

Program EVALPLOT  
(Version 2018-1)

by

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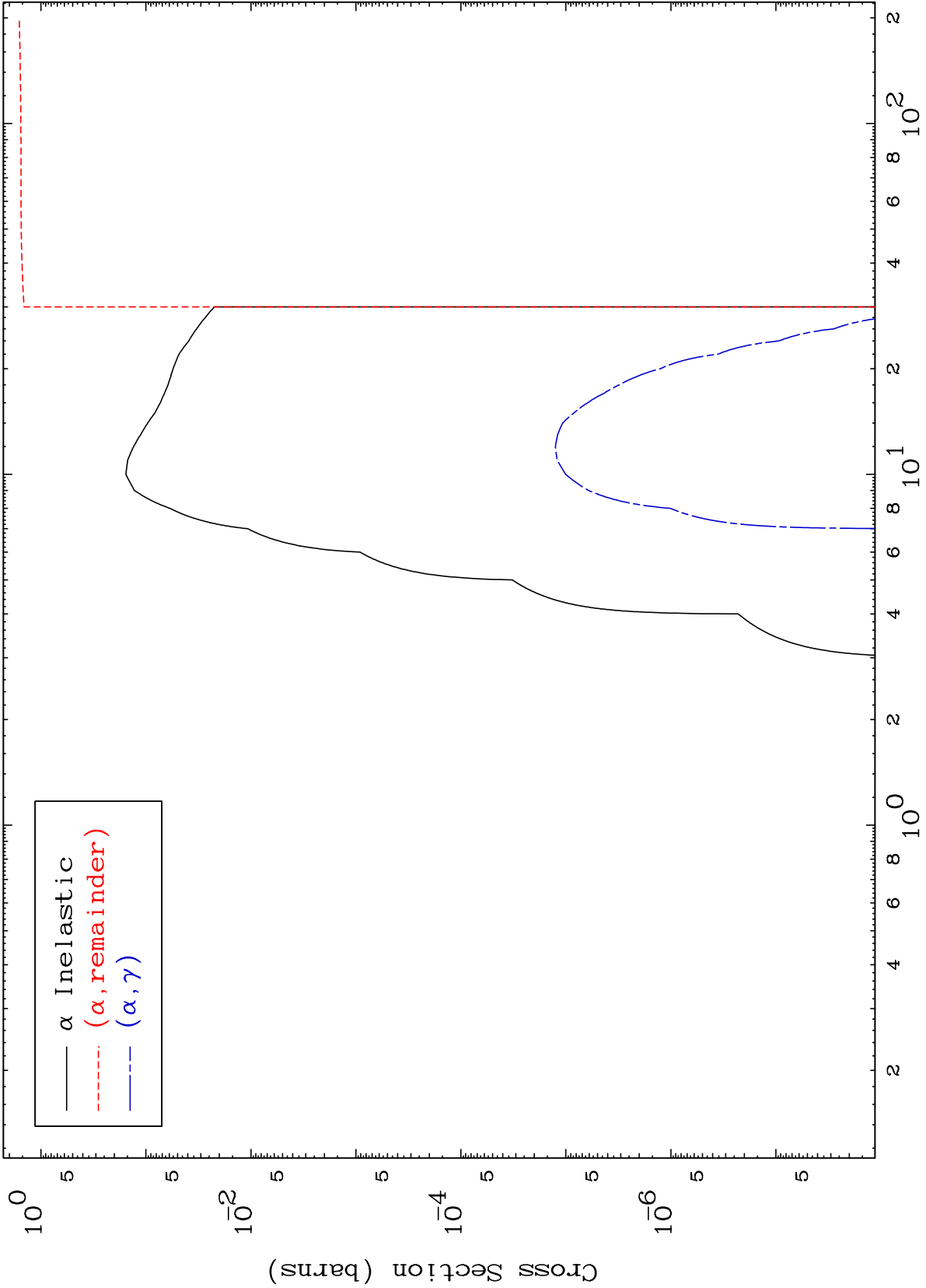
Press Mouse Button to Start

MAT 2646

$\alpha$  Major

26-Fe-61

0 Kelvin Cross Sections

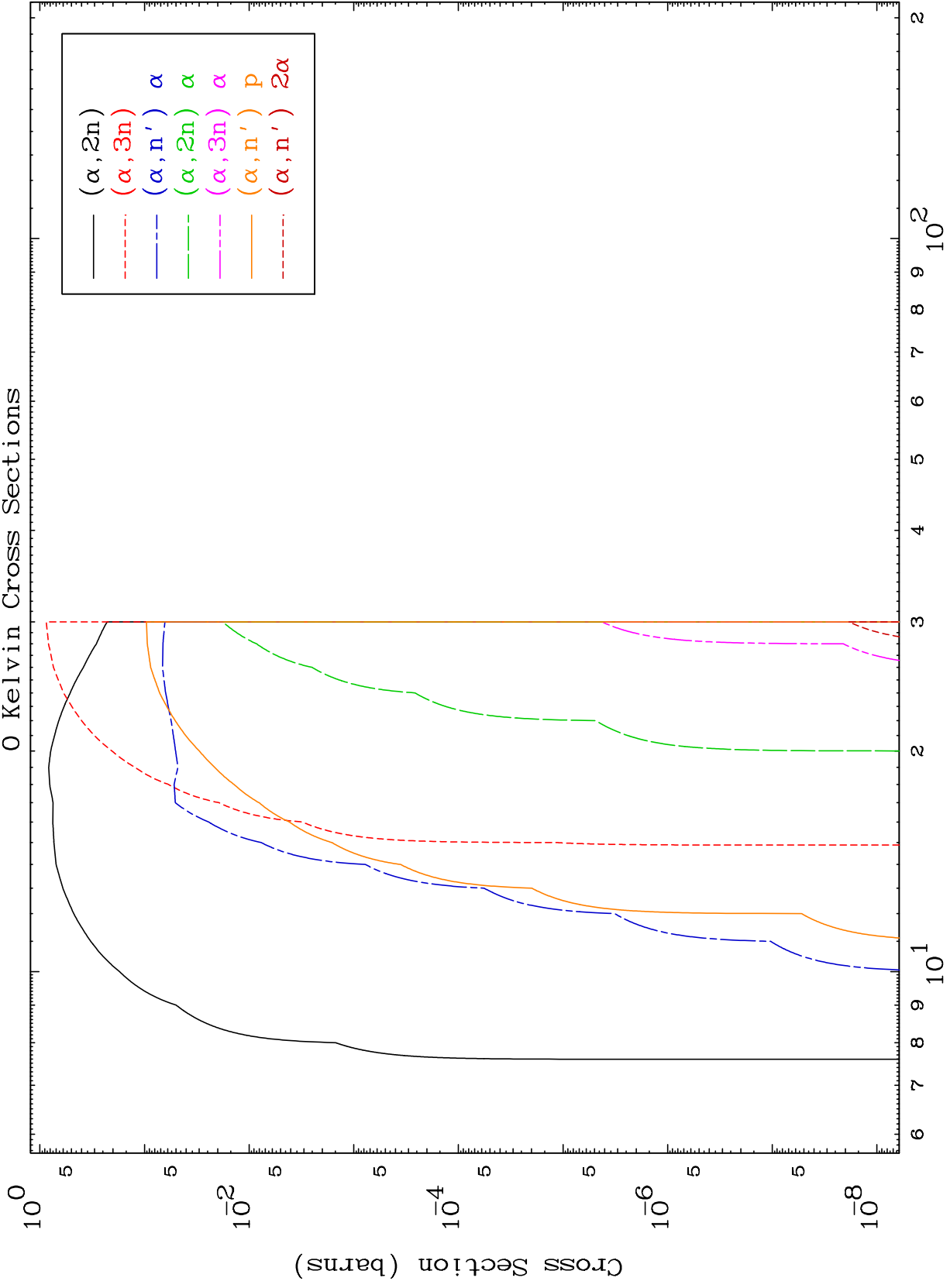


$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha$ ,  $\gamma$ )

