

Program EVALPLOT  
(Version 2018-1)

by

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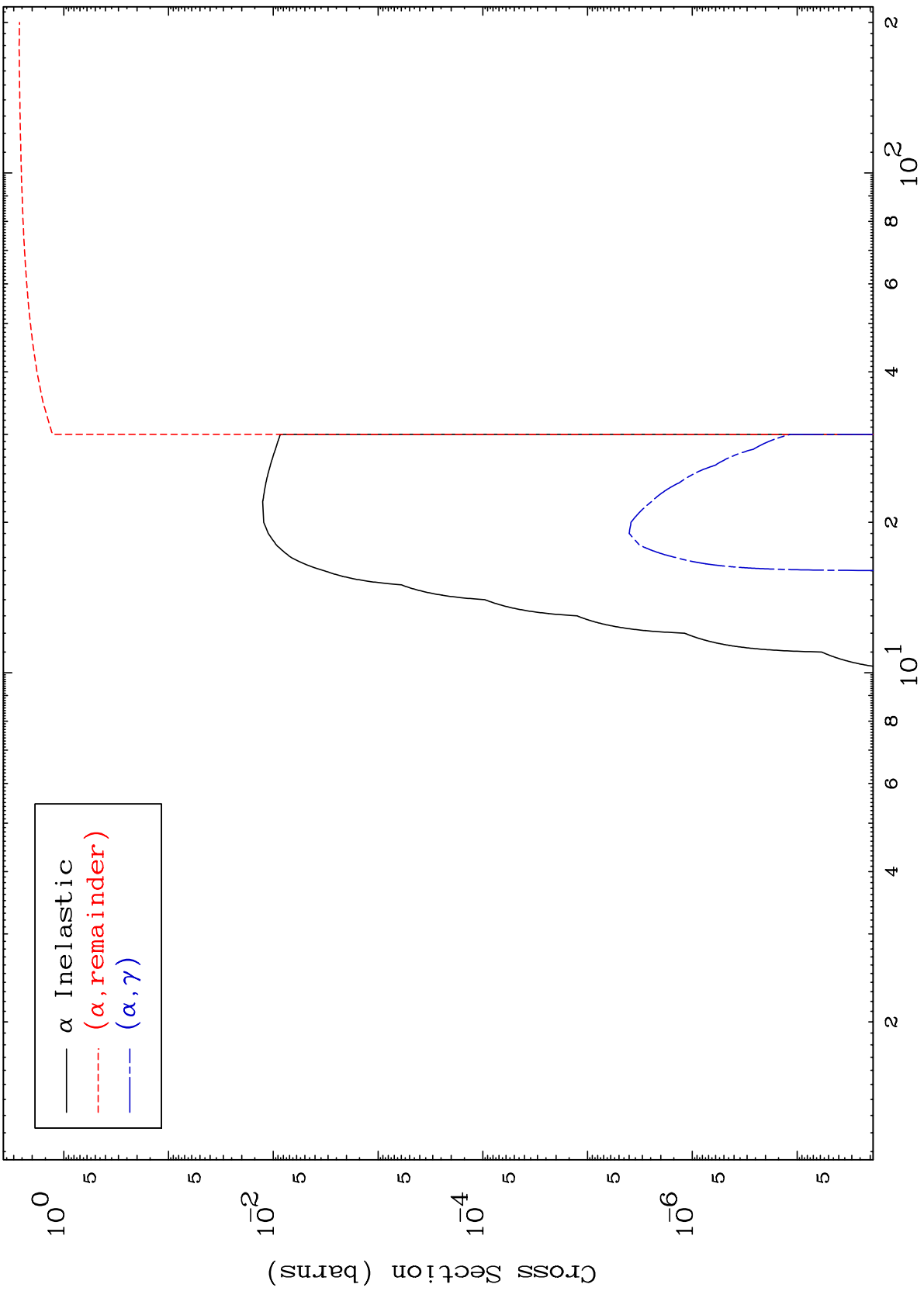
Press Mouse Button to Start

MAT 7249

$\alpha$  Major

72-Hf-182

0 Kelvin Cross Sections

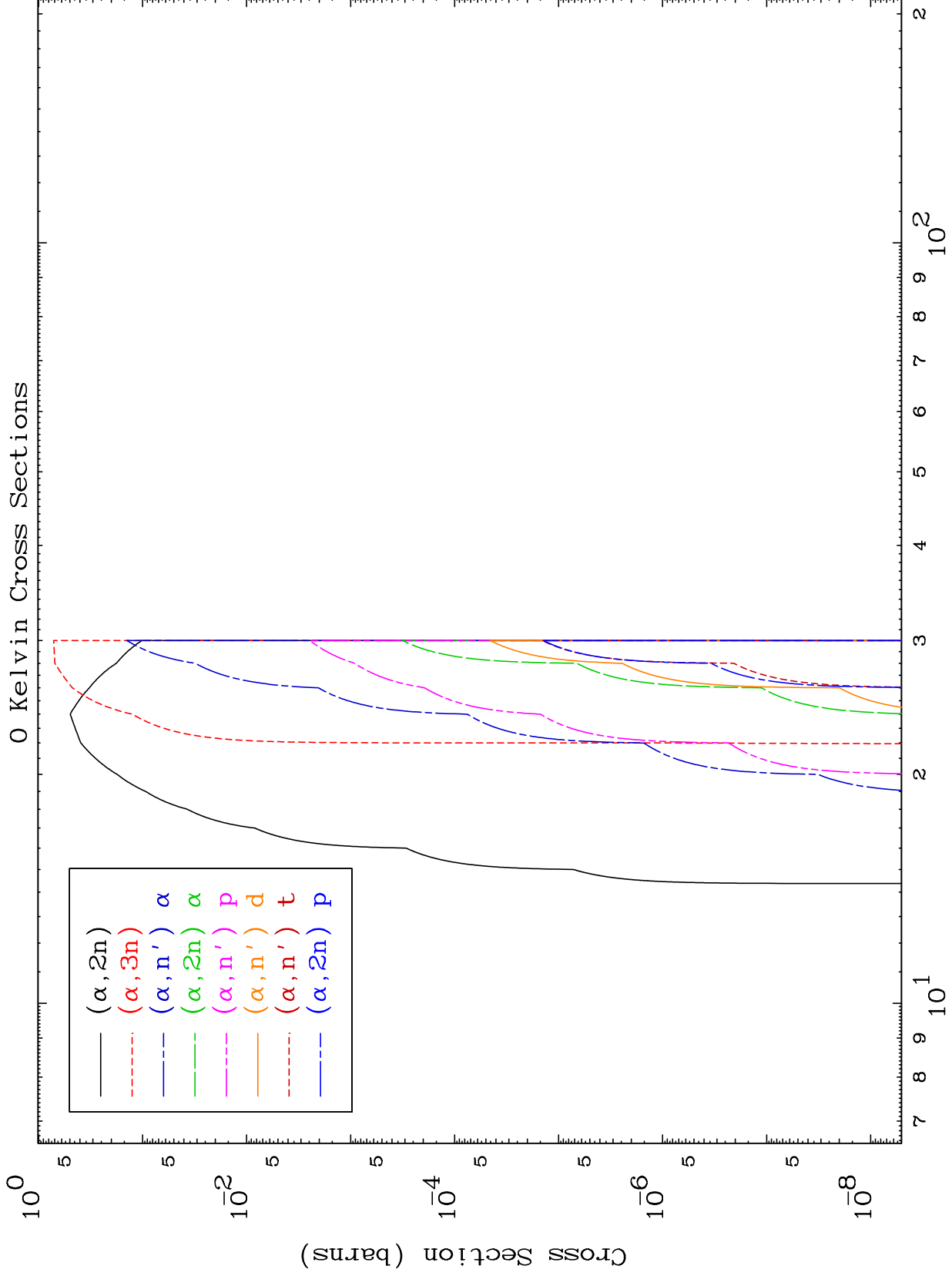


$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha$ ,  $\gamma$ )

