

Program EVALPLOT  
(Version 2018-1)

by

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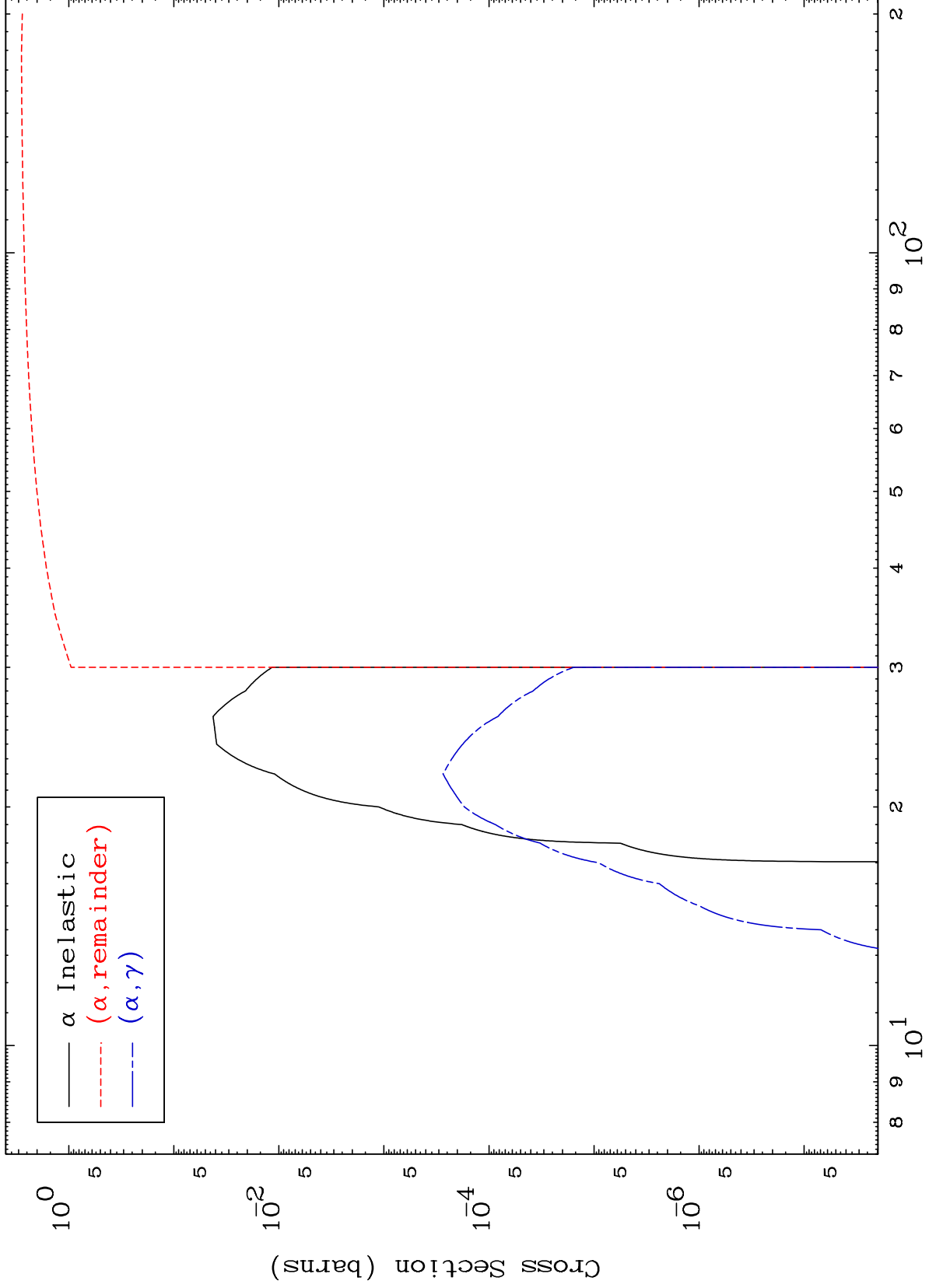
Press Mouse Button to Start