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$\alpha$  Neutron Production  
0 Kelvin Cross Sections

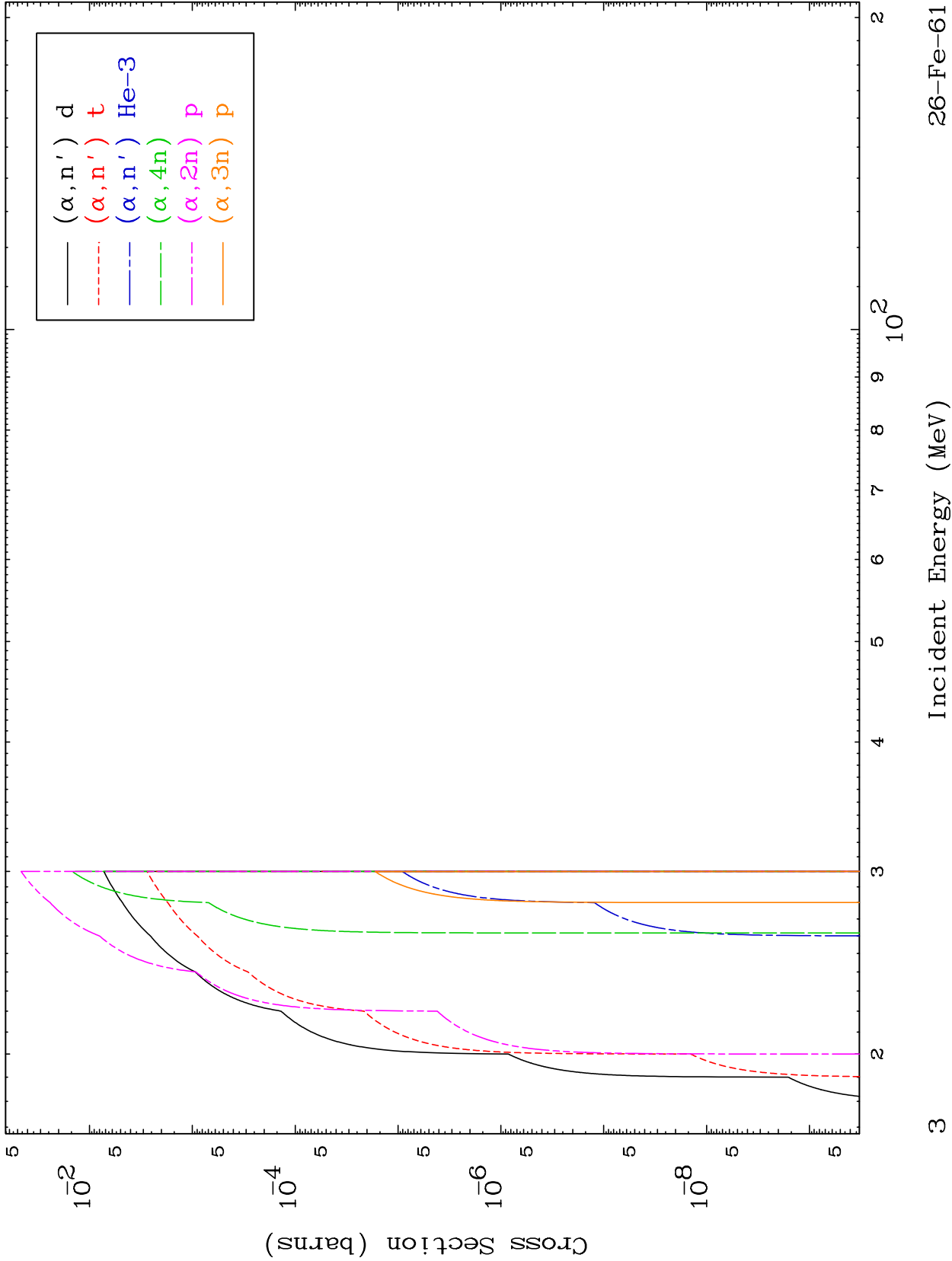
26-Fe-61

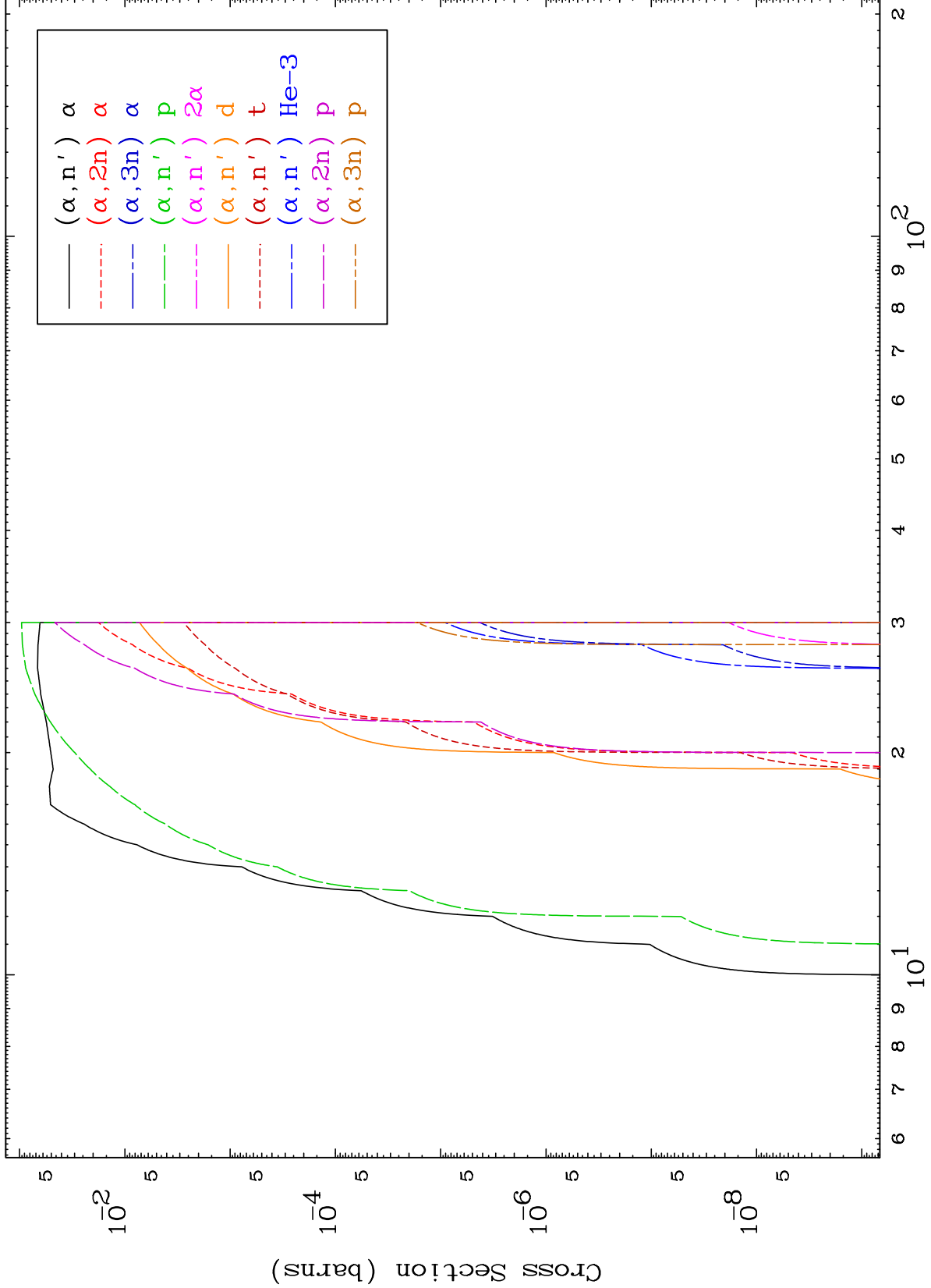


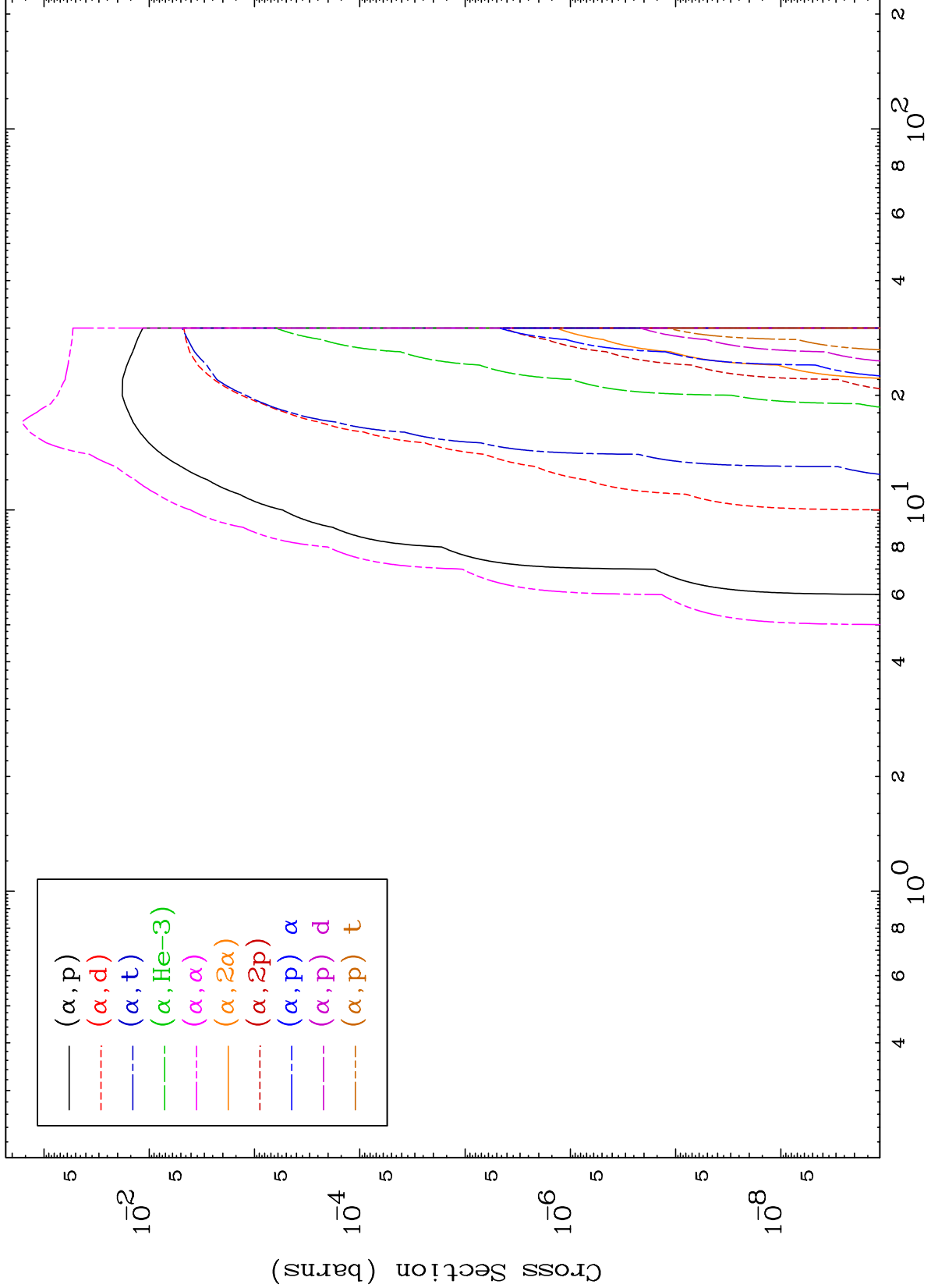
2

Incident Energy (MeV)

