

Program EVALPLOT
(Version 2018-1)

by

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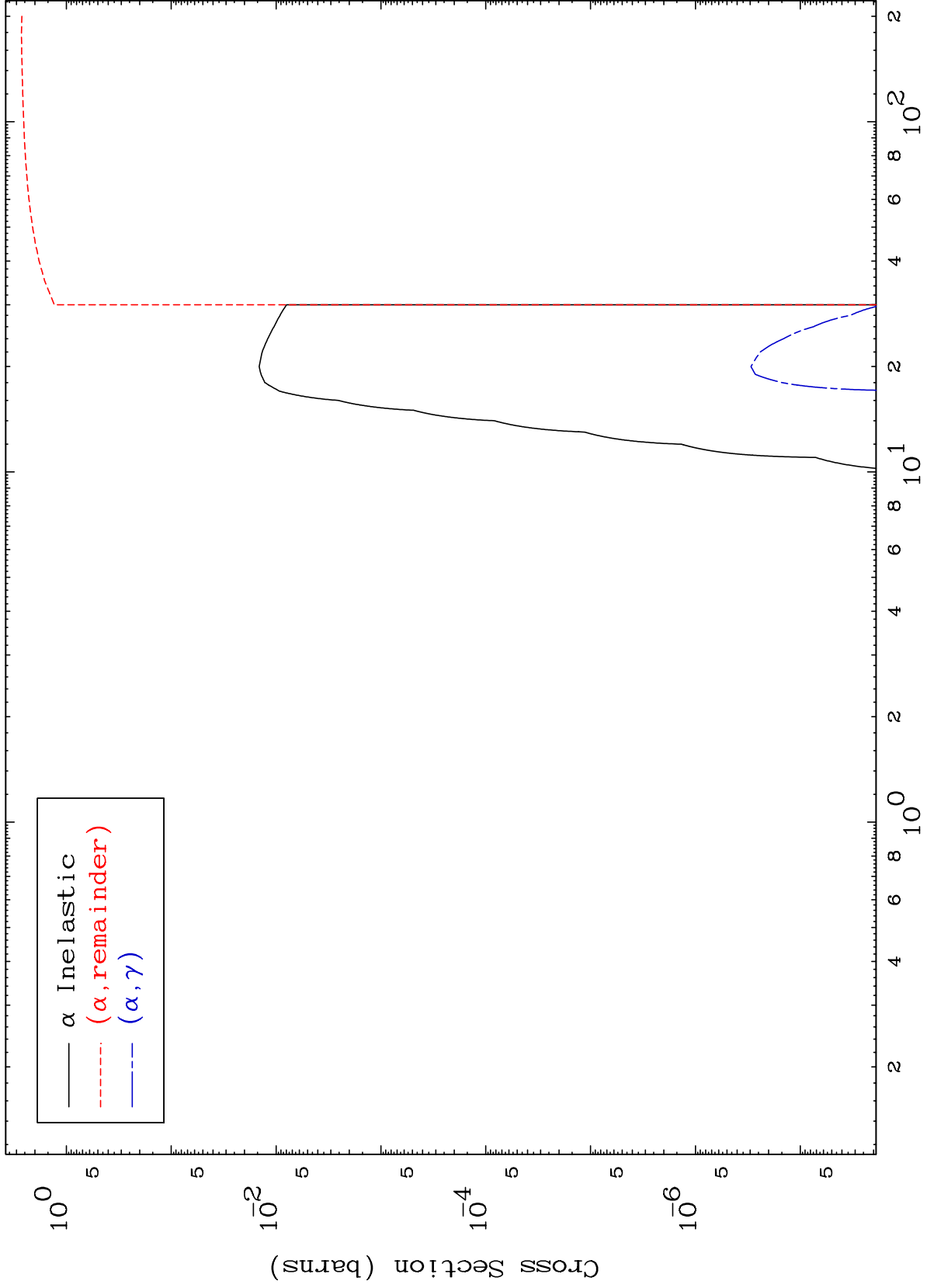
Press Mouse Button to Start

