Program EVALPLOT
(Version 2018-1)
by
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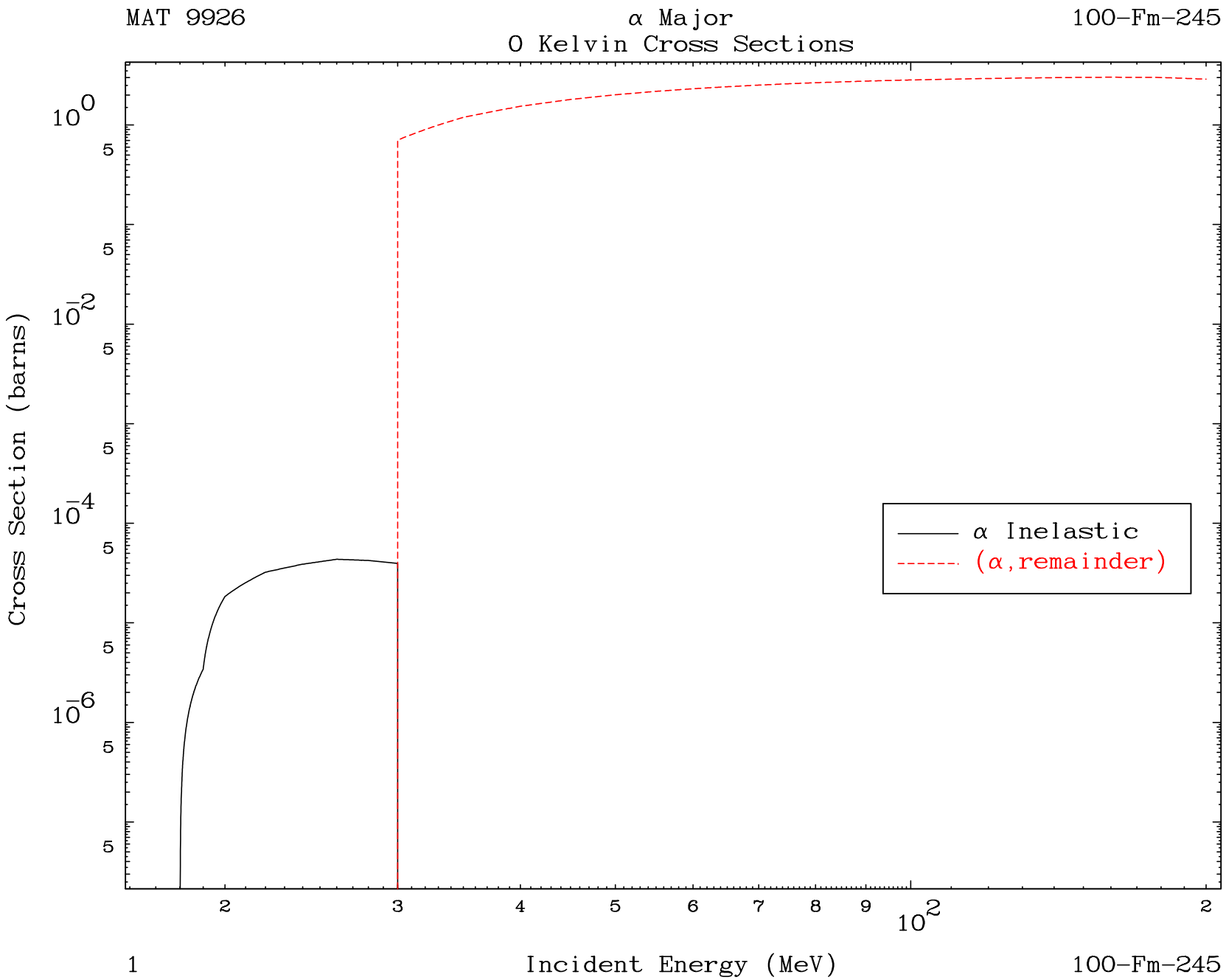
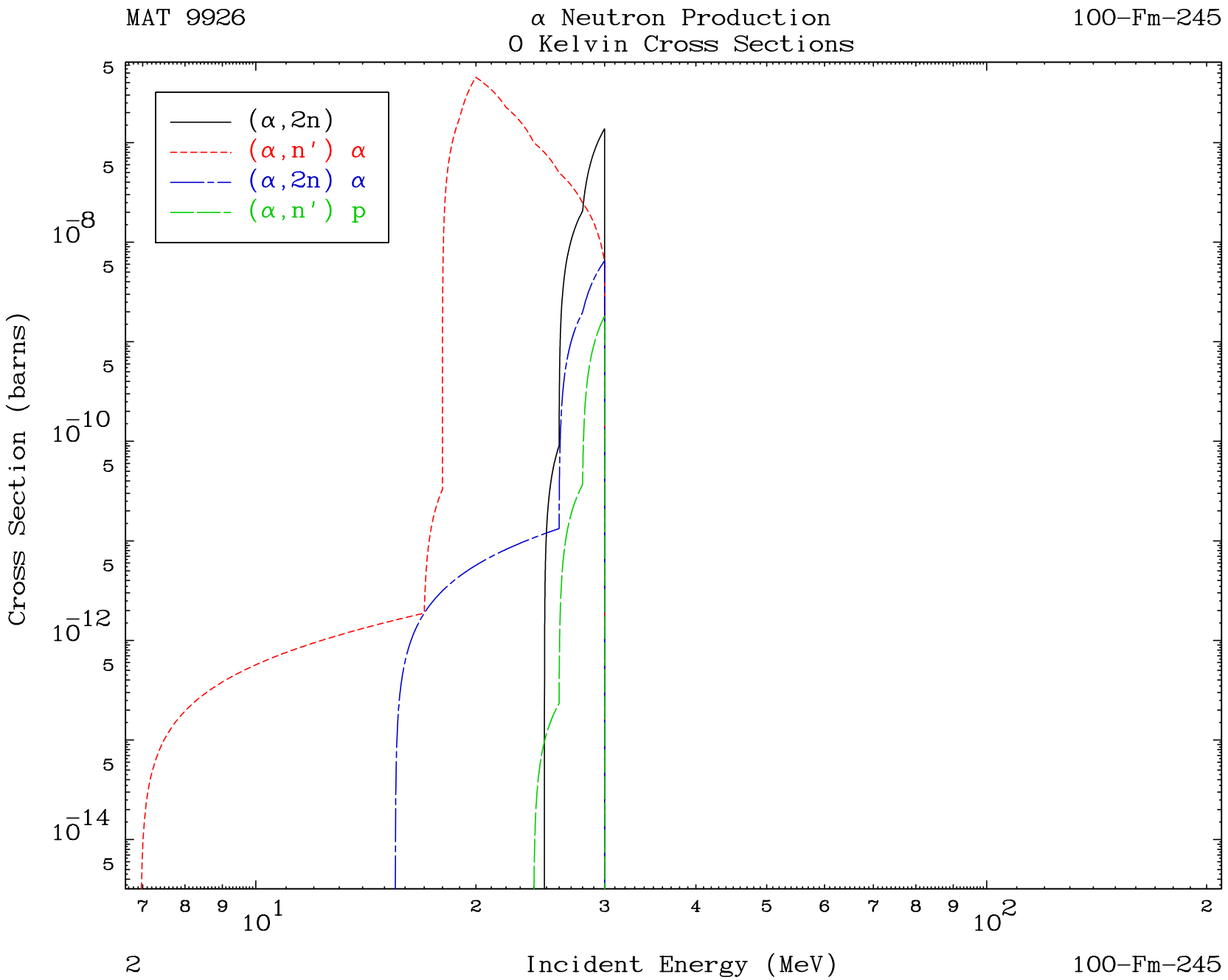
