

Program EVALPLOT  
(Version 2018-1)

by

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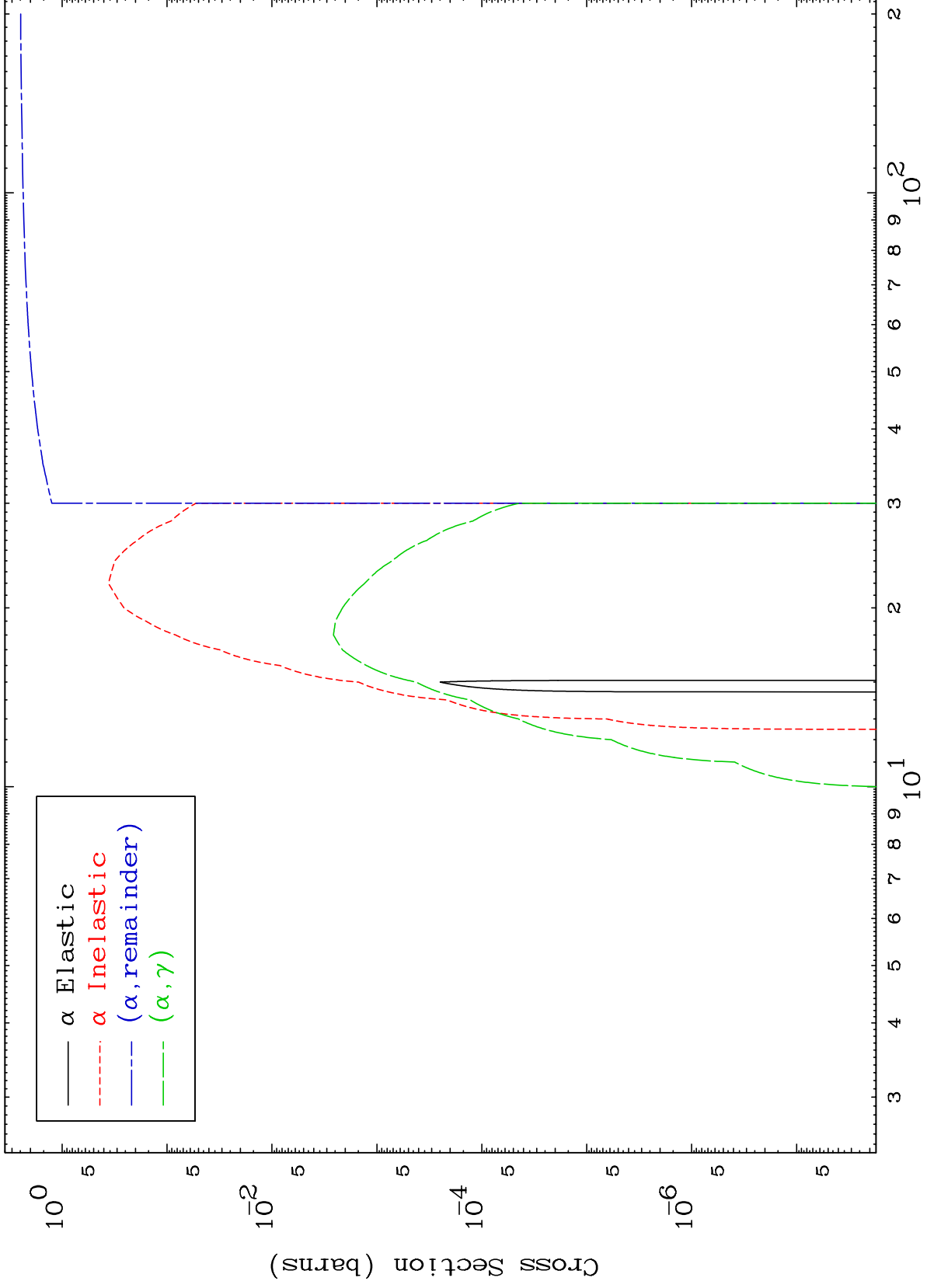
Press Mouse Button to Start

MAT 6701

$\alpha$  Major

67-Ho-157

0 Kelvin Cross Sections



1

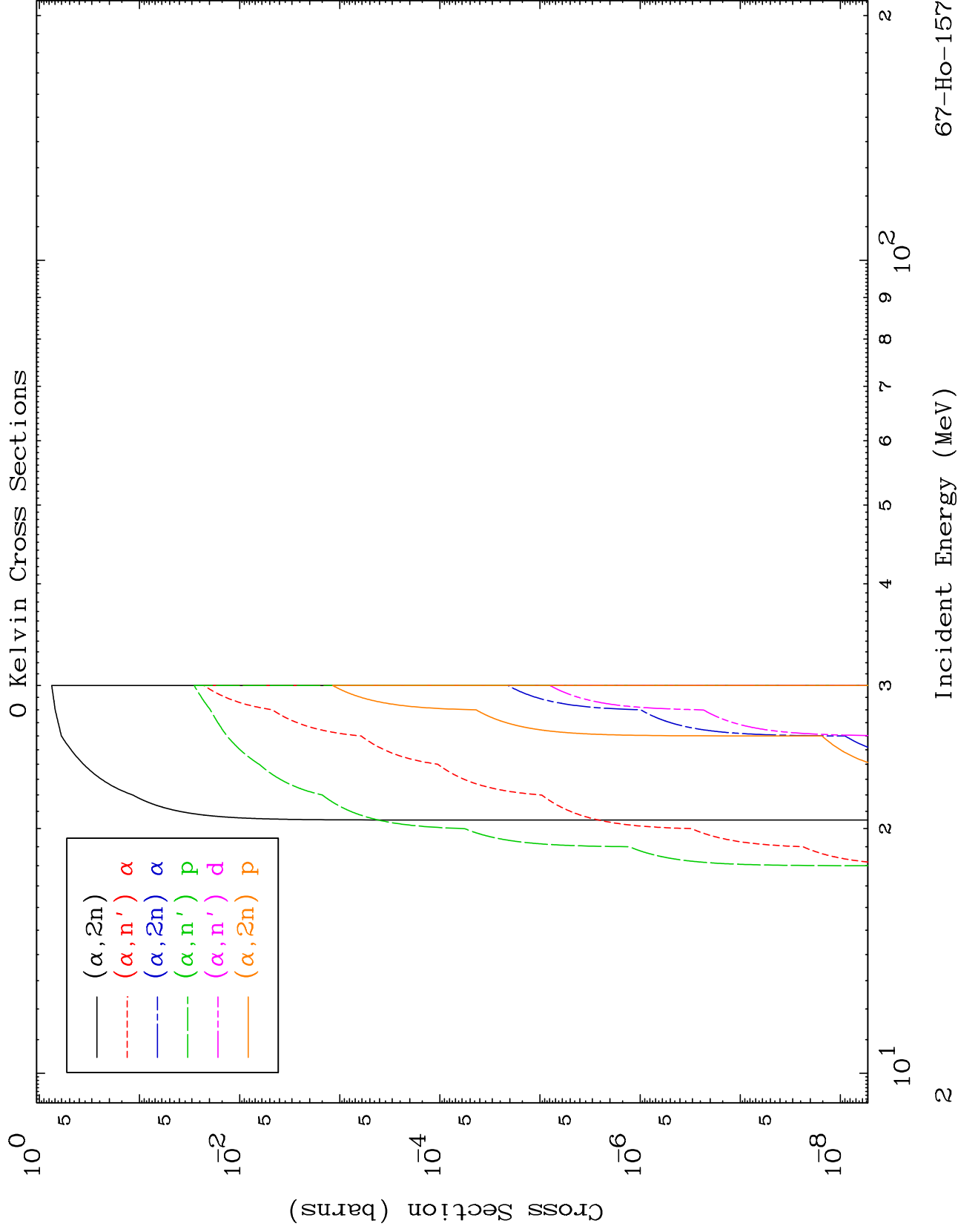
Incident Energy (MeV)