MAT 7256

α Major

72-Hf-184

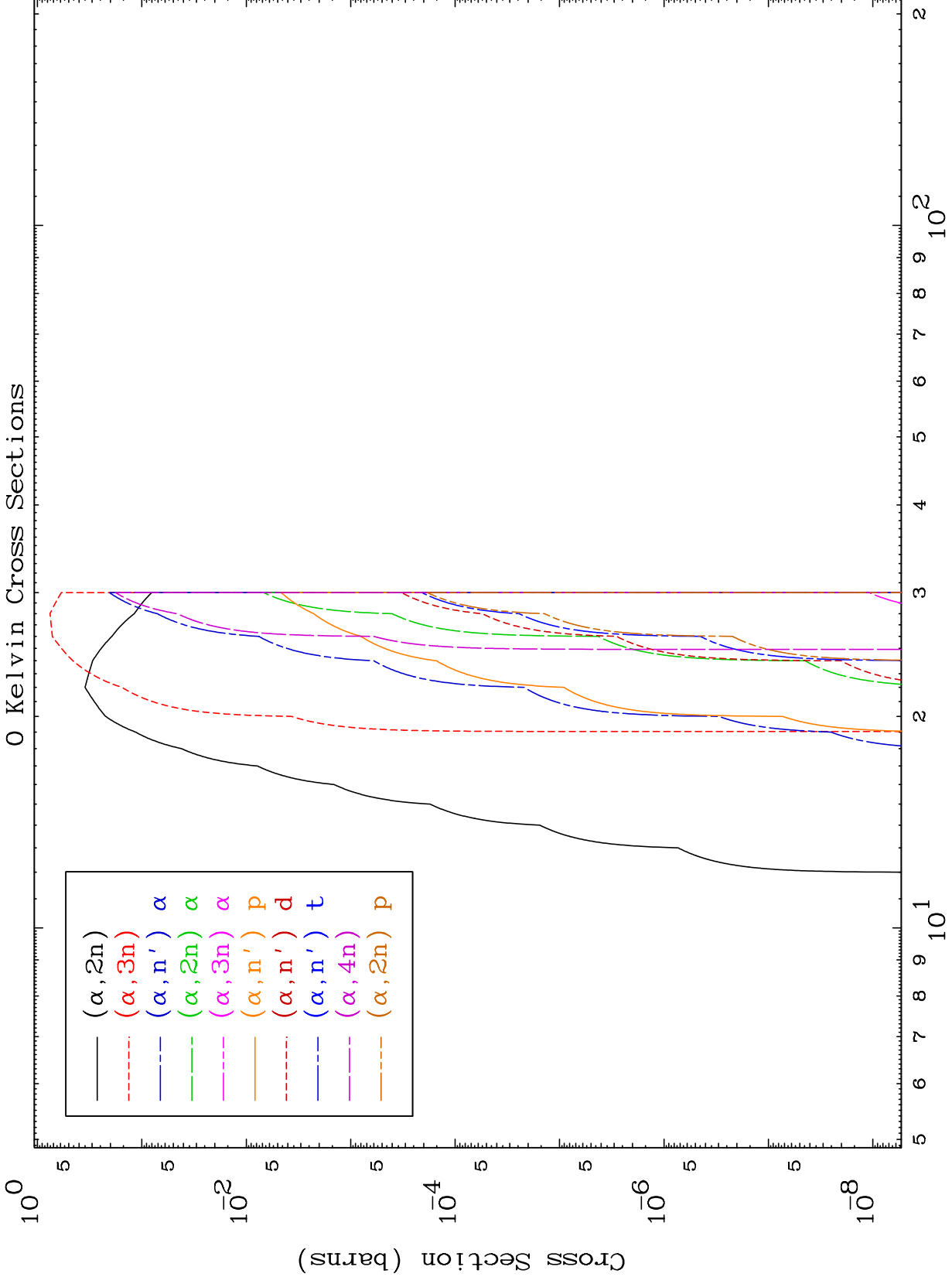
0 Kelvin Cross Sections



MAT 7256

α Neutron Production
