

Program EVALPLOT  
(Version 2018-1)

by

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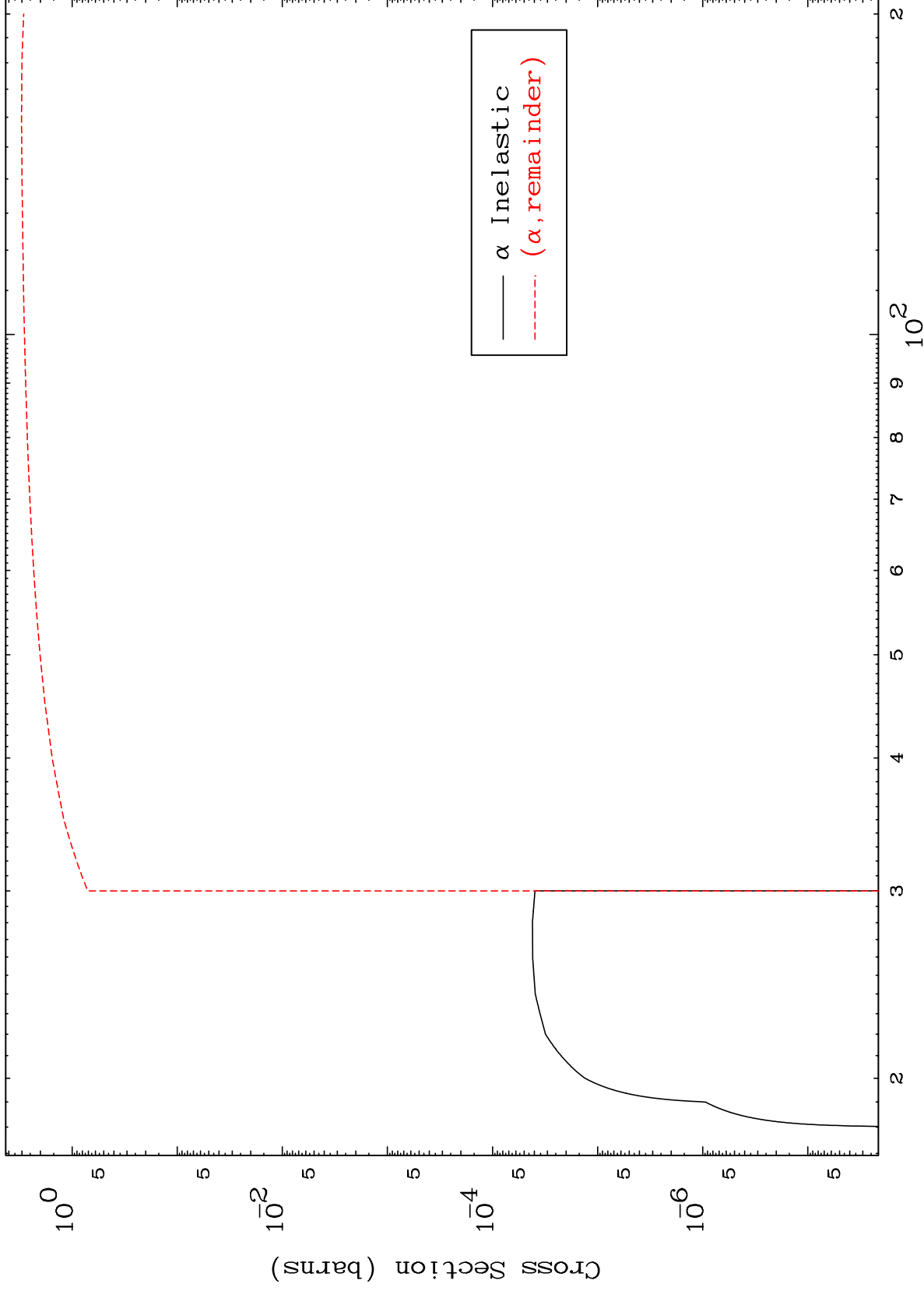
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9927

