

Program EVALPLOT  
(Version 2018-1)

by

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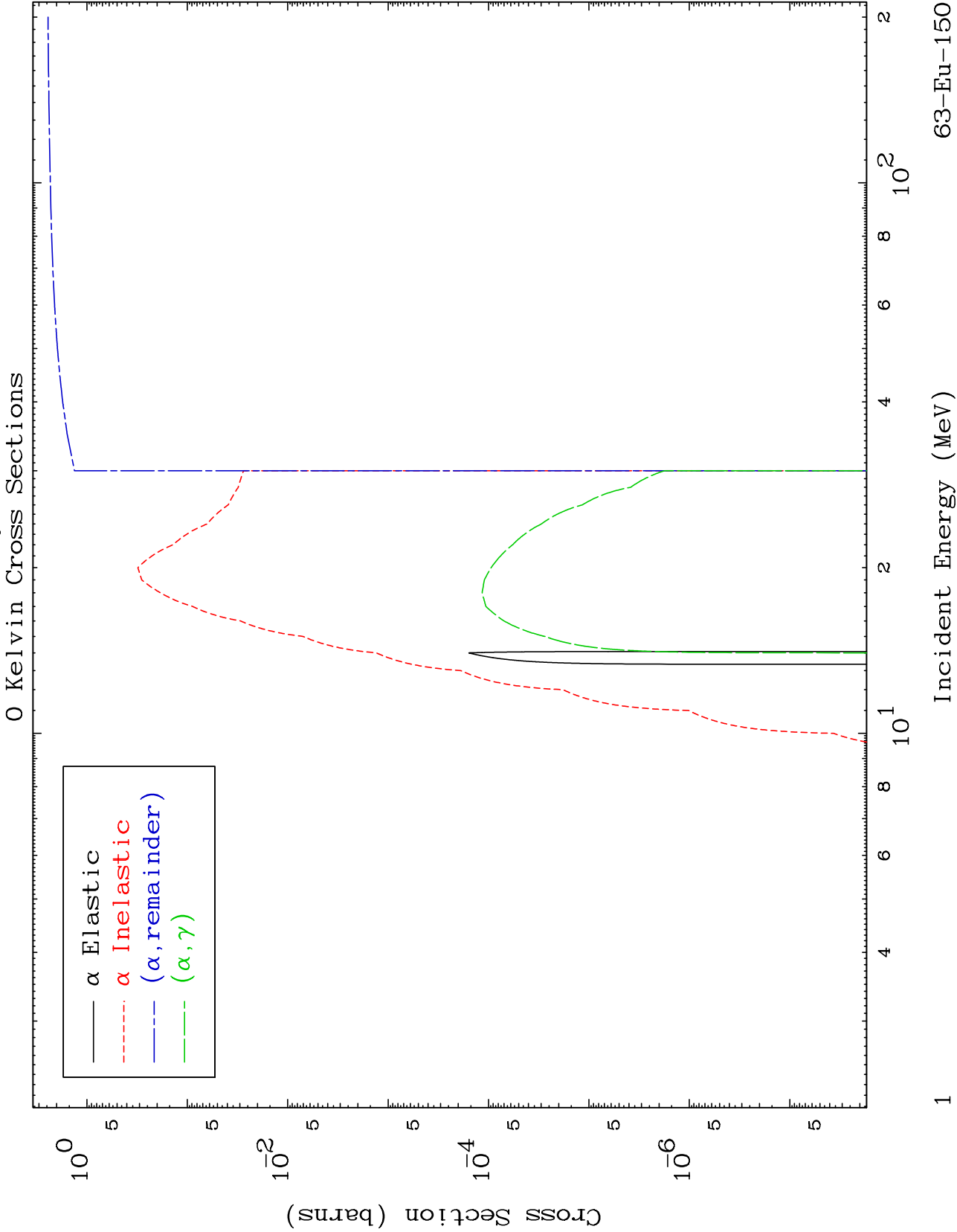
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6323

$\alpha$  Major

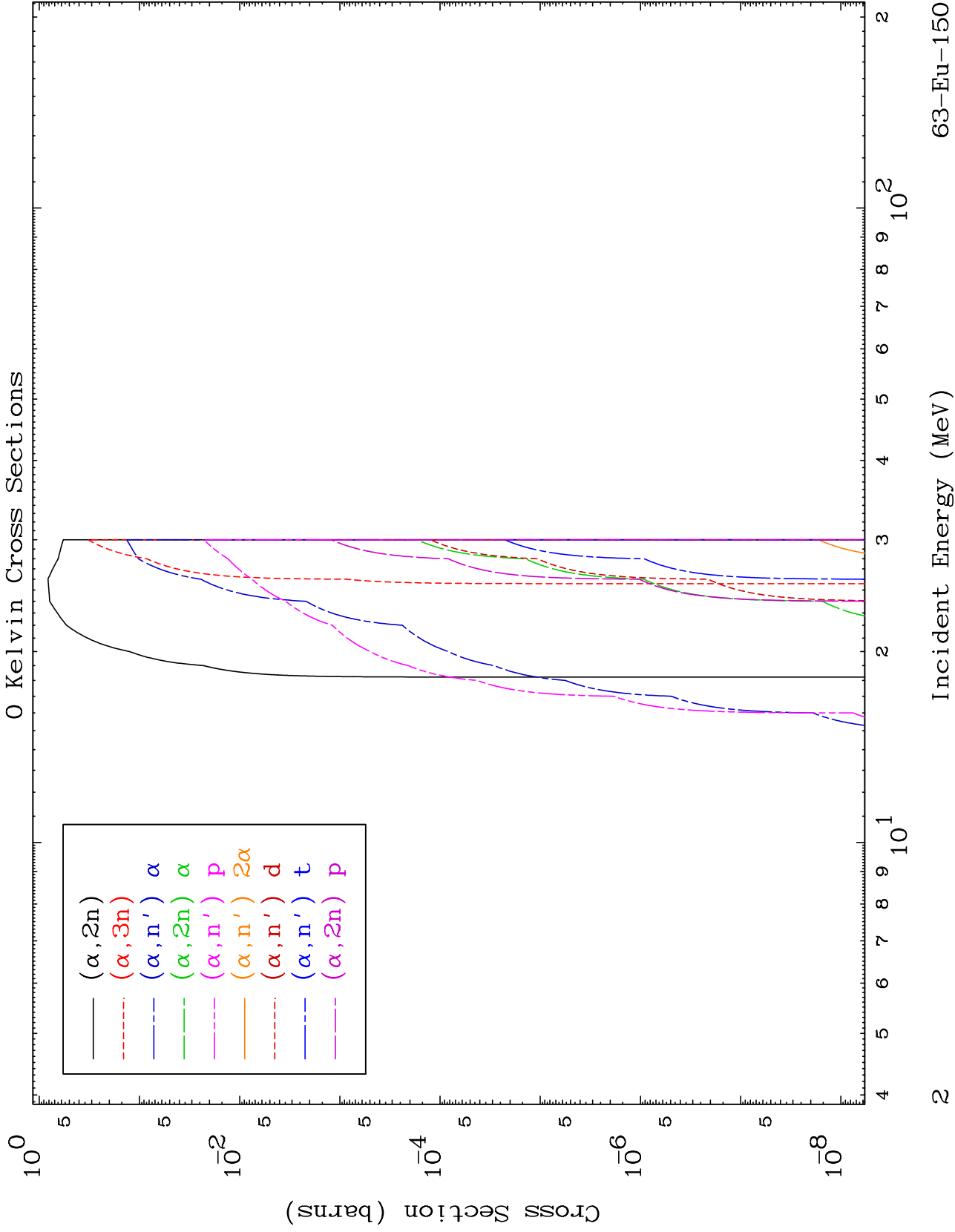
63-Eu-150



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$\alpha$  Neutron Production