MAT 8716

$\alpha$  Major

87-Fr-209

0 Kelvin Cross Sections



1

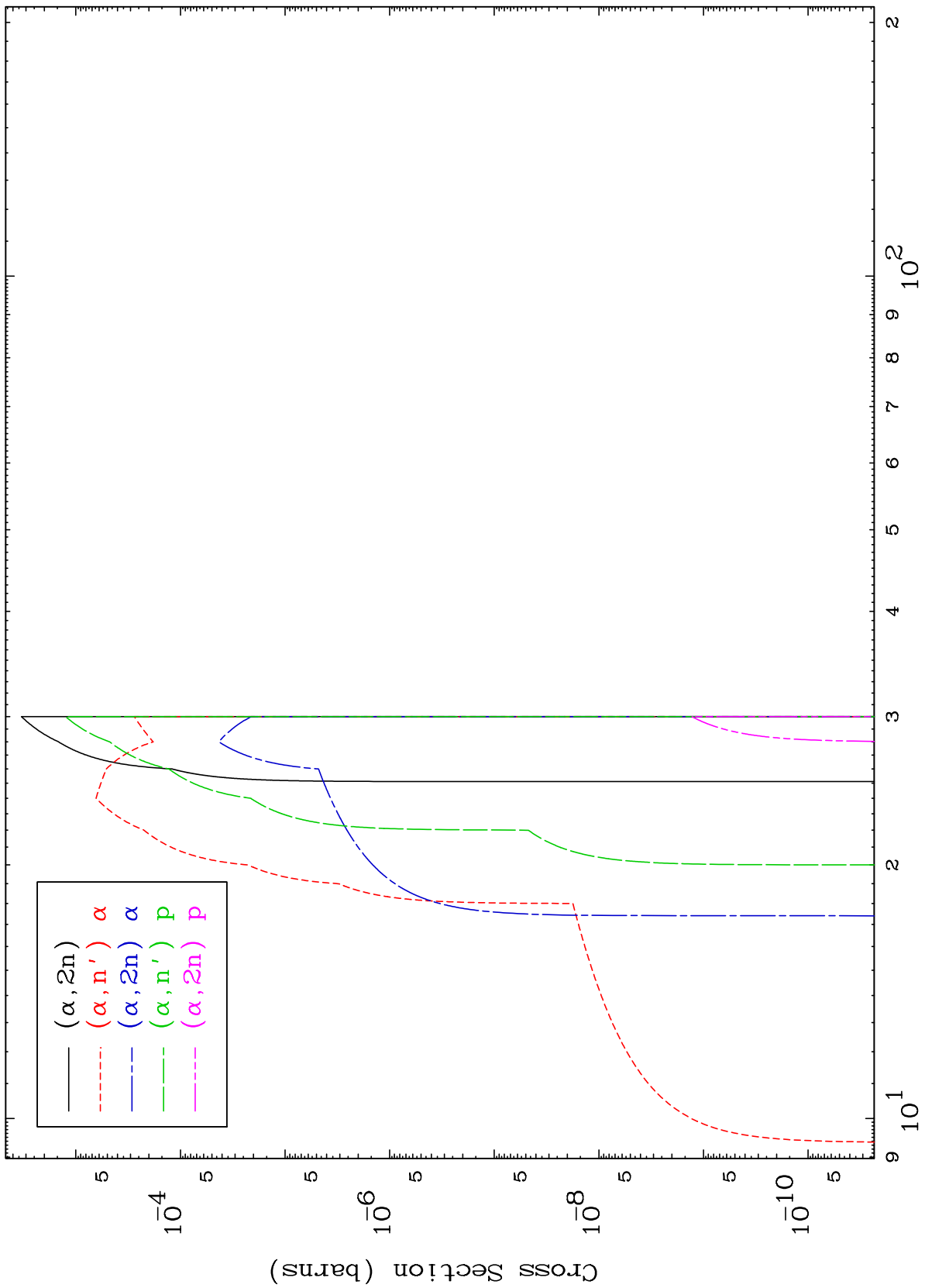
Incident Energy (MeV)