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-246



1

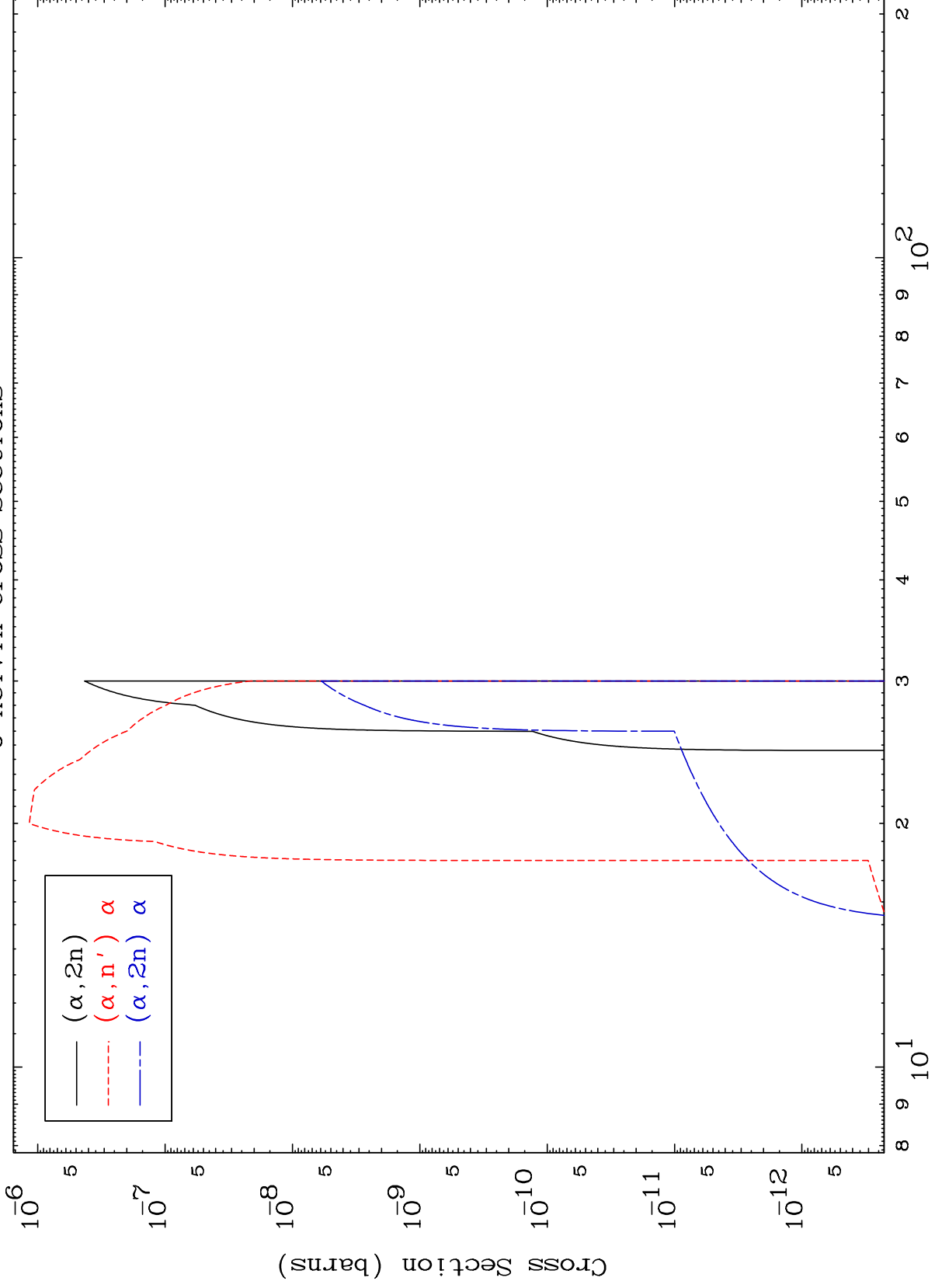
Incident Energy (MeV)