0 Kelvin Cross Sections

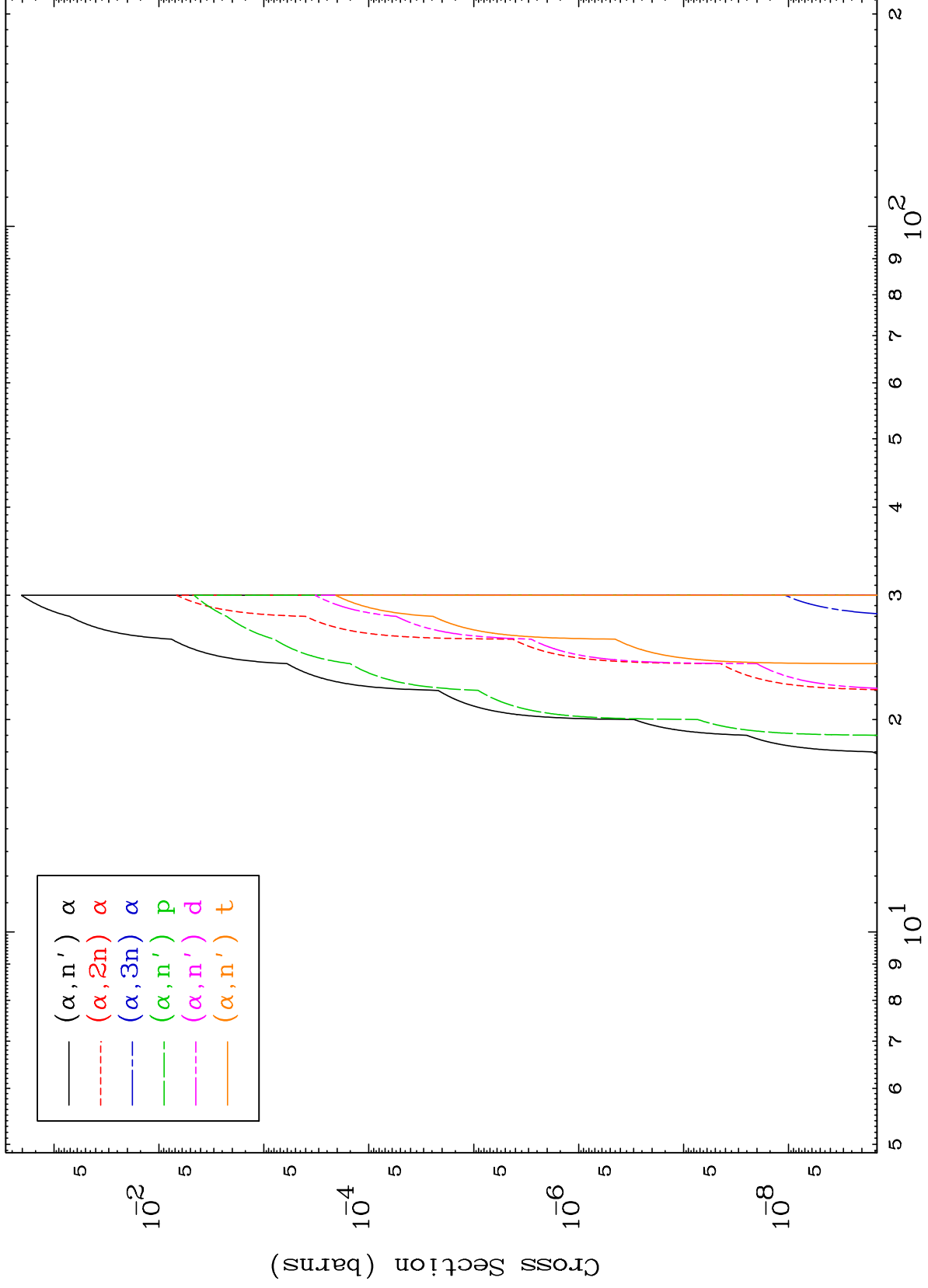
72-Hf-184

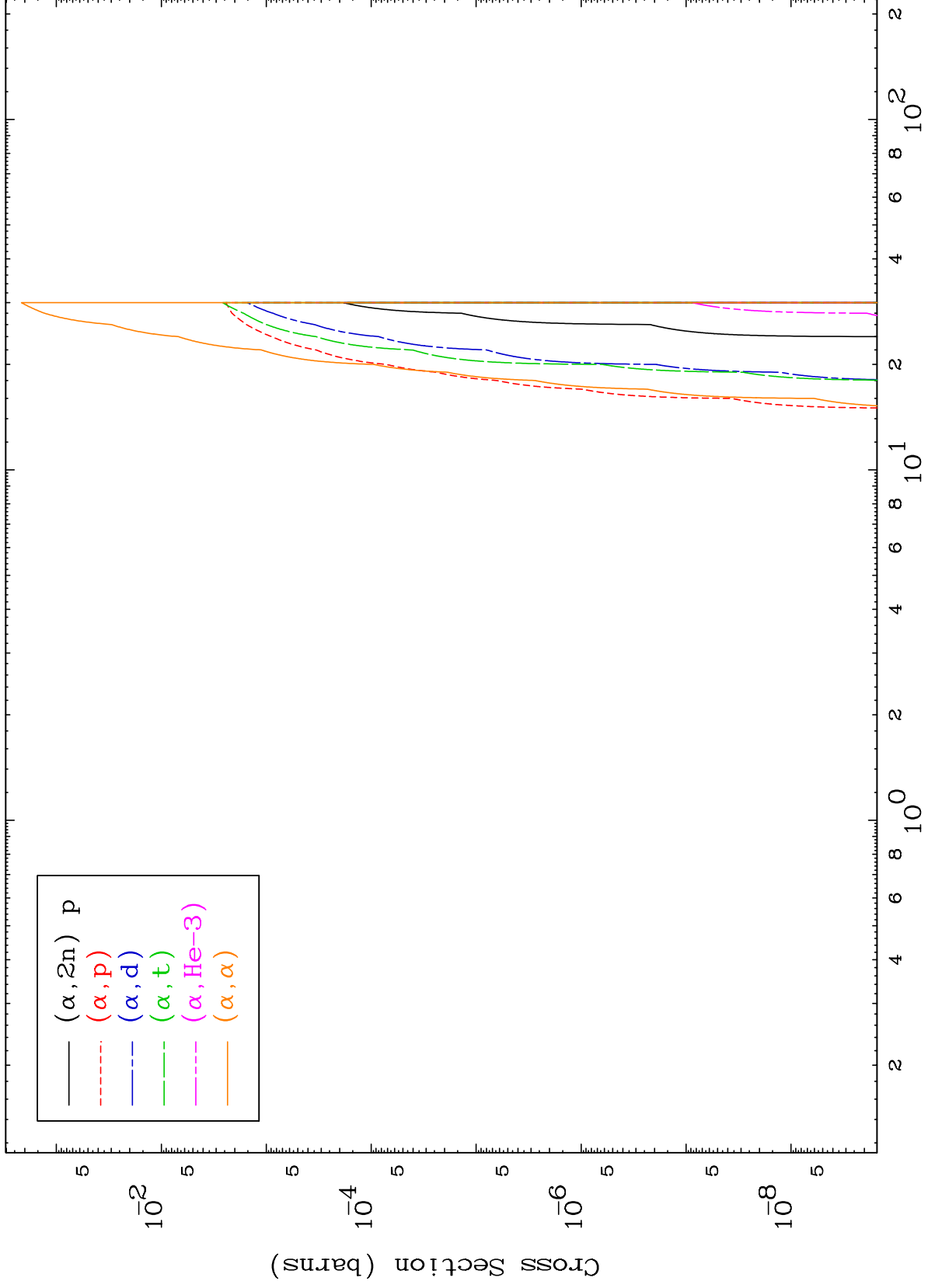


2

Incident Energy (MeV)