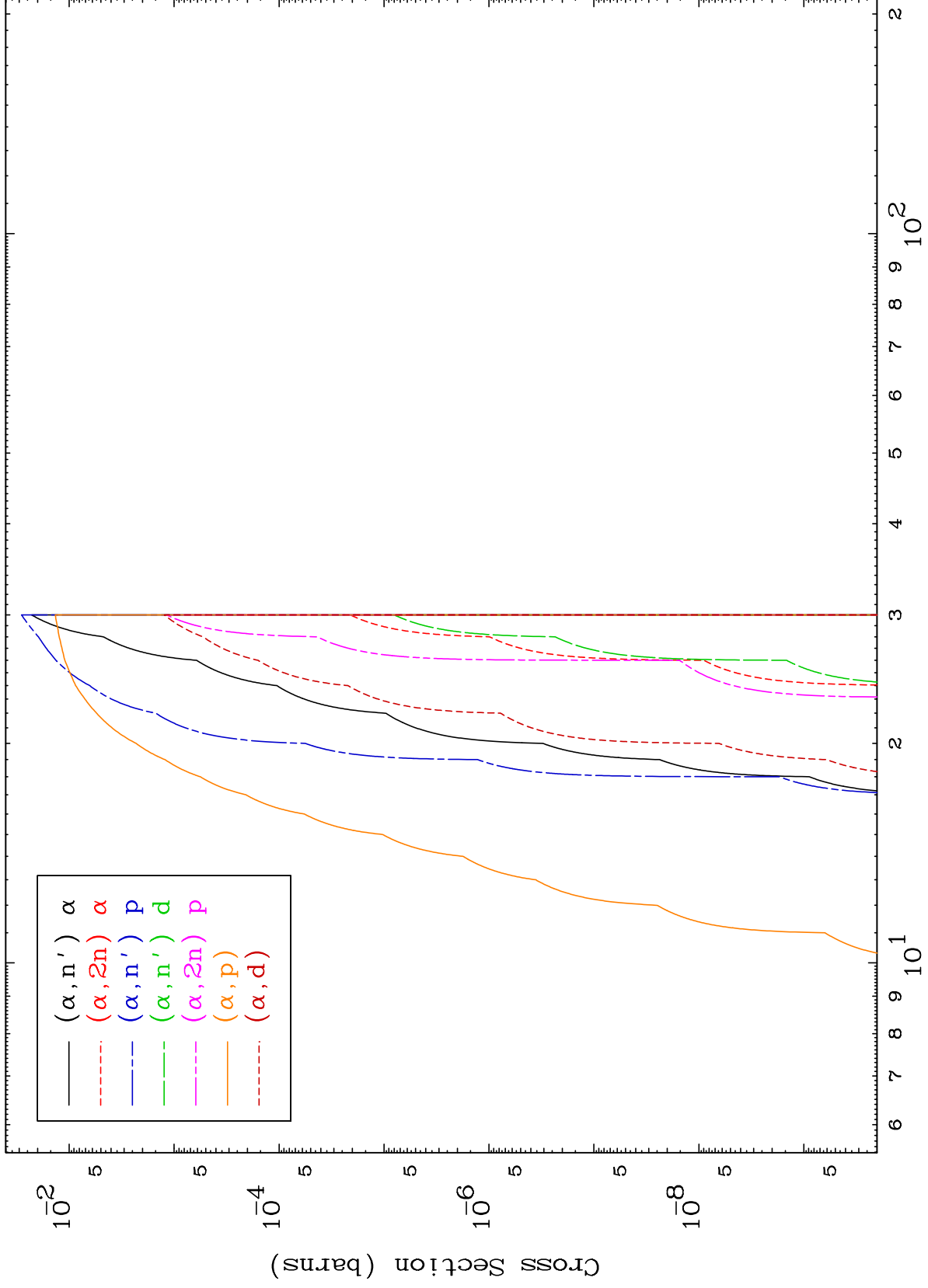
67-Ho-157

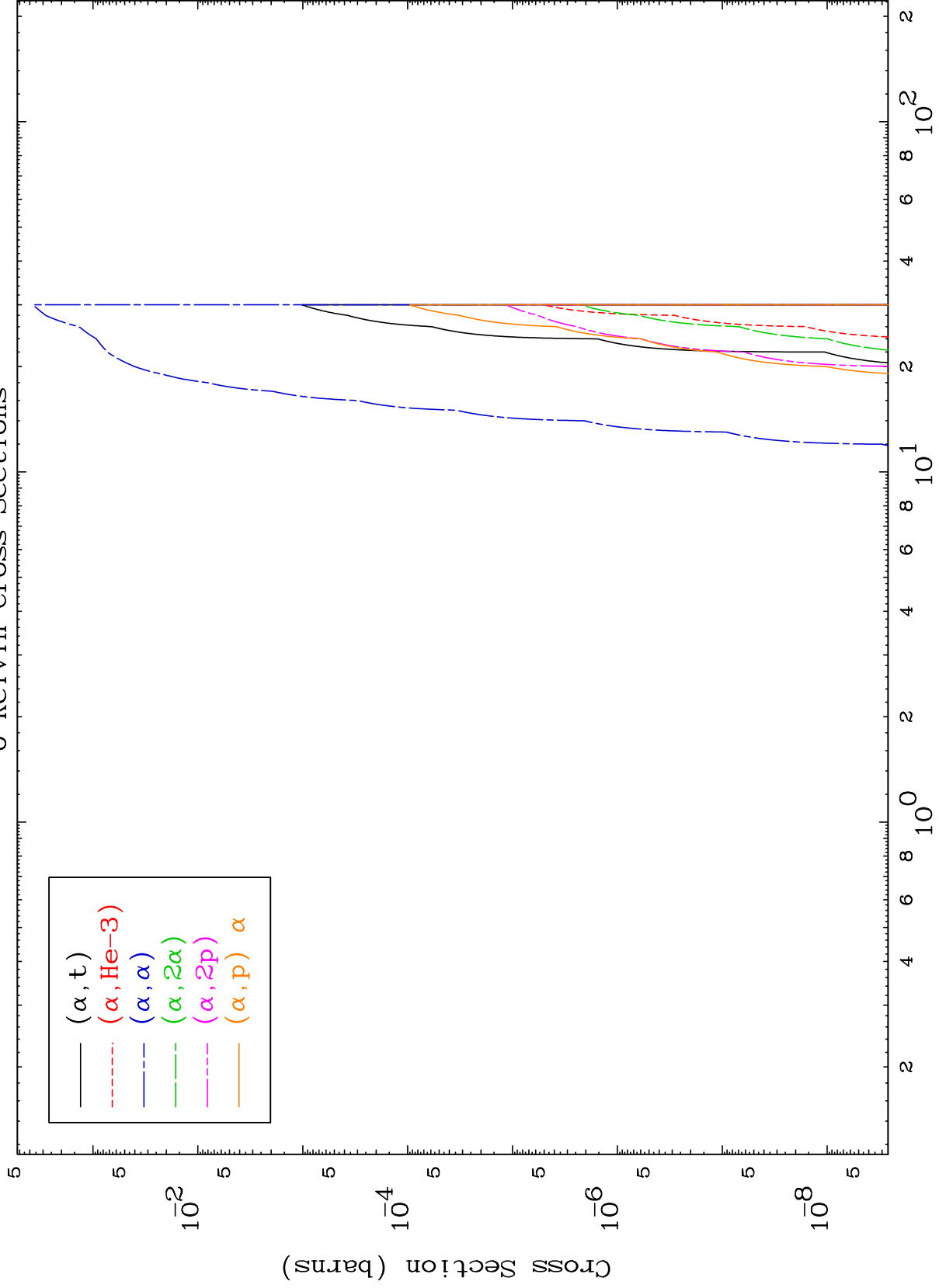
MAT 6701

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