100-Fm-246

MAT 9927

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-246



( $\alpha, 2n$ )  
( $\alpha, n'$ )  $\alpha$   
( $\alpha, 2n$ )  $\alpha$

100-Fm-246

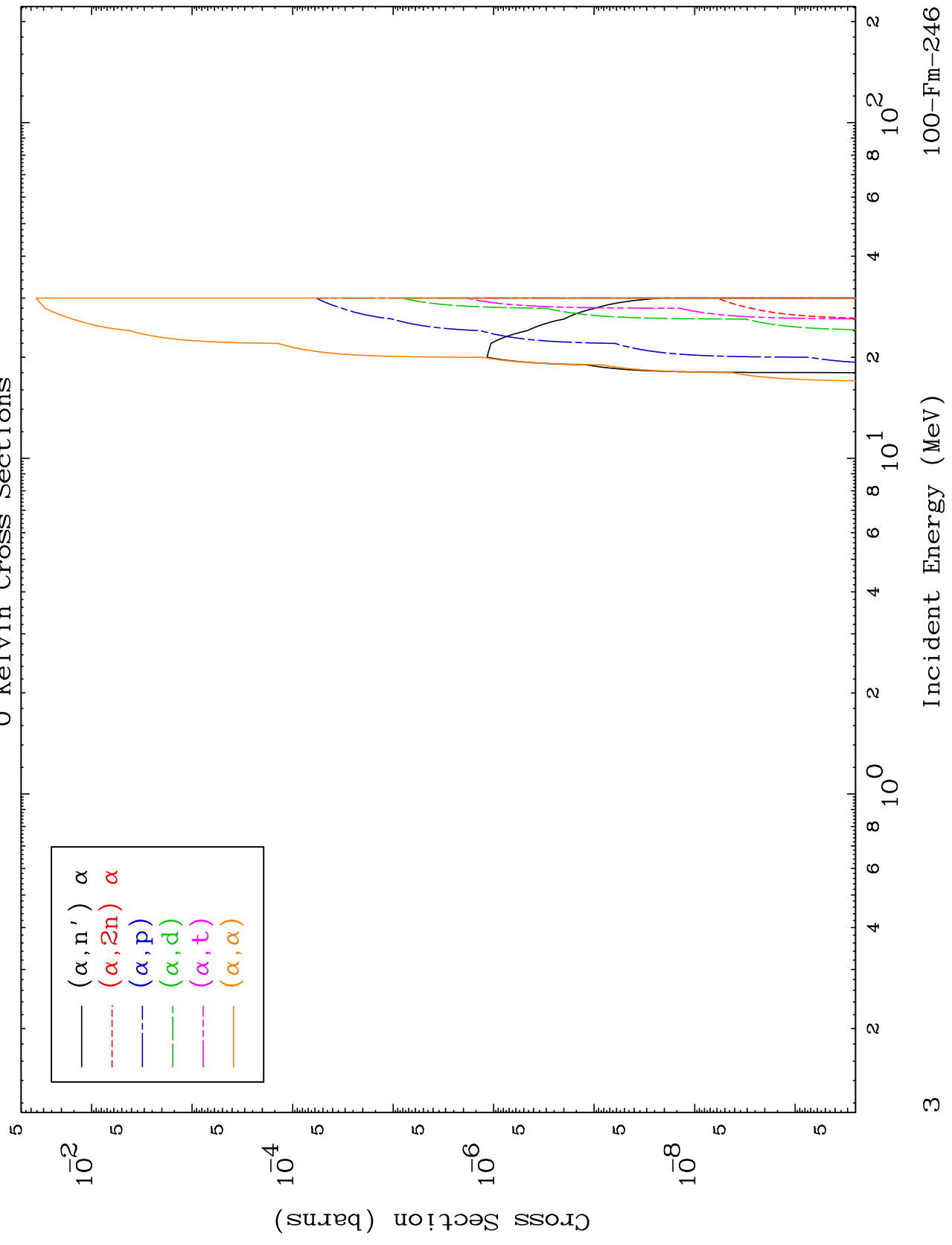
Incident Energy (MeV)

2

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$\alpha$  Charged Particle  
0 Kelvin Cross Sections