72-Hf-184



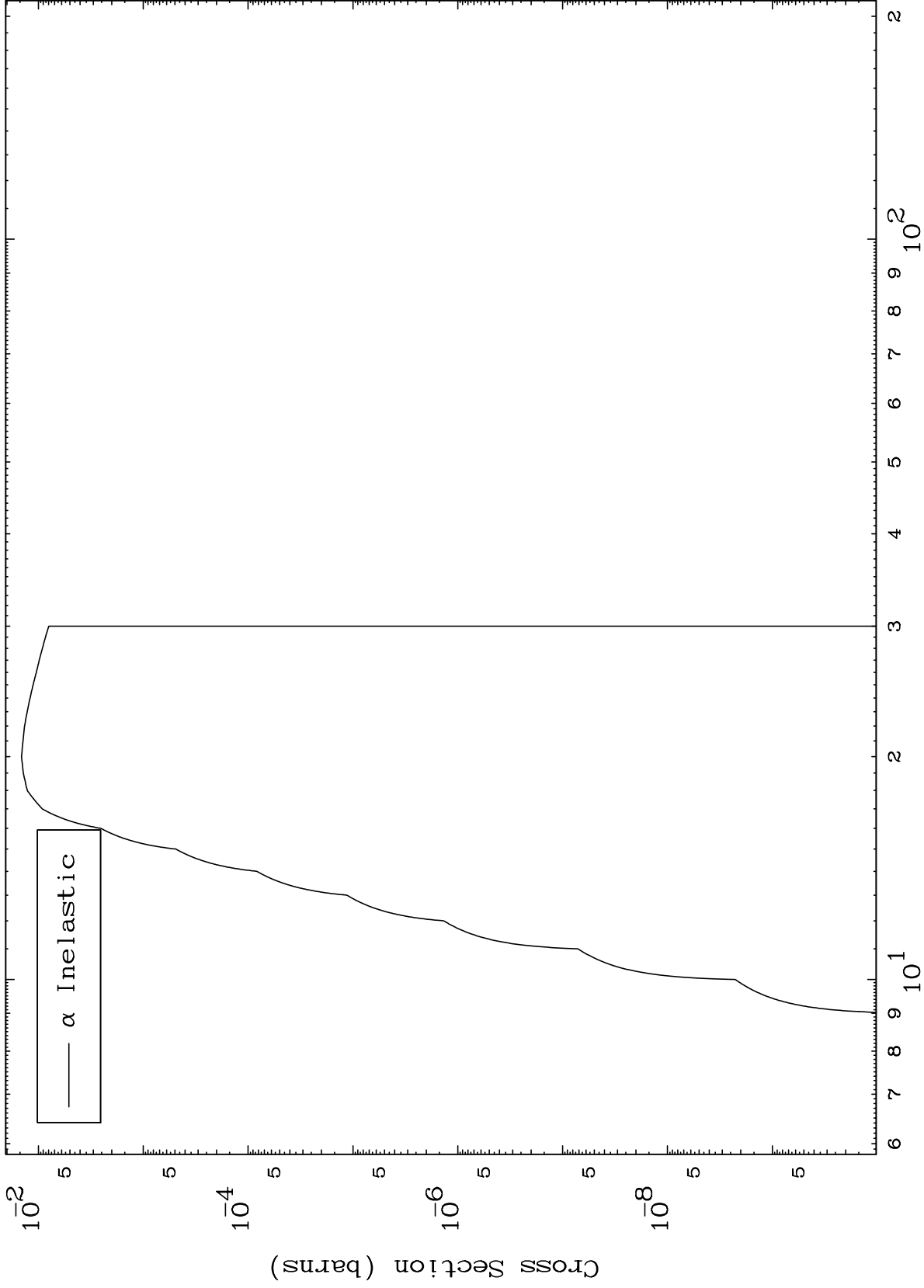


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(α, n') Level

72-Hf-184

0 Kelvin Cross Sections



5

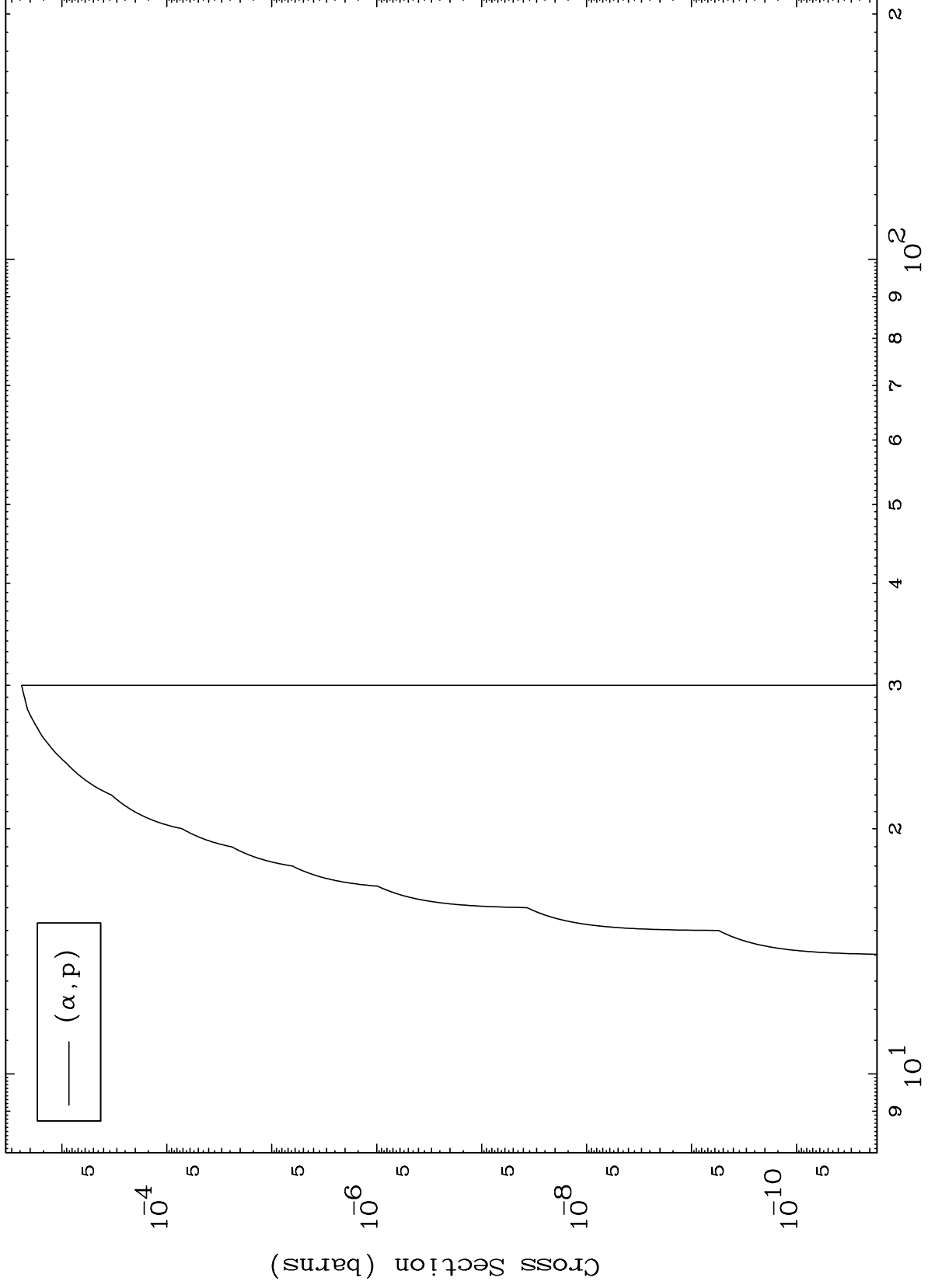
Incident Energy (MeV)