87-Fr-209

MAT 8716

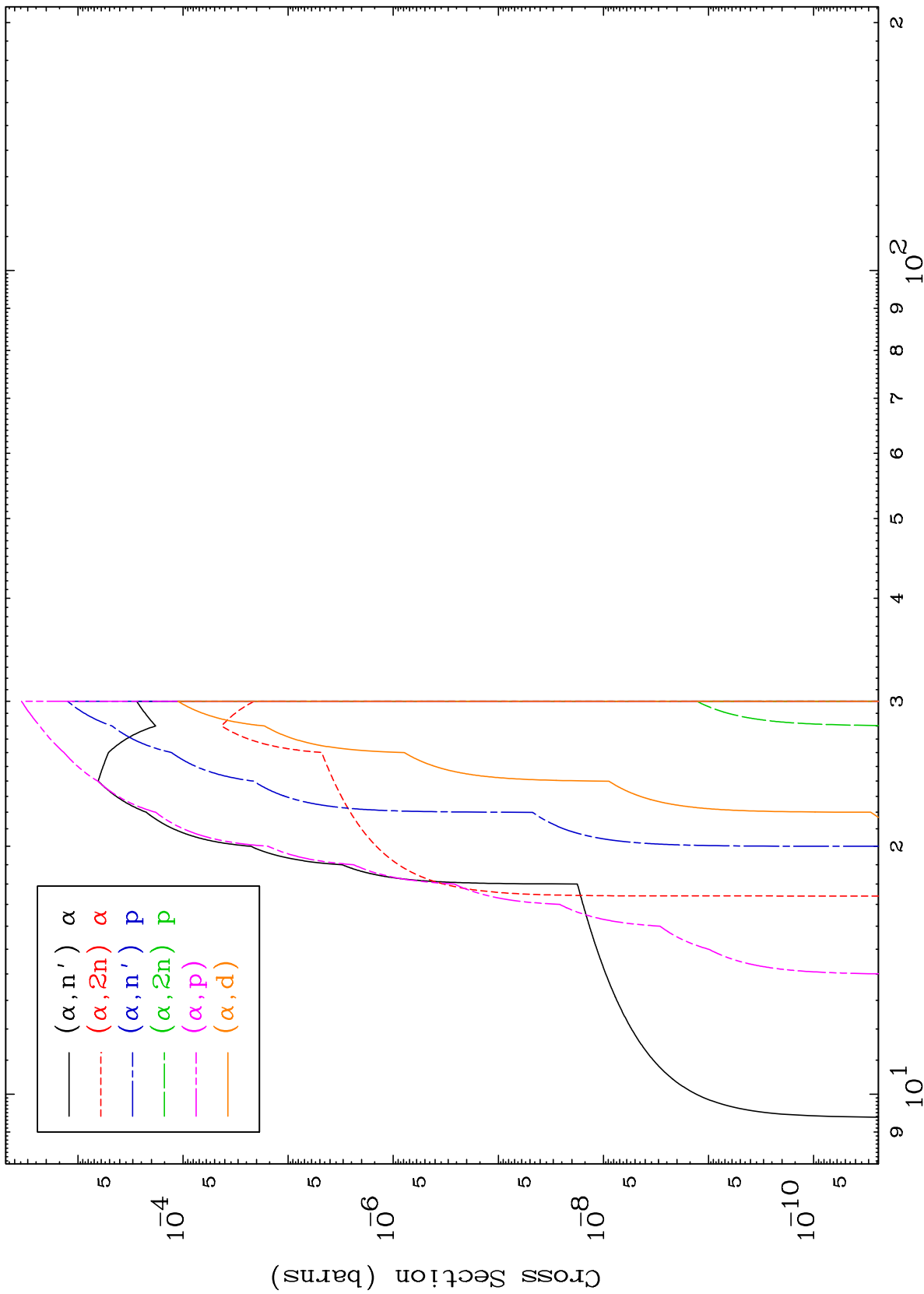
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

87-Fr-209



Incident Energy (MeV)