100-Fm-246

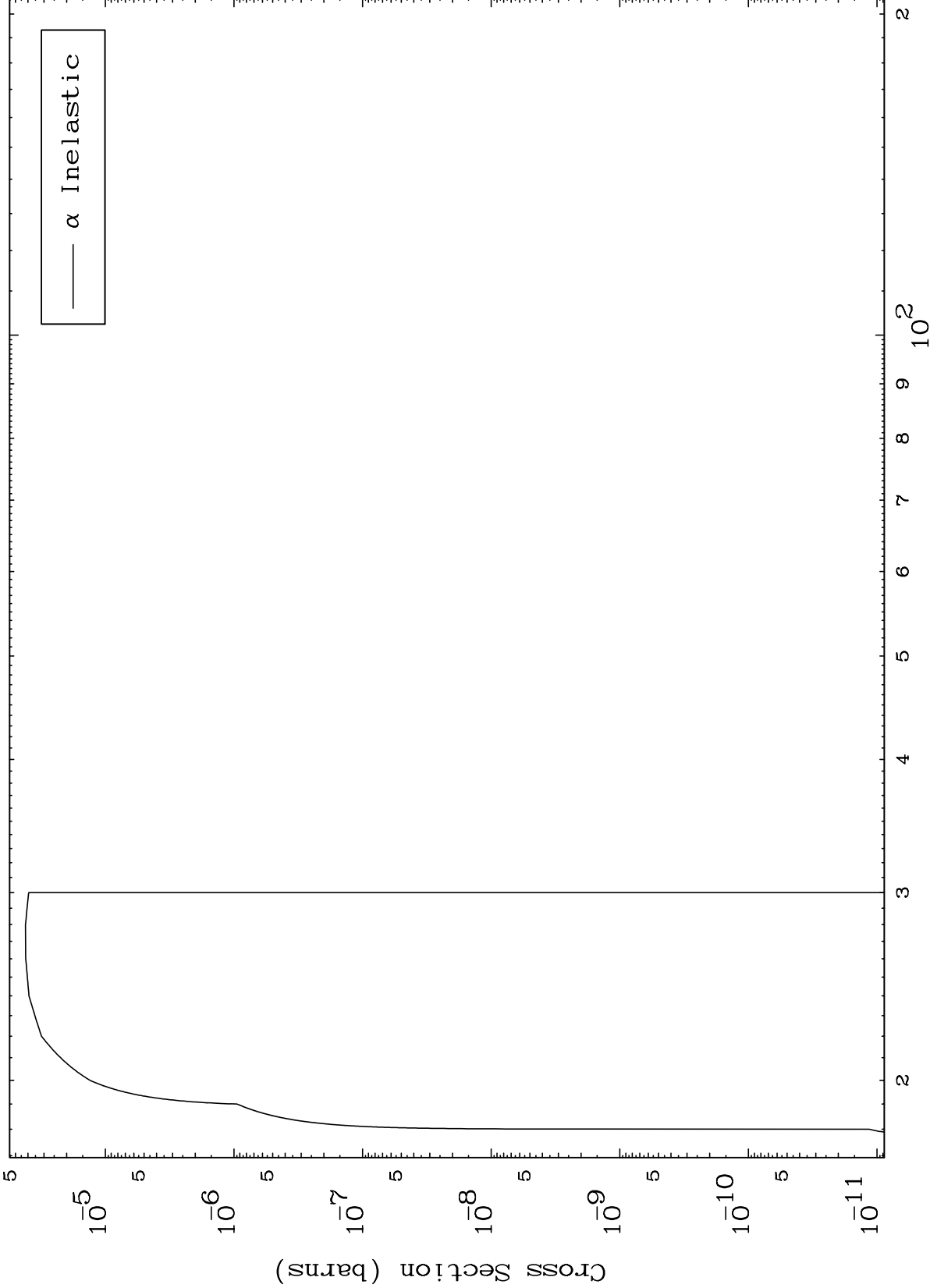


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