67-Ho-157





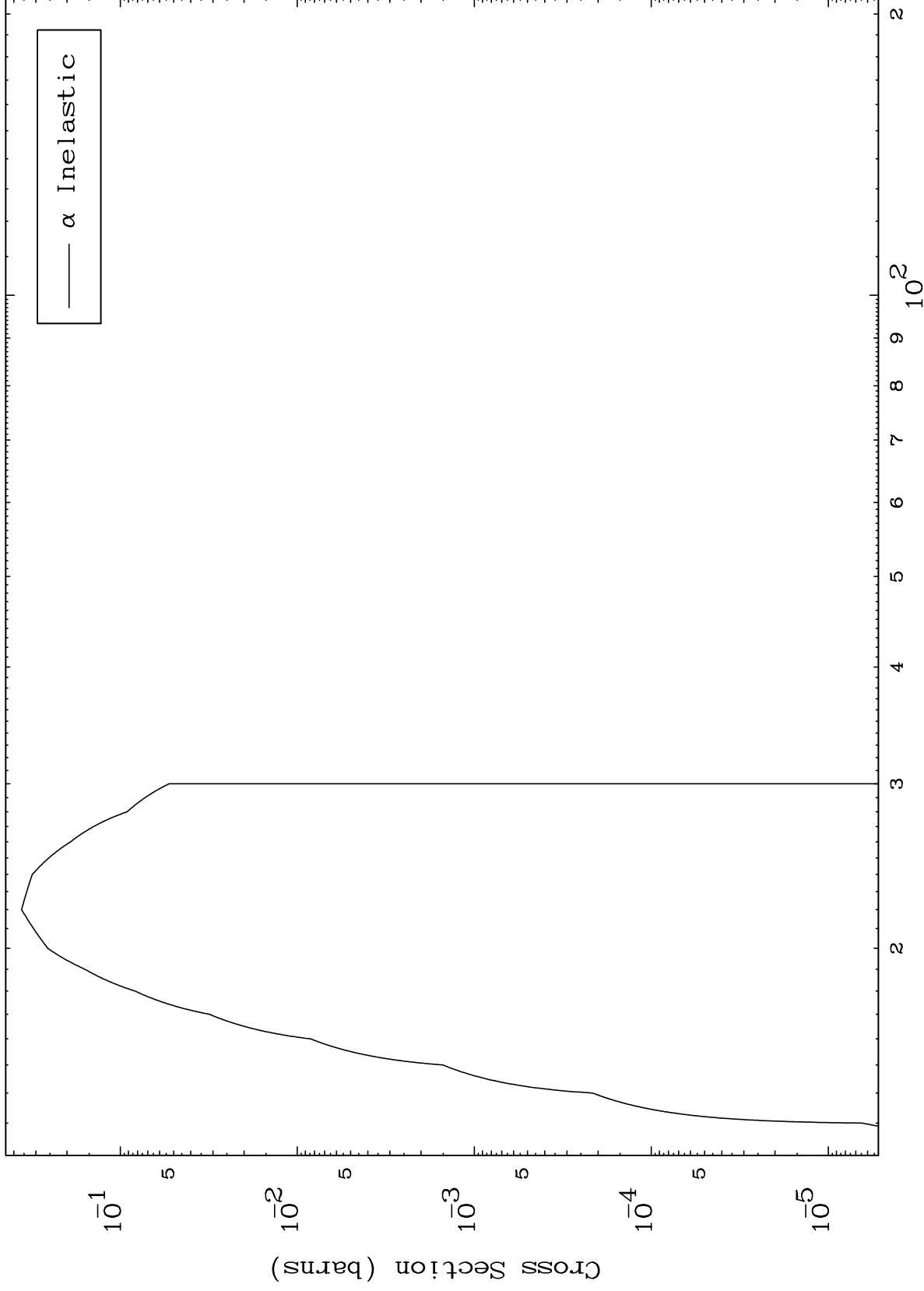


MAT 6701

( $\alpha, n'$ ) Level

67-Ho-157

0 Kelvin Cross Sections



Incident Energy (MeV)