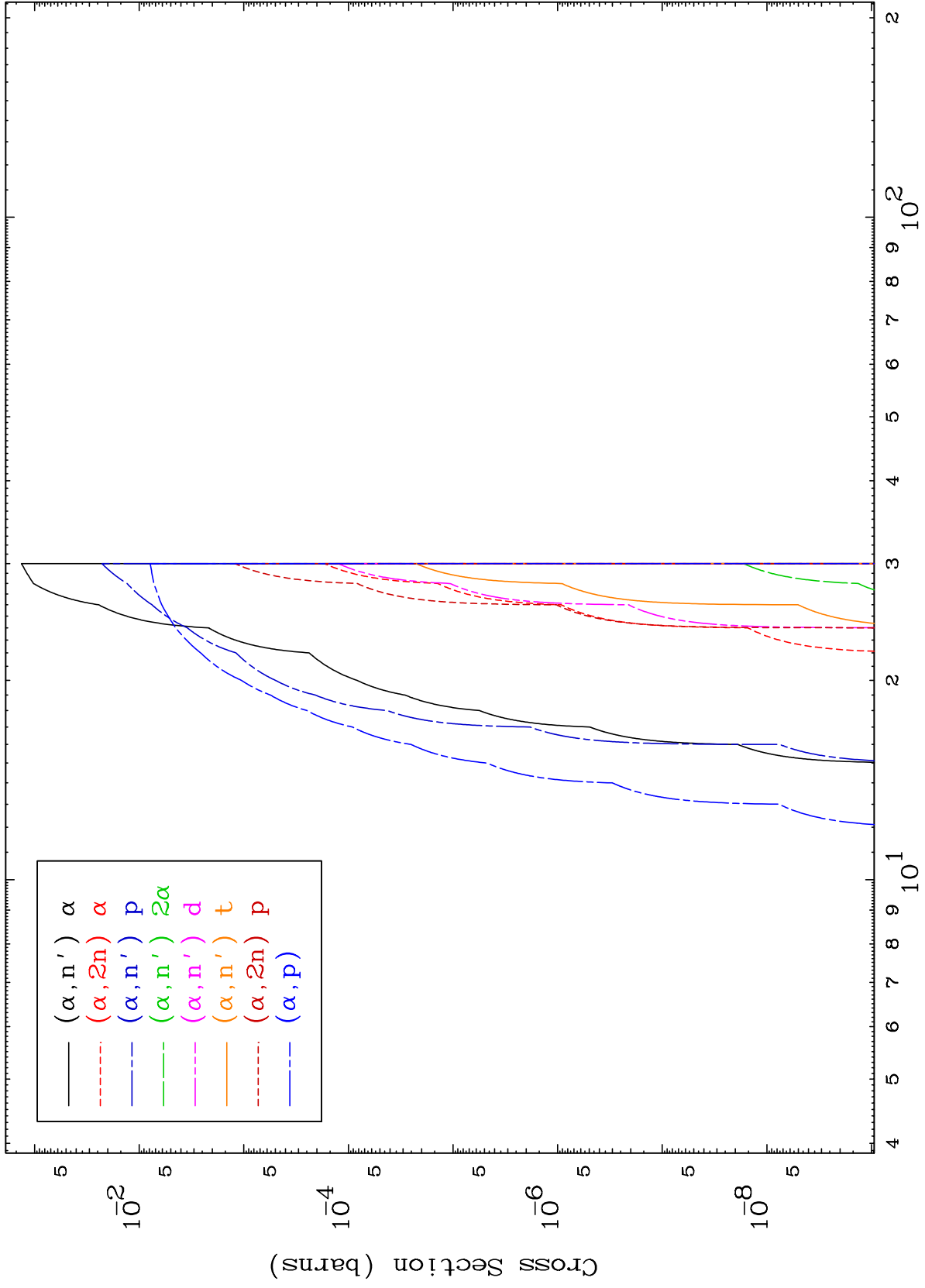
63-Eu-150



2

Incident Energy (MeV)

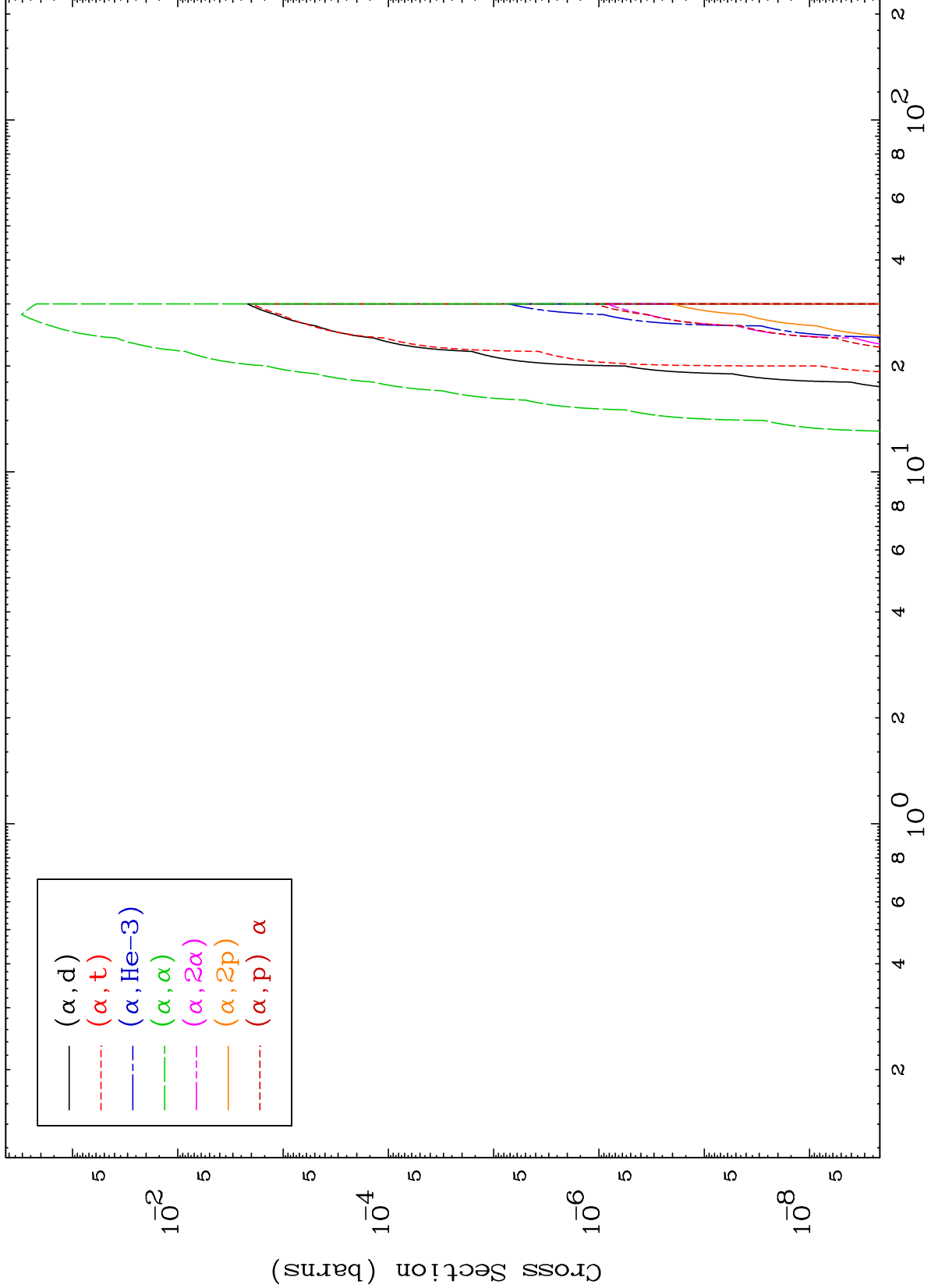
63-Eu-150



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$\alpha$  Charged Particle  
0 Kelvin Cross Sections