( $\alpha, n'$ ) Level

100-Fm-246

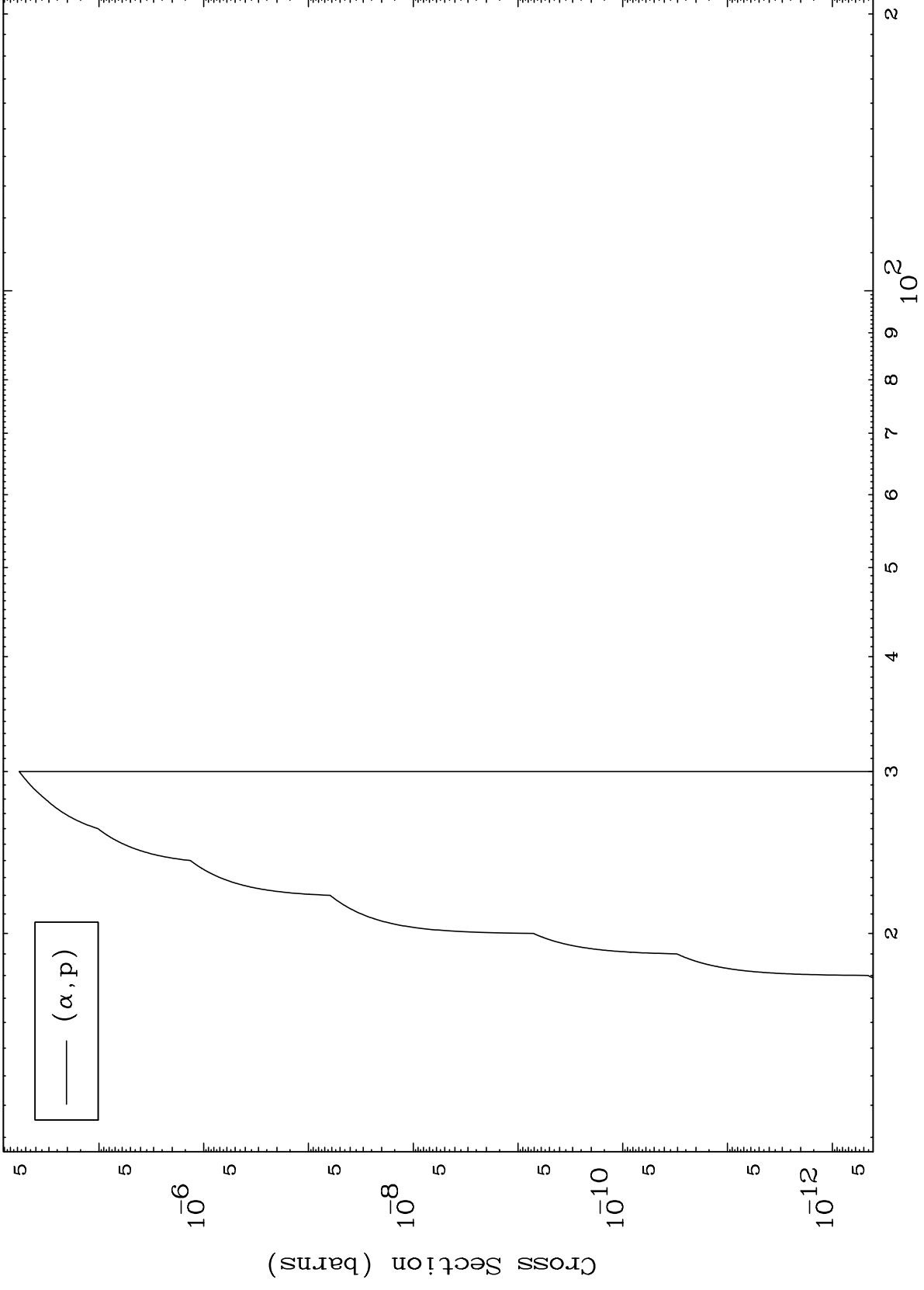
0 Kelvin Cross Sections