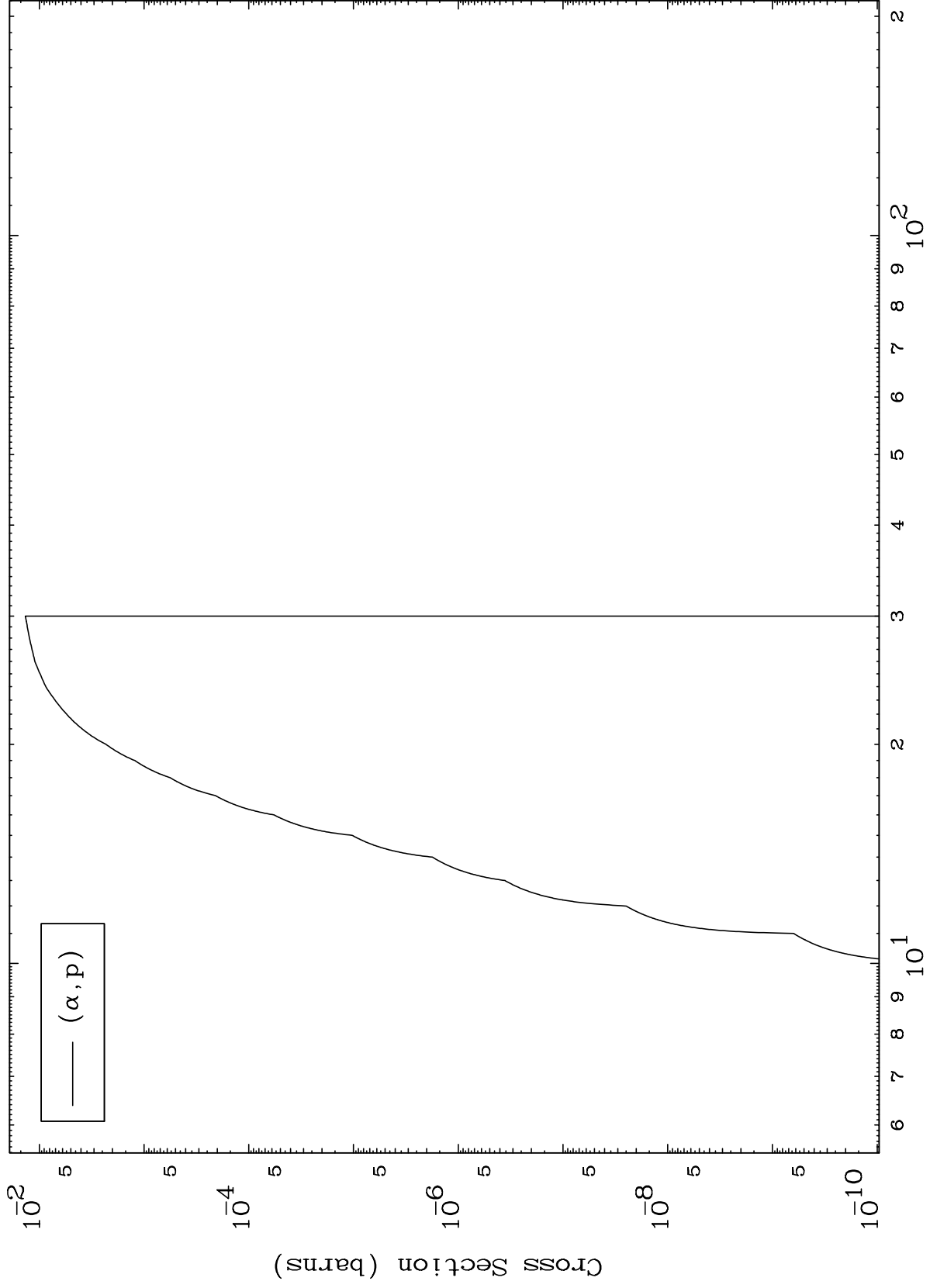
67-Ho-157

5

MAT 6701

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

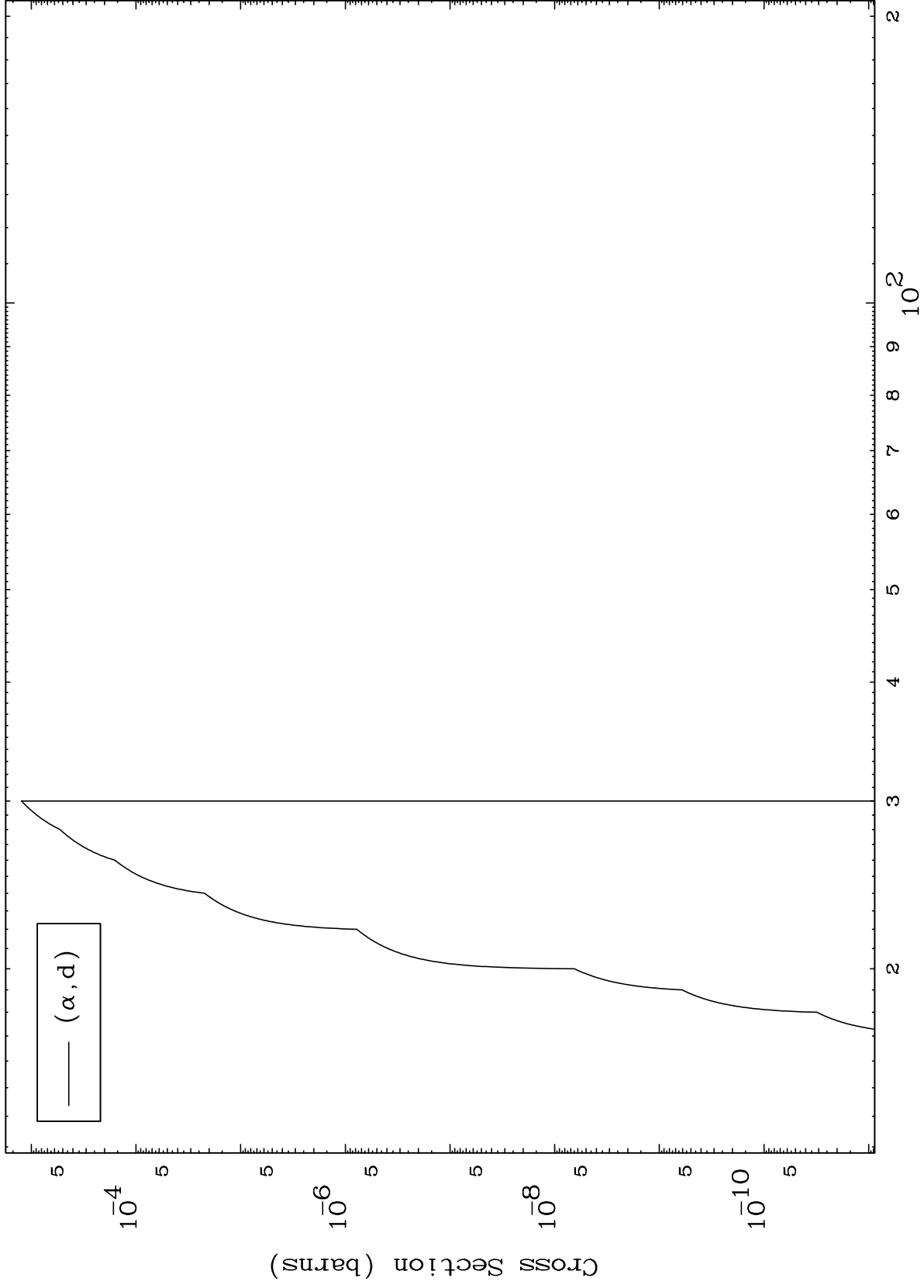
67-Ho-157



MAT 6701

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

67-Ho-157



7

Incident Energy (MeV)

