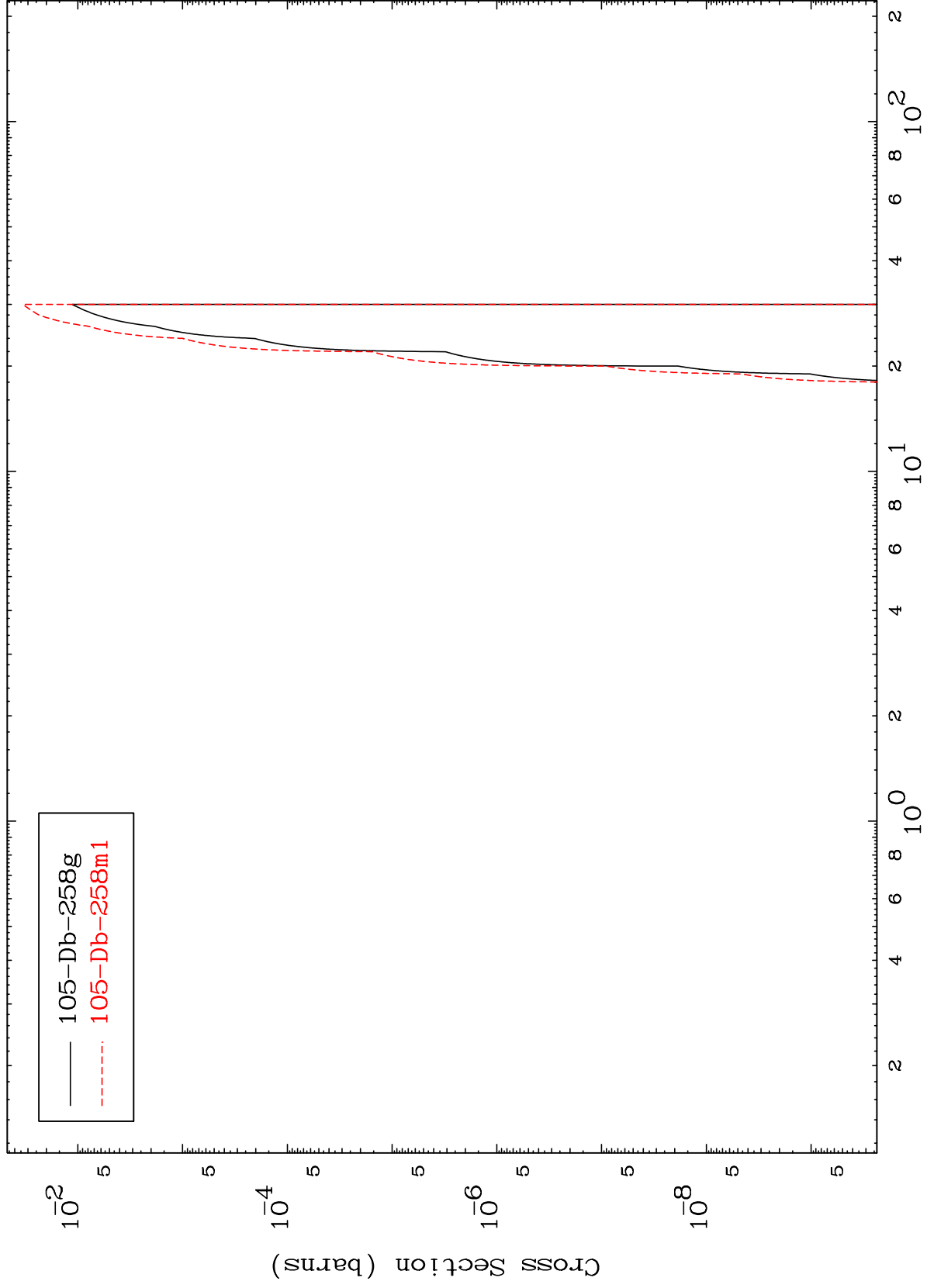
105-Db-258



MAT 558

105-Db-258

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



105-Db-258

Incident Energy (MeV)