26-Fe-61



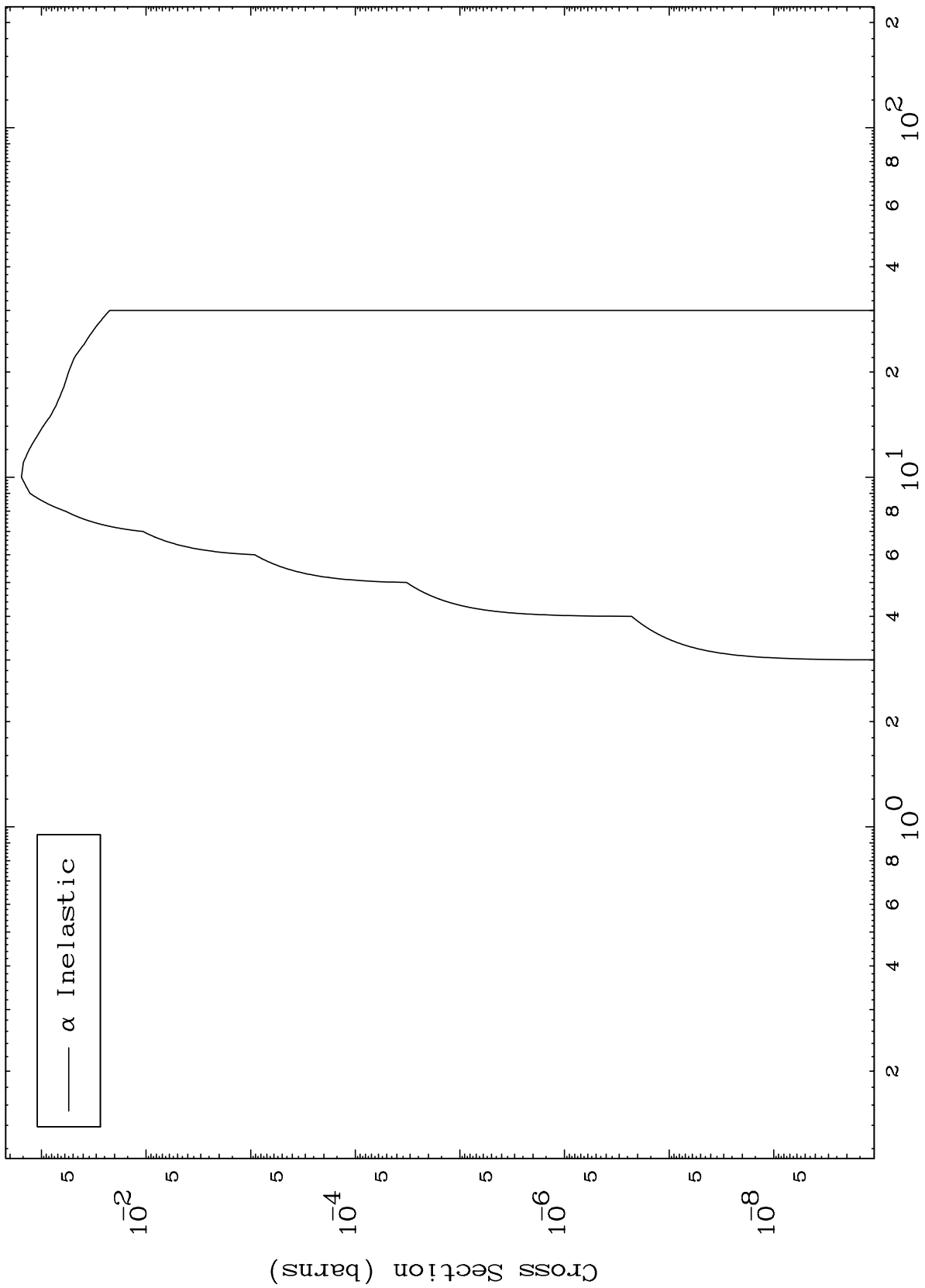




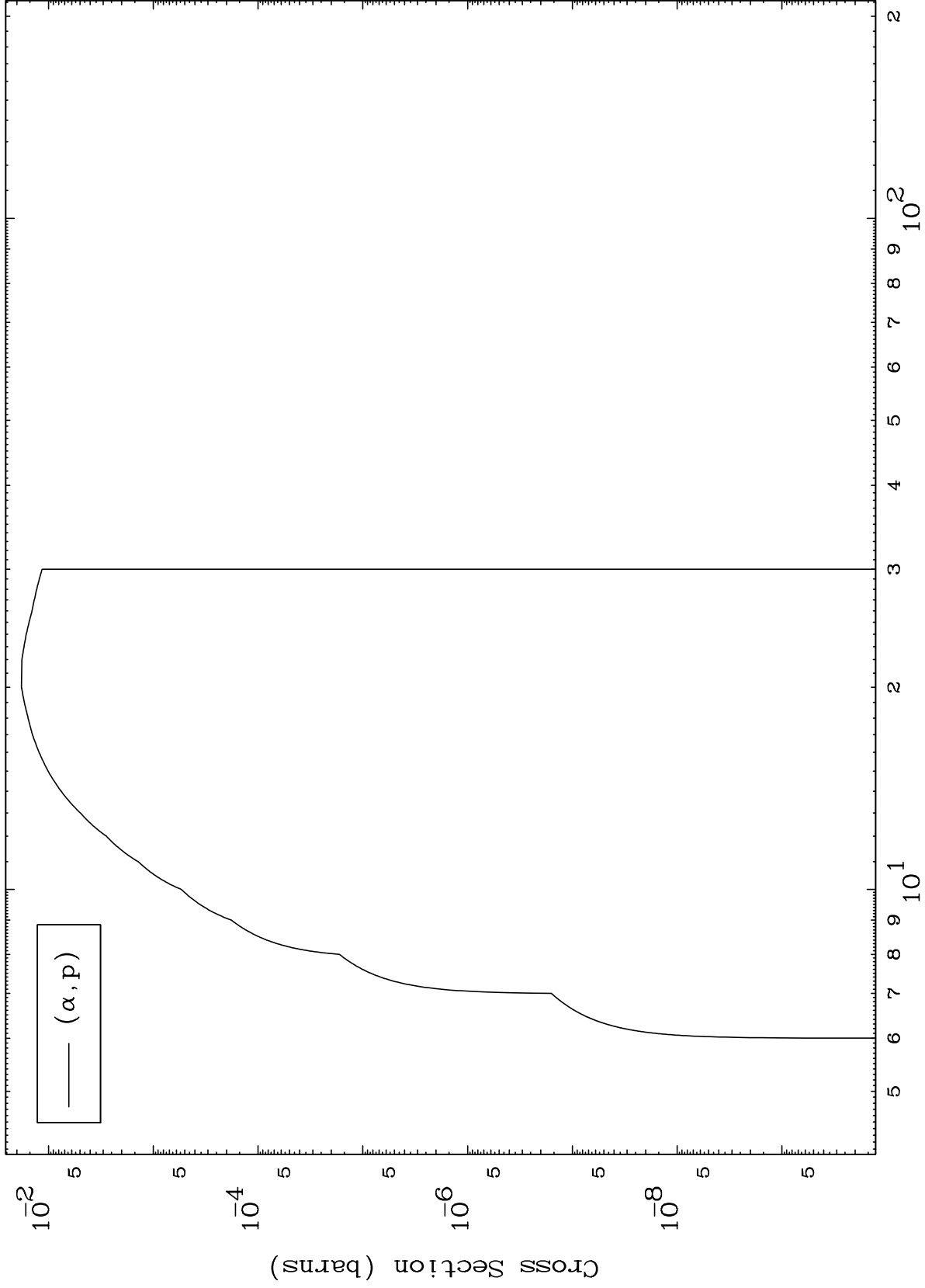
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26-Fe-61