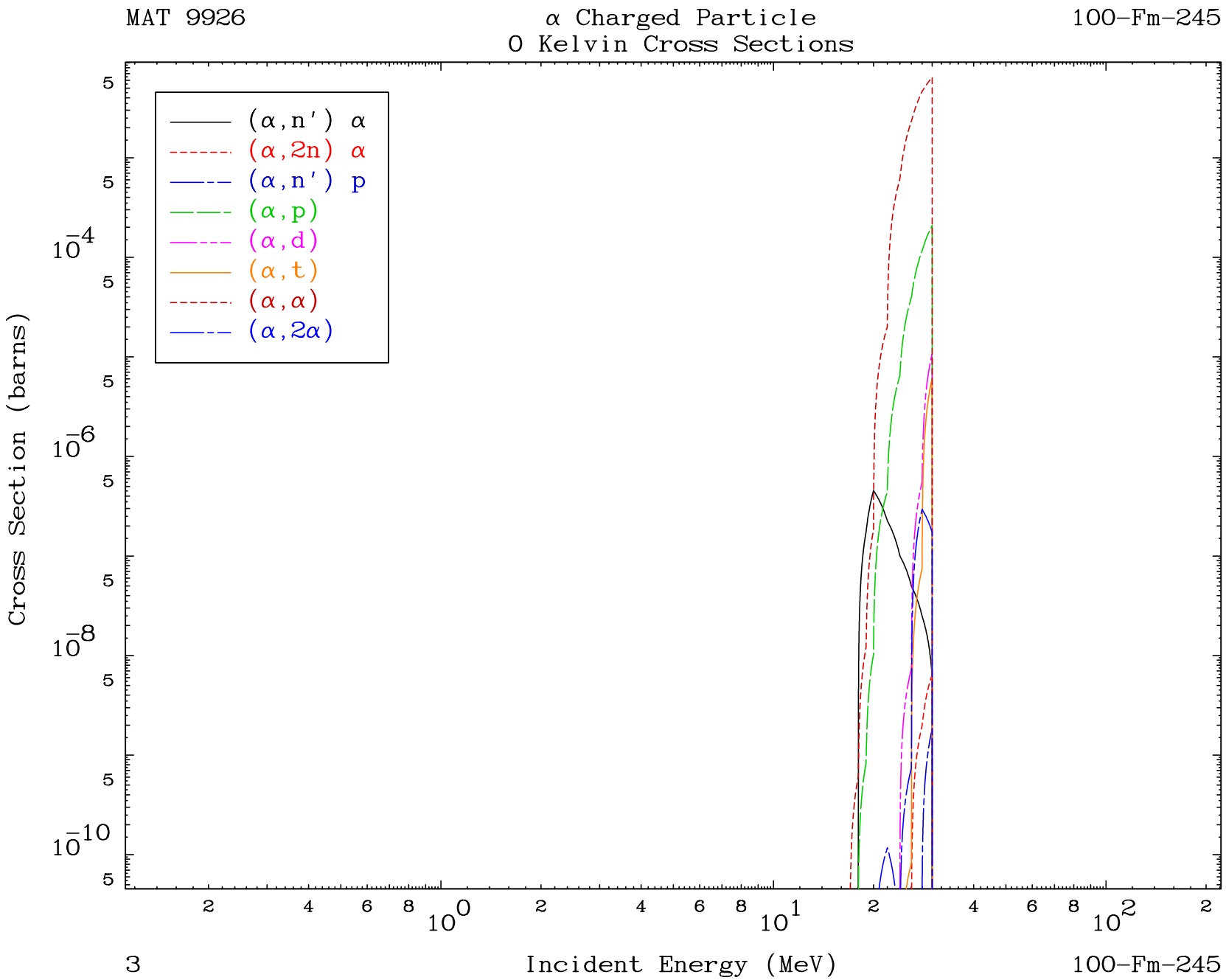
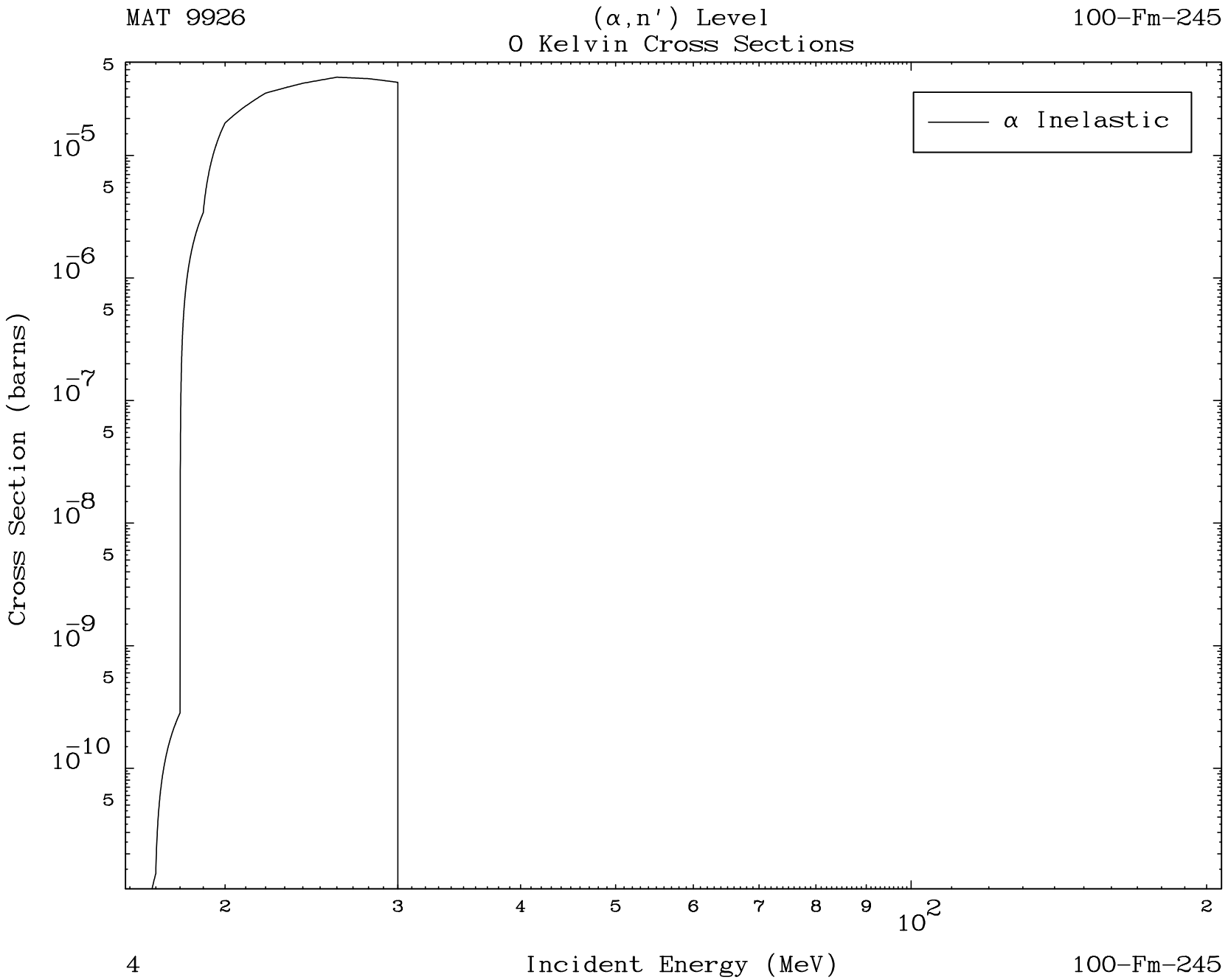