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$\alpha$  Neutron Production  
0 Kelvin Cross Sections

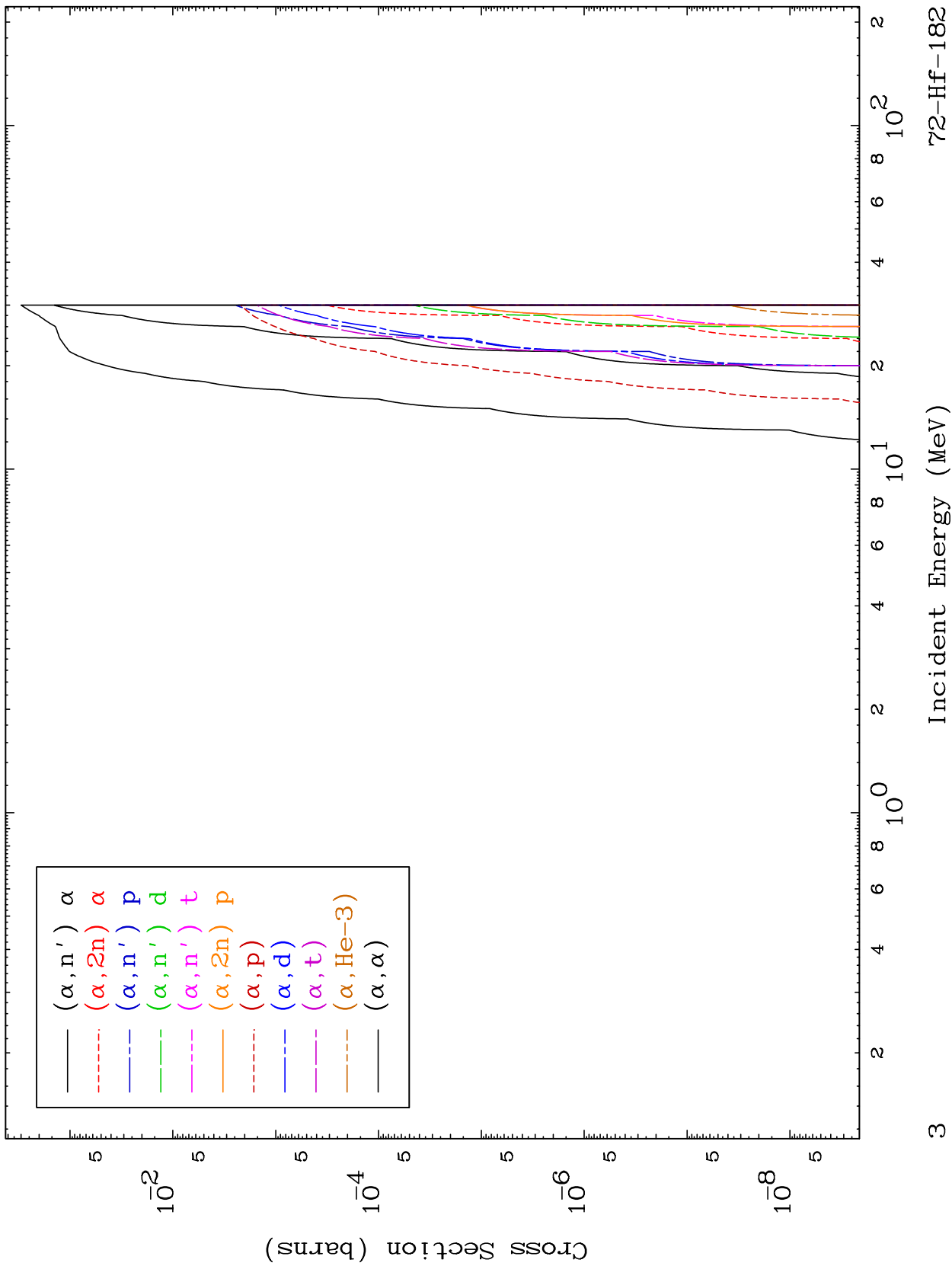
72-Hf-182



2

Incident Energy (MeV)