63-Eu-150

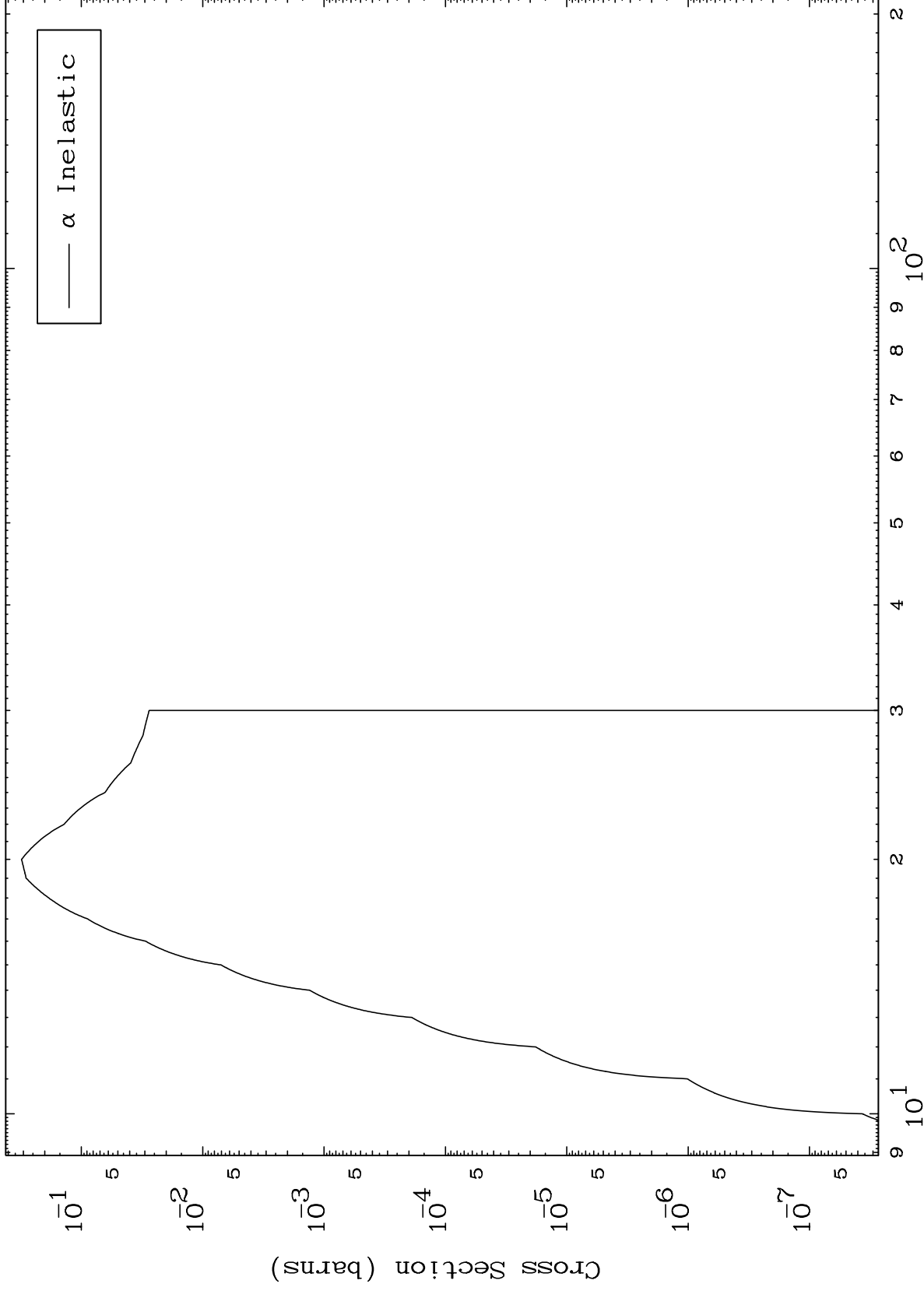


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( $\alpha, n'$ ) Level

63-Eu-150

0 Kelvin Cross Sections



Incident Energy (MeV)