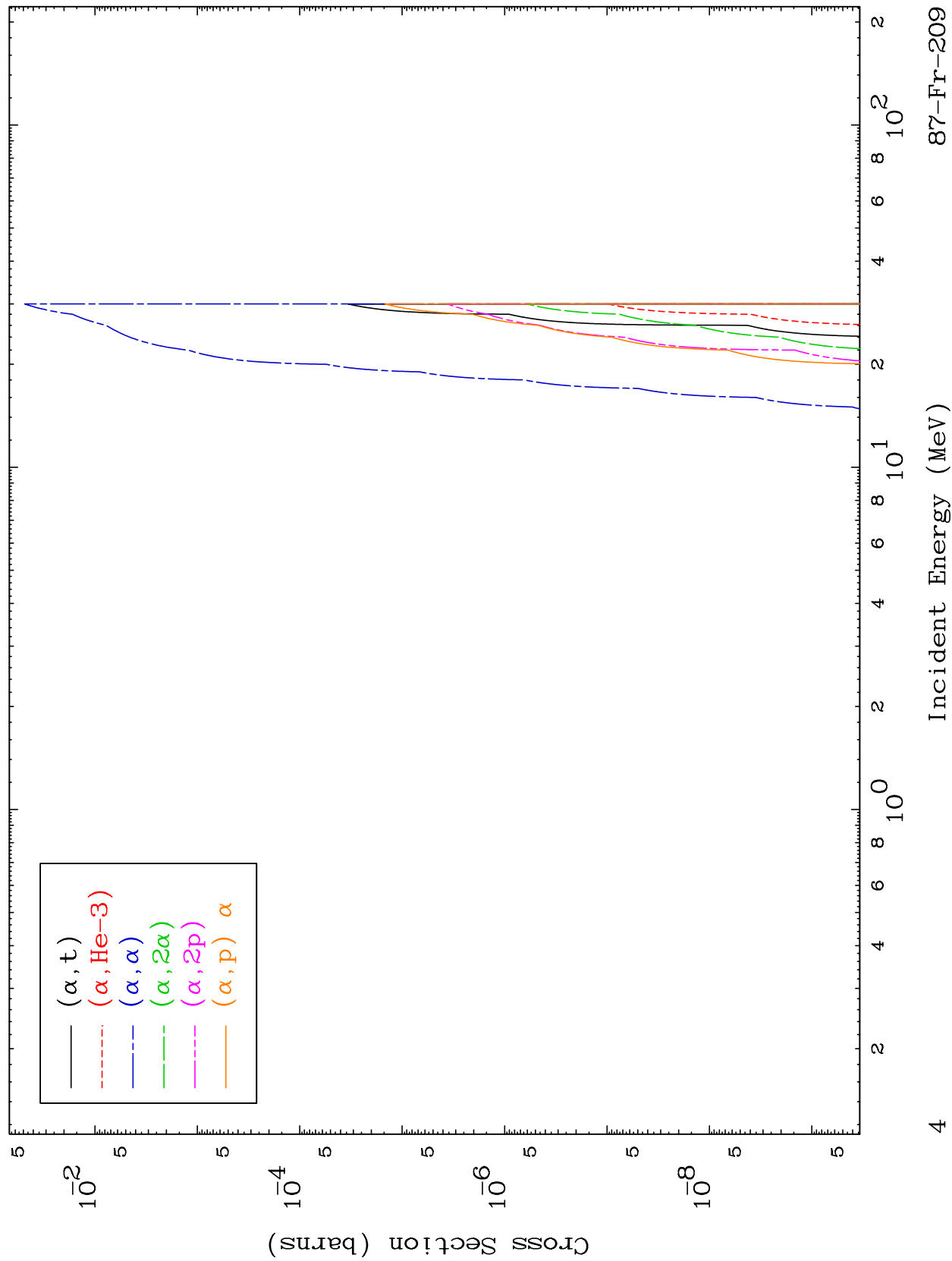
87-Fr-209



MAT 8716

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

87-Fr-209

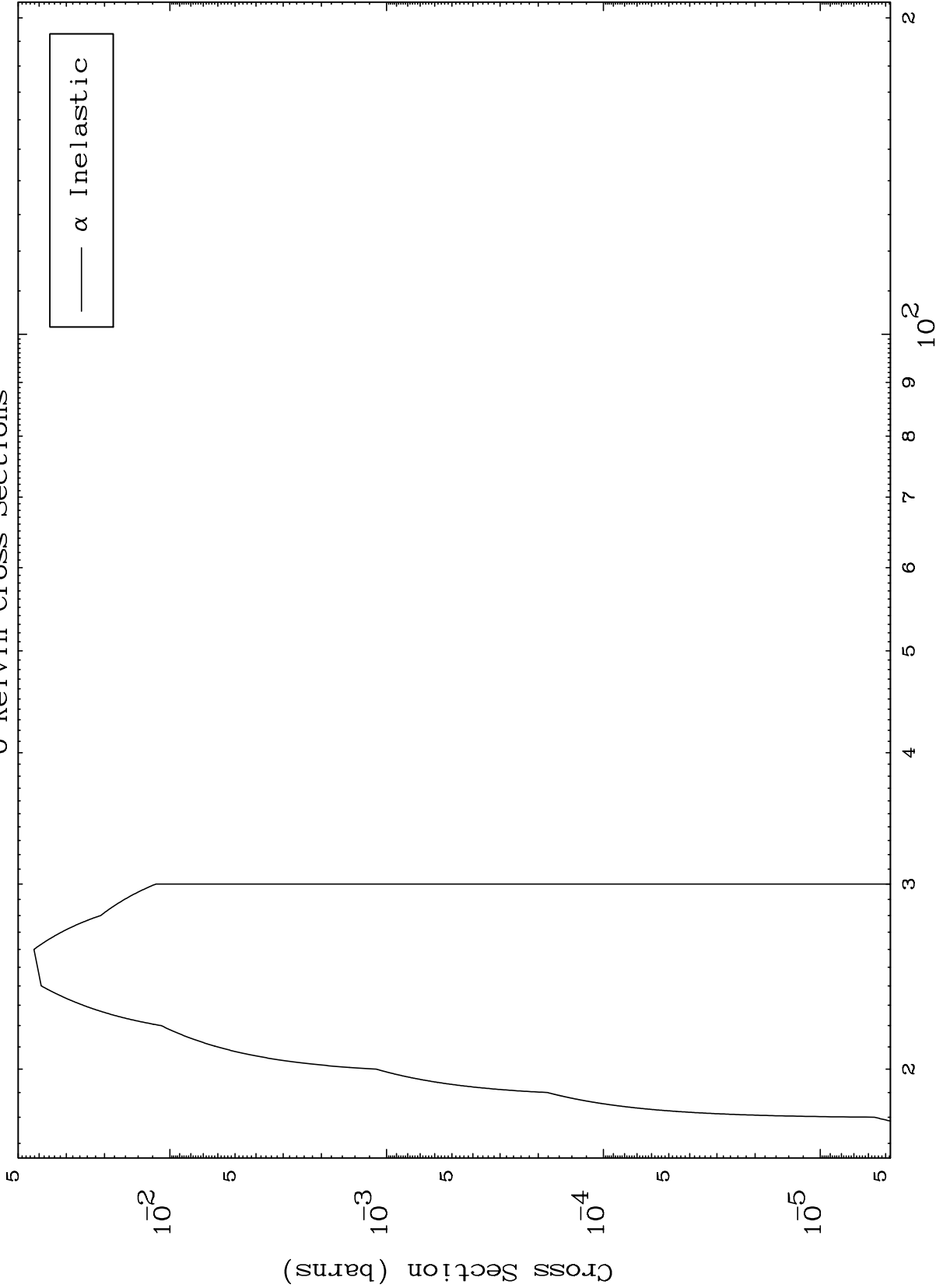


MAT 8716

( $\alpha, n'$ ) Level

87-Fr-209

0 Kelvin Cross Sections



$\alpha$  Inelastic

Incident Energy (MeV)