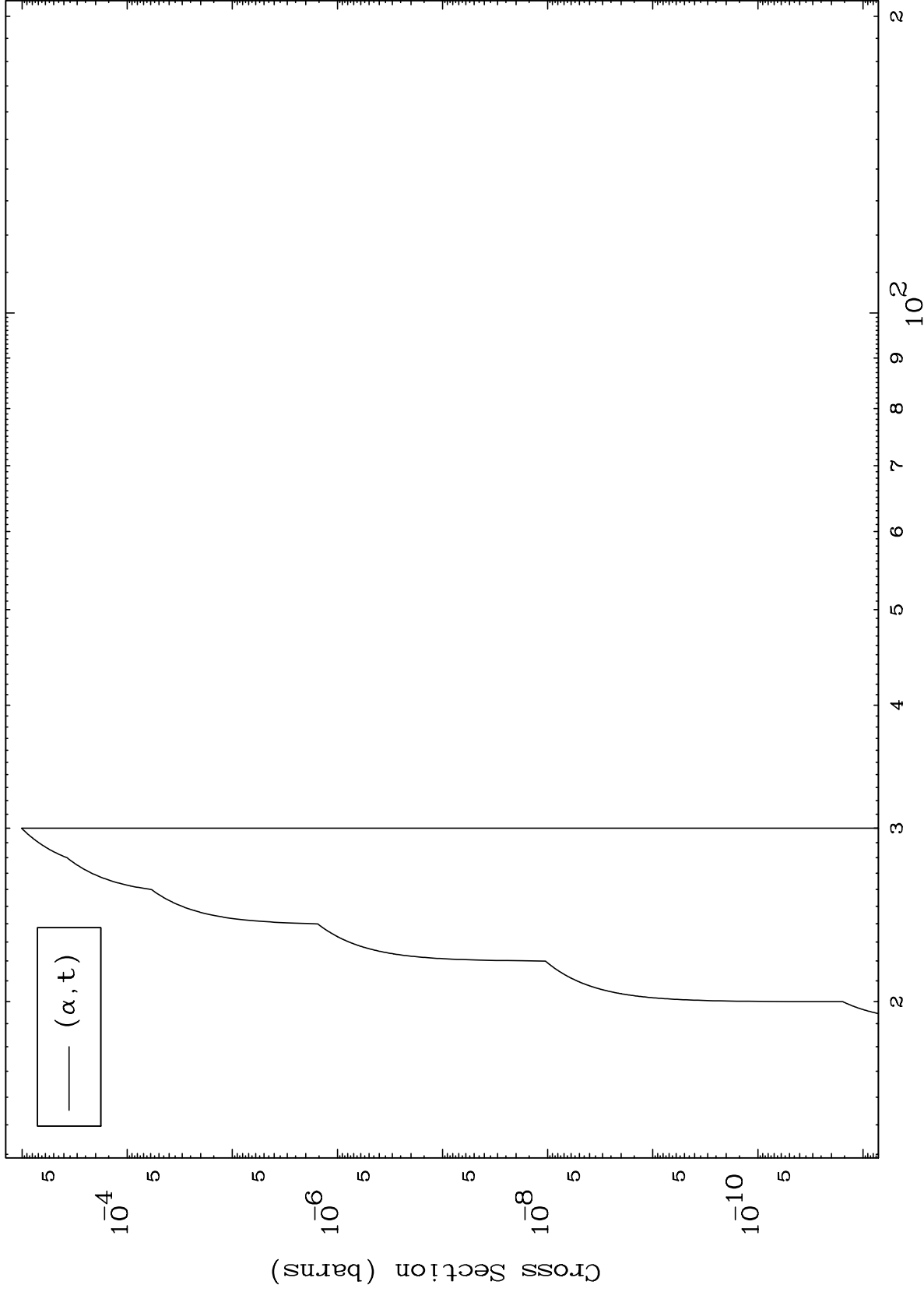
67-Ho-157



MAT 6701

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

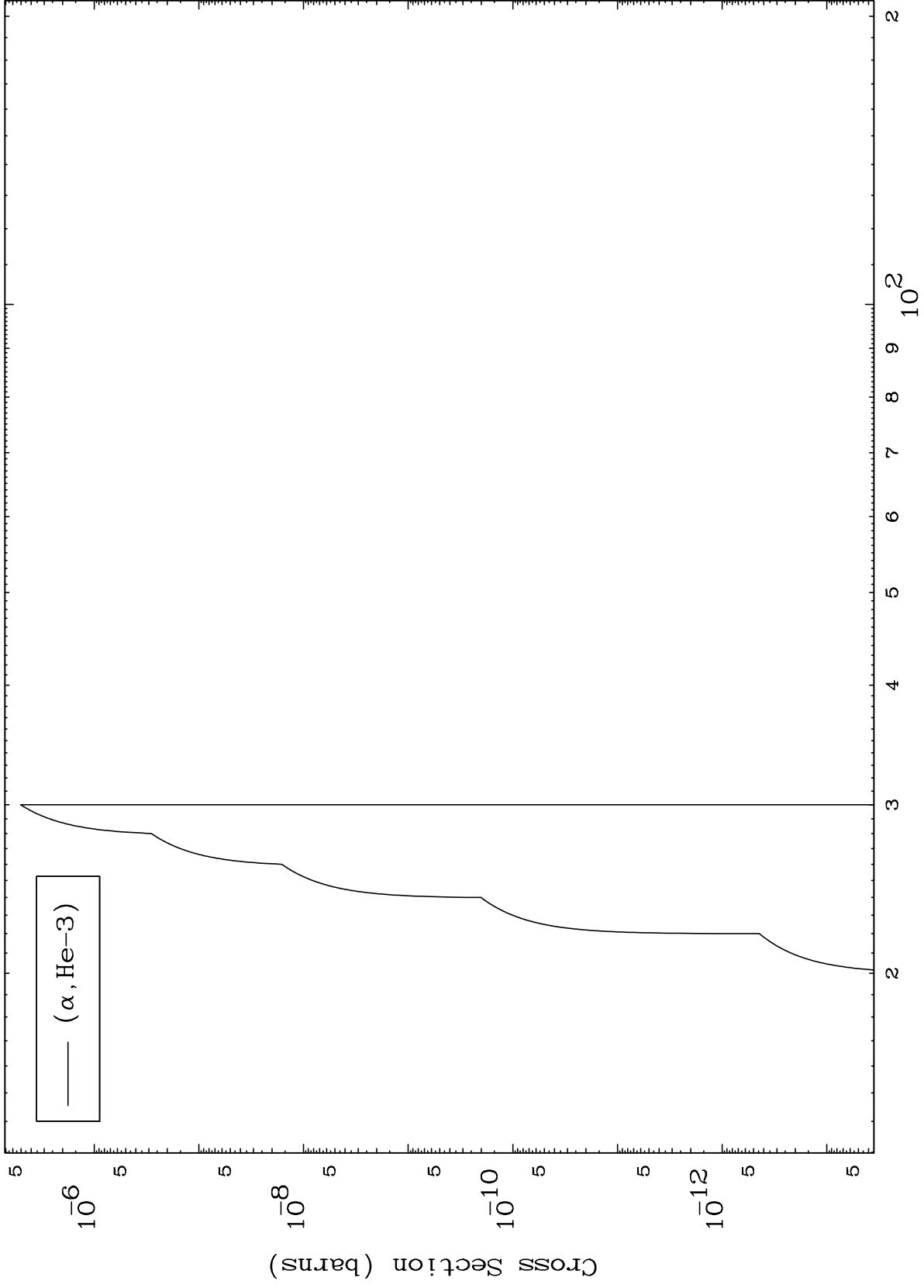
67-Ho-157



8

Incident Energy (MeV)