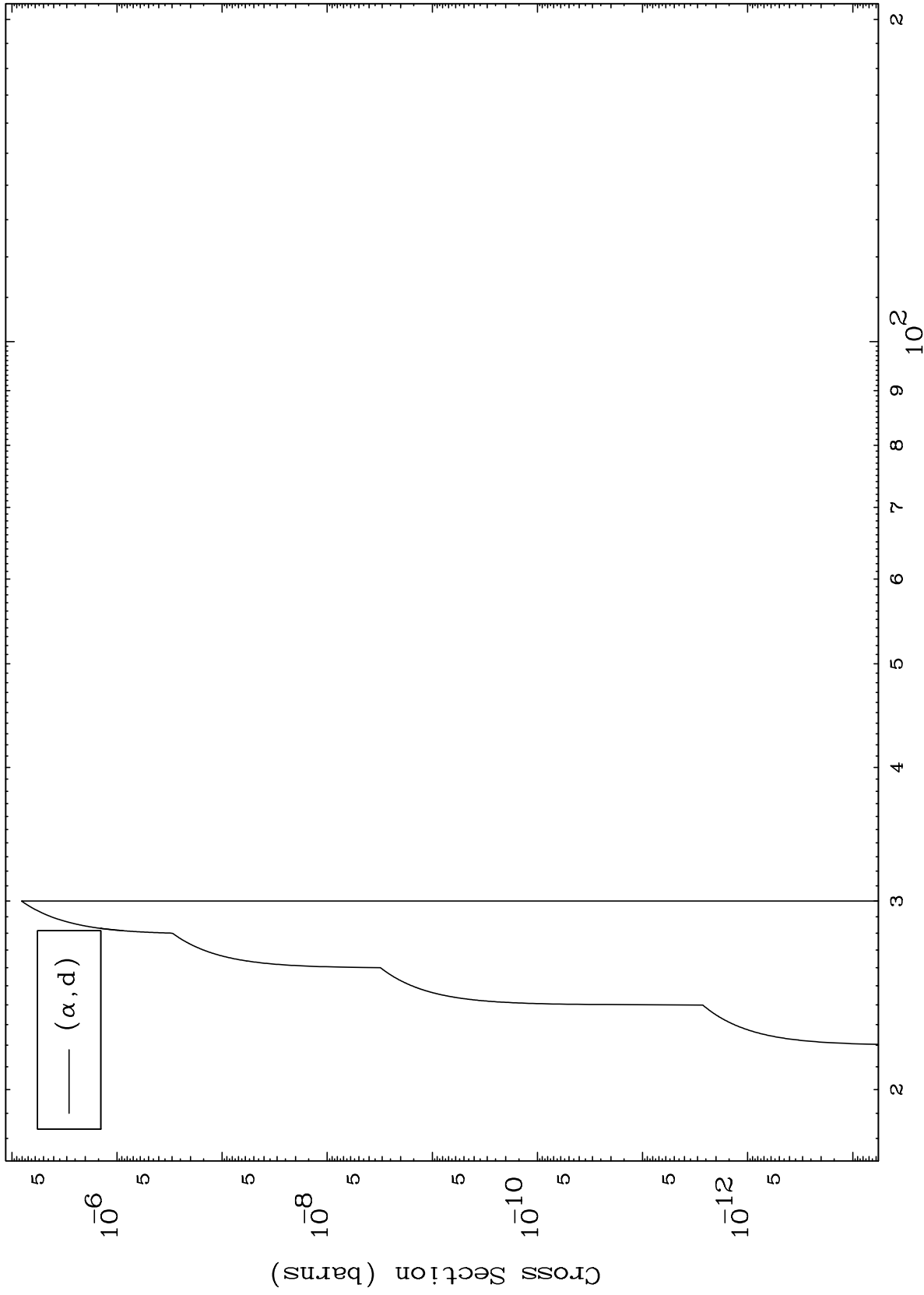


4

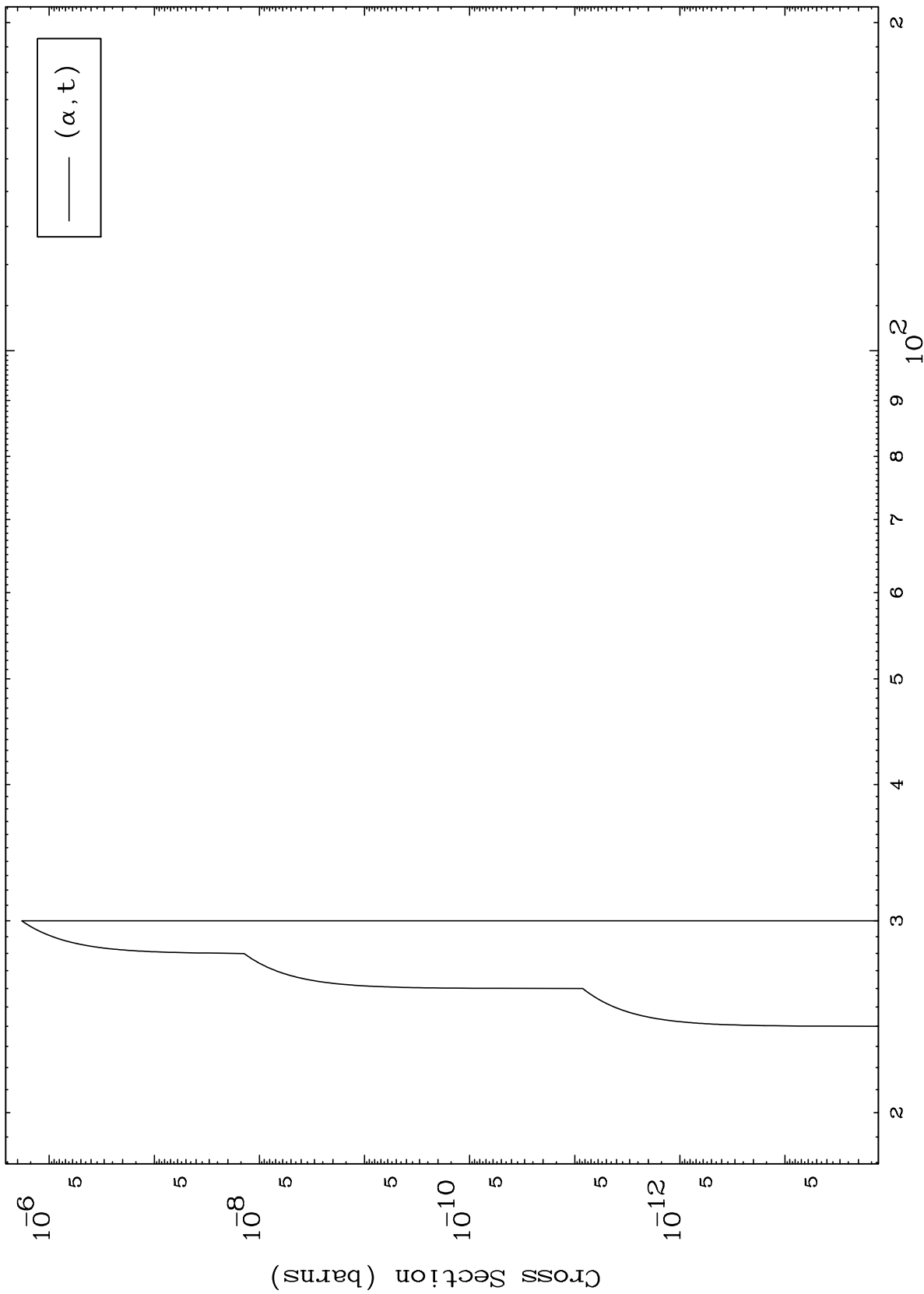
Incident Energy (MeV)