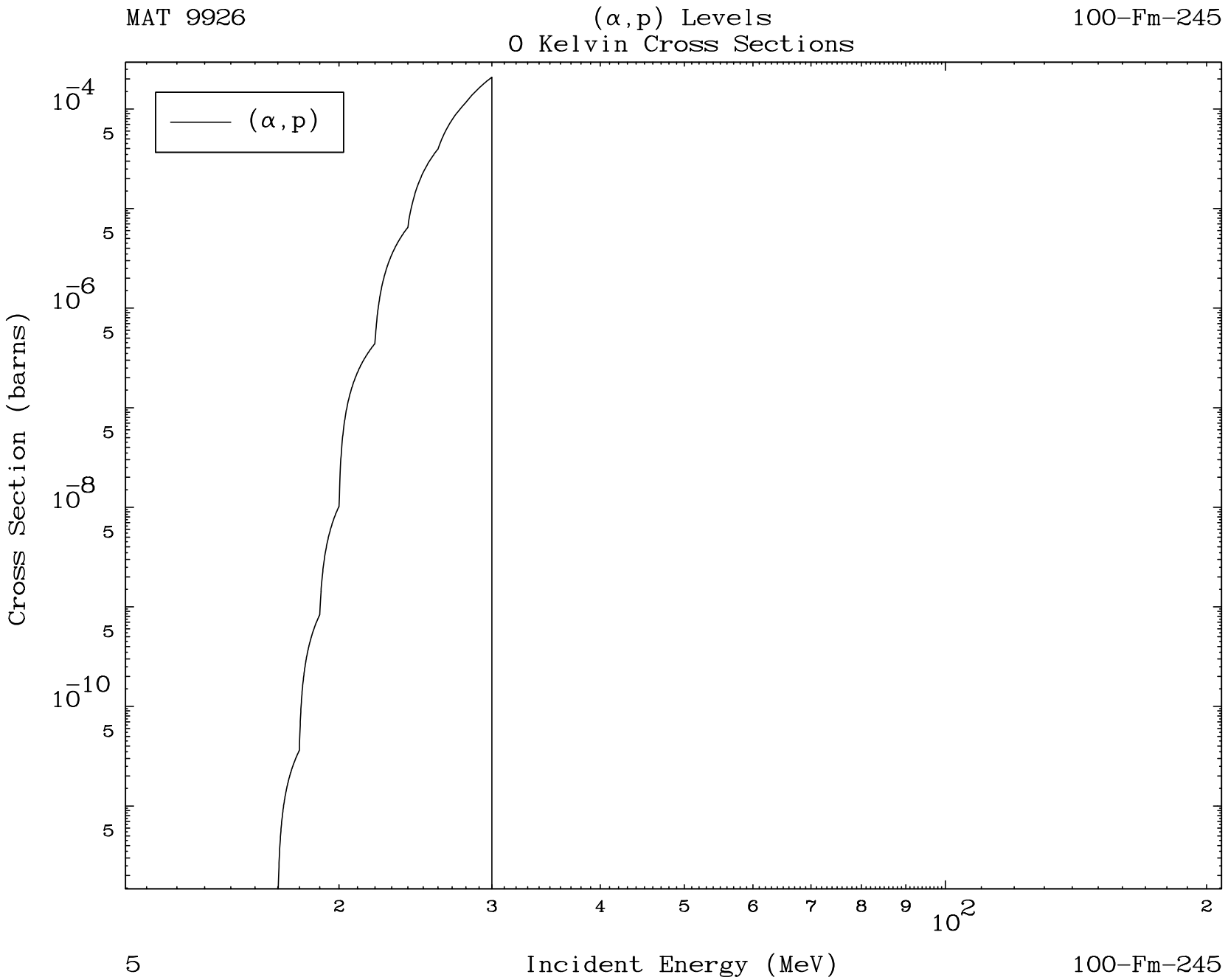
Press Mouse Button to Start