72-Hf-184

MAT 7256

(α, p) Levels
0 Kelvin Cross Sections

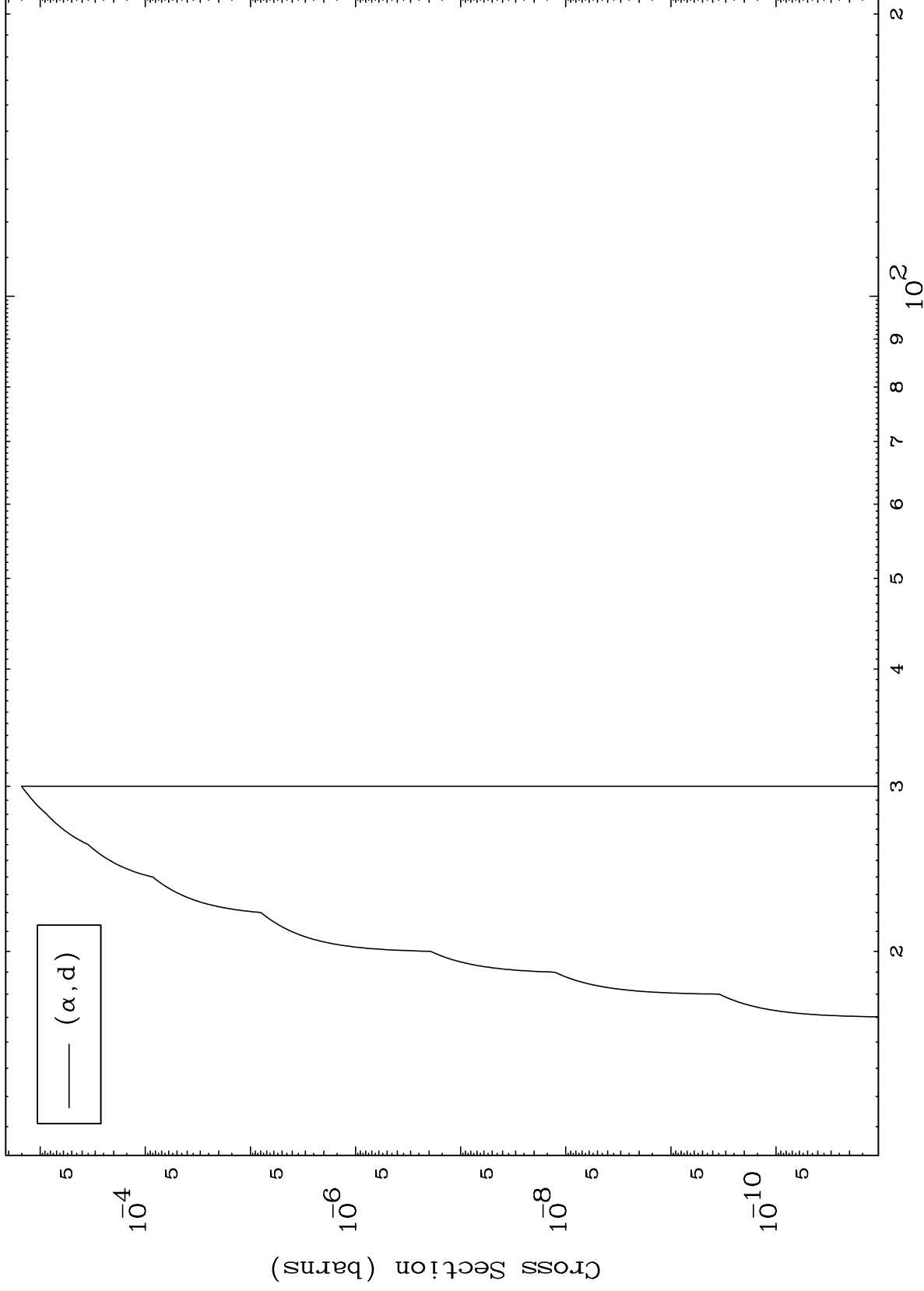
72-Hf-184

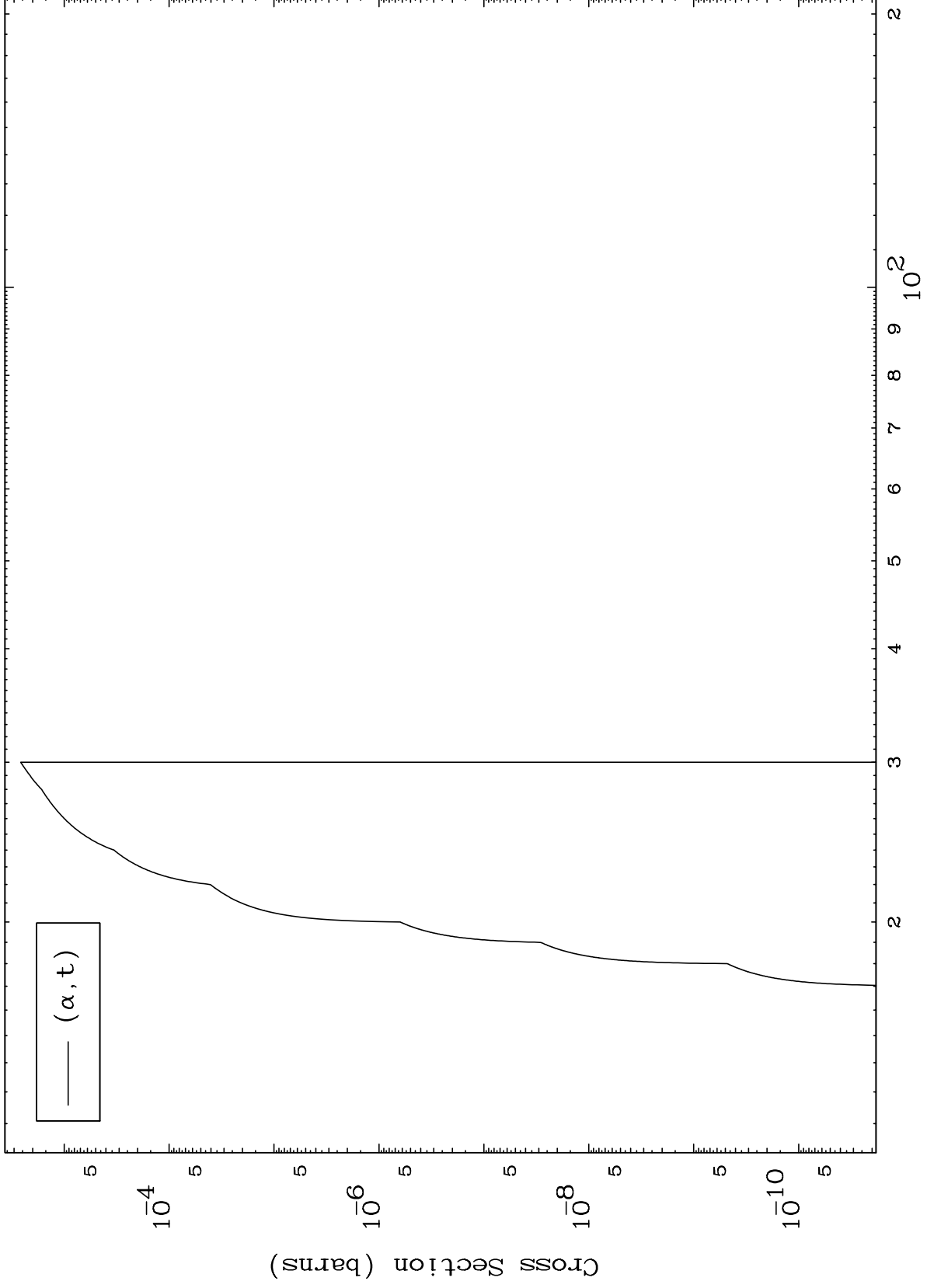


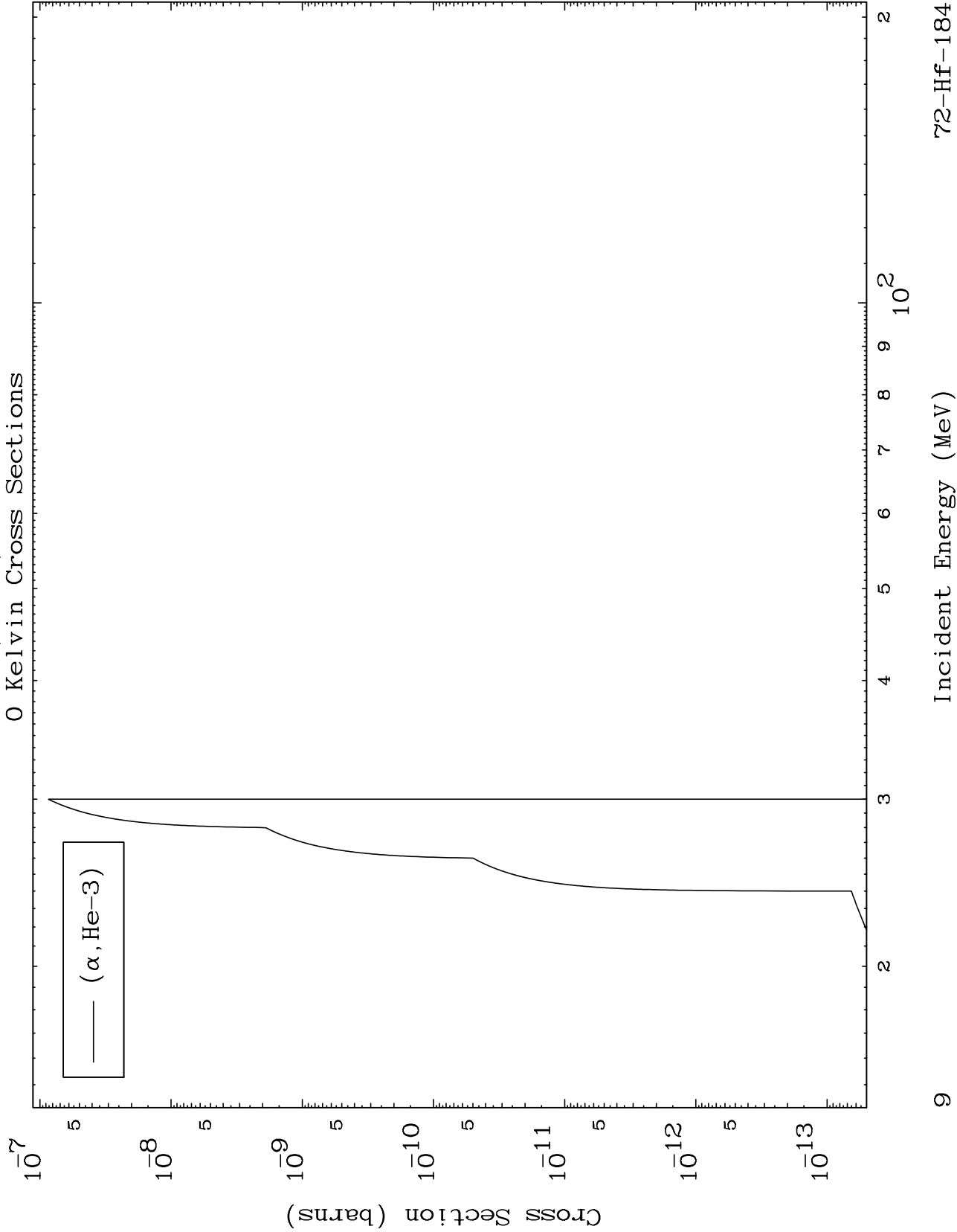
Incident Energy (MeV)