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

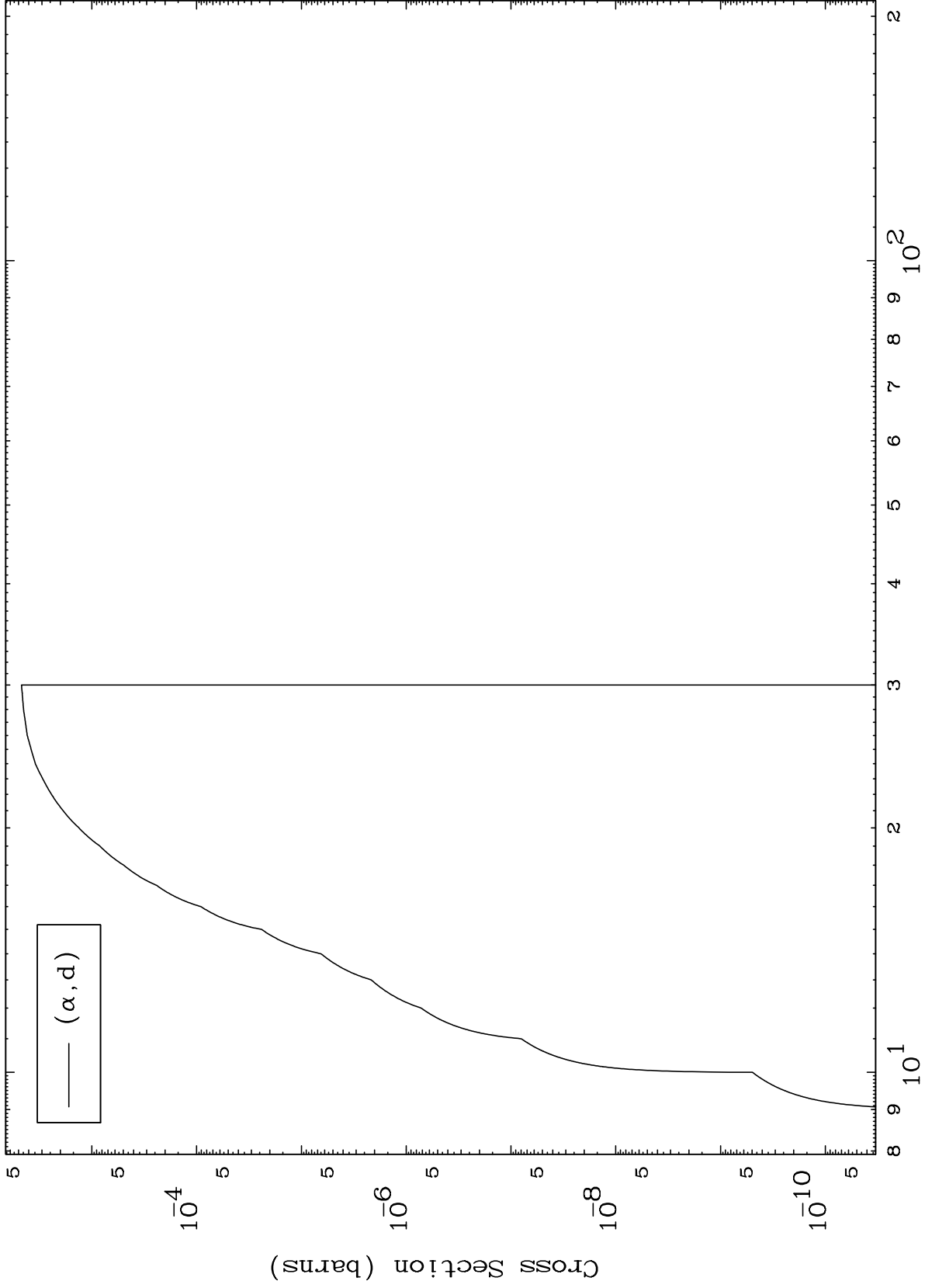




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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

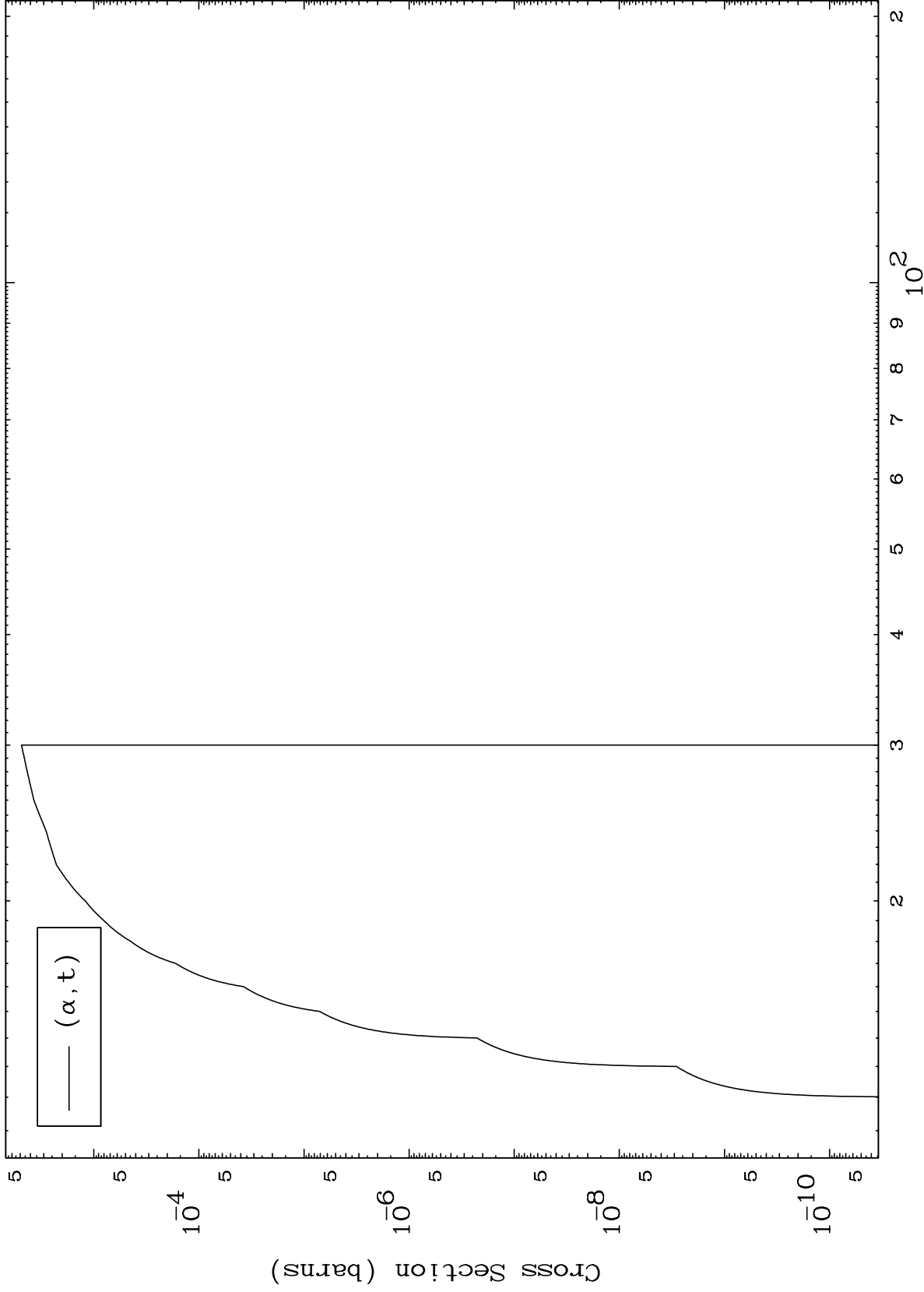
26-Fe-61



8

Incident Energy (MeV)