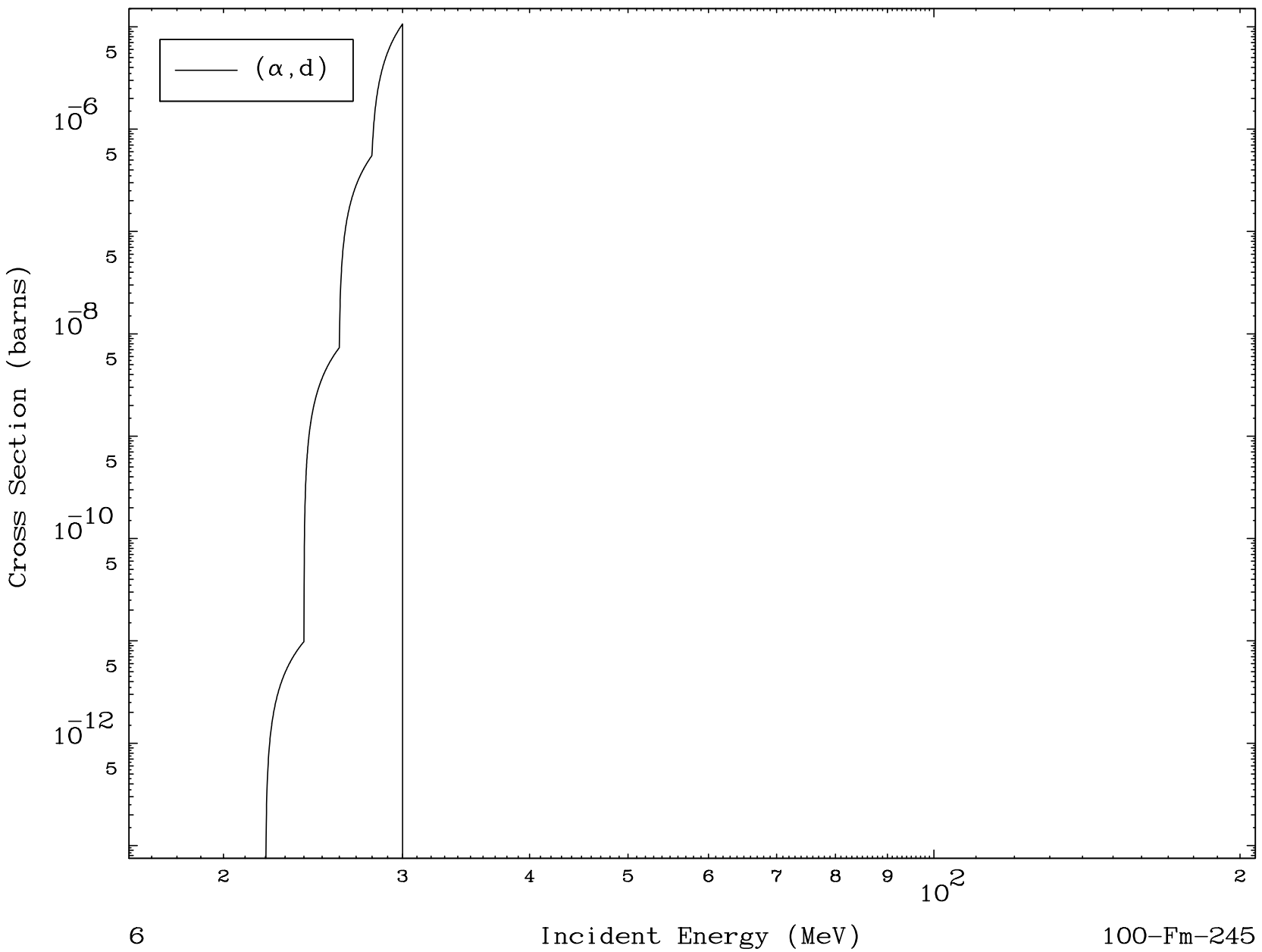

MAT 9926
α Major
100-Fm-245
0 Kelvin Cross Sections
Cross Section (barns)
10^0
10^-2
10^-4
10^-6
5
2
3
4
6
7
8
9
10^2
1
Incident Energy (MeV)
α Inelastic
(α,remainder)
100-Fm-245