72-Hf-184

6





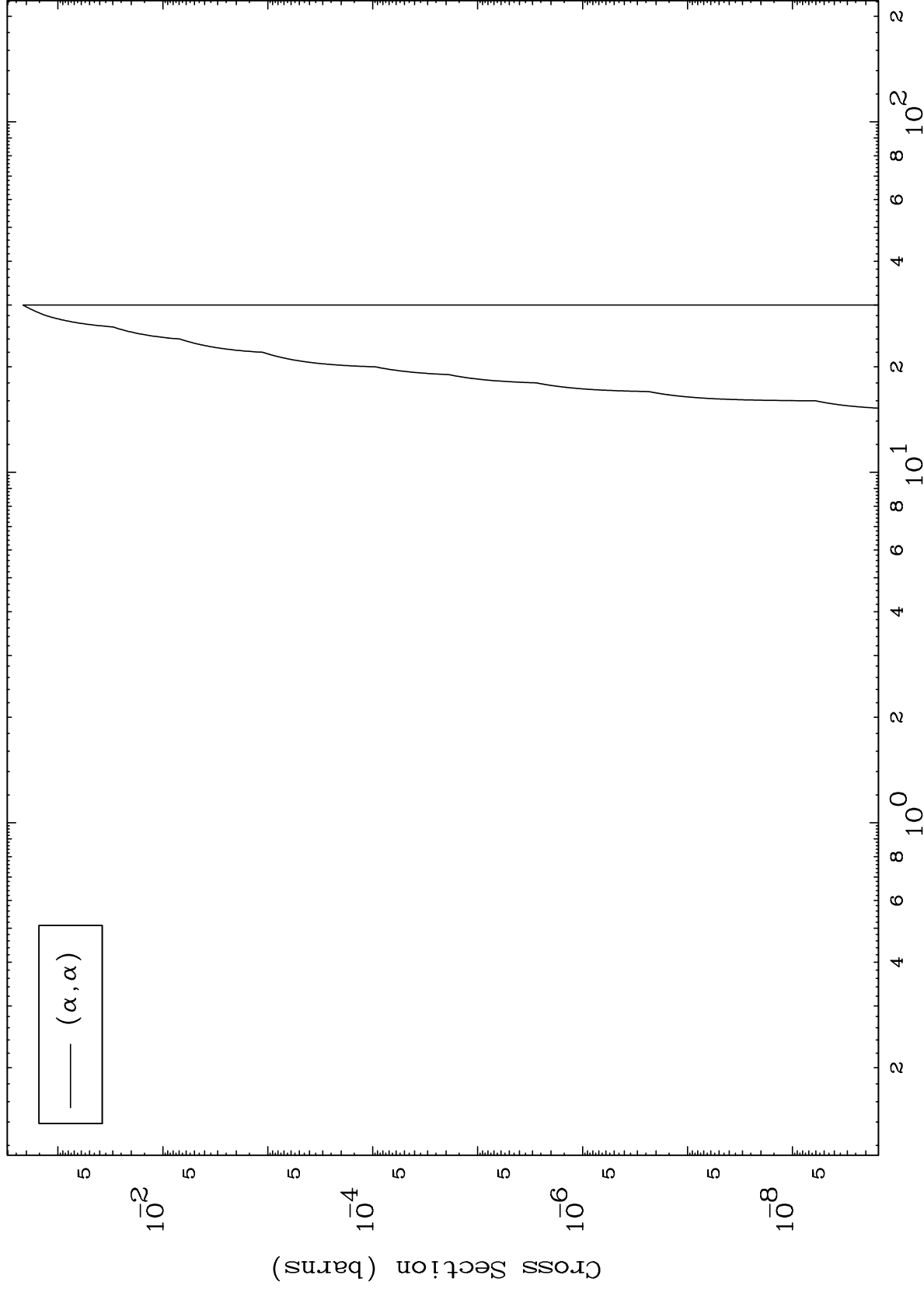


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(α, α) Levels

72-Hf-184

0 Kelvin Cross Sections



10

Incident Energy (MeV)