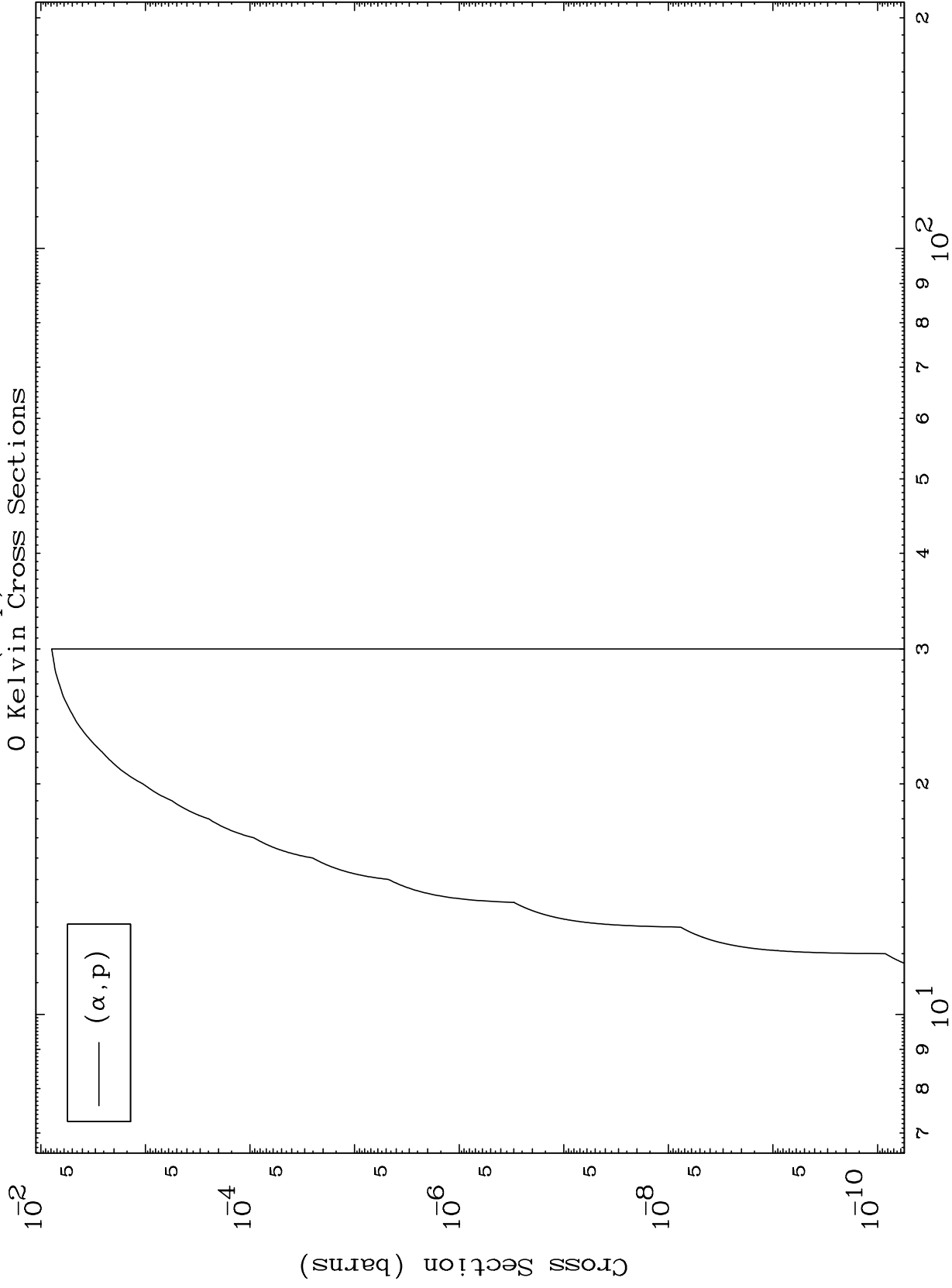
63-Eu-150

5

MAT 6323

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

63-Eu-150



6

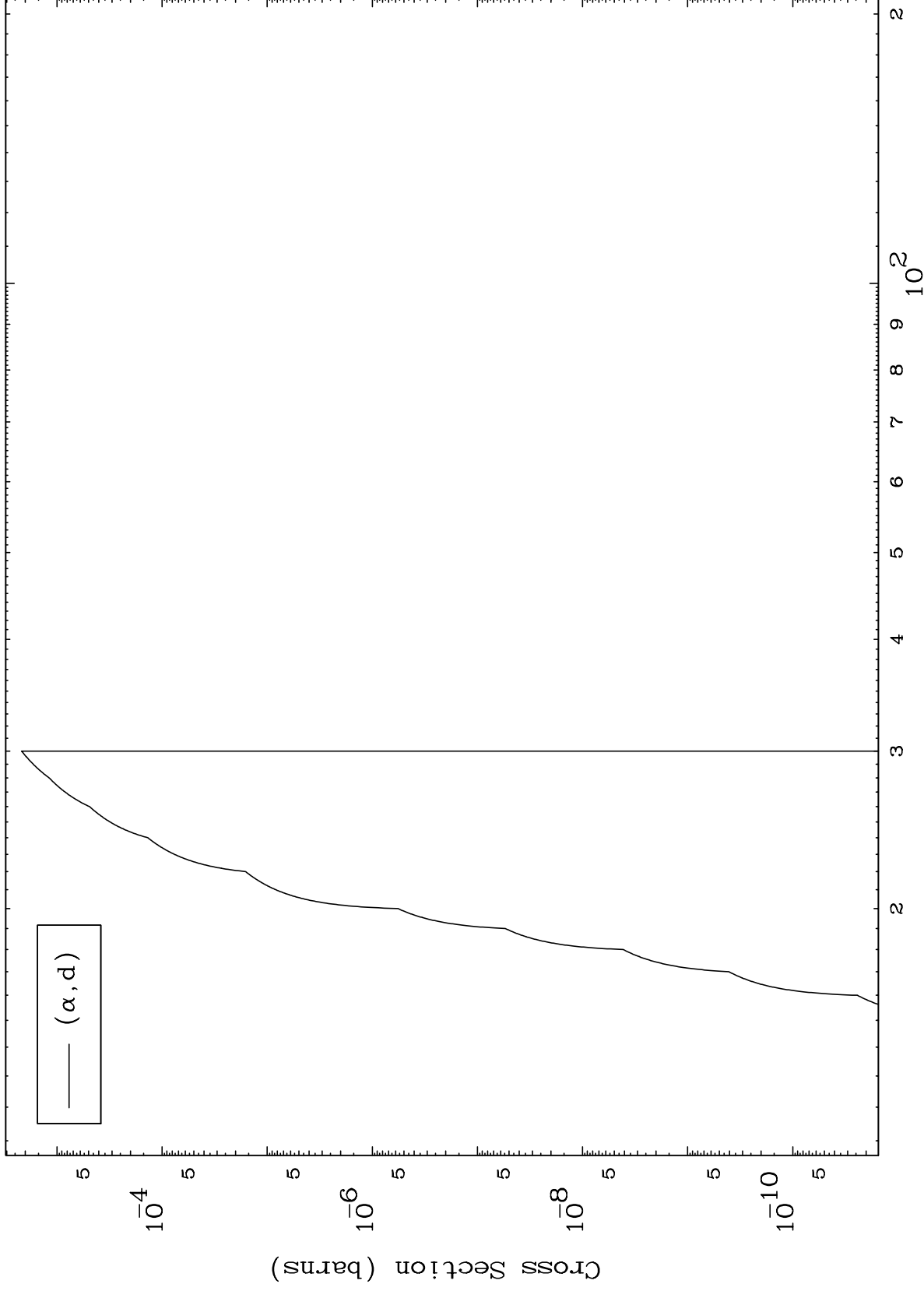
Incident Energy (MeV)