72-Hf-182

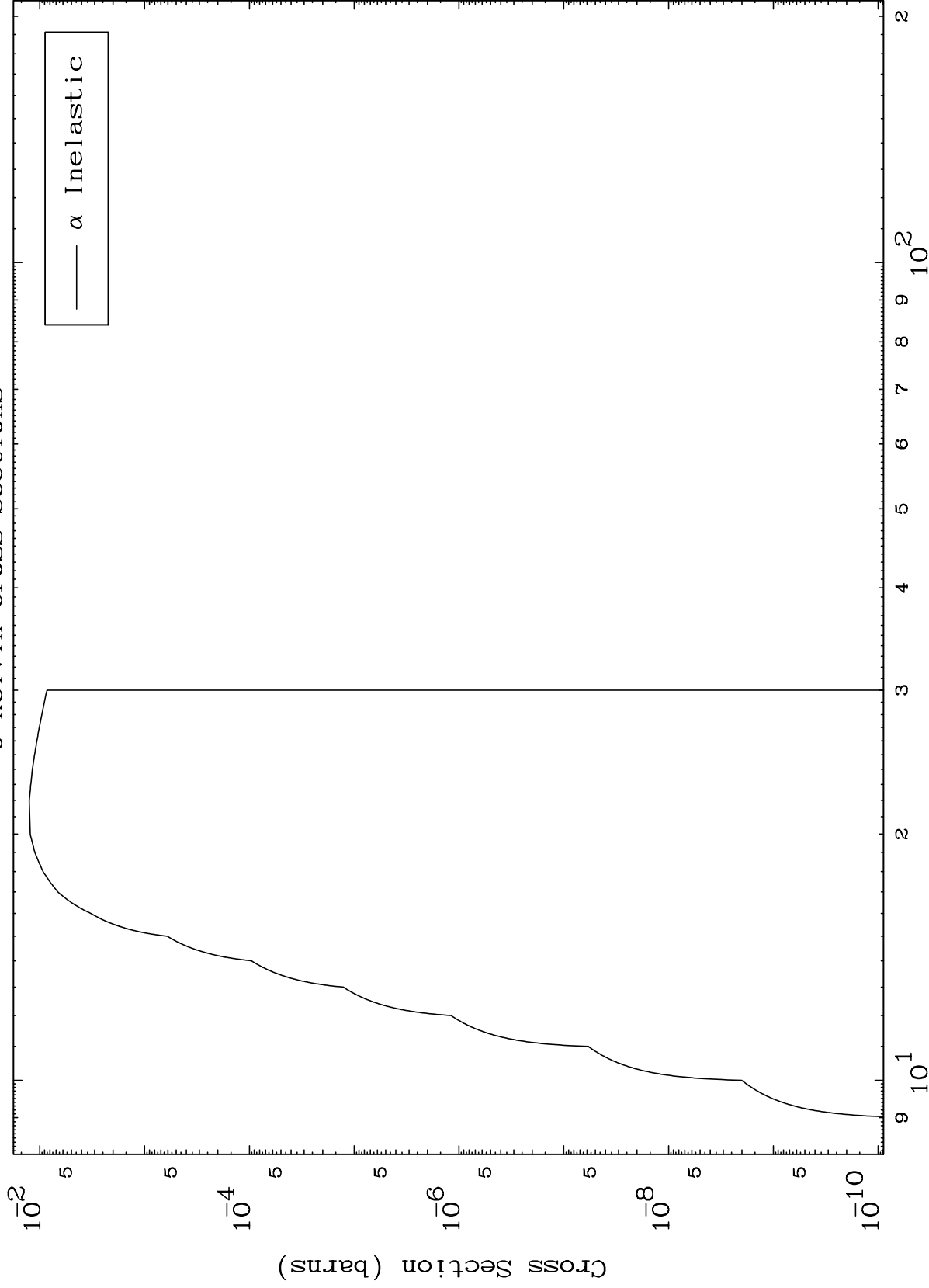


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( $\alpha, n'$ ) Level

72-Hf-182

0 Kelvin Cross Sections



4

Incident Energy (MeV)

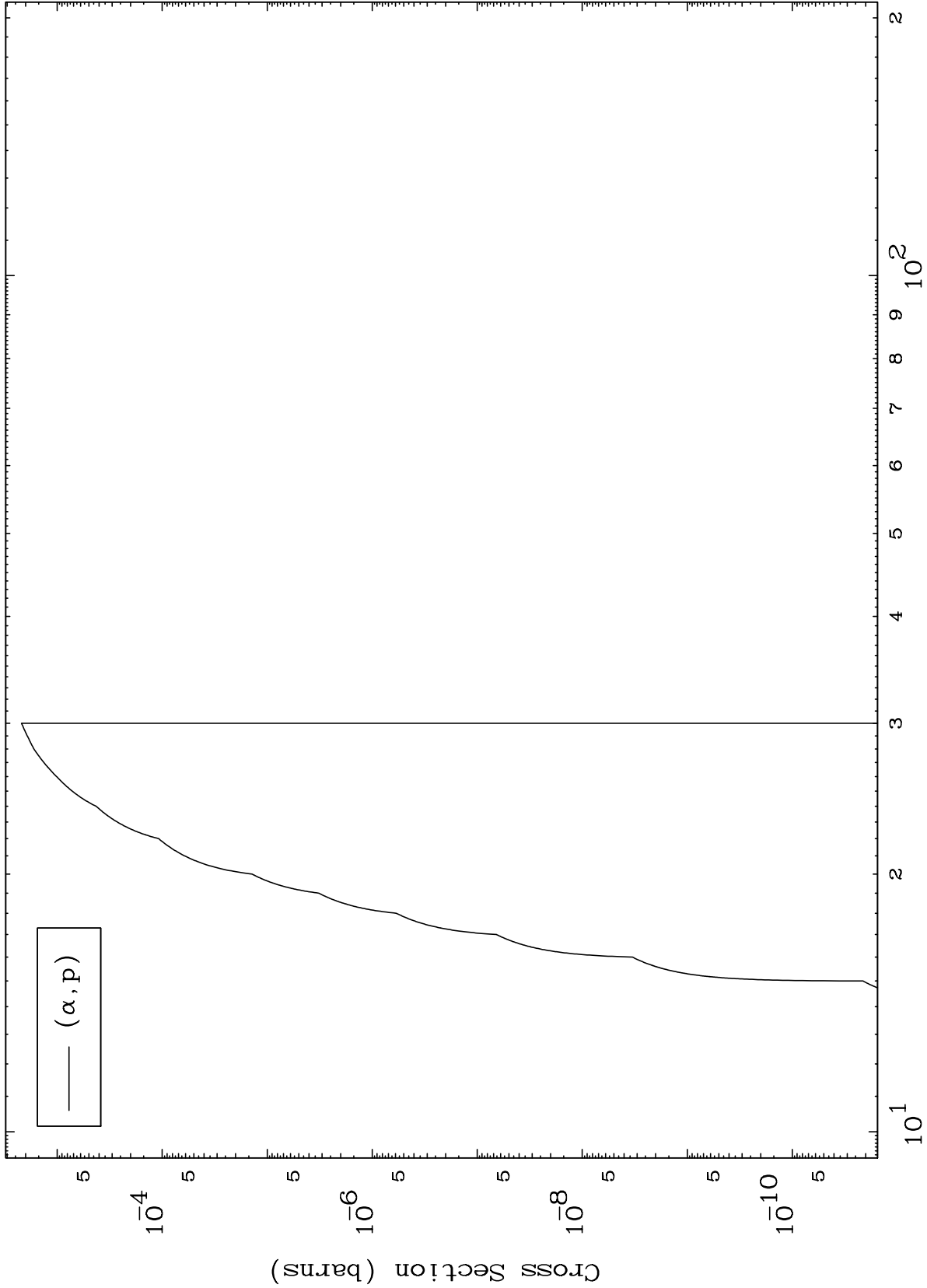
72-Hf-182

MAT 7249

( $\alpha, p$ ) Levels

72-Hf-182

0 Kelvin Cross Sections



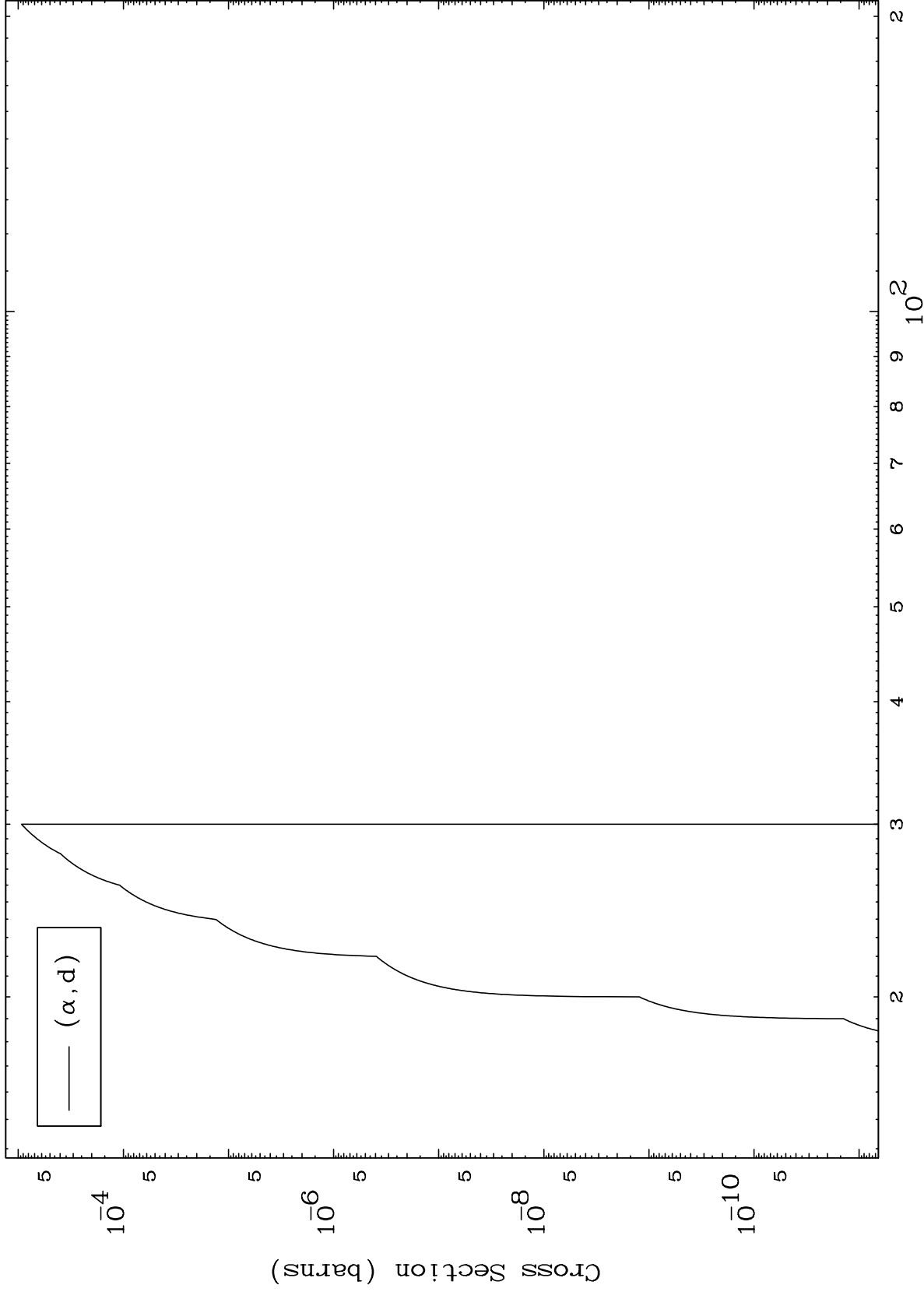
Incident Energy (MeV)