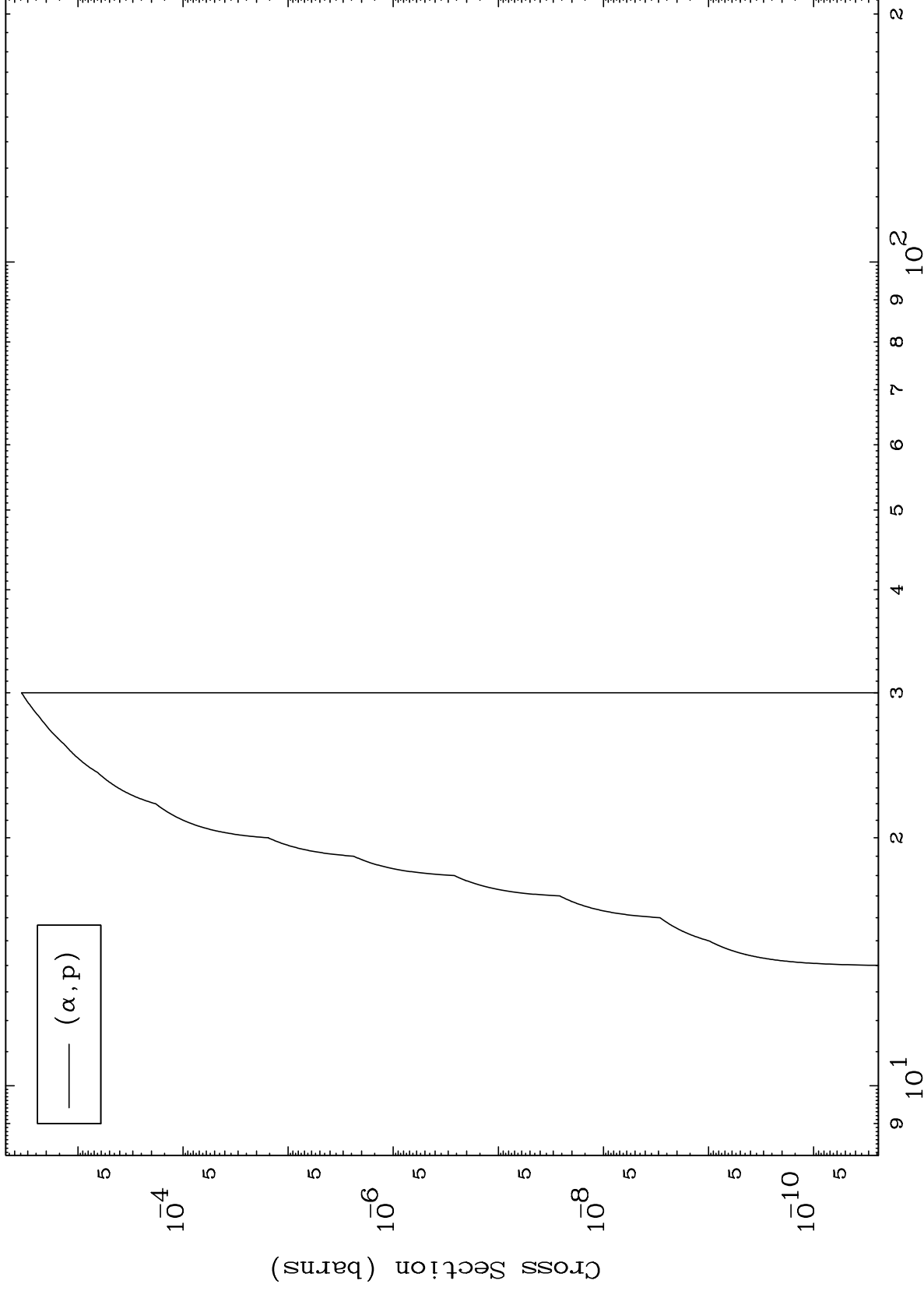
87-Fr-209

5

MAT 8716

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-209



Incident Energy (MeV)