63-Eu-150

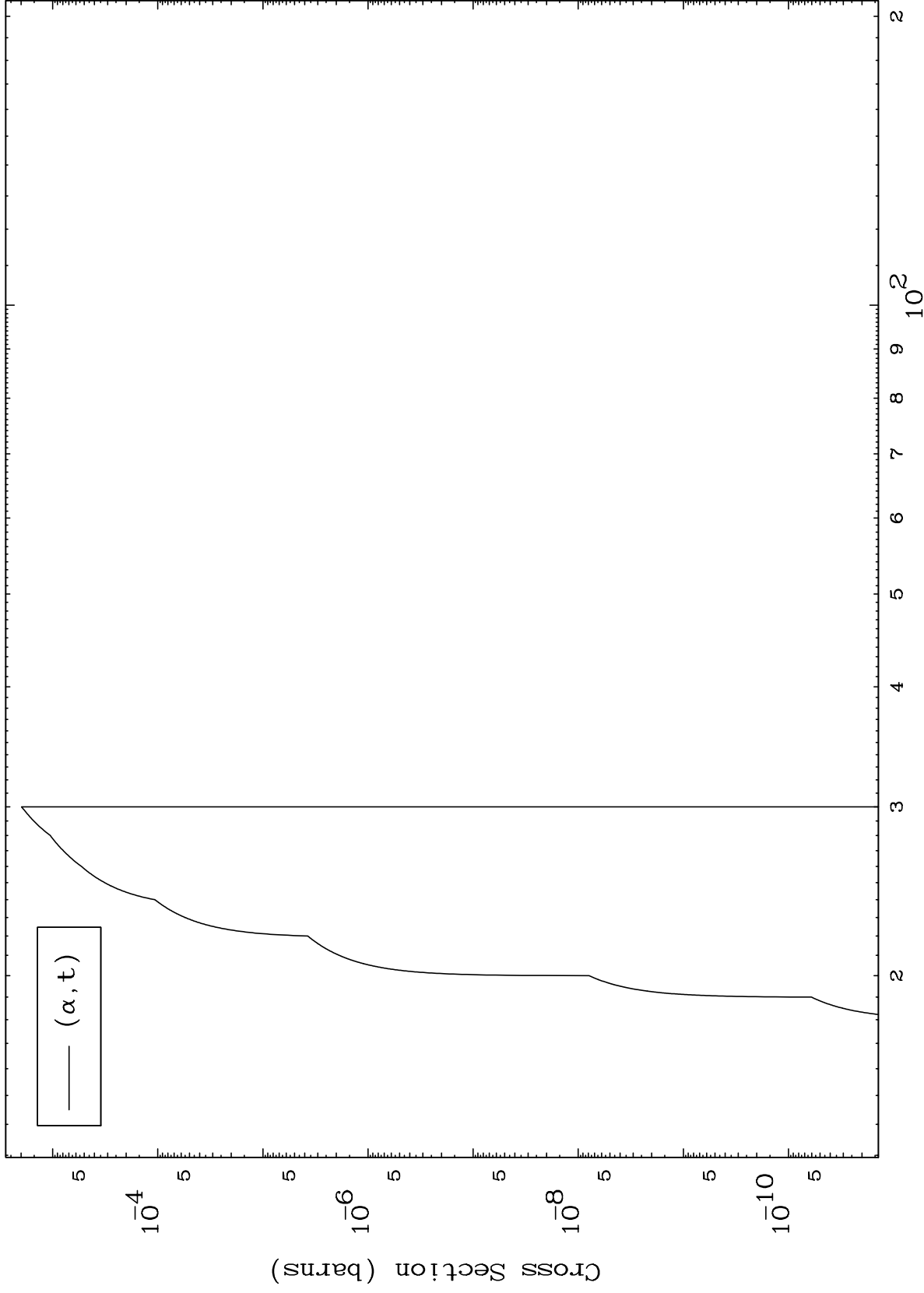
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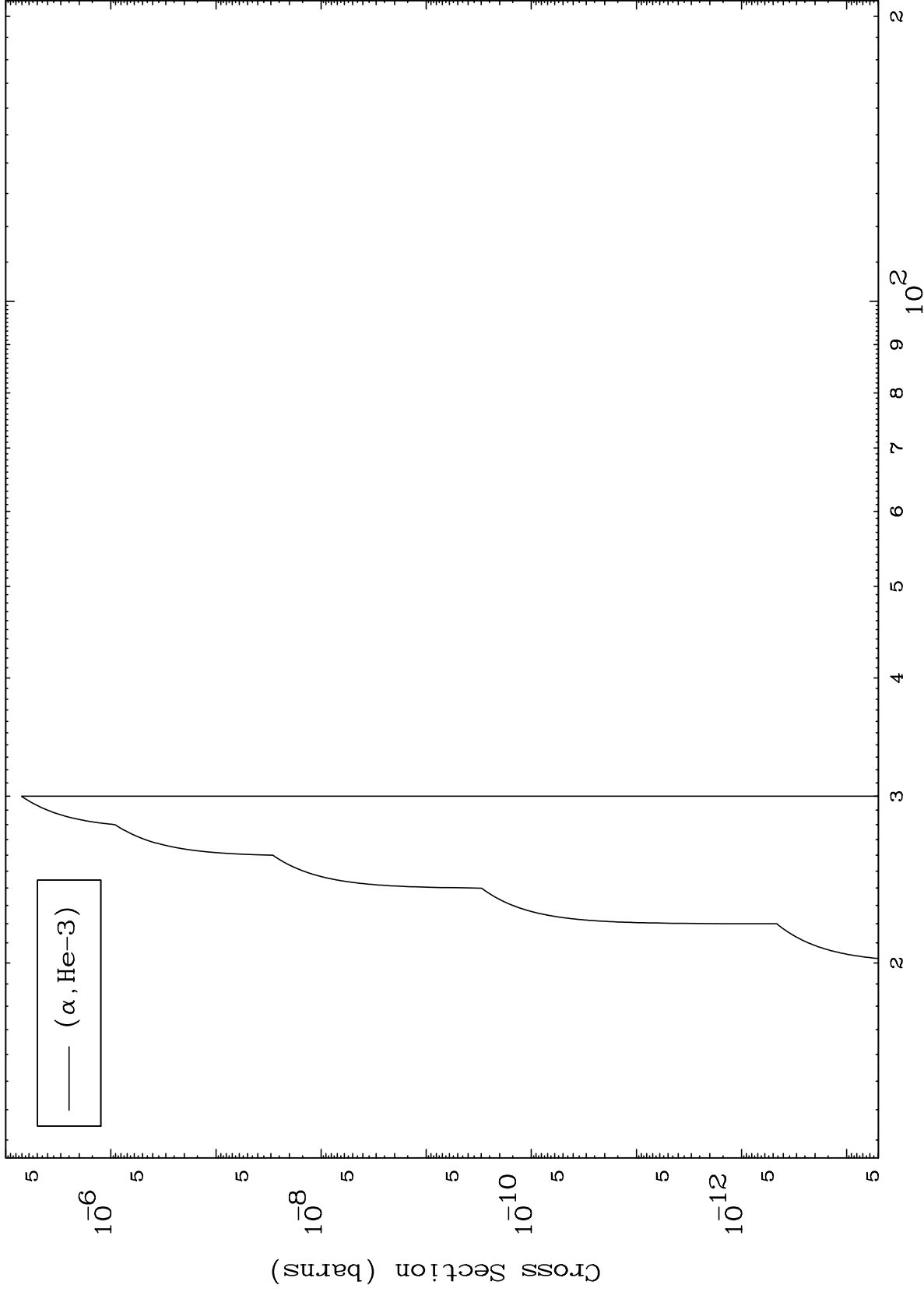
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