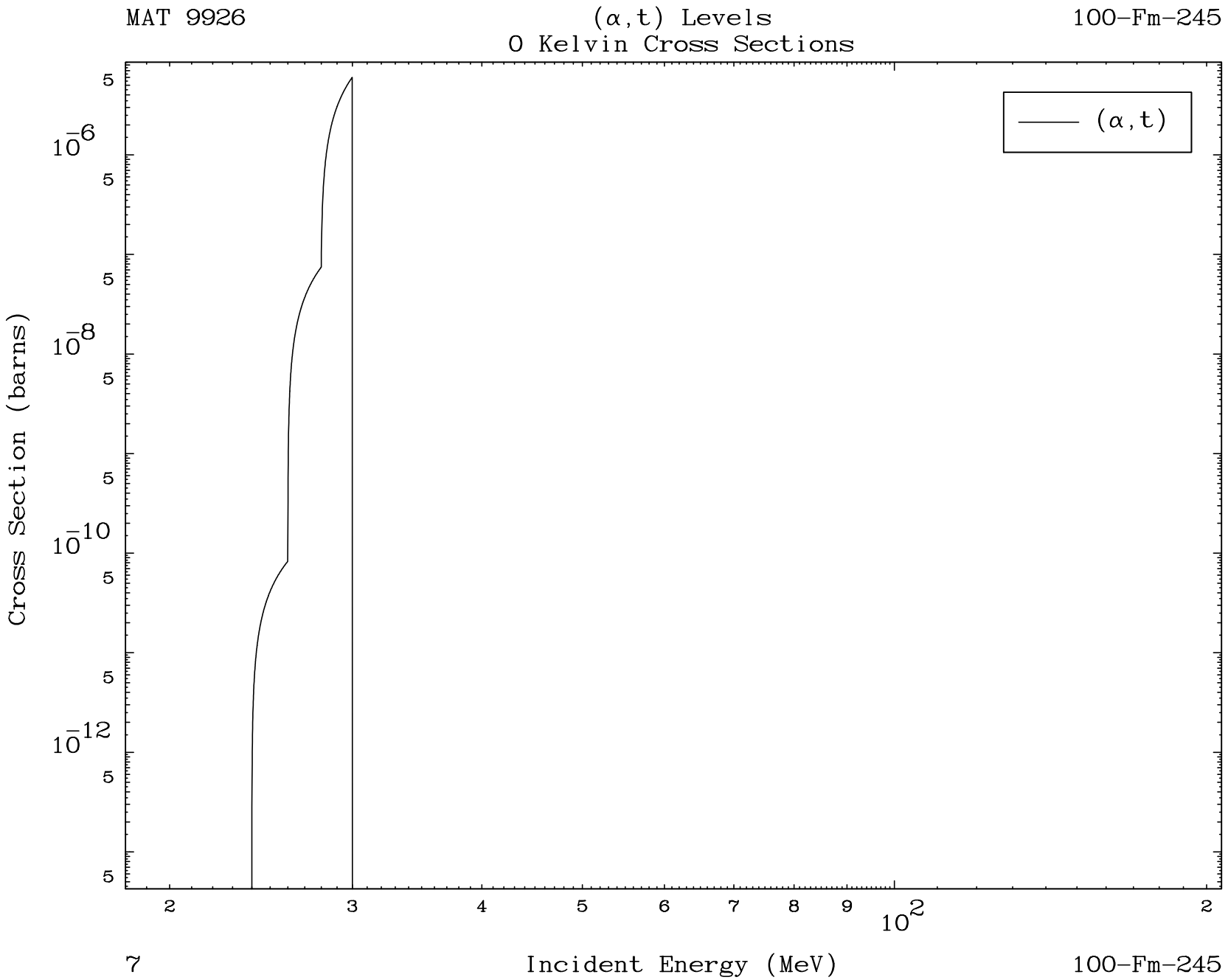

MAT 9926
α Neutron Production
0 Kelvin Cross Sections
100-Fm-245
(α,2n)
(α,n') α
(α,2n) α
(α,n') p
Cross Section (barns)
Incident Energy (MeV)
100-Fm-245