67-Ho-157

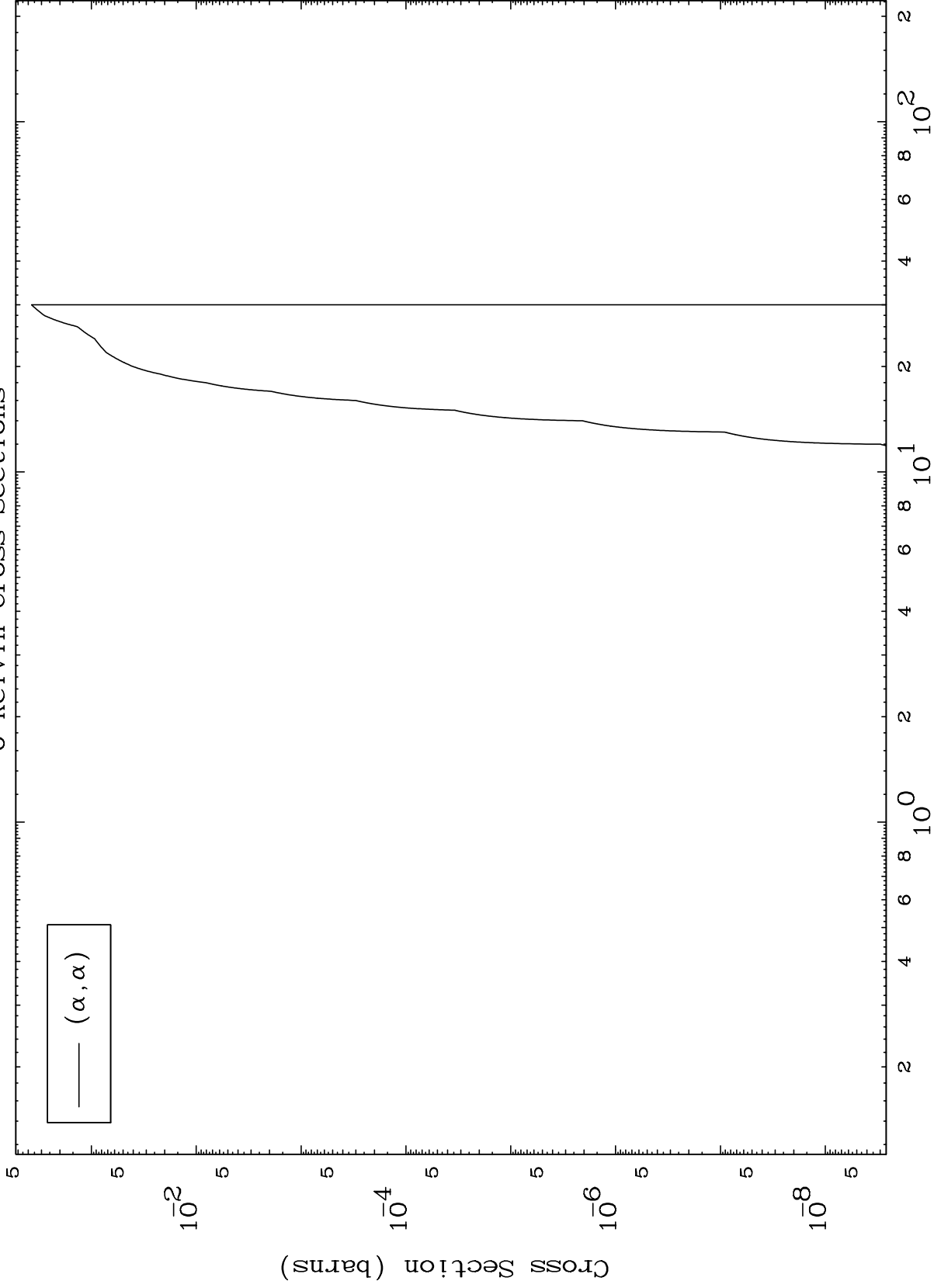


MAT 6701

( $\alpha, \alpha$ ) Levels

67-Ho-157

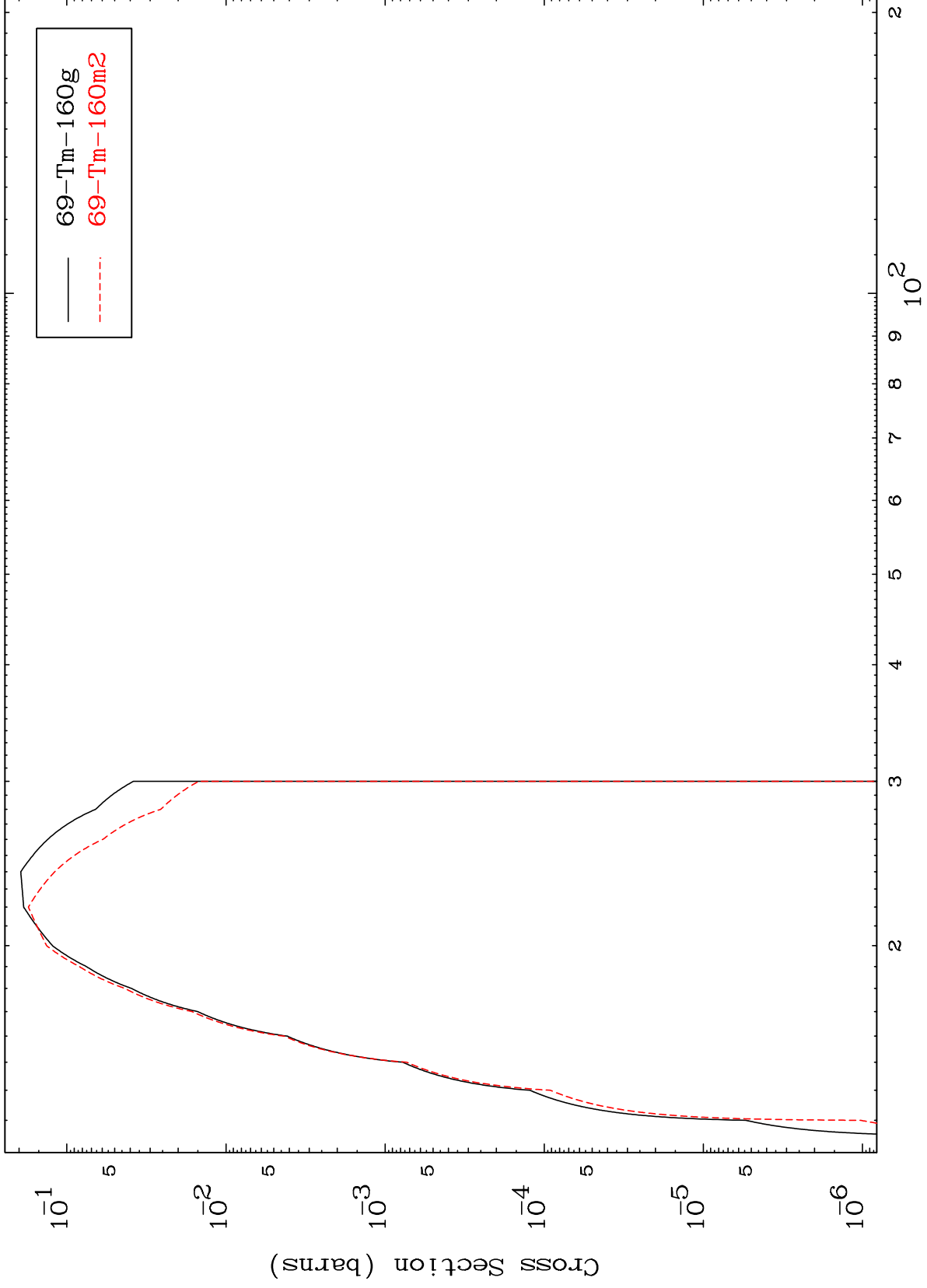
0 Kelvin Cross Sections



10

Incident Energy (MeV)