72-Hf-182

MAT 7249

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

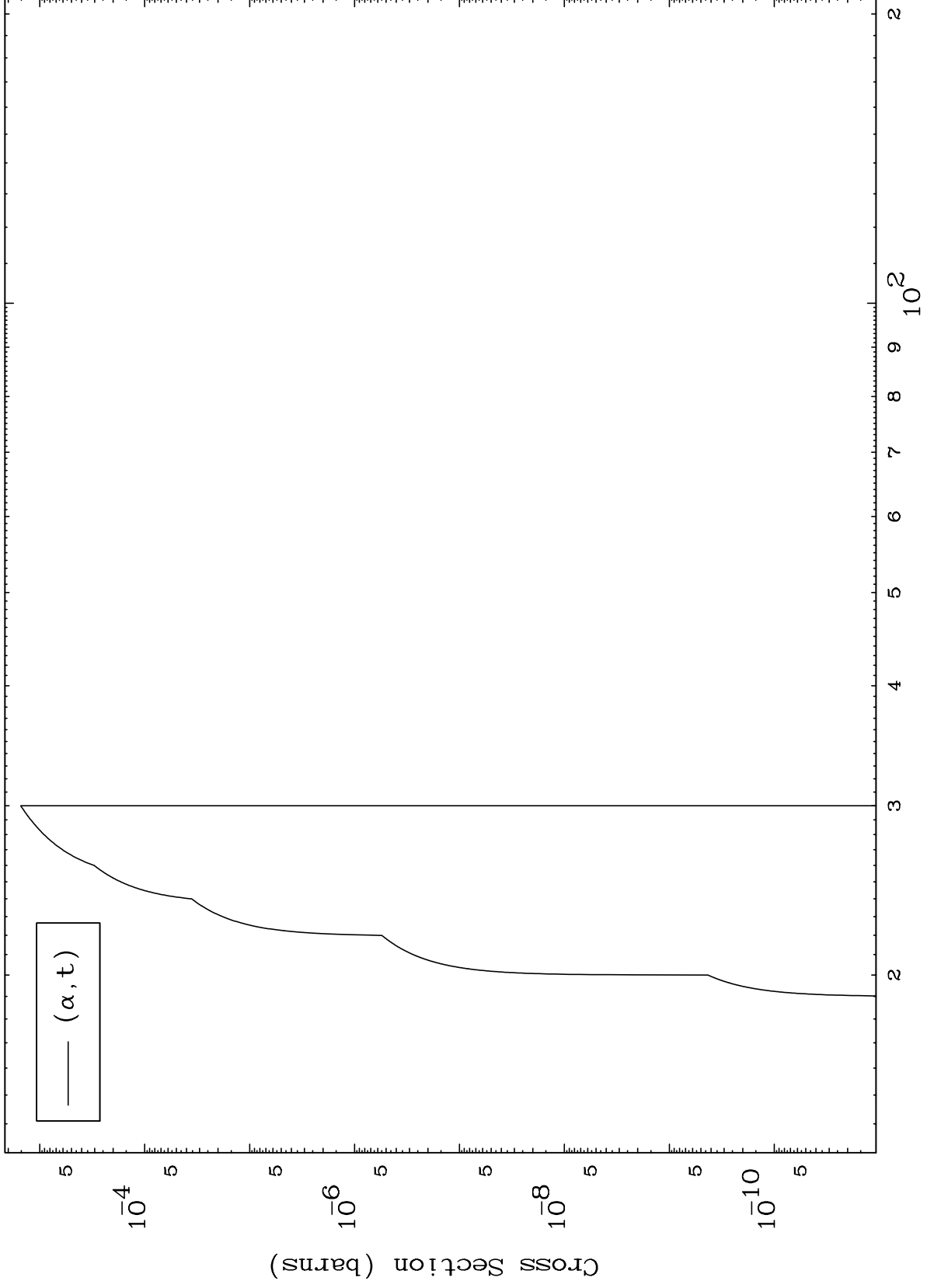
72-Hf-182



6

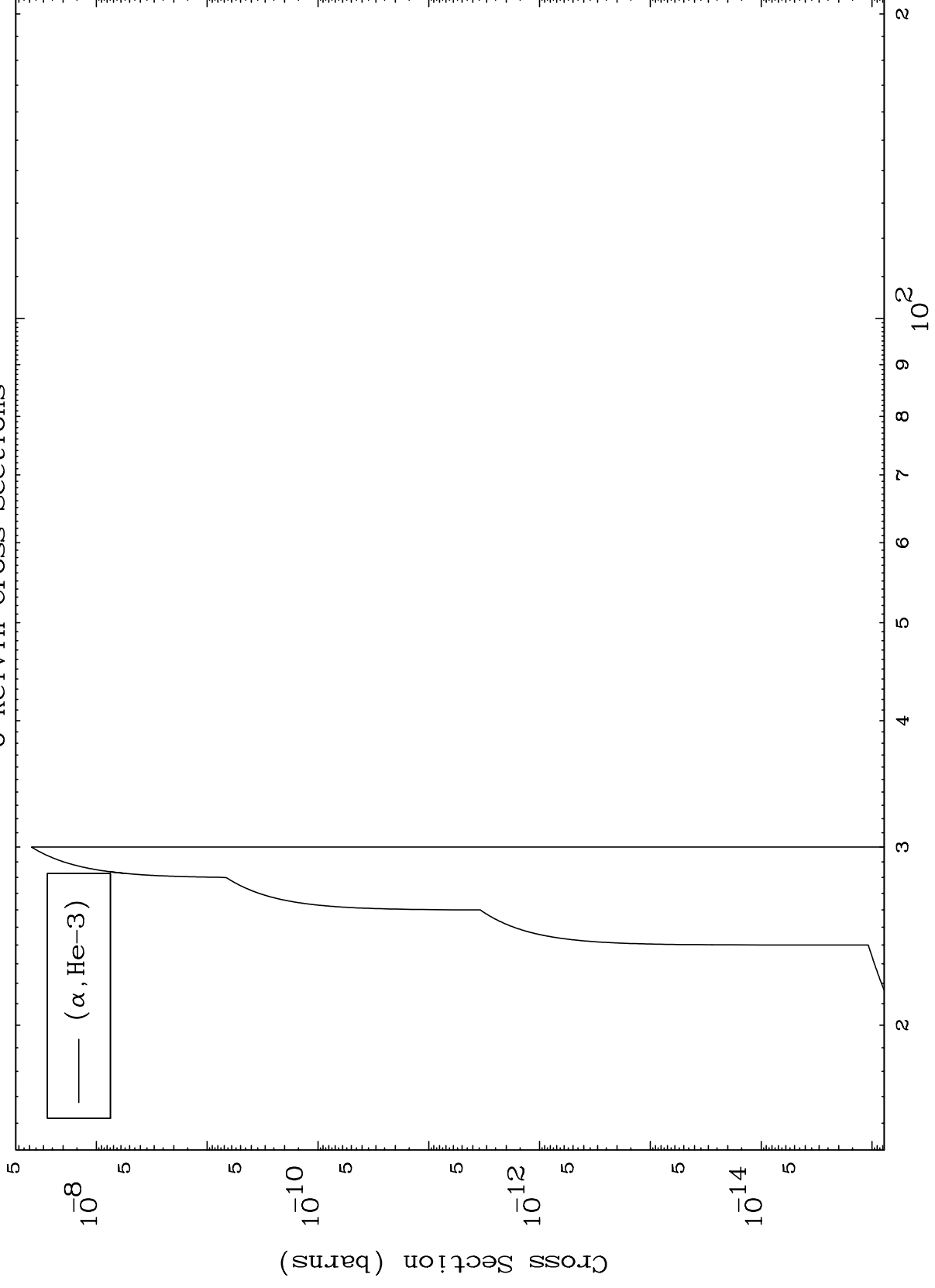
Incident Energy (MeV)