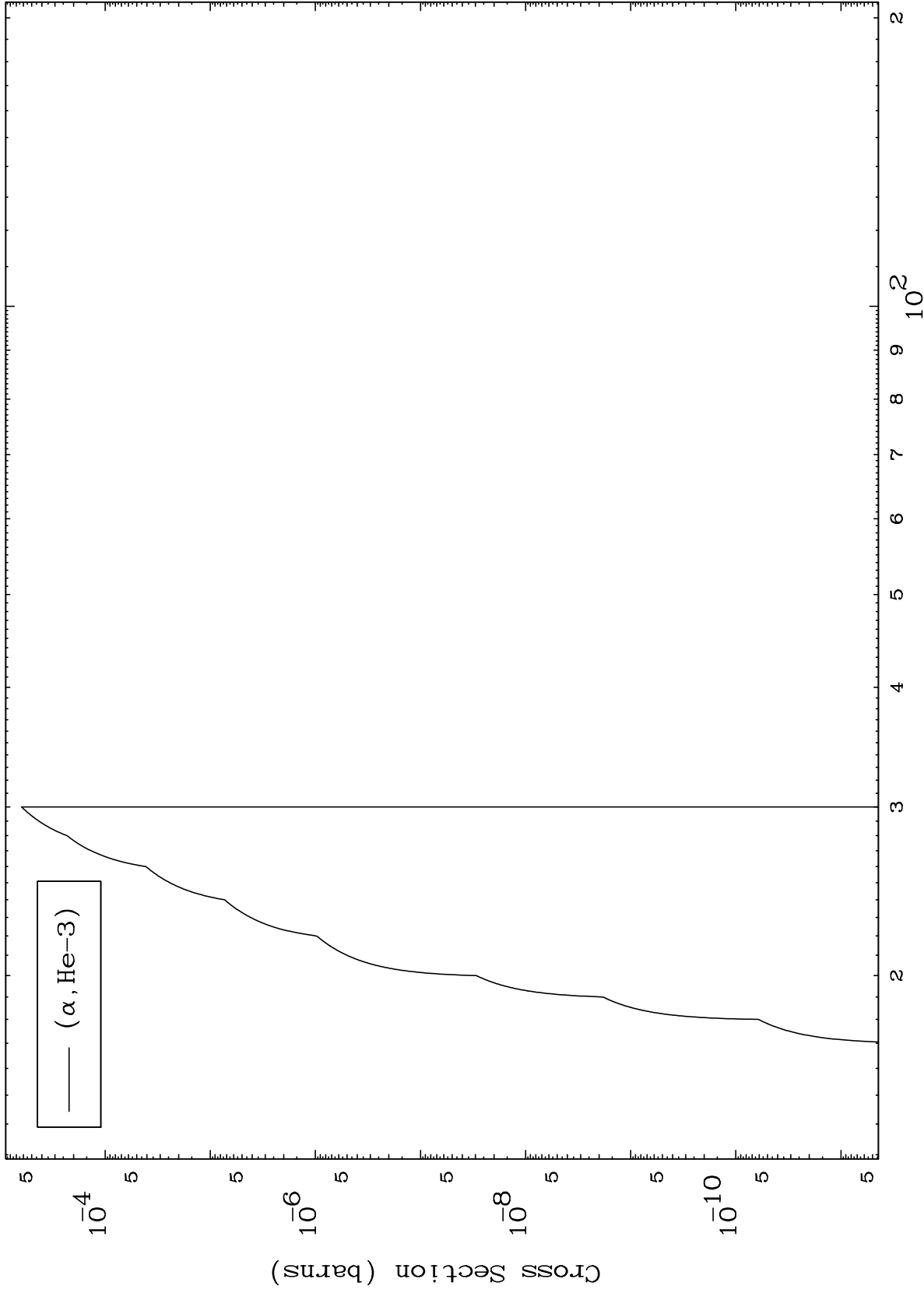
26-Fe-61



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( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