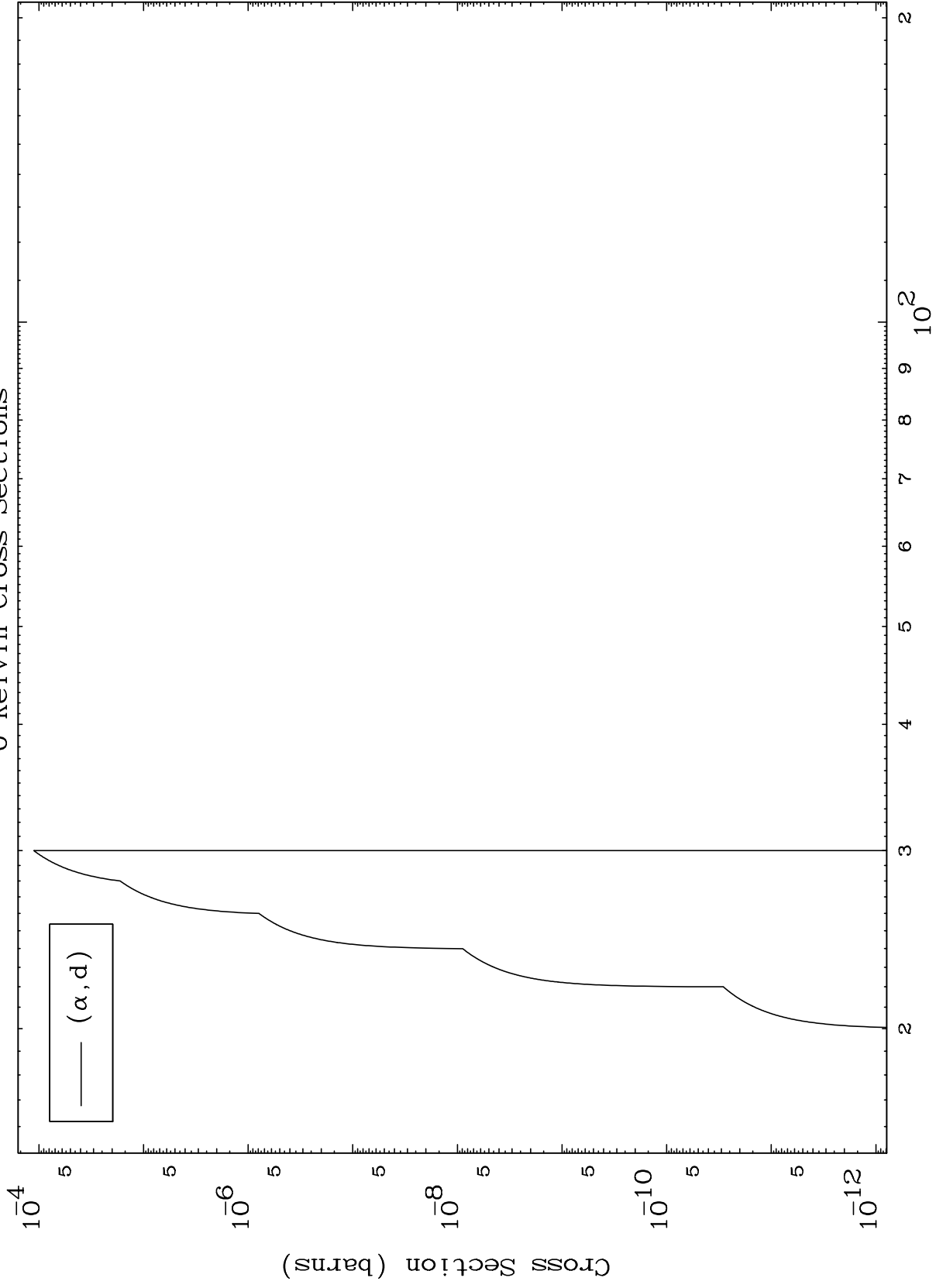
87-Fr-209

6

MAT 8716

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

87-Fr-209

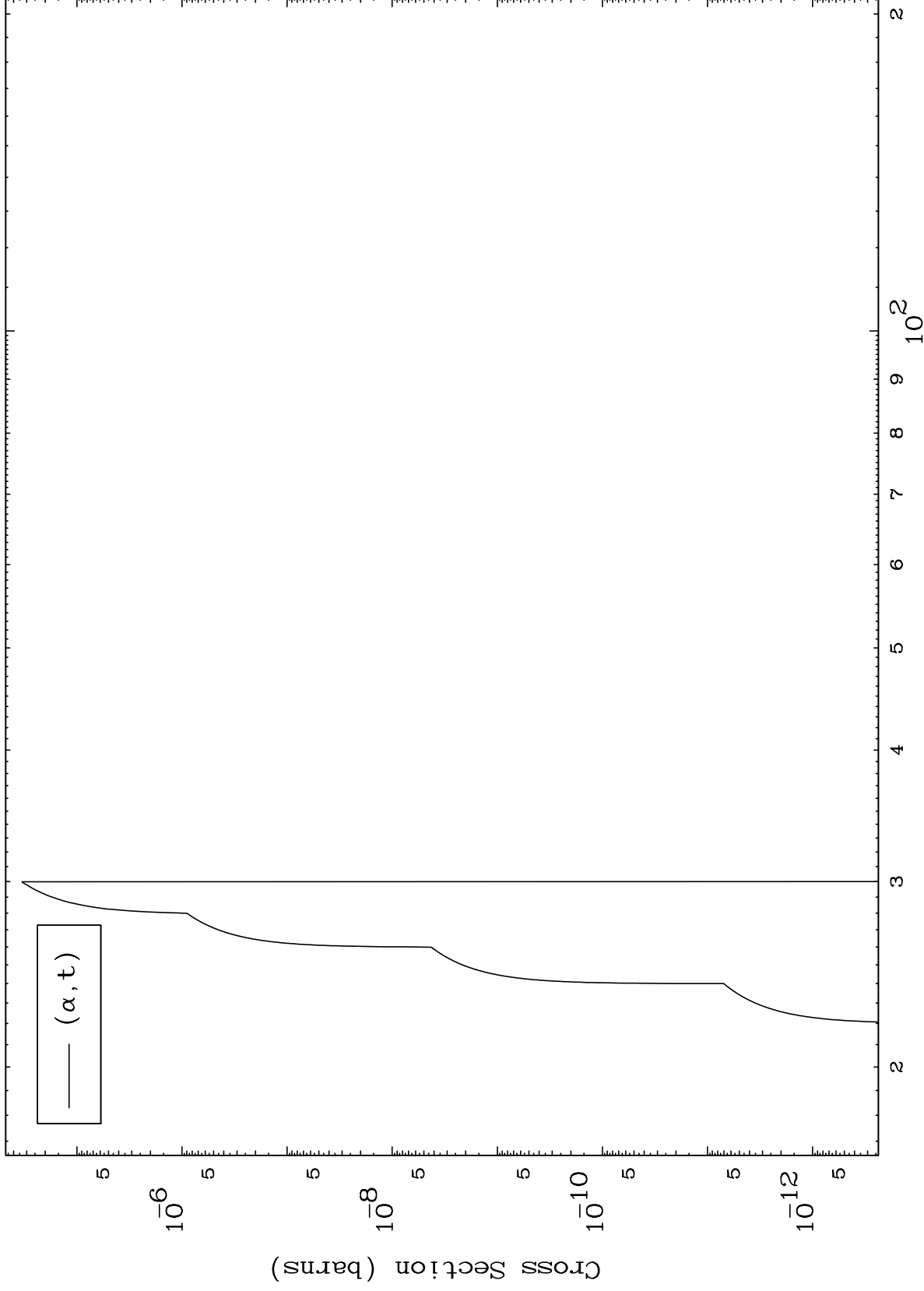




MAT 8716