72-Hf-182





0 Kelvin Cross Sections

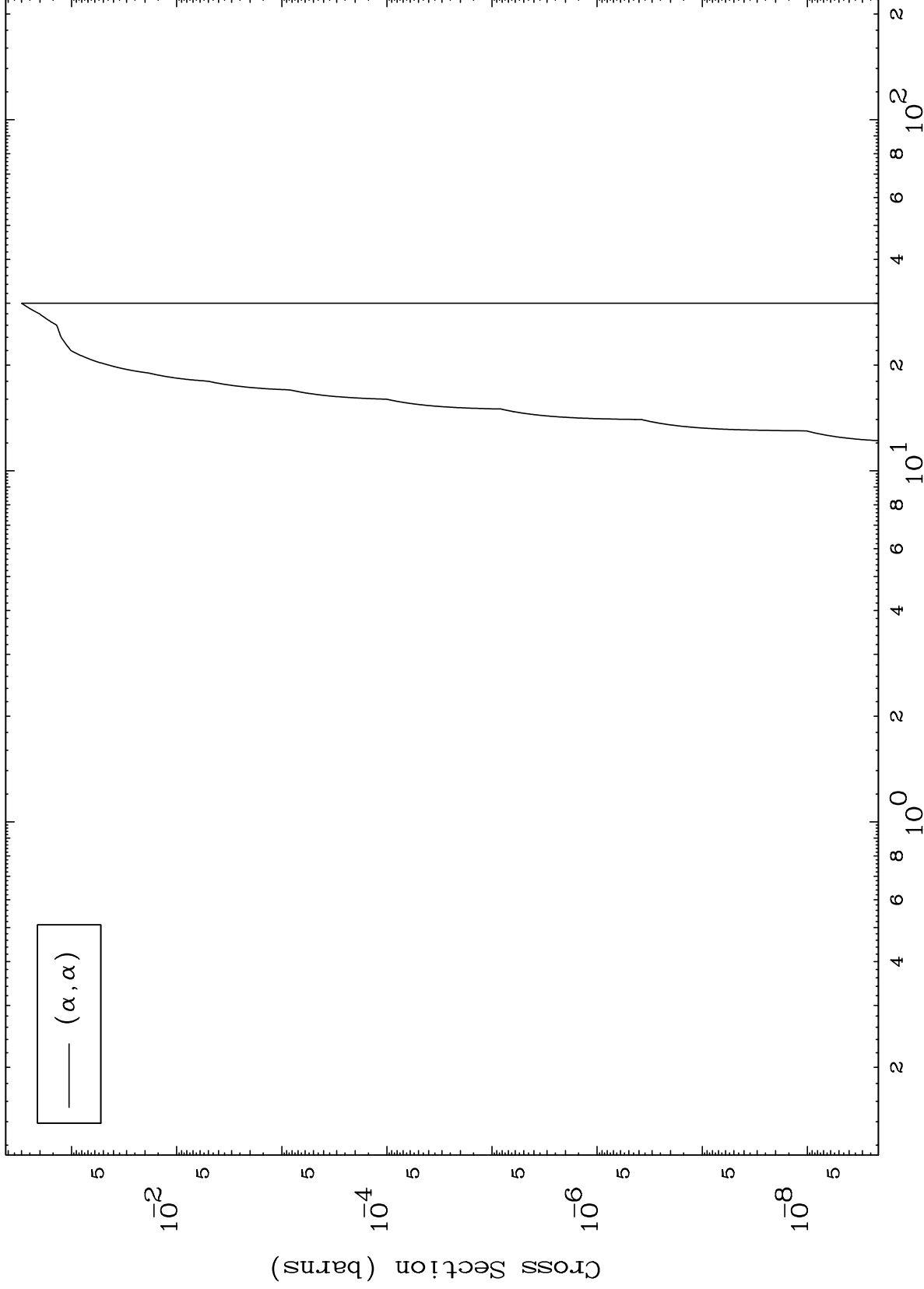


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( $\alpha, \alpha$ ) Levels

72-Hf-182

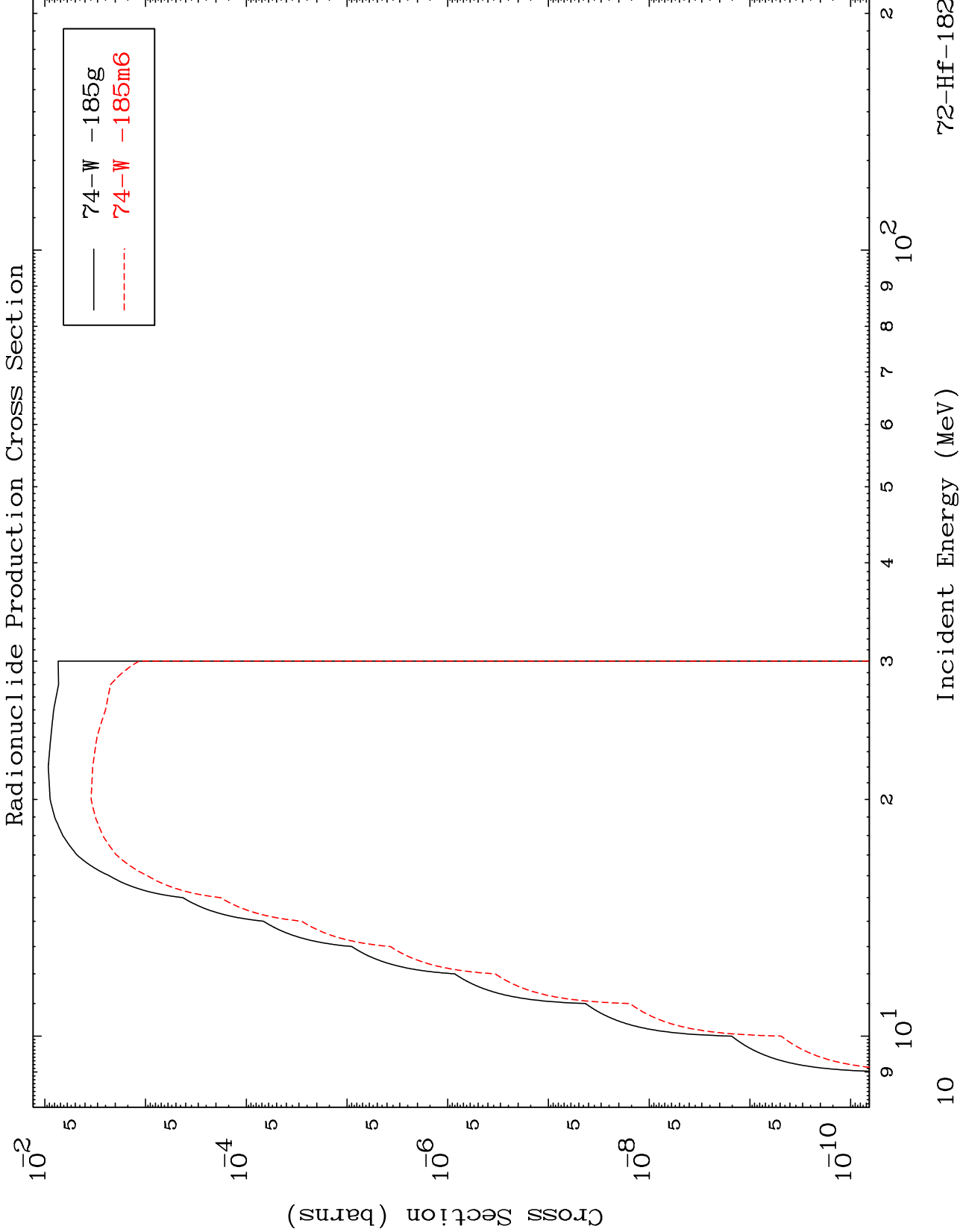
0 Kelvin Cross Sections



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$\alpha$  Inelastic