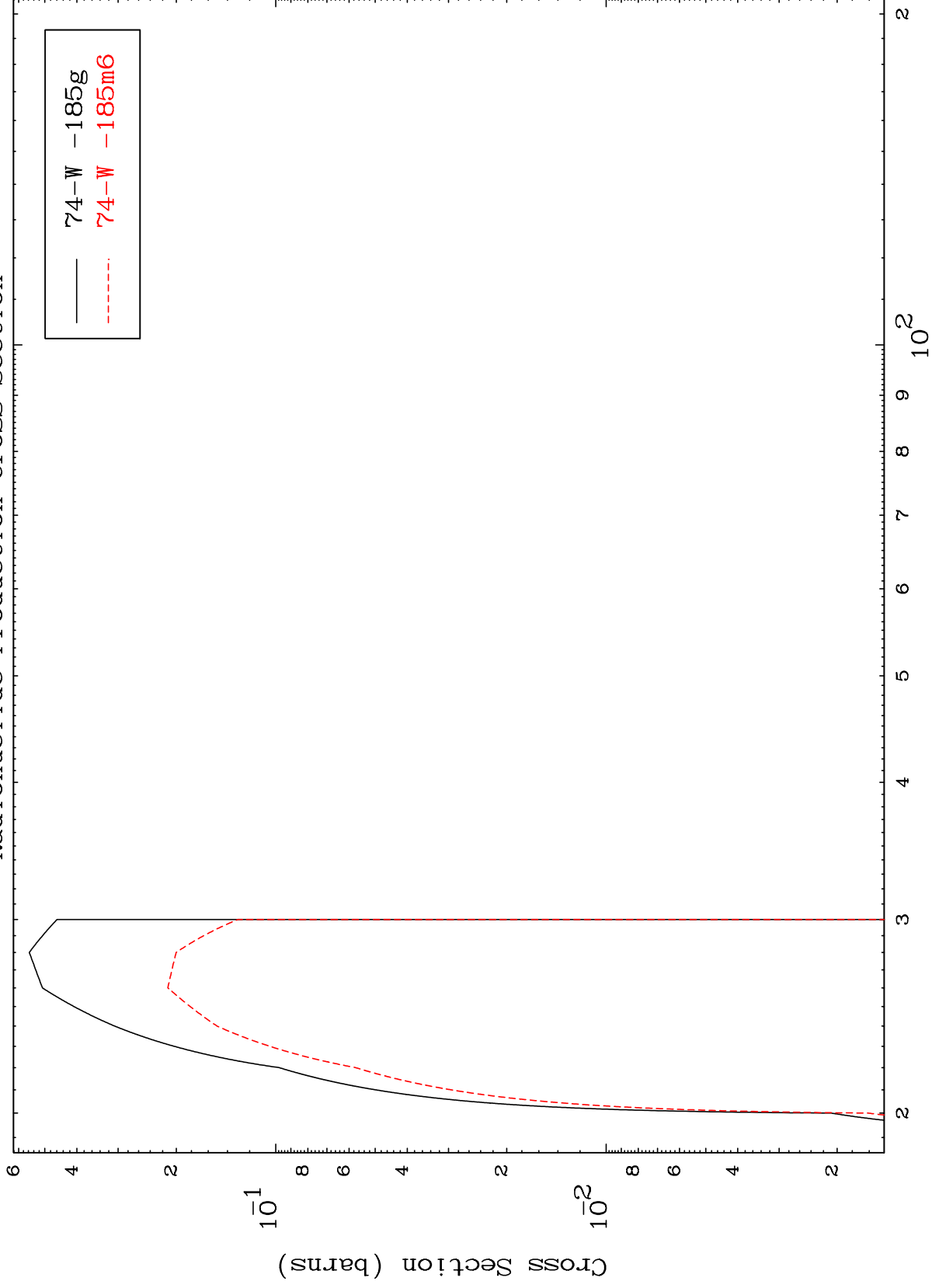
72-Hf-184

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($\alpha, 3n$)

72-Hf-184

Radionuclide Production Cross Section



11

Incident Energy (MeV)

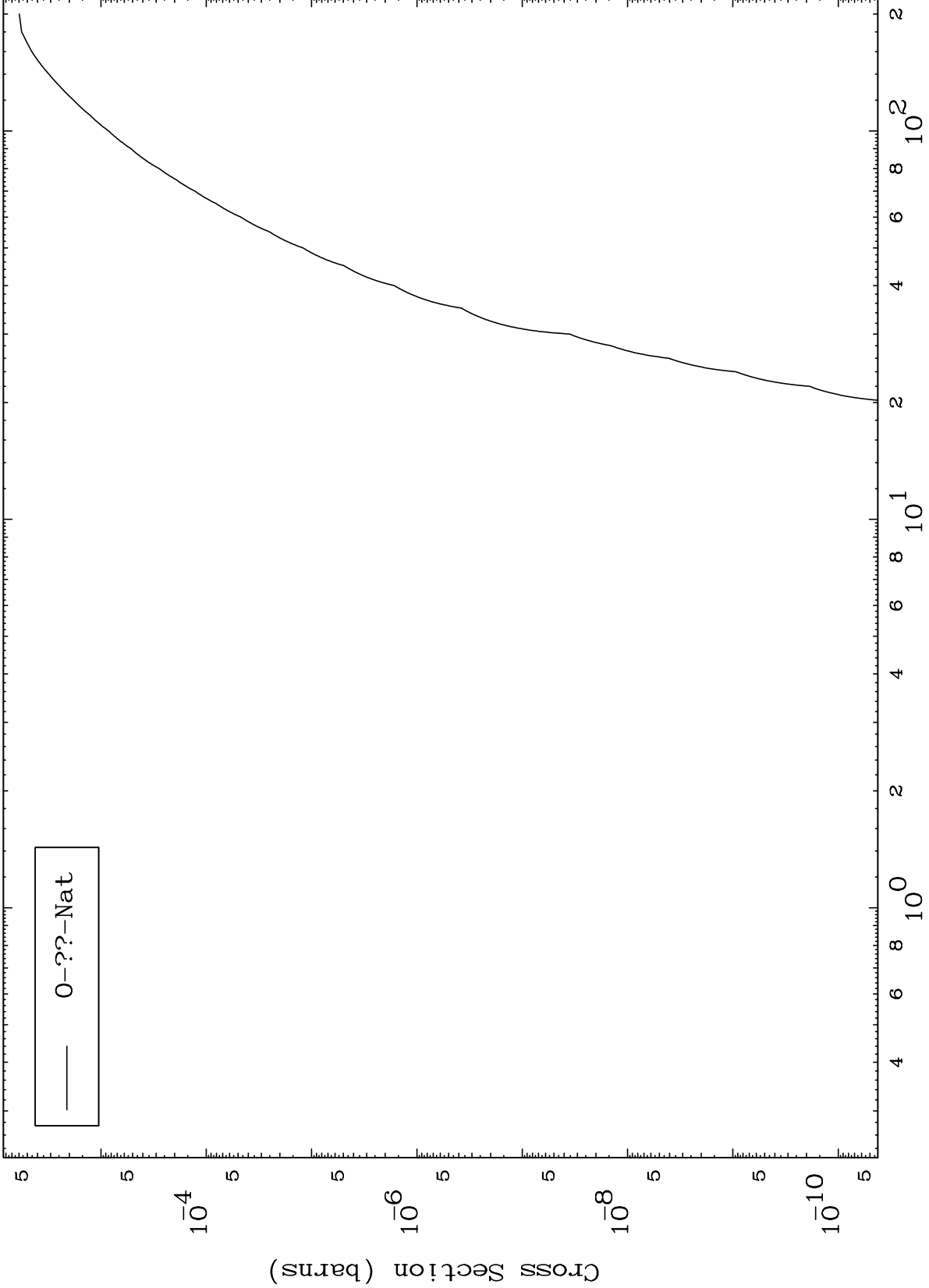
72-Hf-184

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α Fission

72-Hf-184

Radionuclide Production Cross Section



12

Incident Energy (MeV)