63-Eu-150







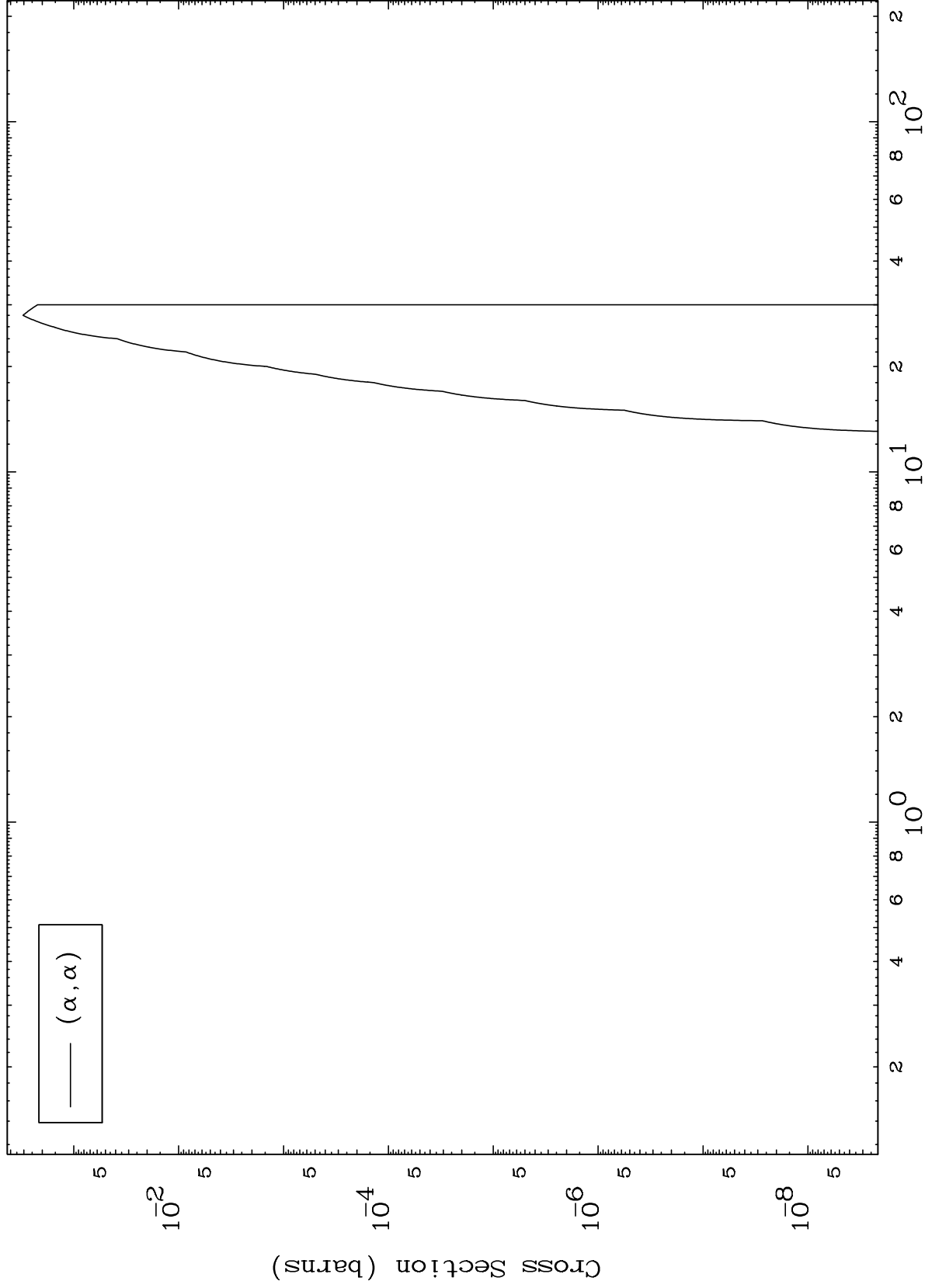


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( $\alpha, \alpha$ ) Levels

63-Eu-150

0 Kelvin Cross Sections



10

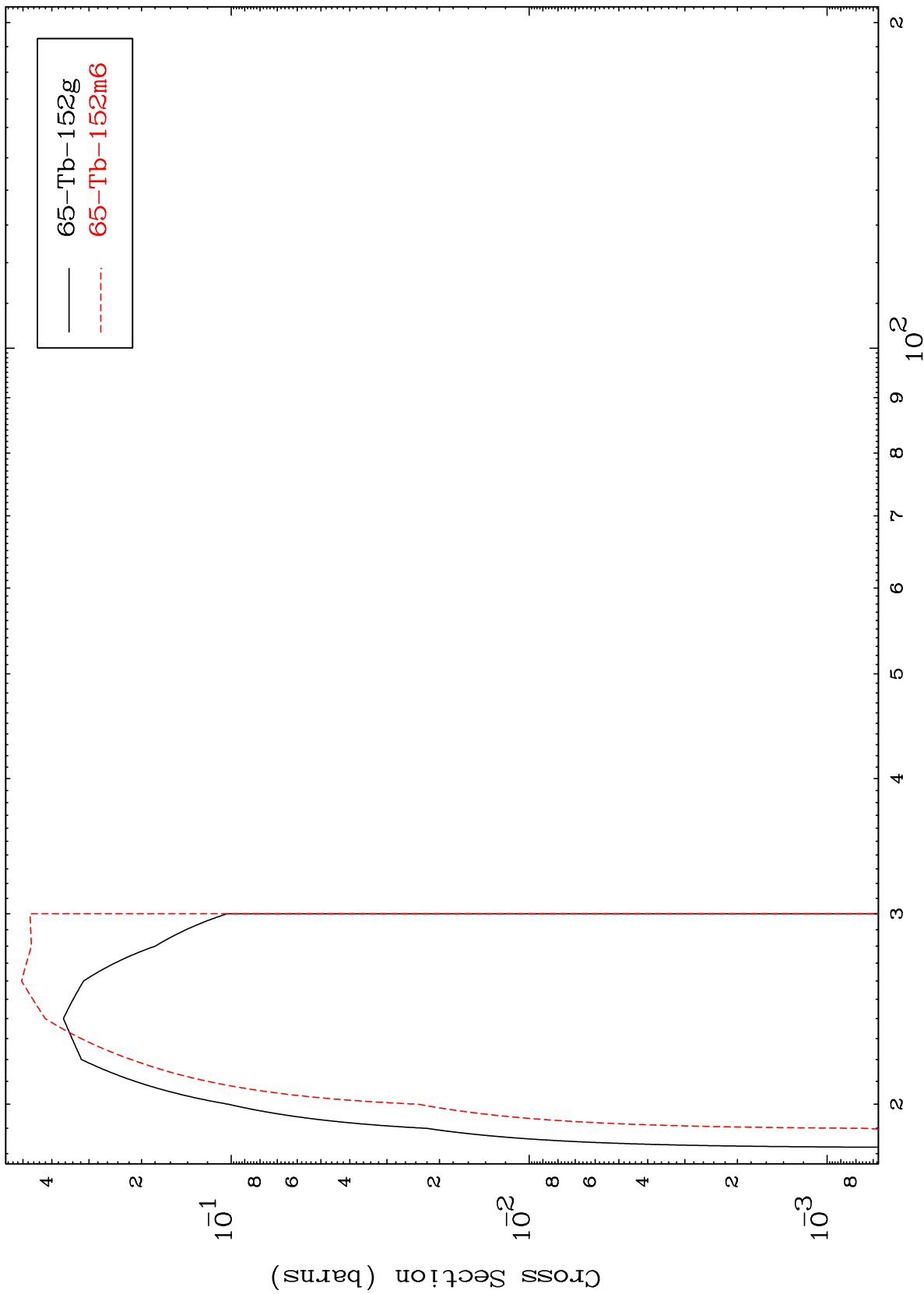
Incident Energy (MeV)