26-Fe-61

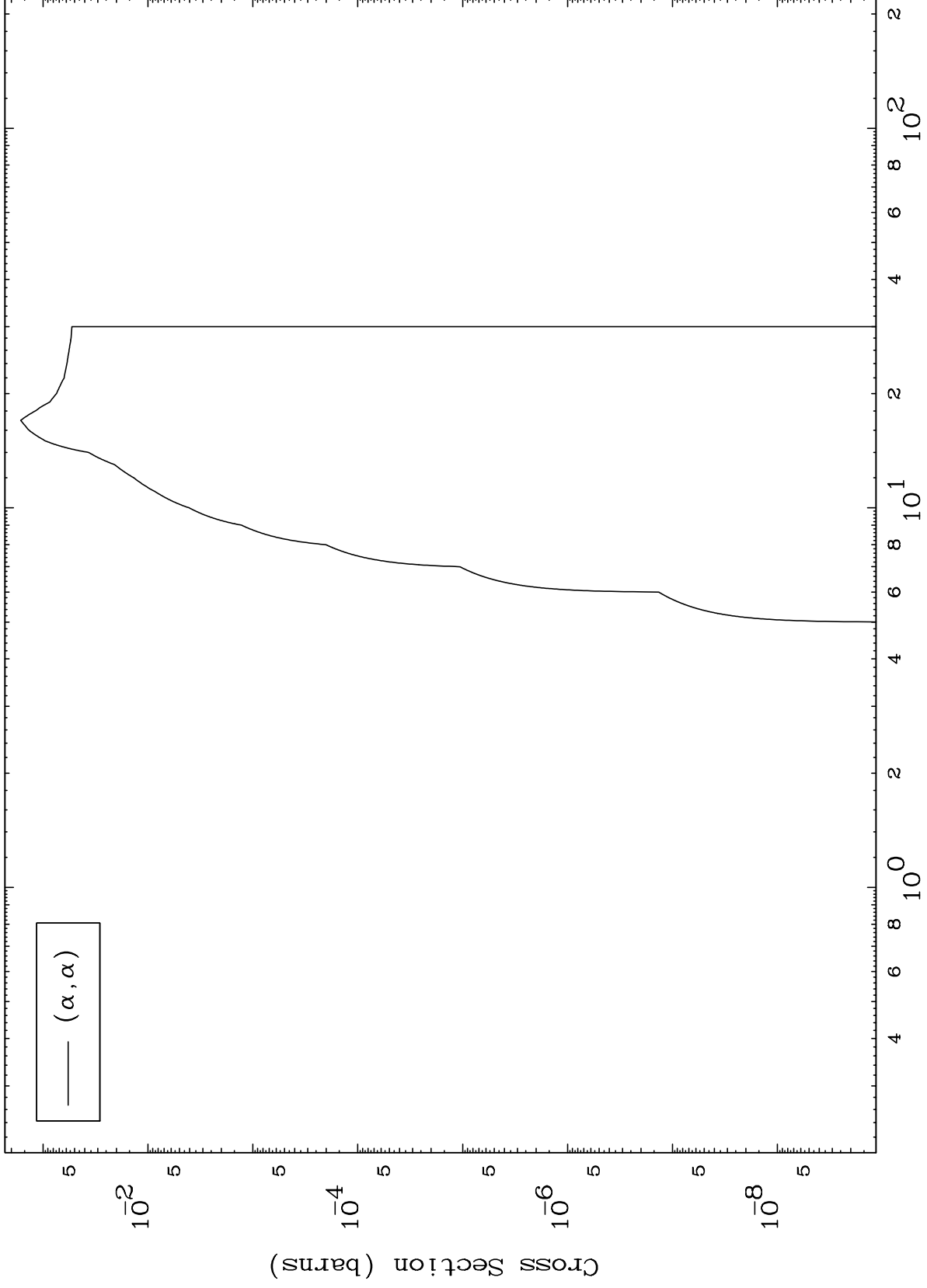


10

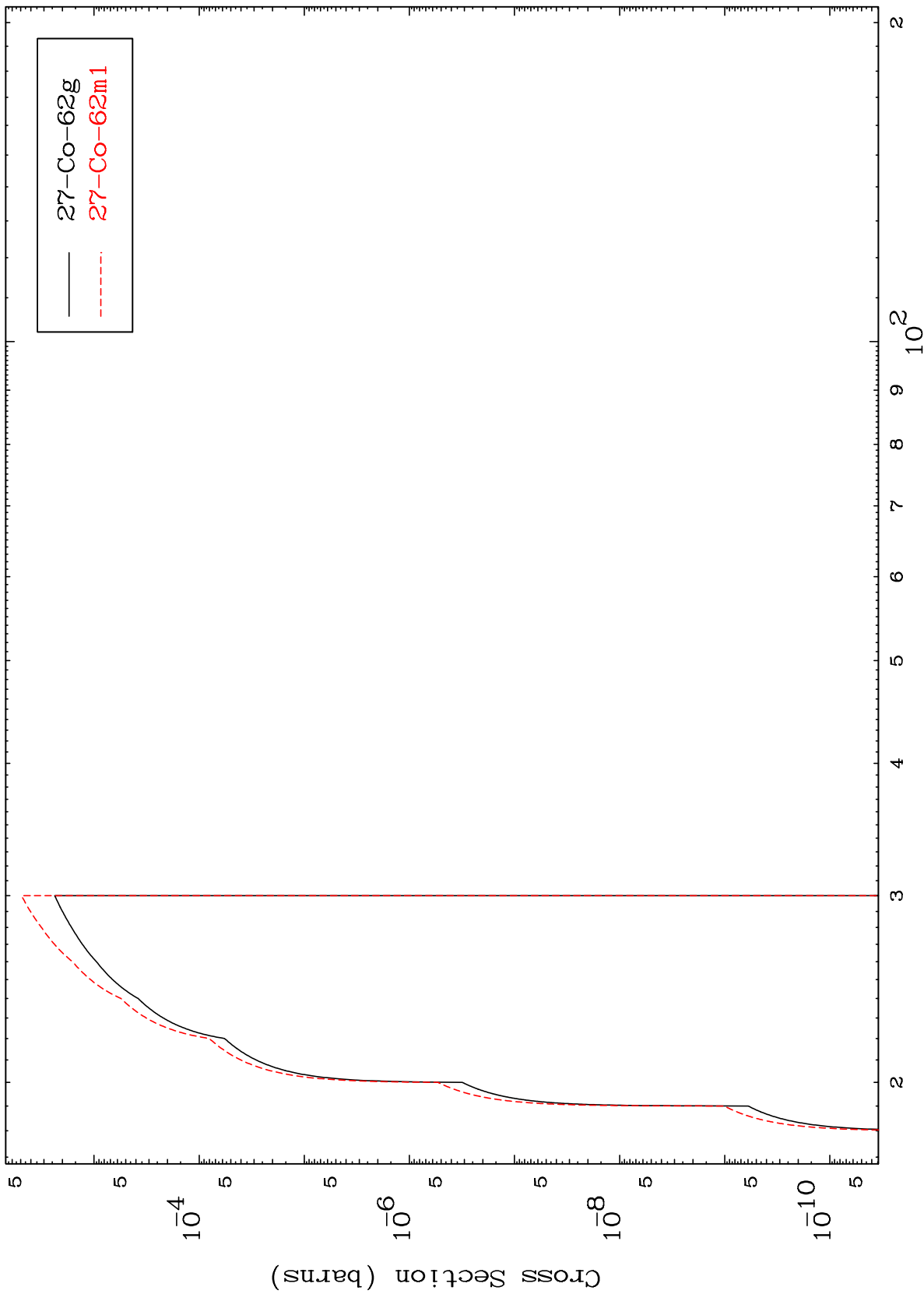
Incident Energy (MeV)

26-Fe-61

0 Kelvin Cross Sections



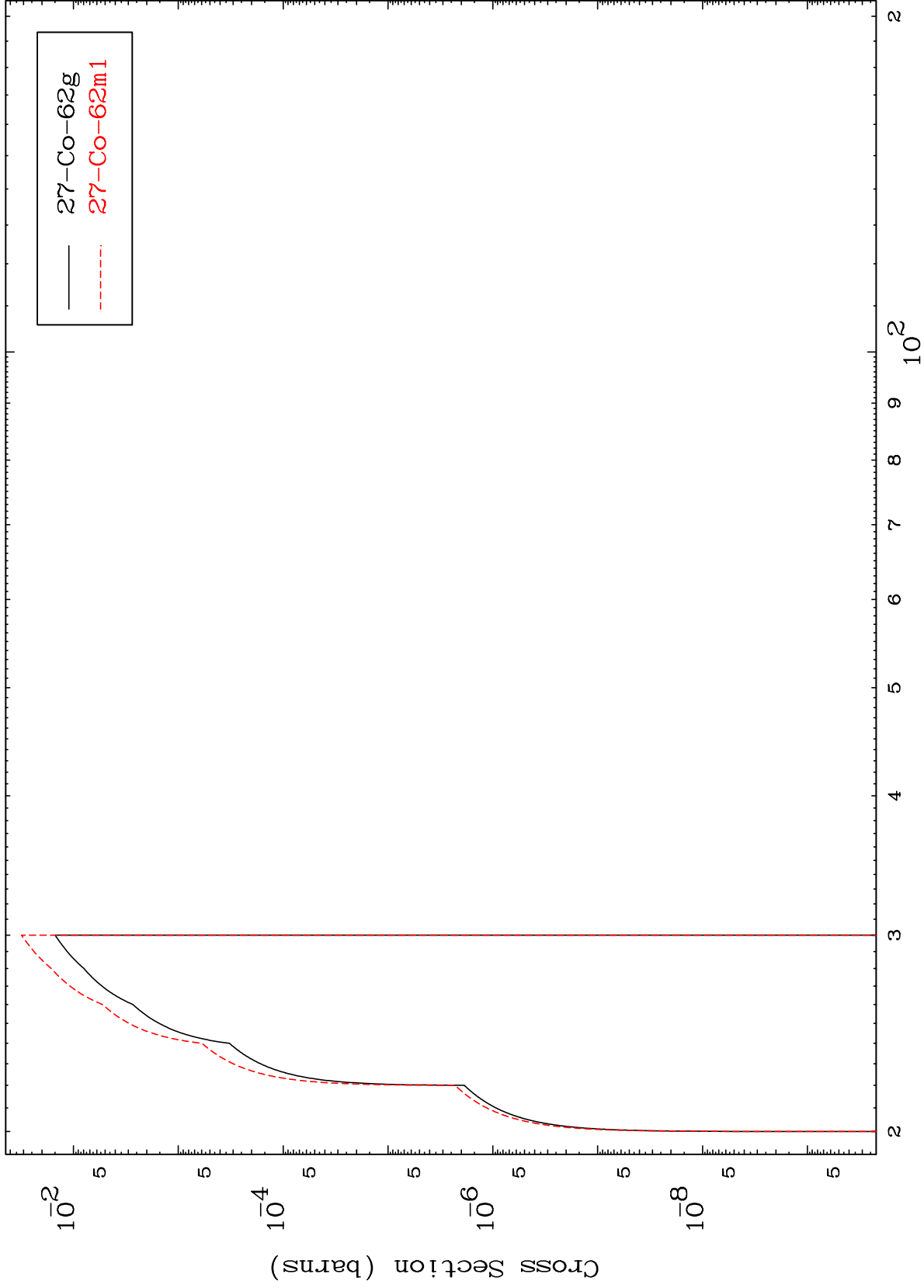
Radionuclide Production Cross Section



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26-Fe-61

$(\alpha, 2n)$  p  
Radionuclide Production Cross Section



13

Incident Energy (MeV)