100-Fm-246





( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



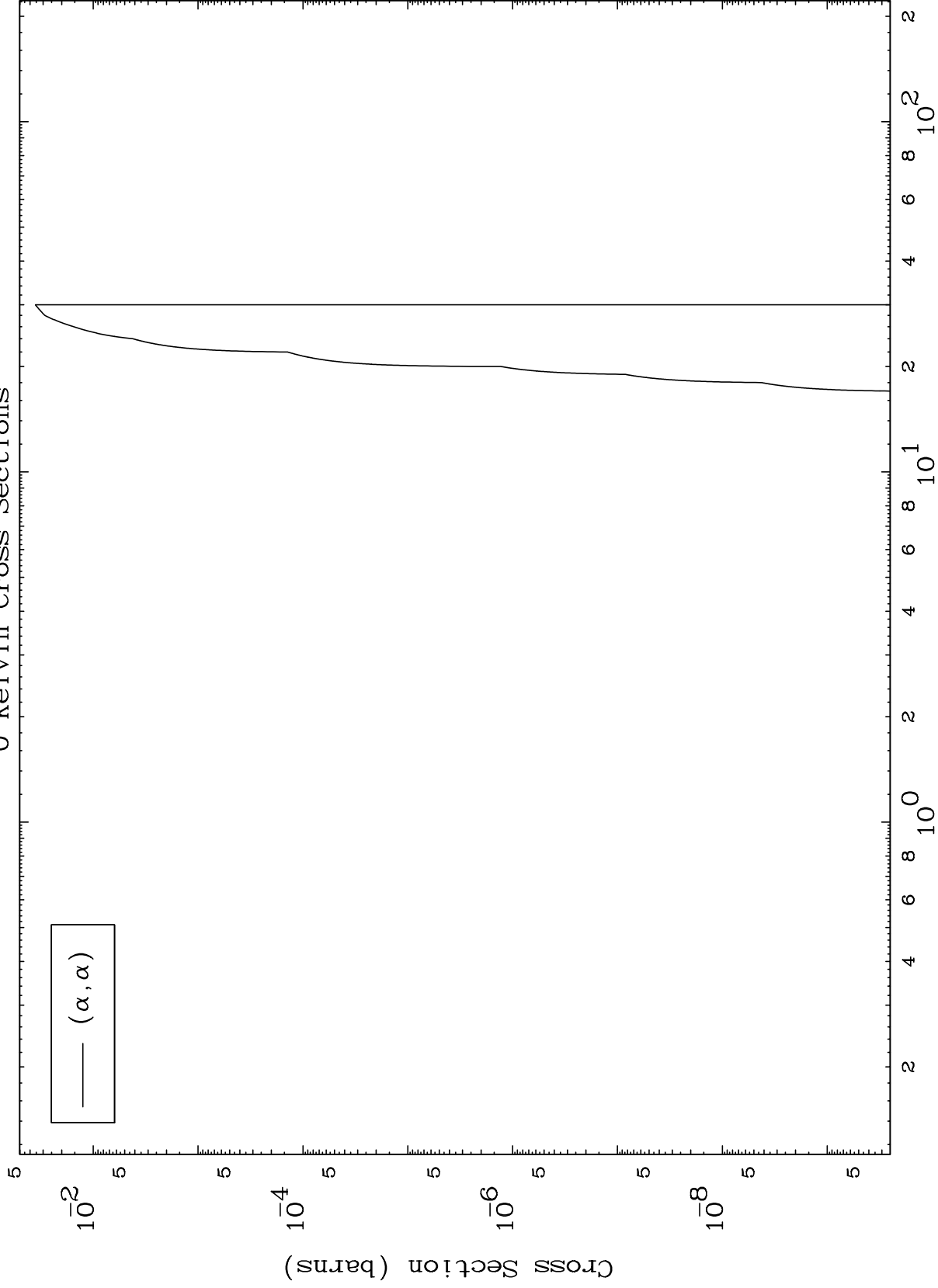


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( $\alpha, \alpha$ ) Levels

100-Fm-246

0 Kelvin Cross Sections