63-Eu-150

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63-Eu-150

$(\alpha, 2n)$   
Radionuclide Production Cross Section



63-Eu-150

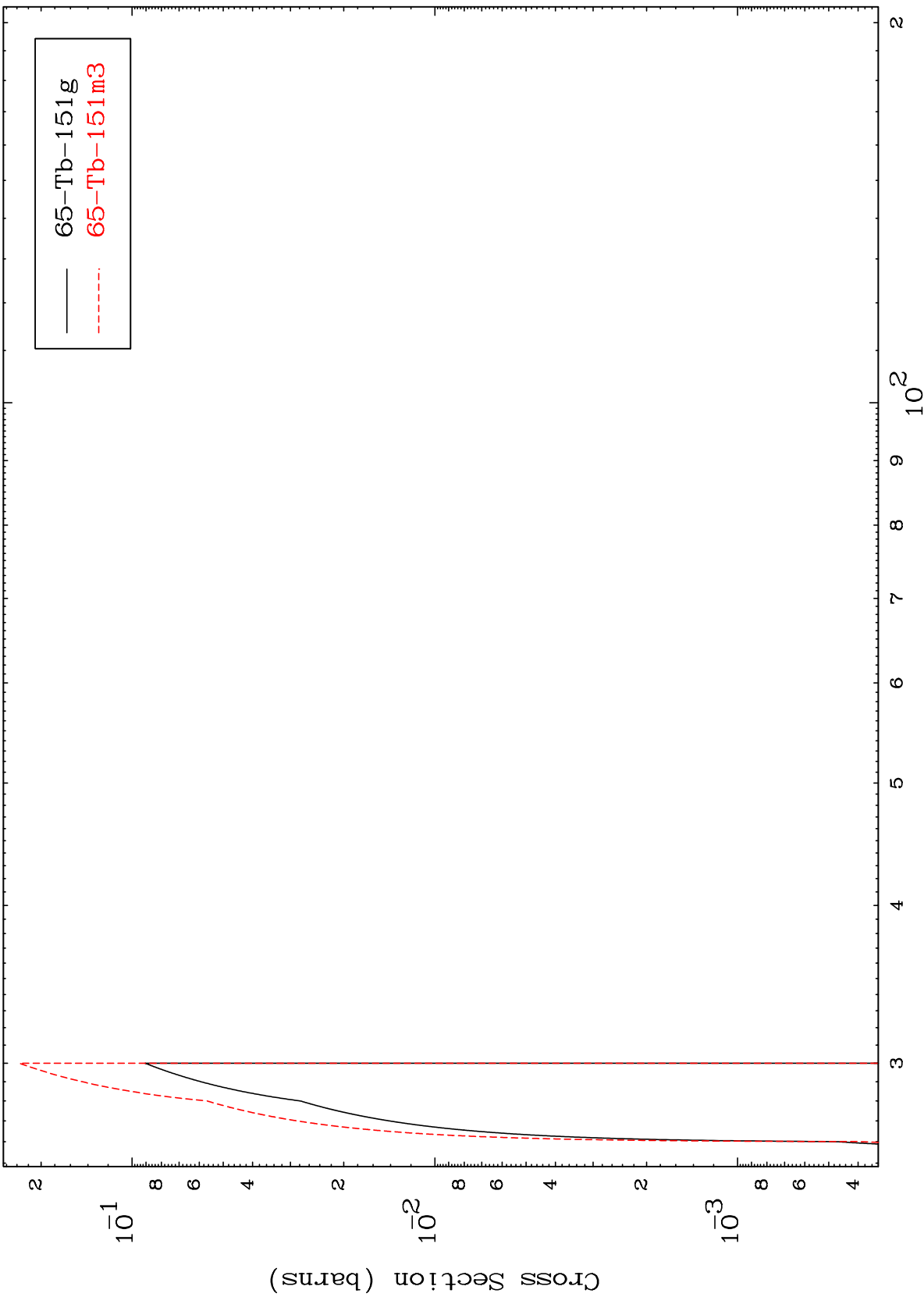
Incident Energy (MeV)

11

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63-Eu-150

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



63-Eu-150

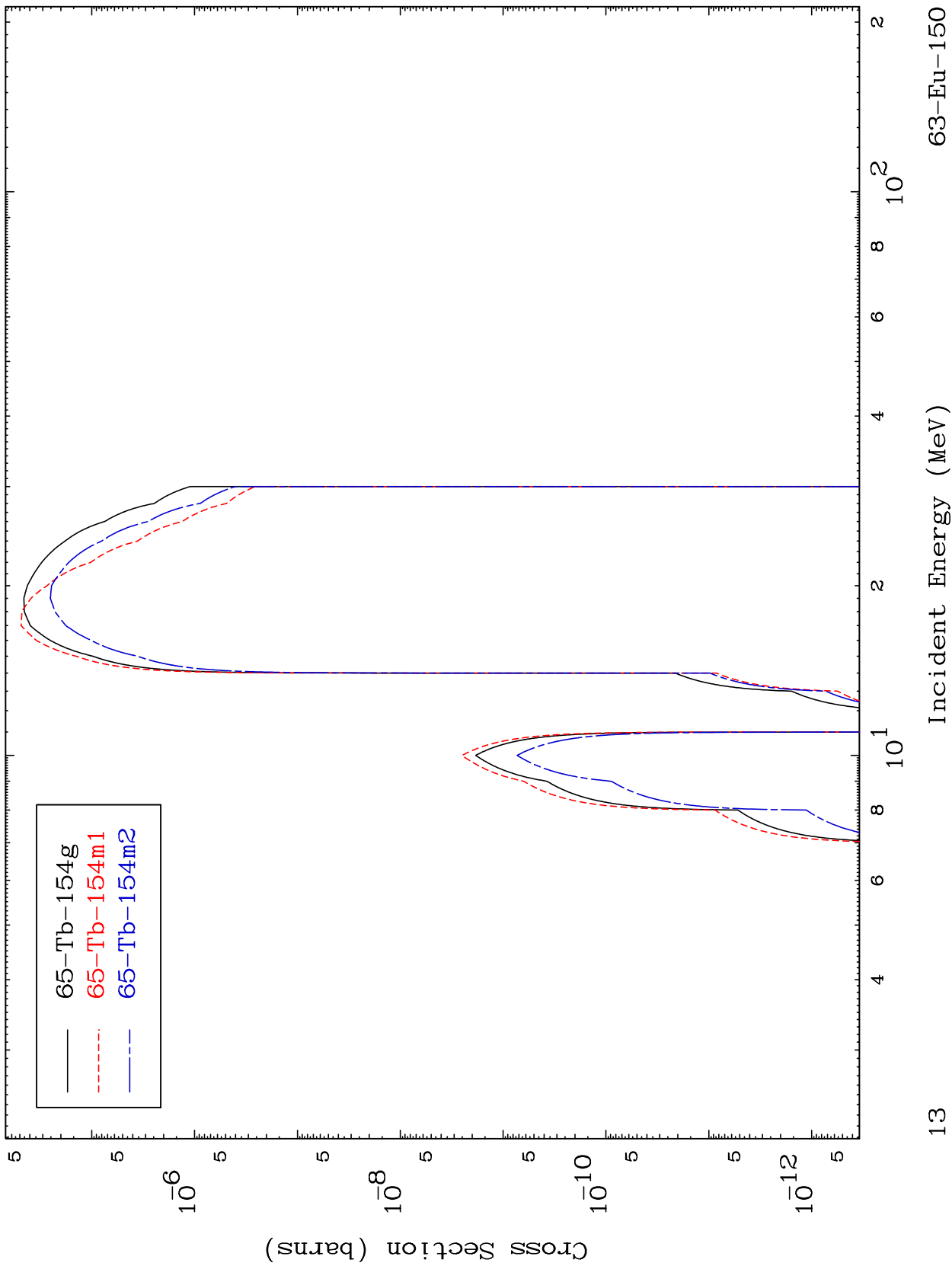
Incident Energy (MeV)