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Incident Energy (MeV)

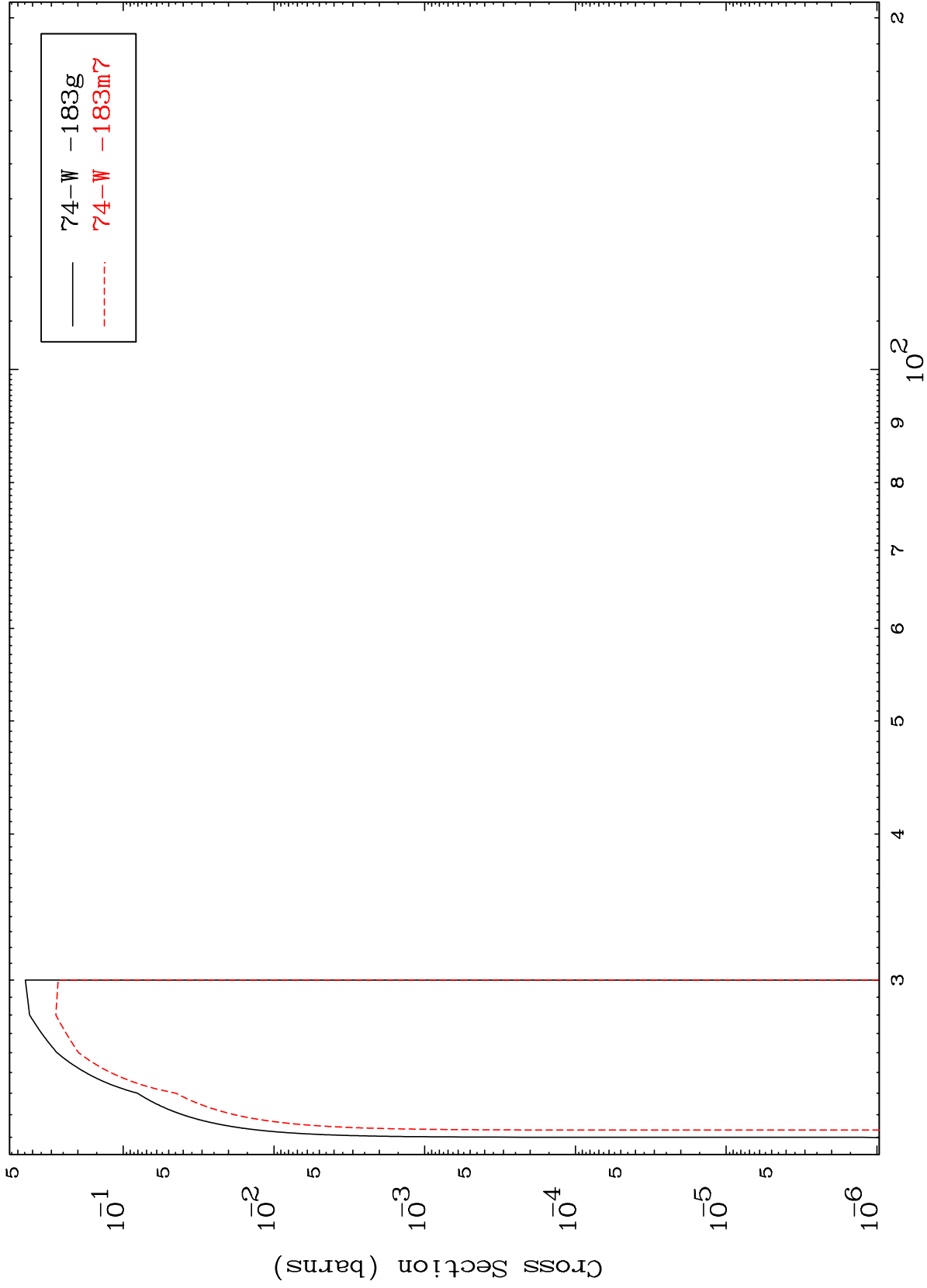
72-Hf-182

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( $\alpha, 3n$ )

72-Hf-182

Radionuclide Production Cross Section



11

Incident Energy (MeV)