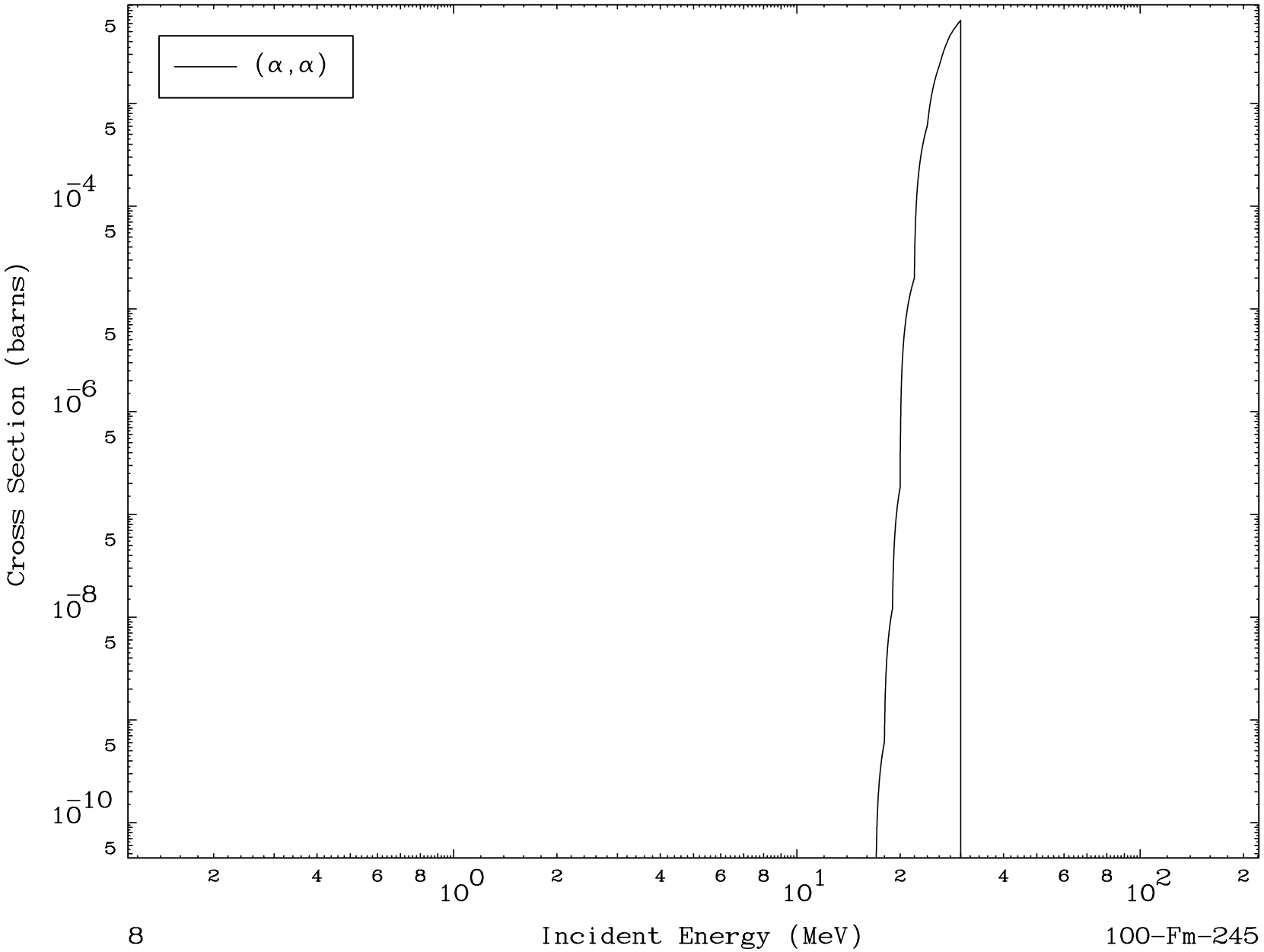

MAT 9926
α Charged Particle
0 Kelvin Cross Sections
100-Fm-245
(α,n') α
(α,2n) α
(α,n') p
(α,p)
(α,d)
(α,t)
(α,α)
(α,2α)
Cross Section (barns)
Incident Energy (MeV)
3
100-Fm-245

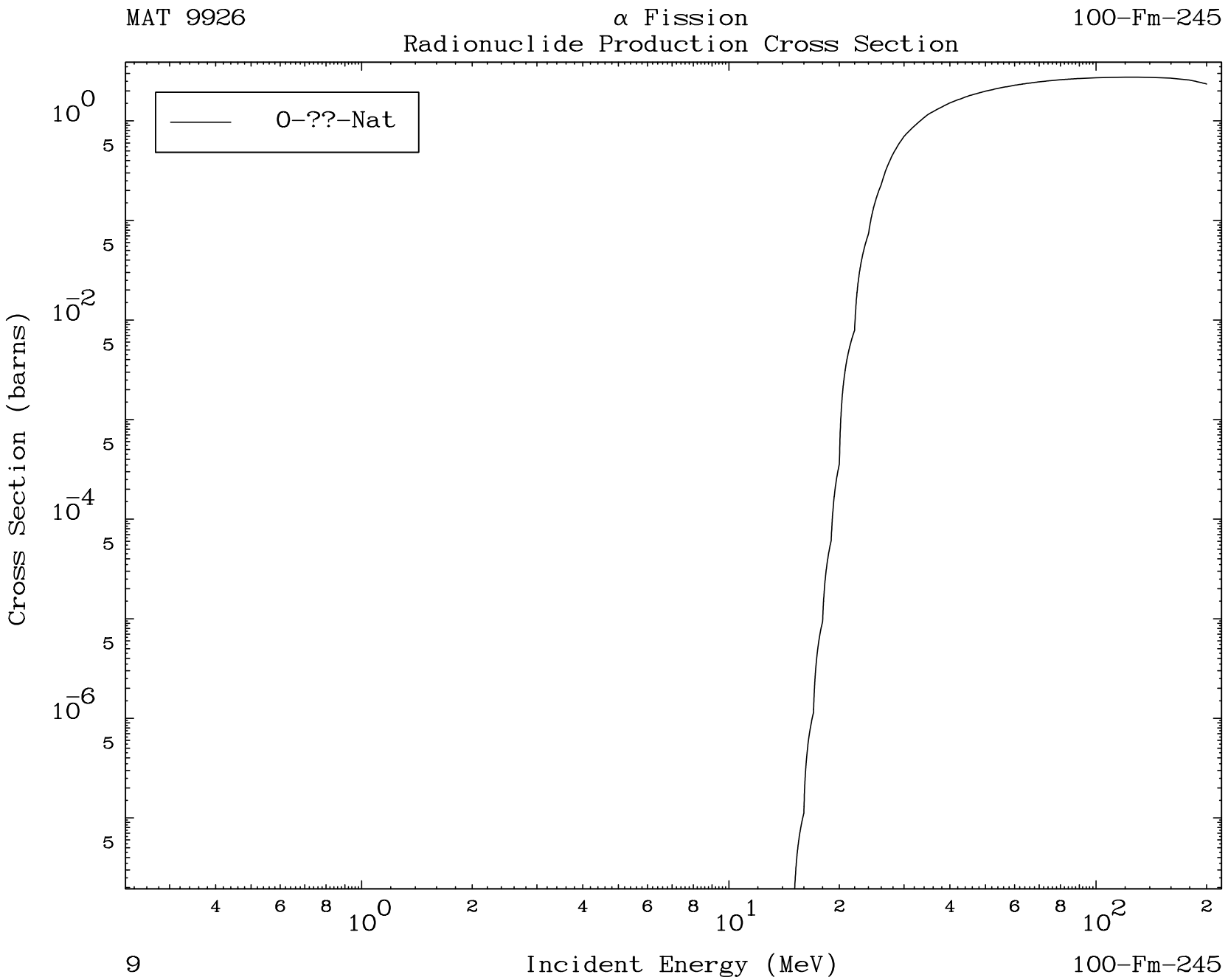

MAT 9926
(α,n') Level
100-Fm-245
0 Kelvin Cross Sections
α Inelastic
Cross Section (barns)
10-5
10-6
10-7
10-8
10-9
10-10
2
3
4
5
6
7
8
9
10^2
2
Incident Energy (MeV)