12

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63-Eu-150

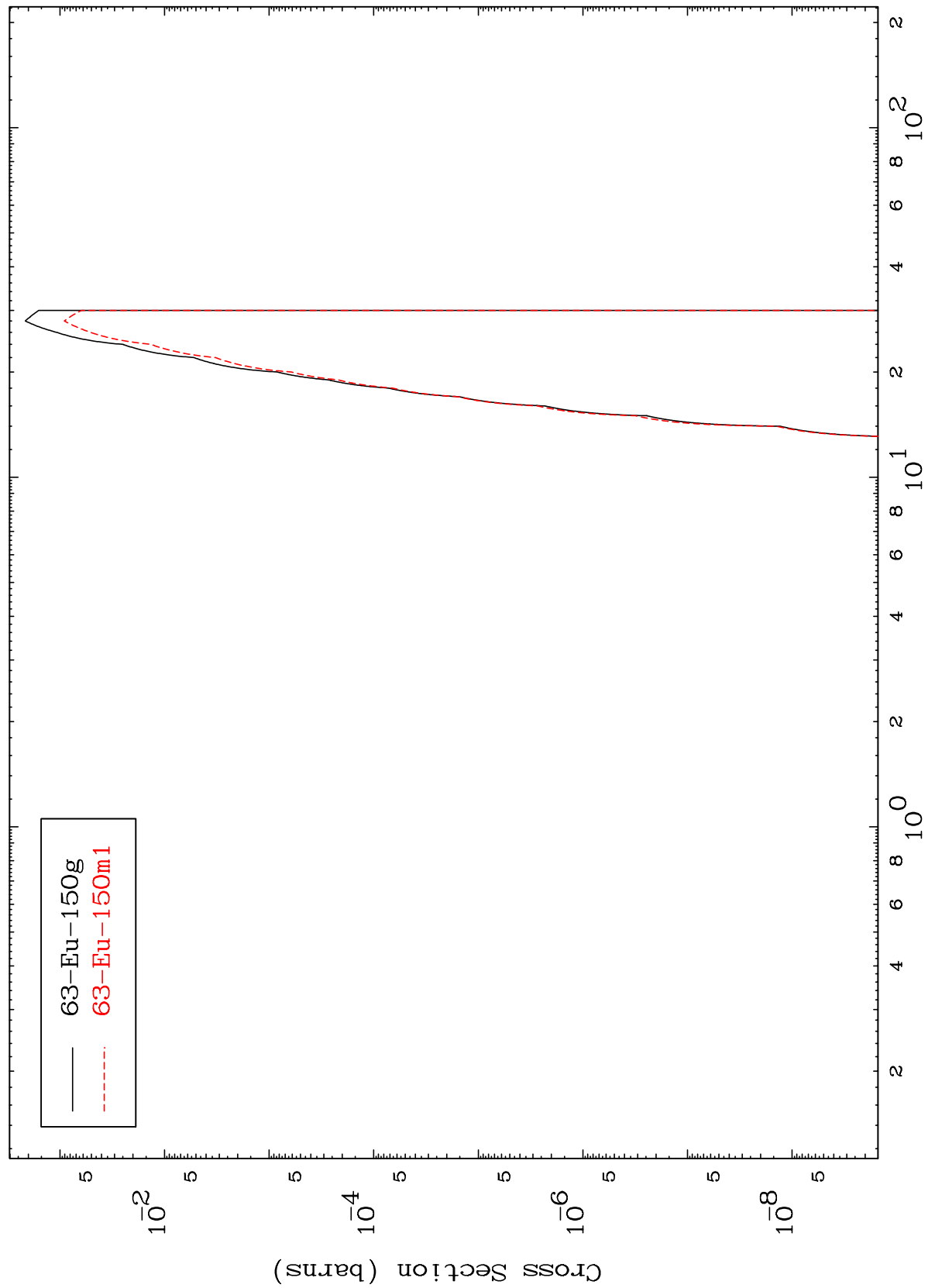
Radionuclide Production Cross Section  
( $\alpha, \gamma$ )



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63-Eu-150

Radionuclide Production Cross Section  
( $\alpha, \alpha$ )



14

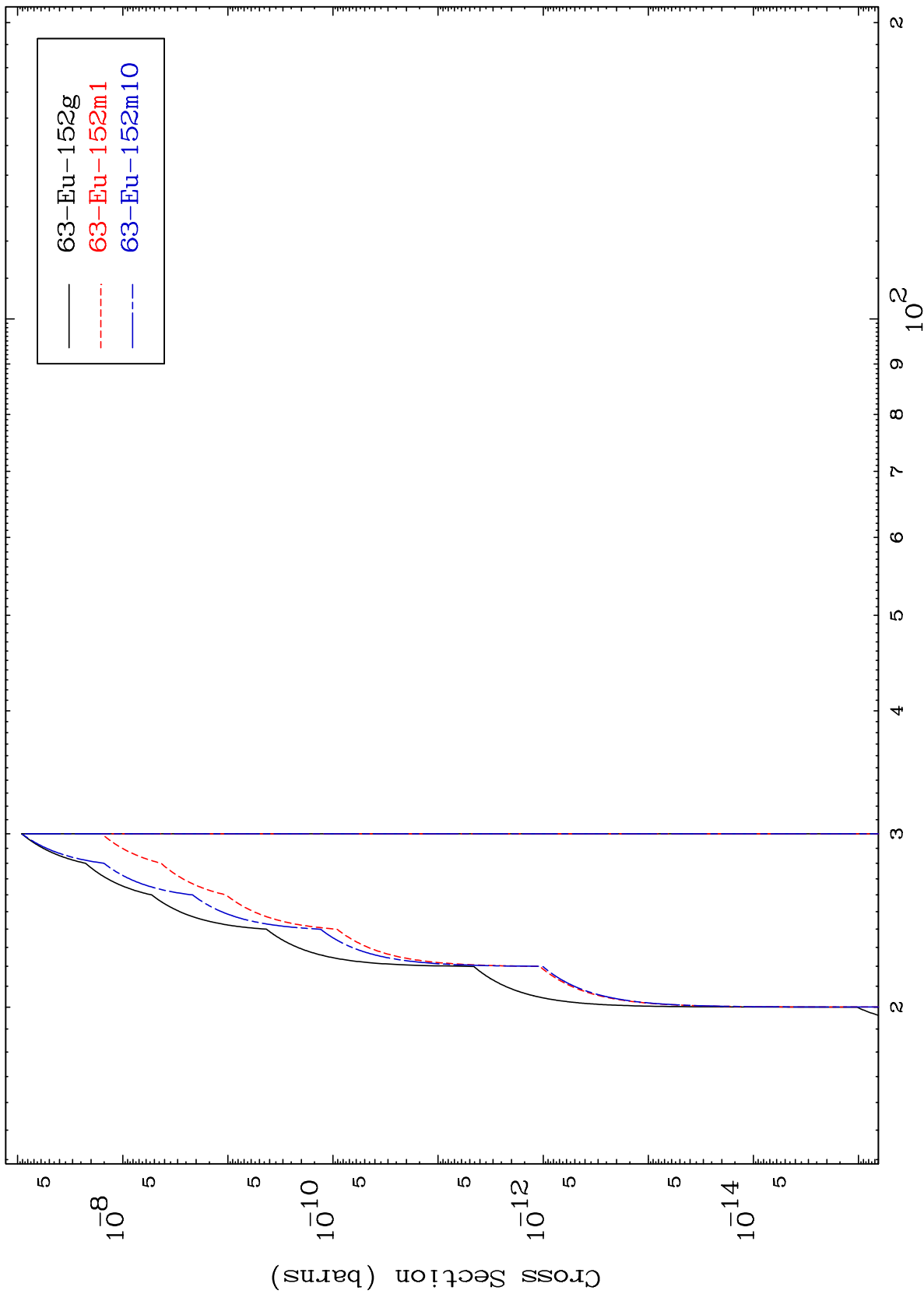
63-Eu-150

Incident Energy (MeV)

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63-Eu-150

$(\alpha, 2p)$   
Radionuclide Production Cross Section



15

63-Eu-150

Incident Energy (MeV)