

Program EVALPLOT  
(Version 2018-1)

by

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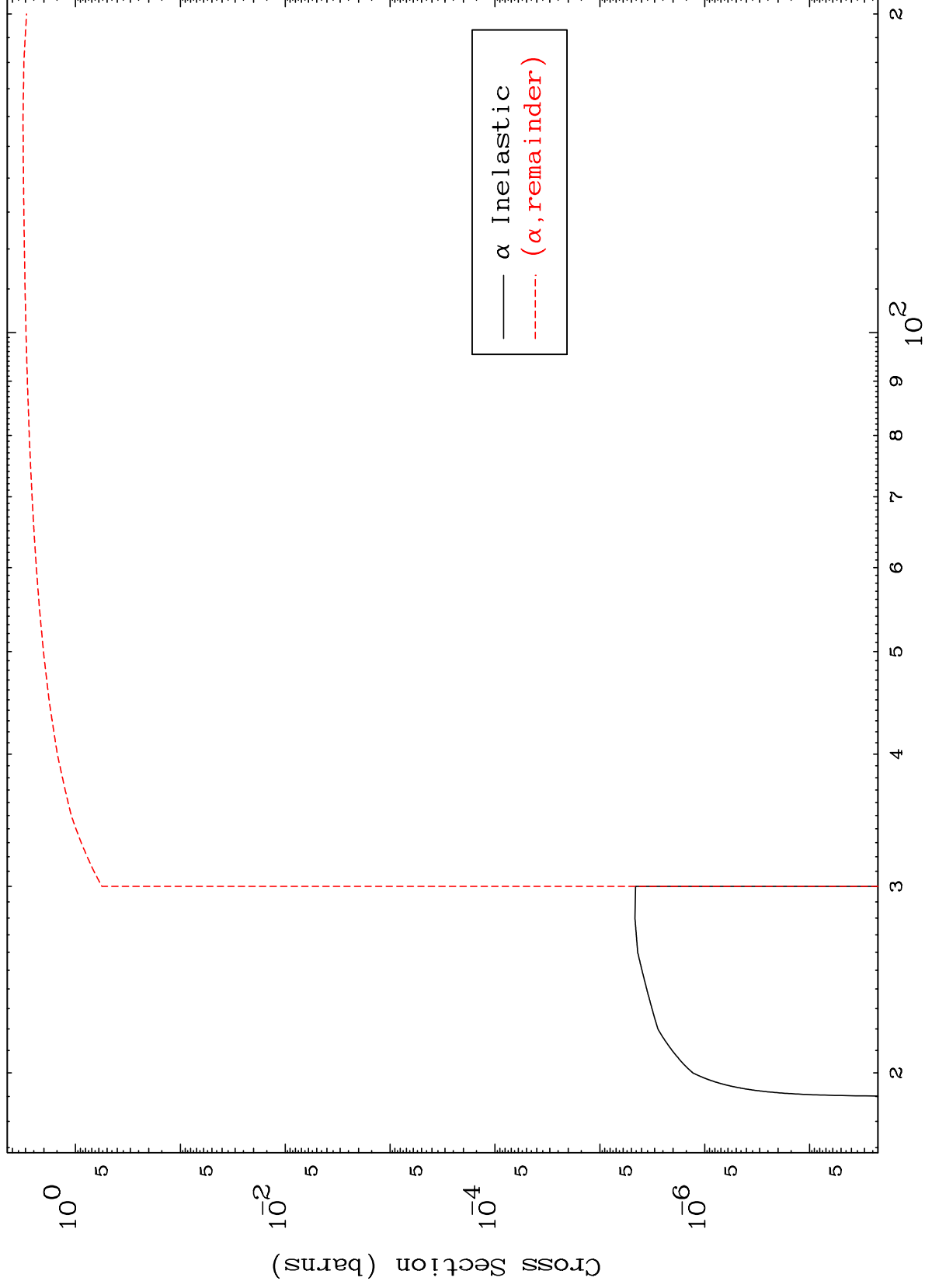
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 871