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

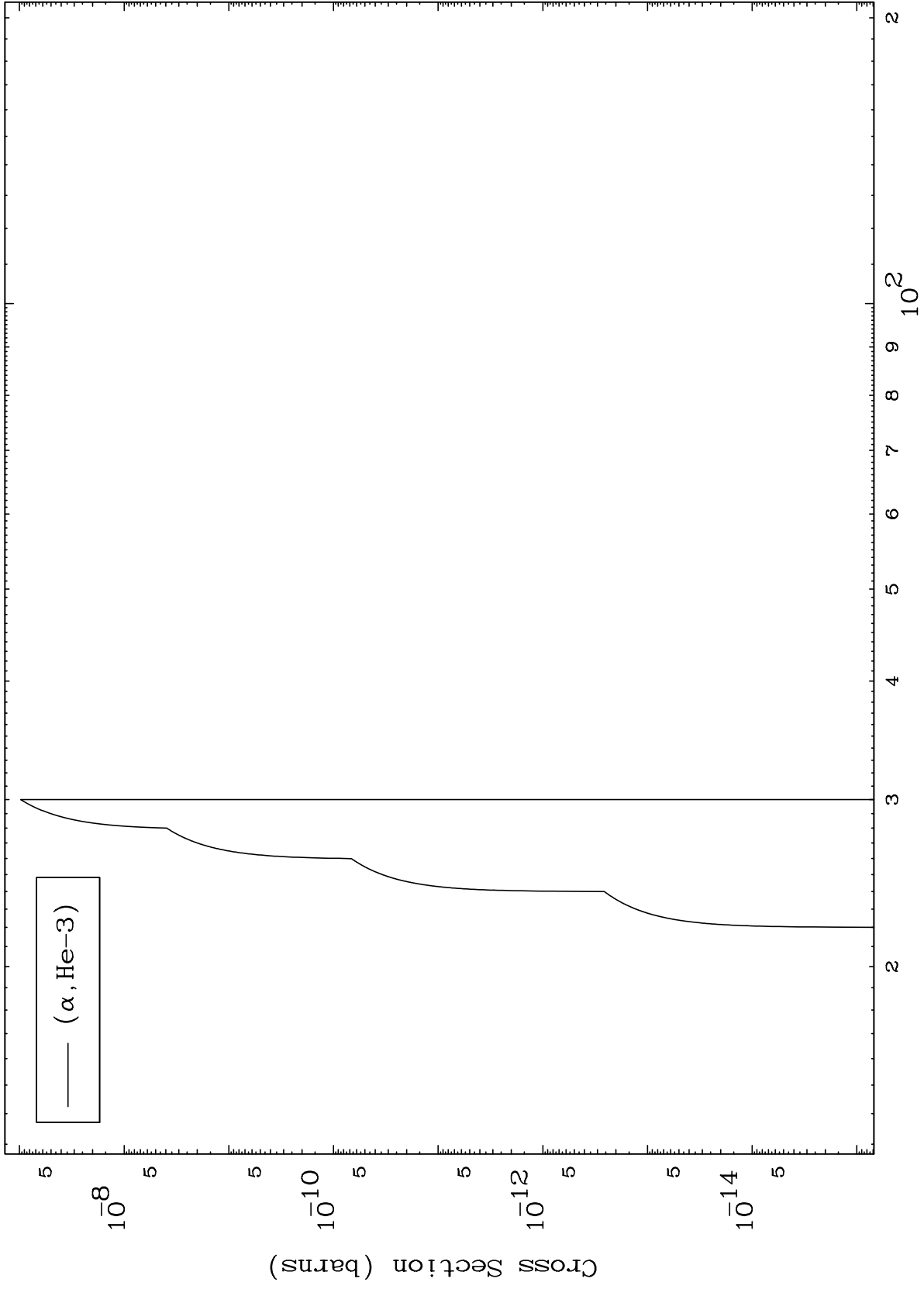
87-Fr-209



8

Incident Energy (MeV)

87-Fr-209

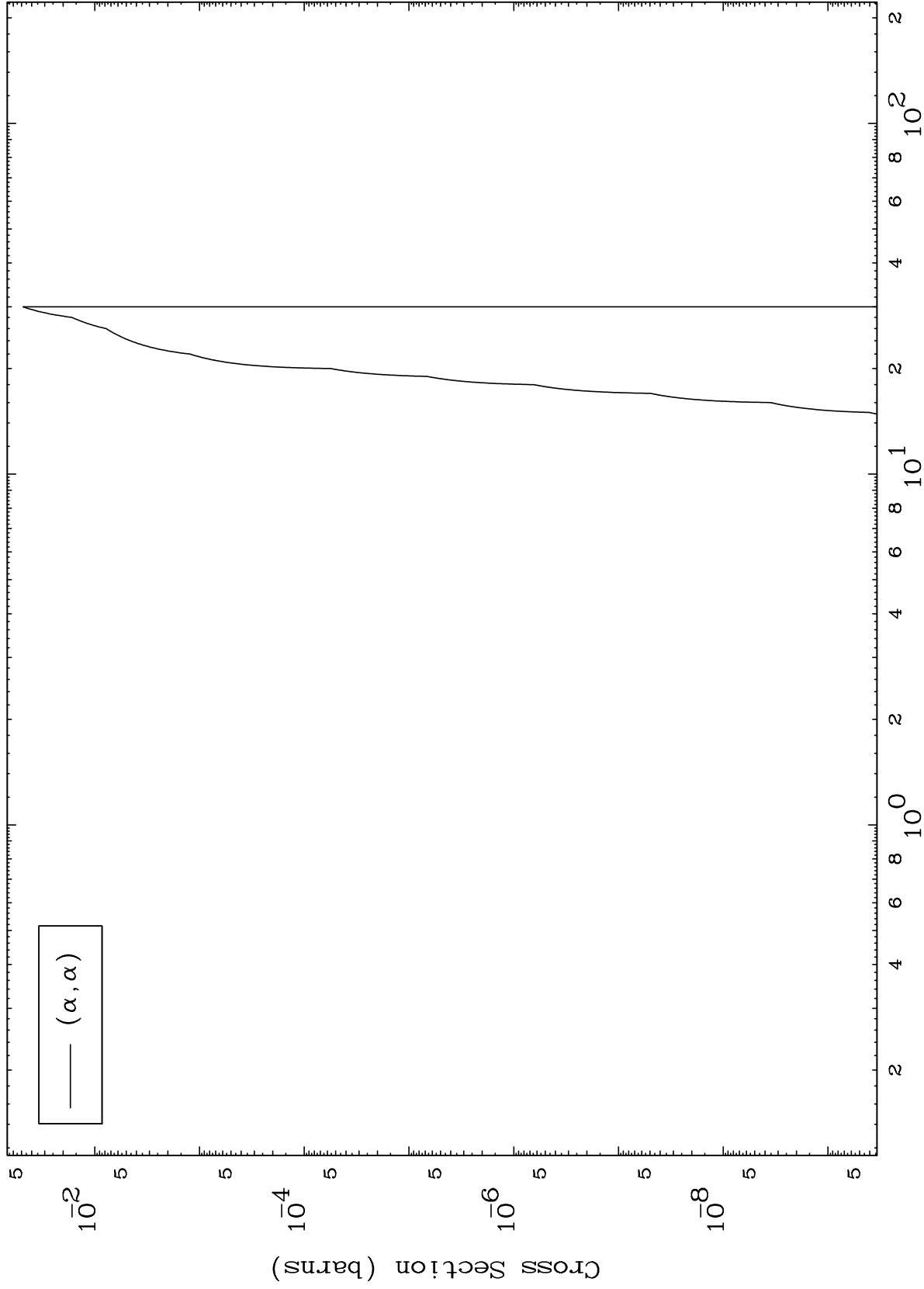


MAT 8716

( $\alpha, \alpha$ ) Levels

87-Fr-209

0 Kelvin Cross Sections



10

Incident Energy (MeV)

87-Fr-209

MAT 8716

87-Fr-209

$\alpha$  Fission  
Radionuclide Production Cross Section