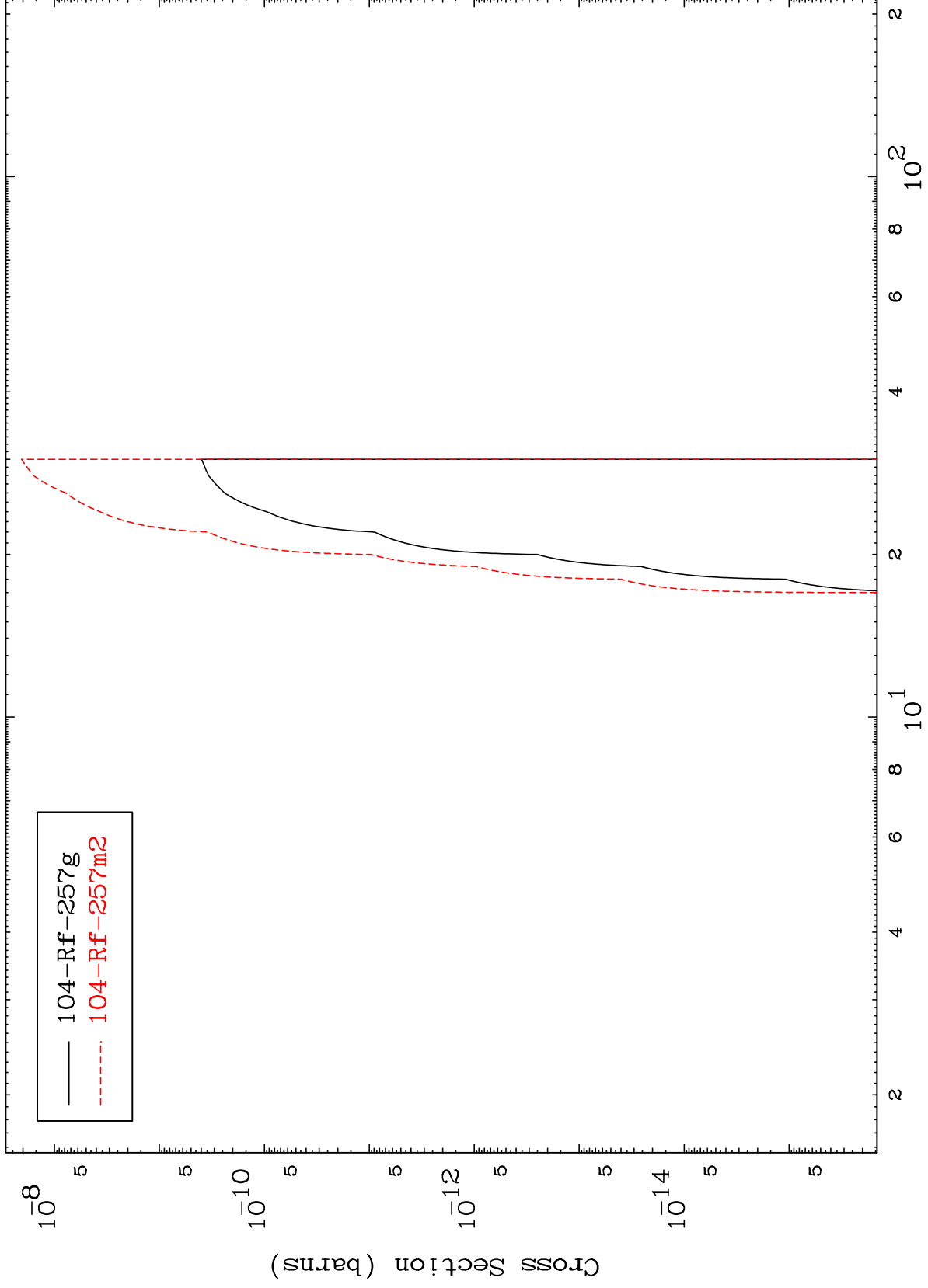
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( $\alpha, p$ )  $\alpha$

105-Db-258

Radionuclide Production Cross Section



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Incident Energy (MeV)

105-Db-258