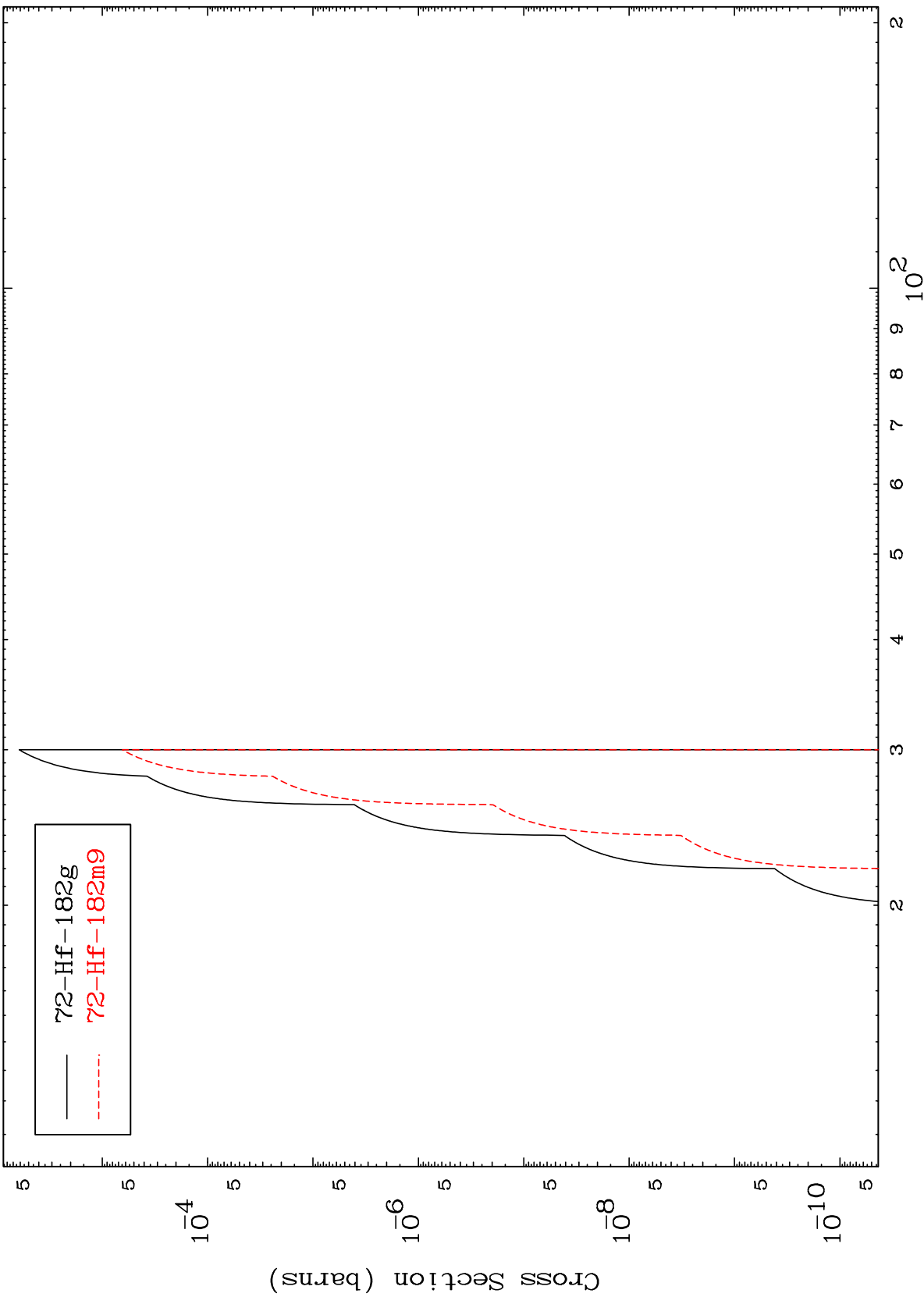
72-Hf-184

MAT 7256

$(\alpha, 2n)$ α

$^{72}\text{Hf-184}$

Radionuclide Production Cross Section

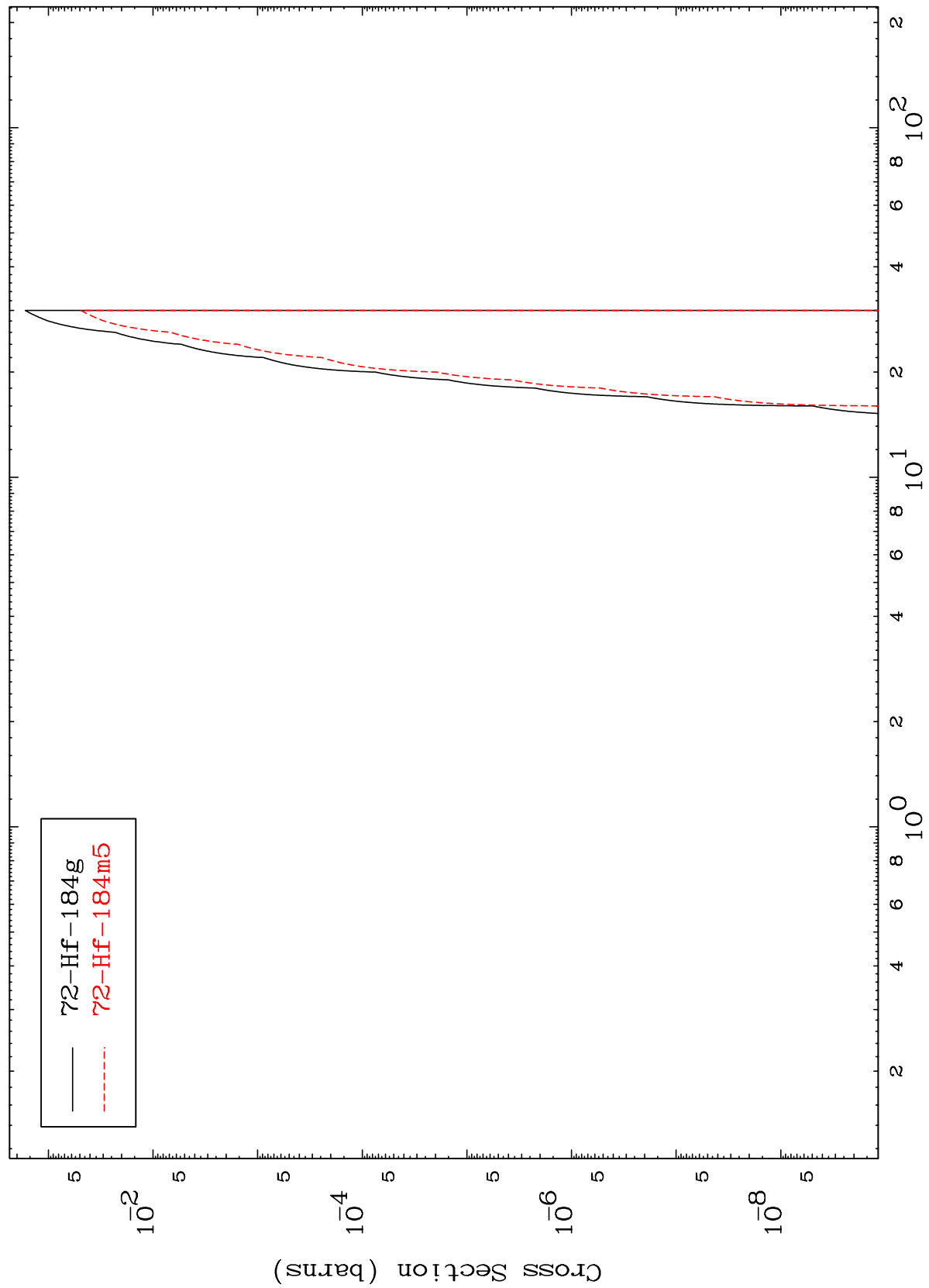


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(α, α)

⁷²Hf-184

Radionuclide Production Cross Section



14

Incident Energy (MeV)

⁷²Hf-184