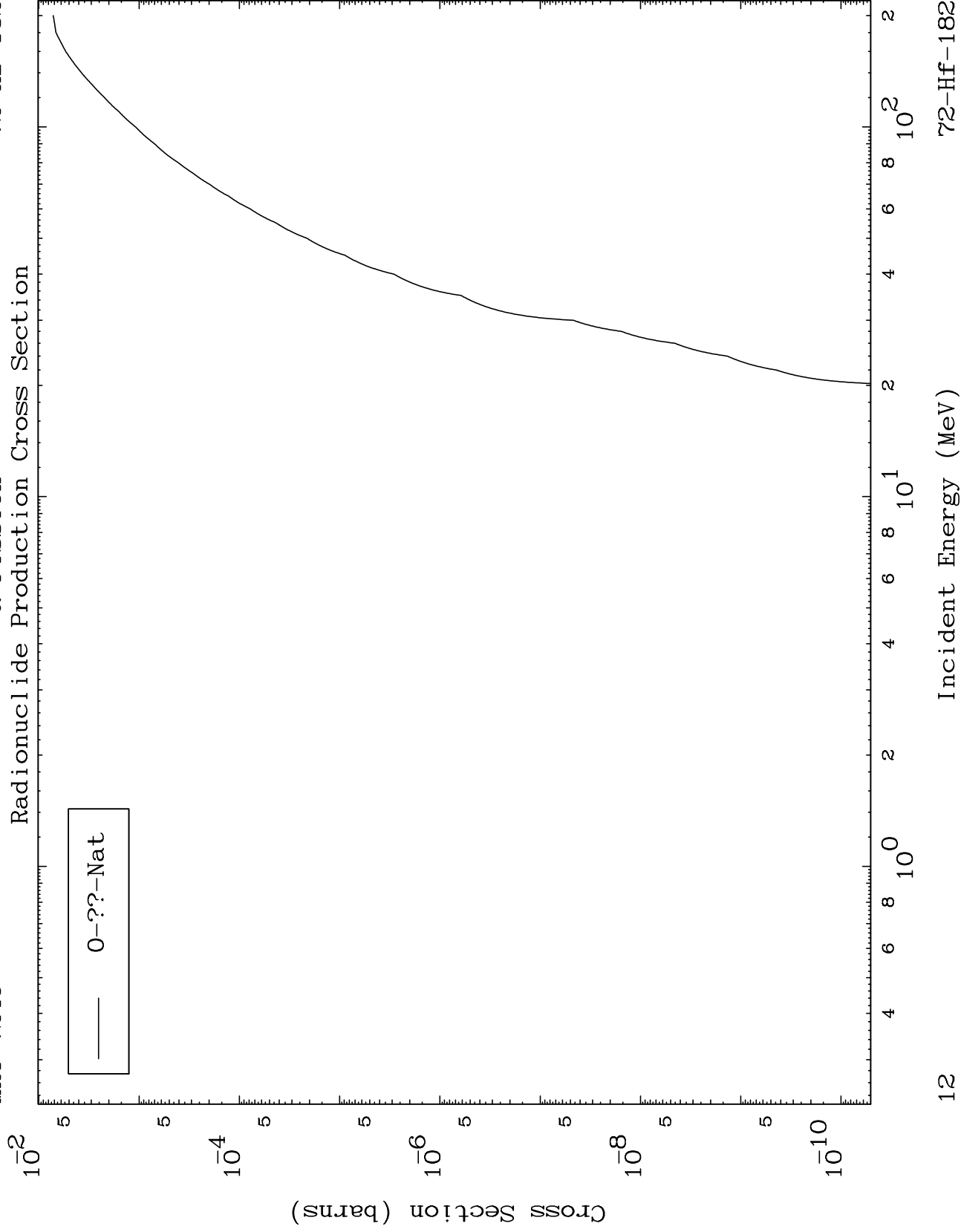
72-Hf-182

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$\alpha$  Fission

72-Hf-182

Radionuclide Production Cross Section



12

Incident Energy (MeV)