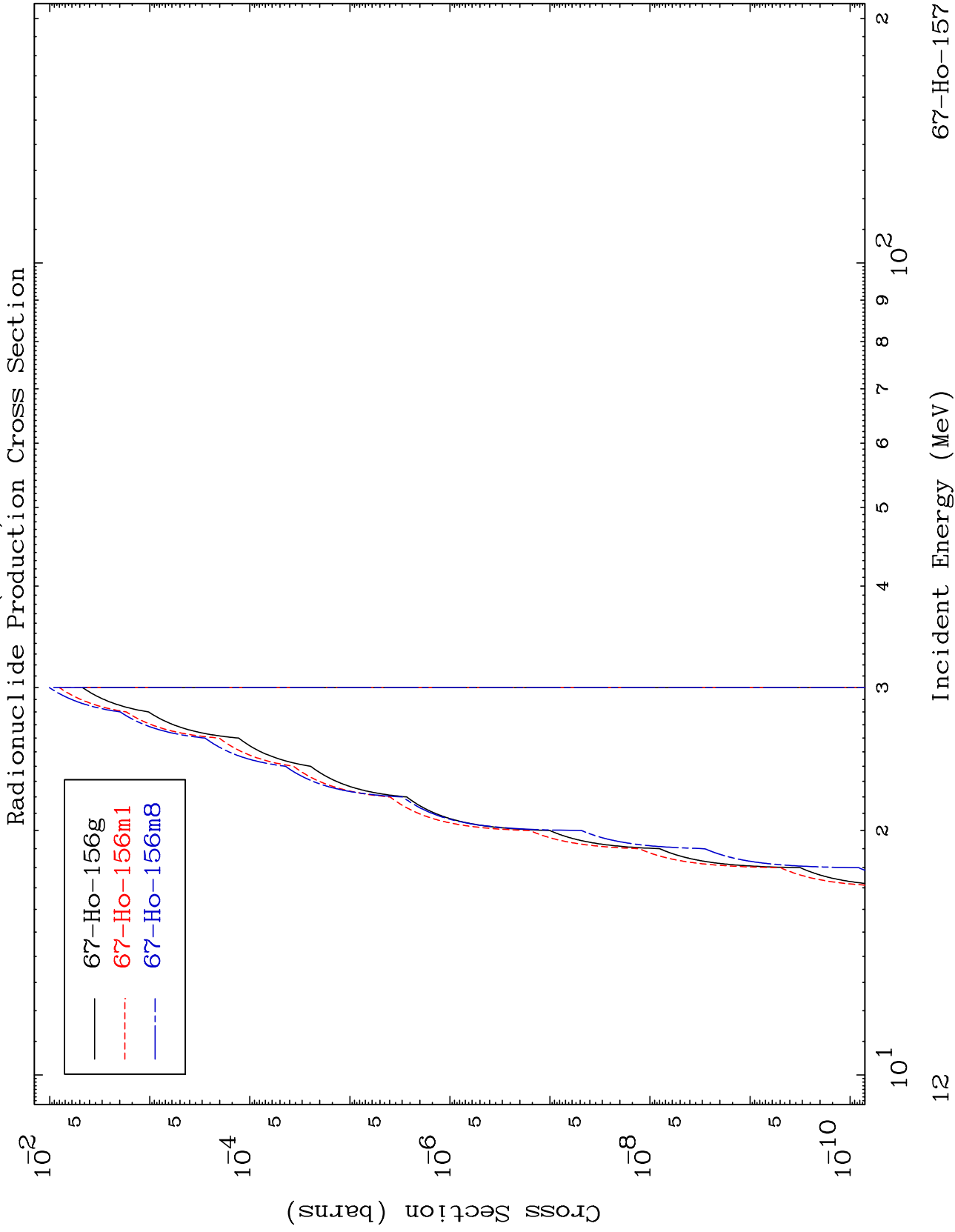
67-Ho-157



MAT 6701

$(\alpha, n')$   $\alpha$

67-Ho-157



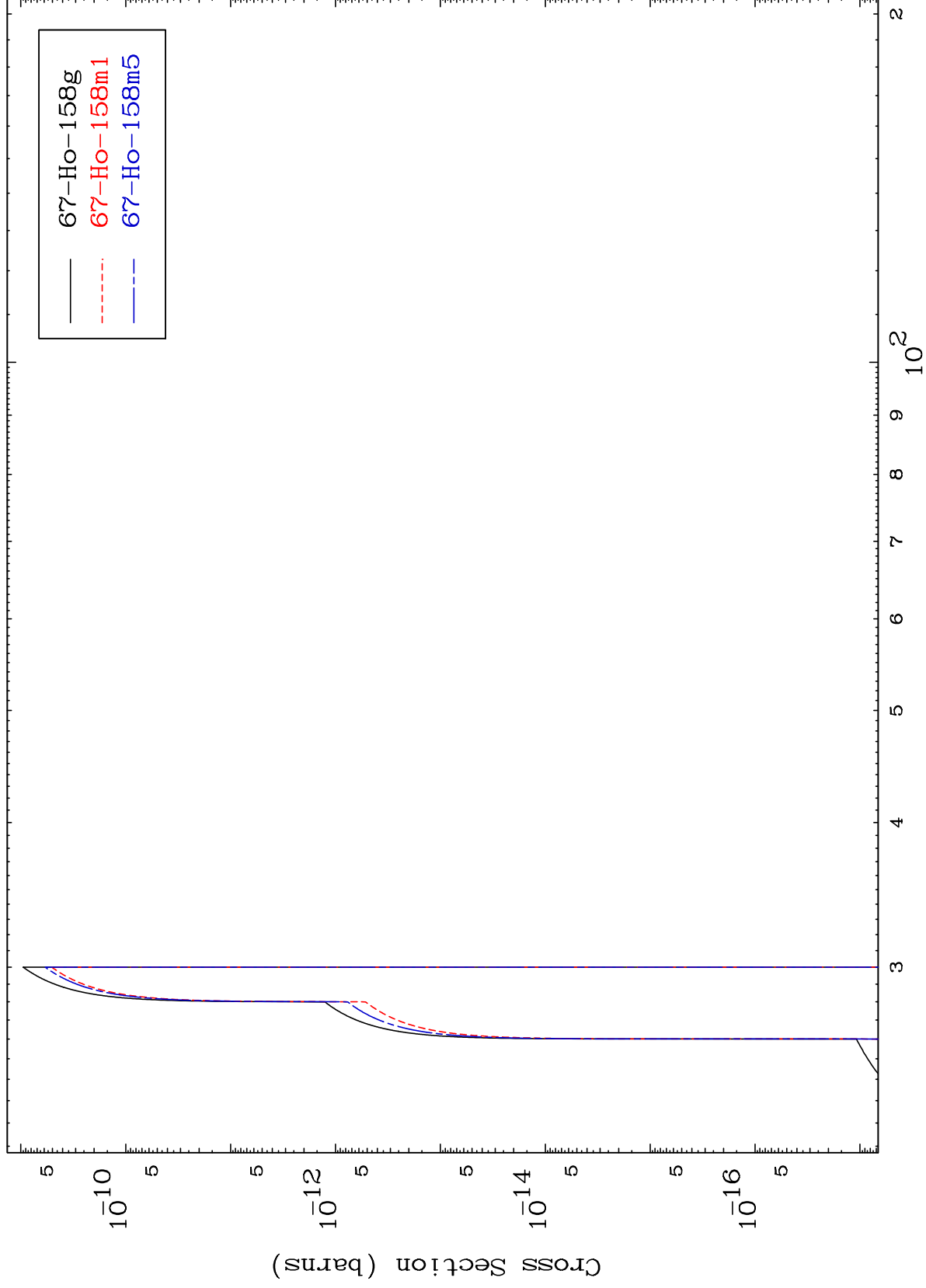
67-Ho-157

MAT 6701

67-Ho-157

( $\alpha, 2n$ ) p

Radionuclide Production Cross Section



67-Ho-157

Incident Energy (MeV)