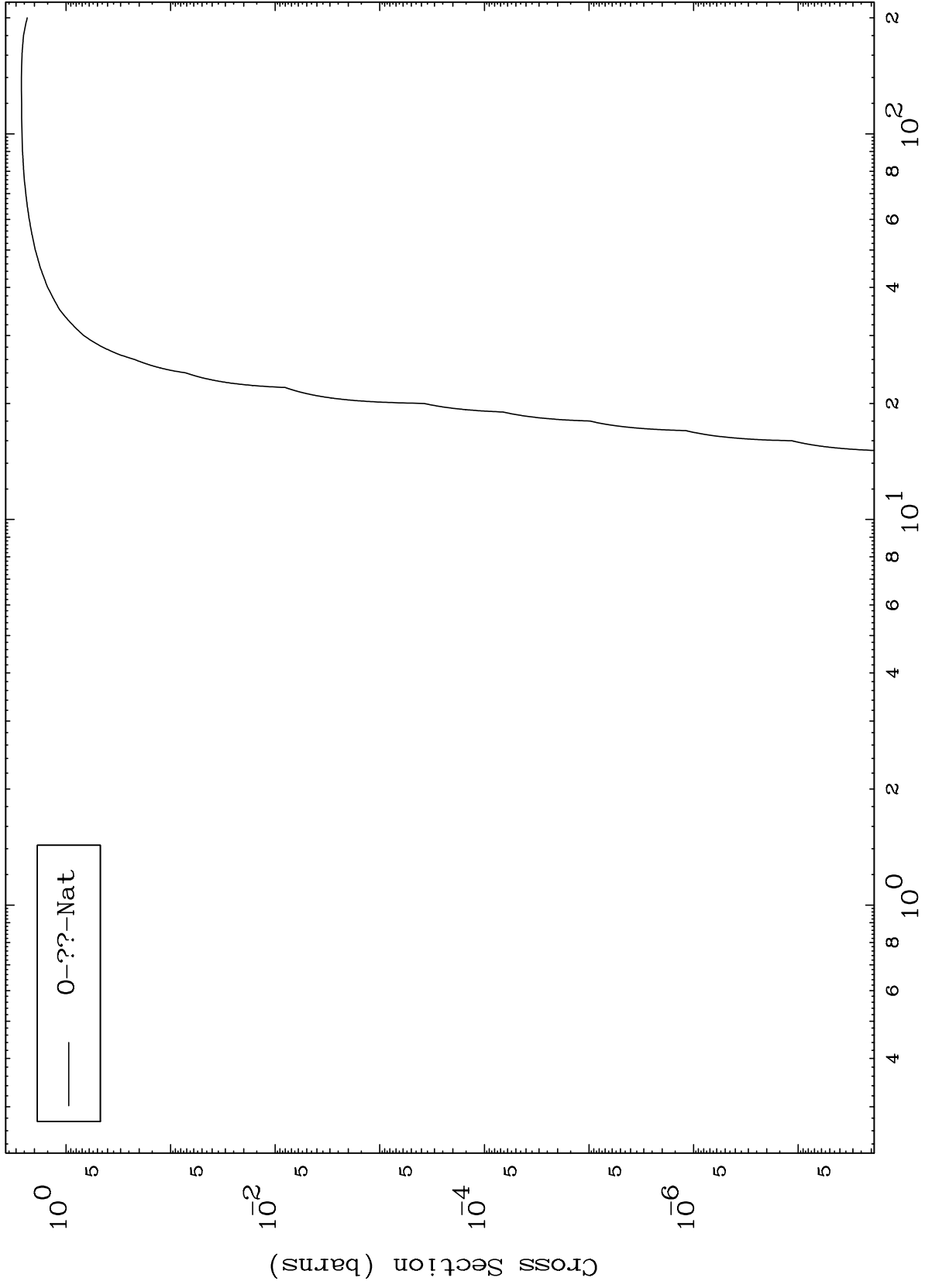


MAT 9927

100-Fm-246

$\alpha$  Fission

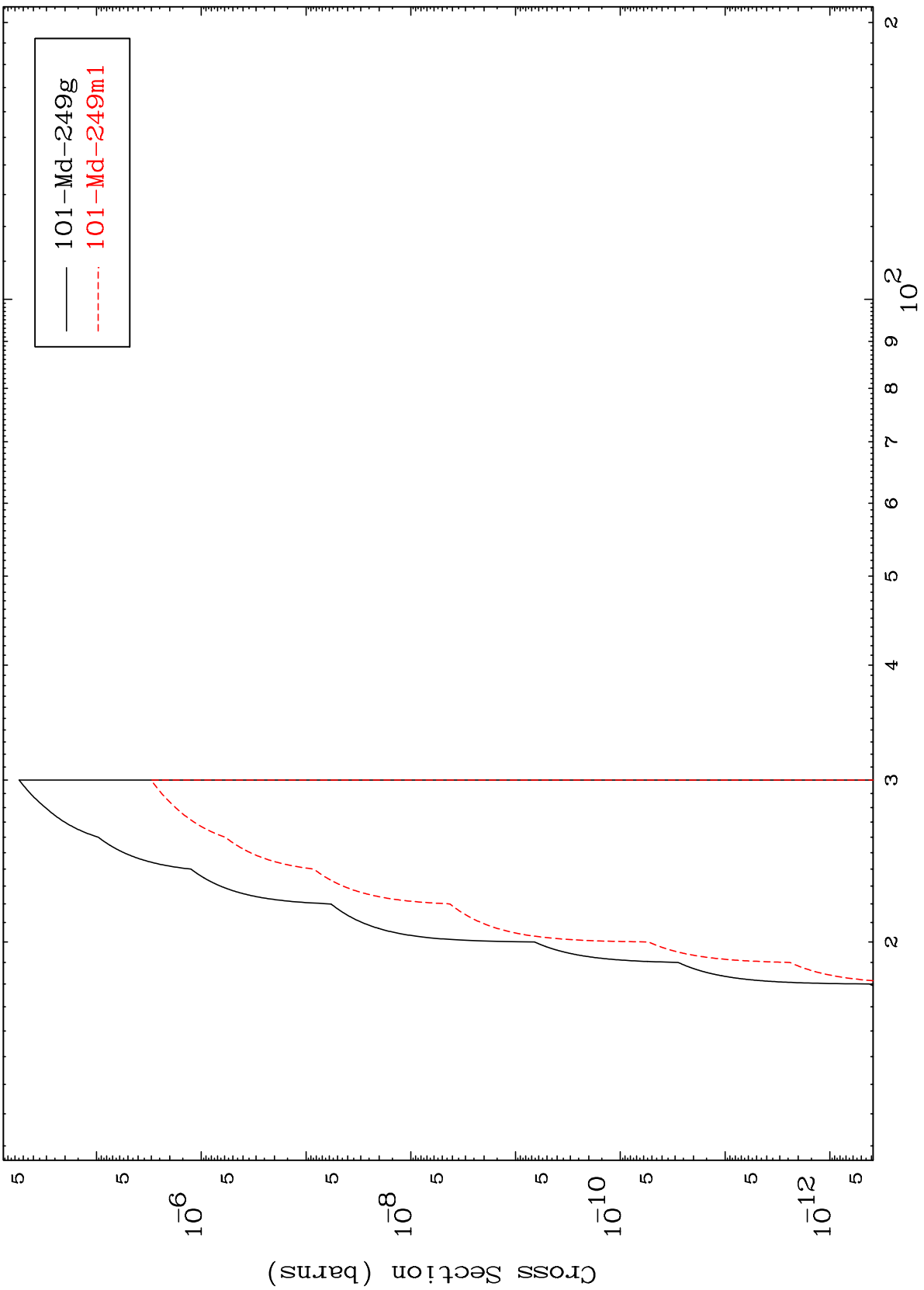
Radionuclide Production Cross Section



MAT 9927

100-Fm-246

( $\alpha, p$ )  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

100-Fm-246