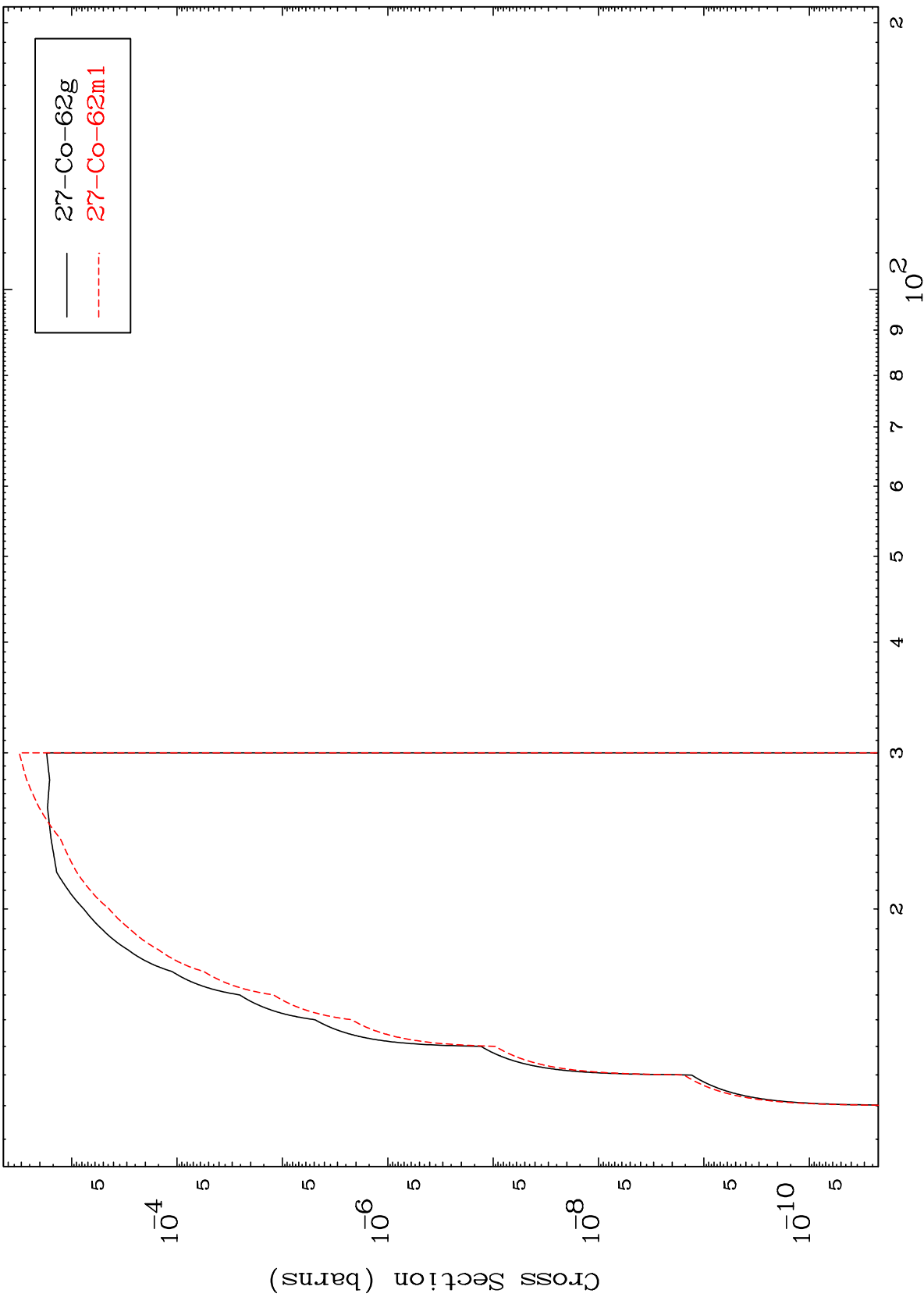
26-Fe-61

MAT 2646

( $\alpha, t$ )

26-Fe-61

Radionuclide Production Cross Section



14

Incident Energy (MeV)

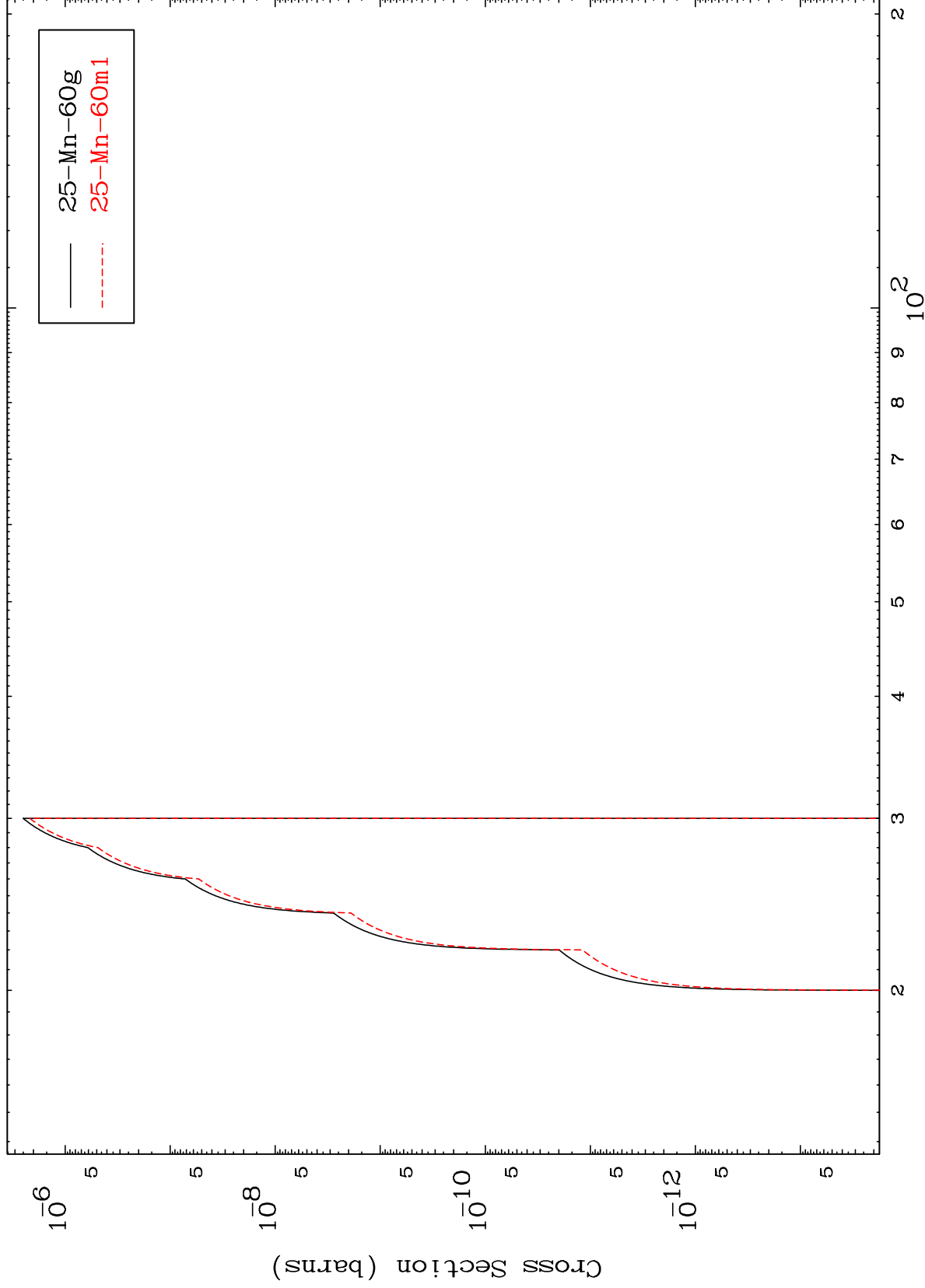
26-Fe-61

MAT 2646

( $\alpha, p$ )  $\alpha$

$^{26}\text{Fe-61}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

$^{26}\text{Fe-61}$