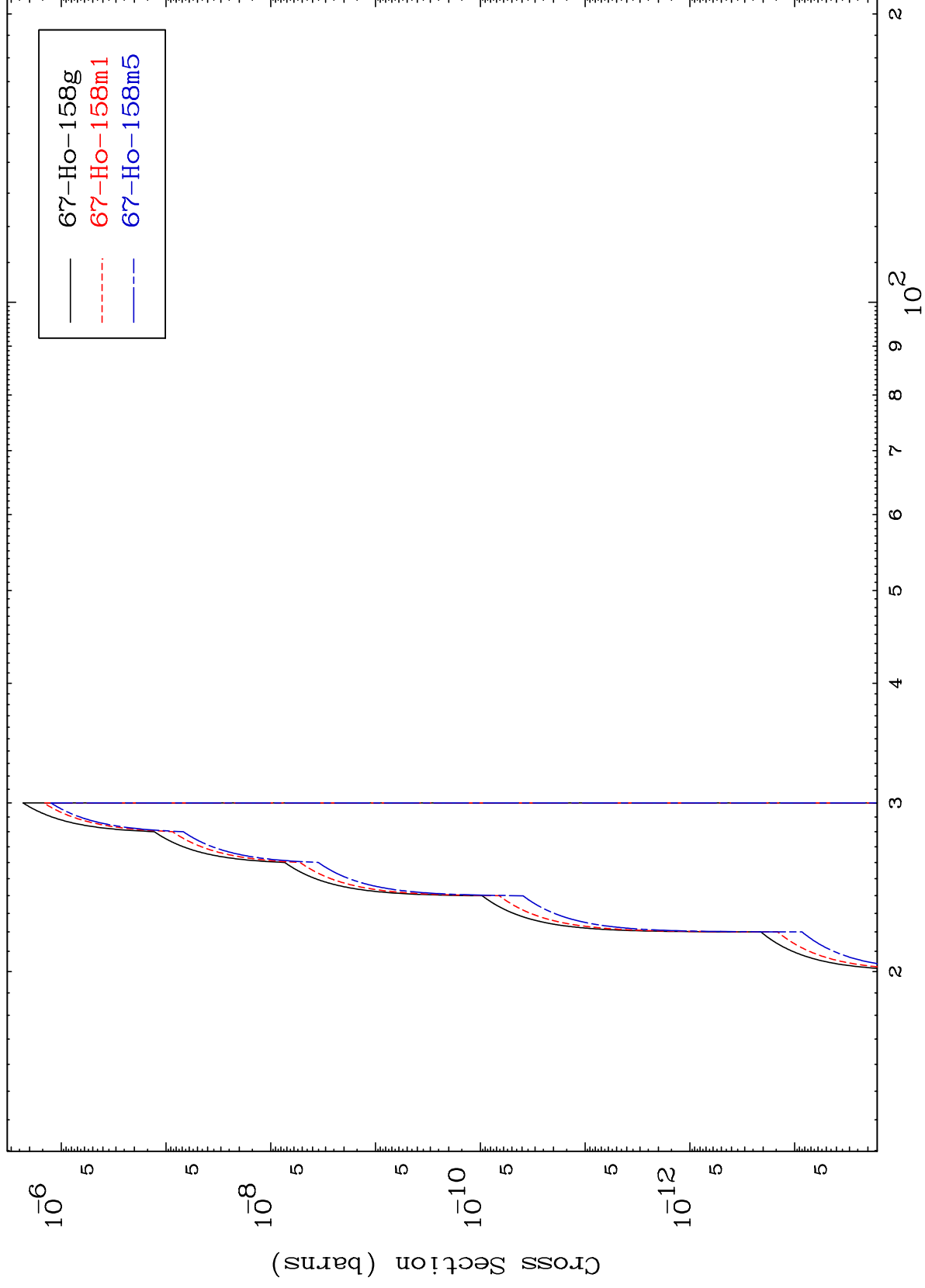
13

MAT 6701

( $\alpha, \text{He-3}$ )

67-Ho-157

Radionuclide Production Cross Section



14

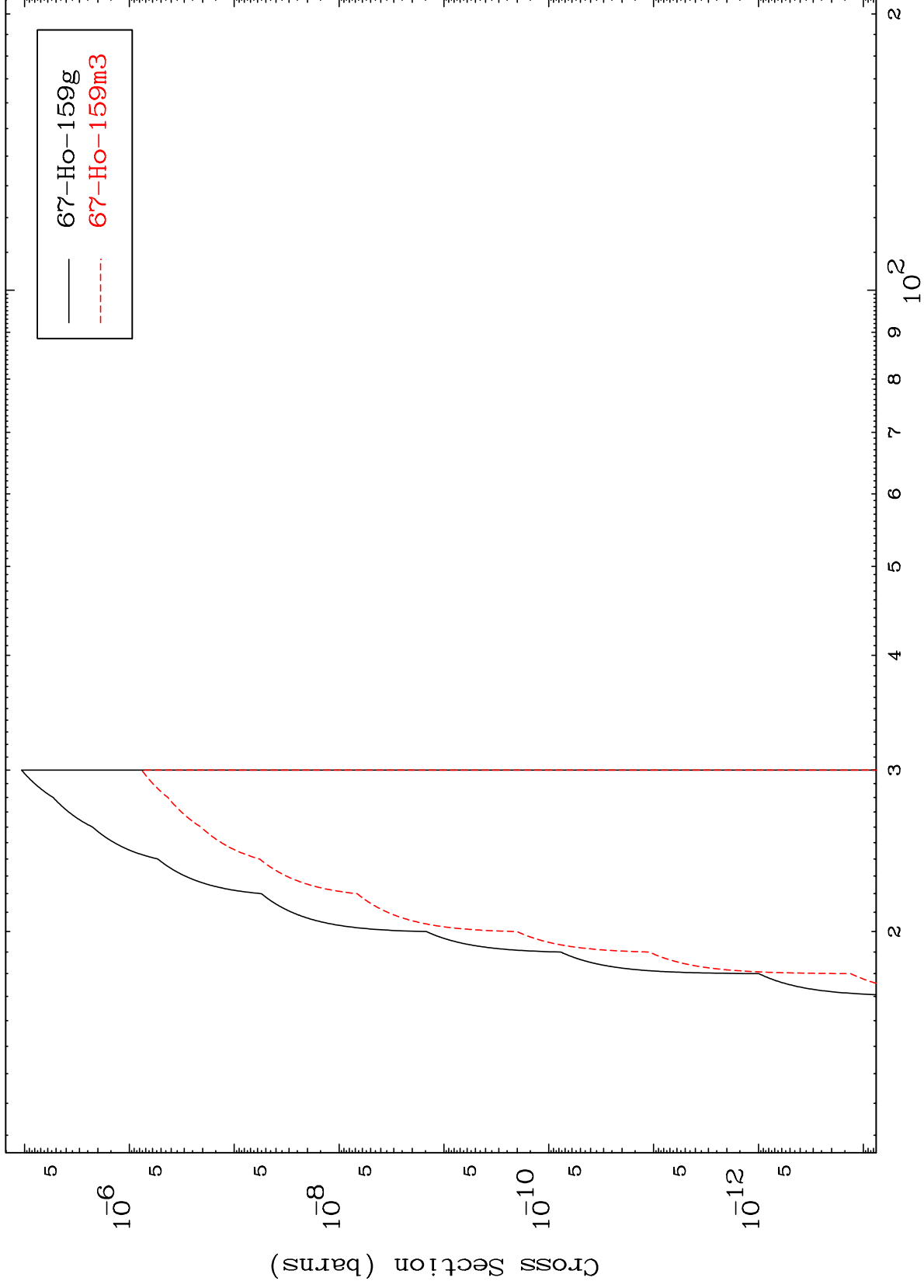
Incident Energy (MeV)

67-Ho-157

MAT 6701

67-Ho-157

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

67-Ho-157