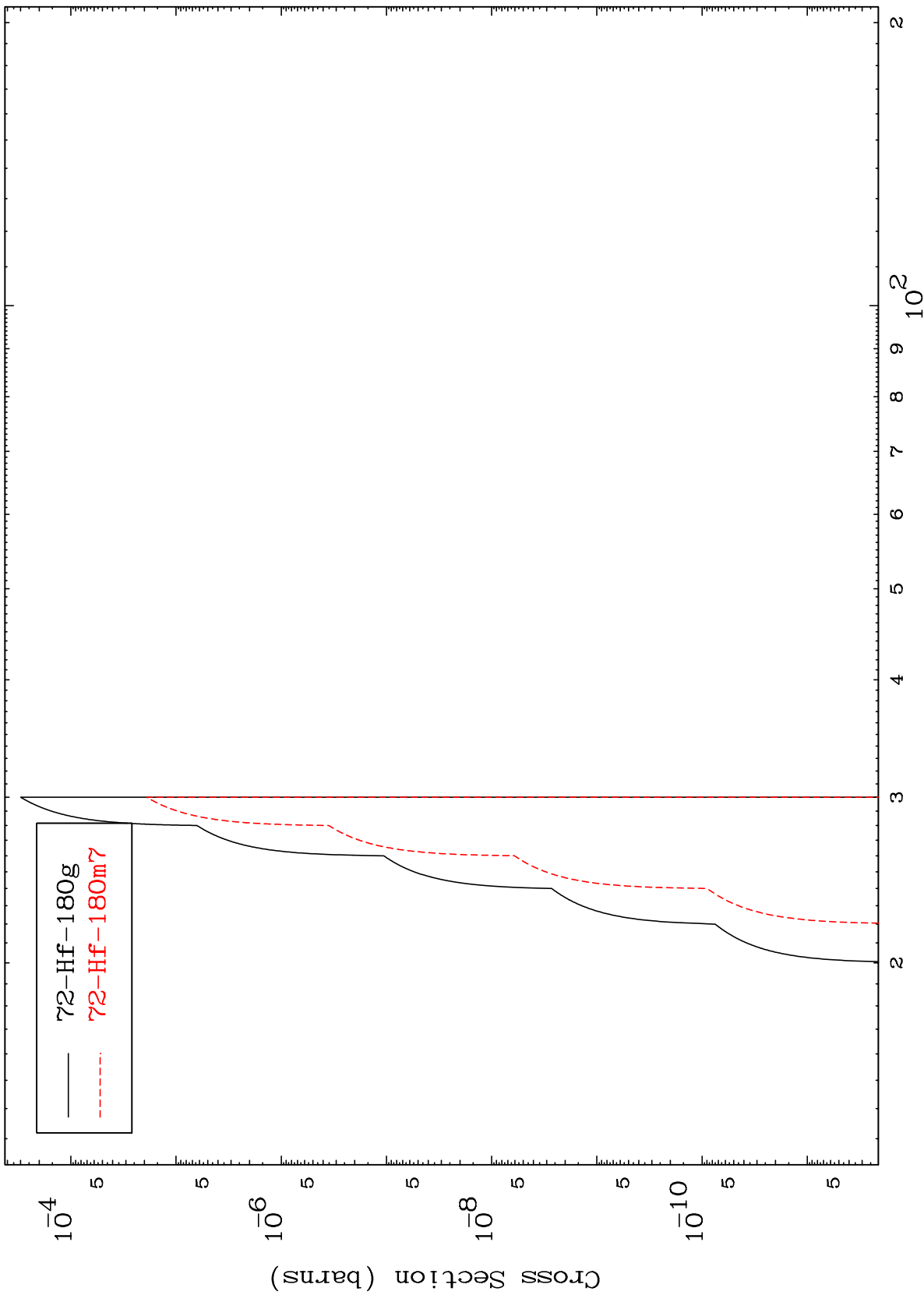
72-Hf-182

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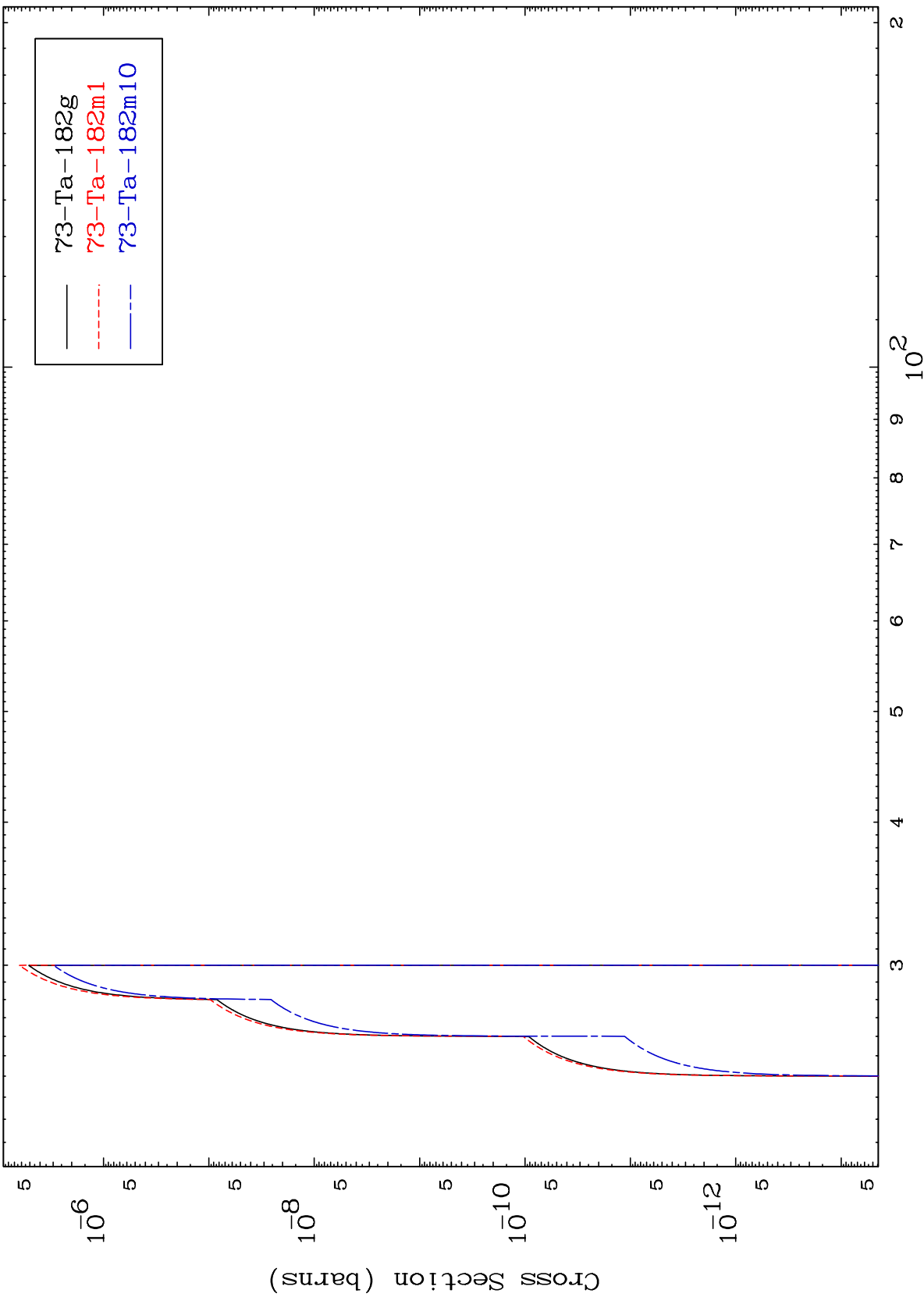
( $\alpha, 2n$ )  $\alpha$

72-Hf-182

Radionuclide Production Cross Section



Radionuclide Production Cross Section

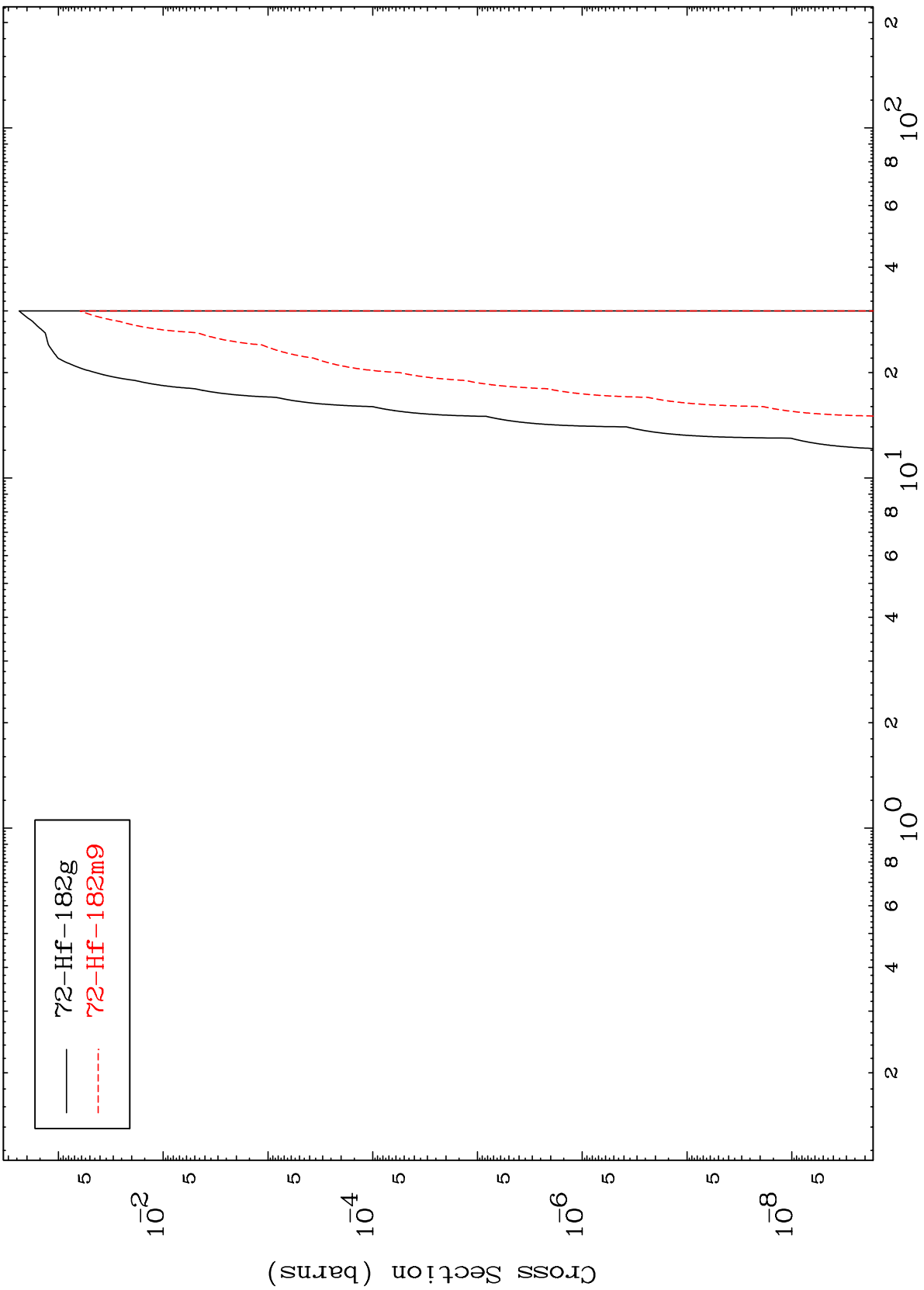


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( $\alpha, \alpha$ )

<sup>72</sup>Hf-182

Radionuclide Production Cross Section



15

Incident Energy (MeV)

<sup>72</sup>Hf-182