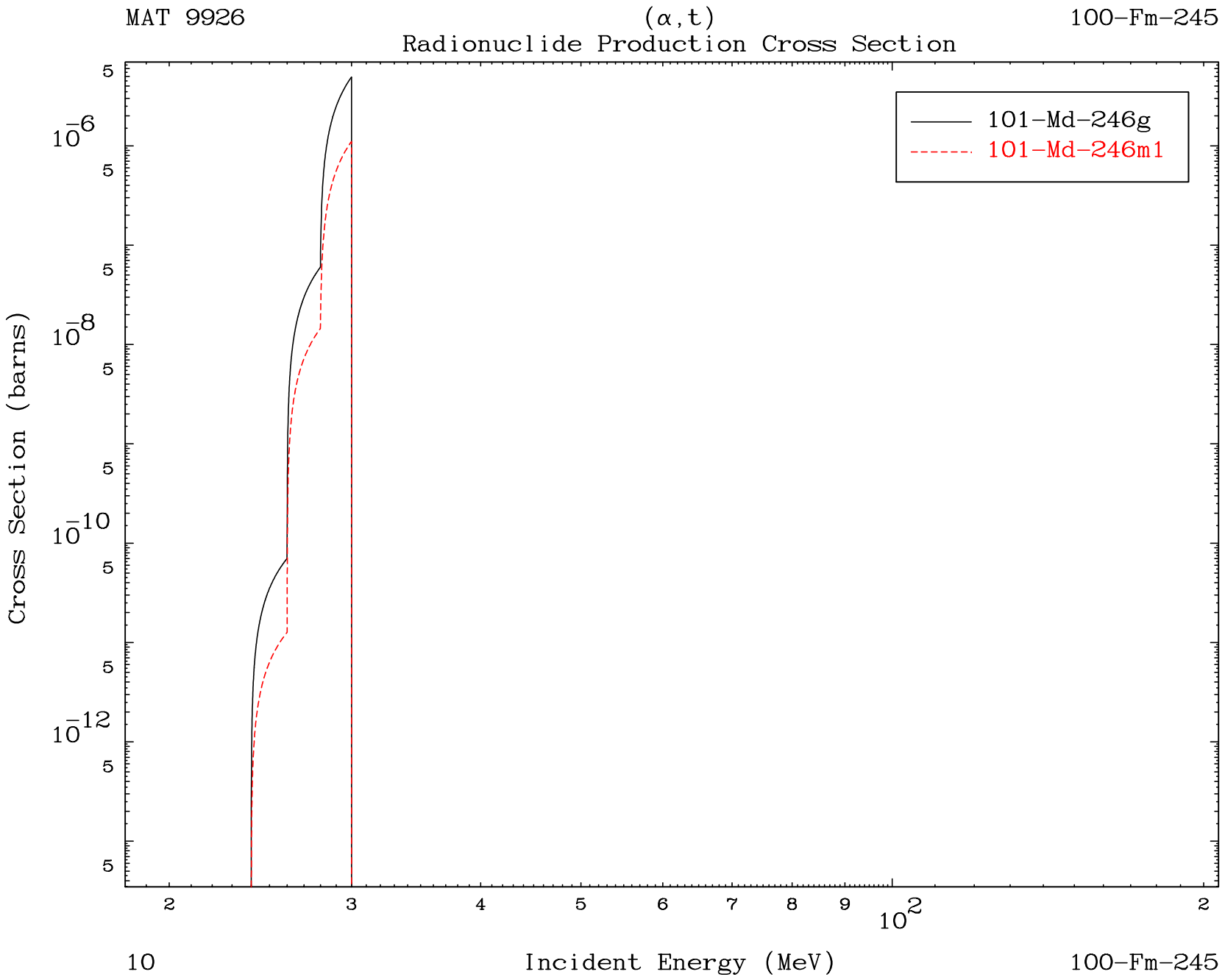

MAT 9926
(α,p) Levels
100-Fm-245
0 Kelvin Cross Sections
(α,p)
Cross Section (barns)
10-4
10-6
10-8
10-10
2
3
4
5
6
7
8
9
10 2
2
5
Incident Energy (MeV)
100-Fm-245


0 Kelvin Cross Sections
(α,d)
Cross Section (barns)
10-6
10-8
10-10
10-12
5
2
3
4
5
6
7
8
9
10^2
2
Incident Energy (MeV)


MAT 9926
(α,t) Levels
100-Fm-245
0 Kelvin Cross Sections
(α,t)
Cross Section (barns)
10-6
10-8
10-10
10-12
Incident Energy (MeV)
10^2
7
100-Fm-245


0 Kelvin Cross Sections
(α,α)
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
10^0
10^1
10^2
Incident Energy (MeV)


MAT 9926
α Fission
100-Fm-245
Radionuclide Production Cross Section
0-??-Nat
Cross Section (barns)
10^0
10^-2
10^-4
10^-6
5
4 6 8 10^0 2 4 6 8 10^1 2 4 6 8 10^2 2
Incident Energy (MeV)
9
100-Fm-245


MAT 9926
(α,t)
100-Fm-245
Radionuclide Production Cross Section
101-Md-246g
101-Md-246m1
Cross Section (barns)
Incident Energy (MeV)
100-Fm-245