$\alpha$  Major  
0 Kelvin Cross Sections

108-Hs-271



1

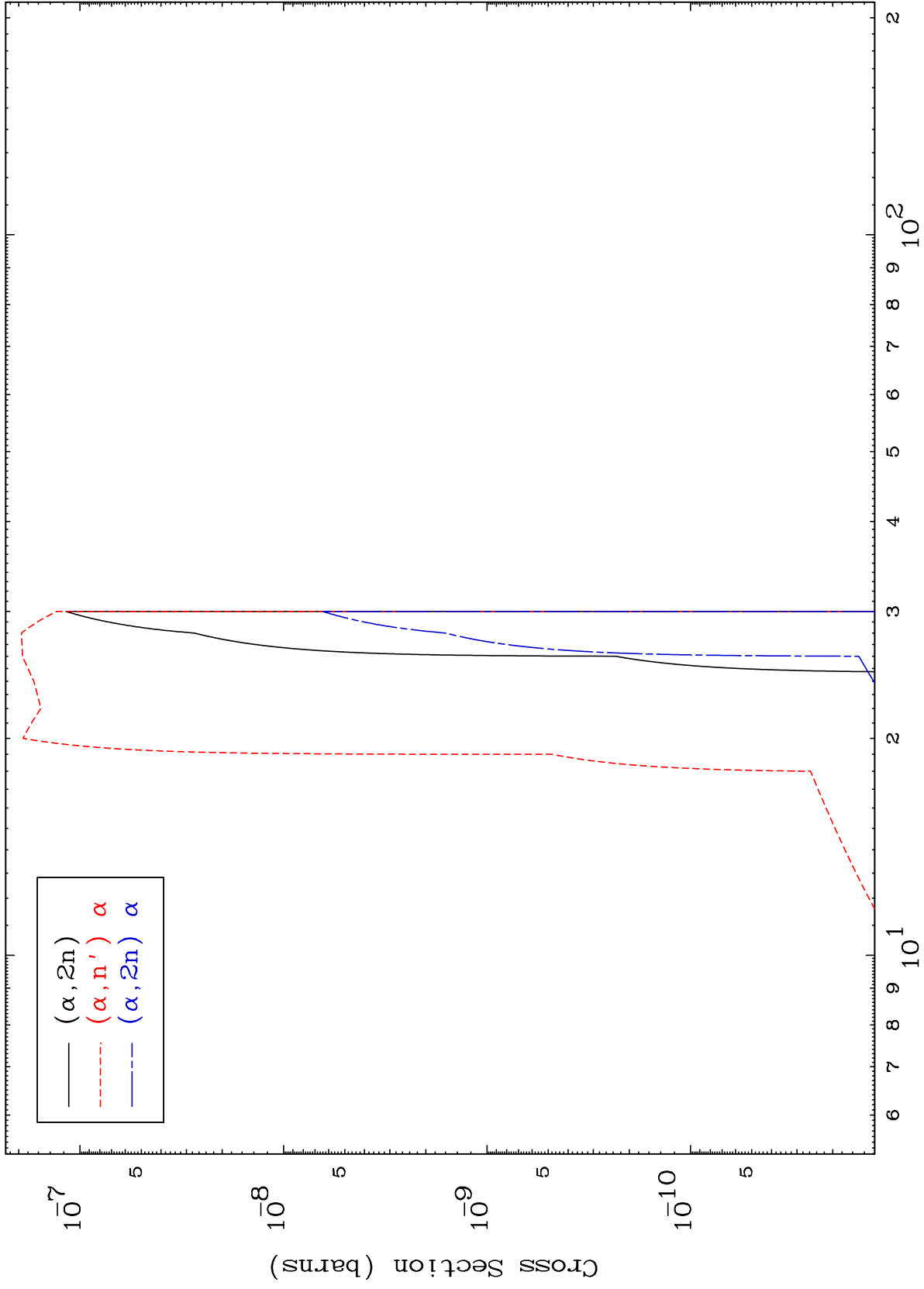
Incident Energy (MeV)

108-Hs-271

MAT 871

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

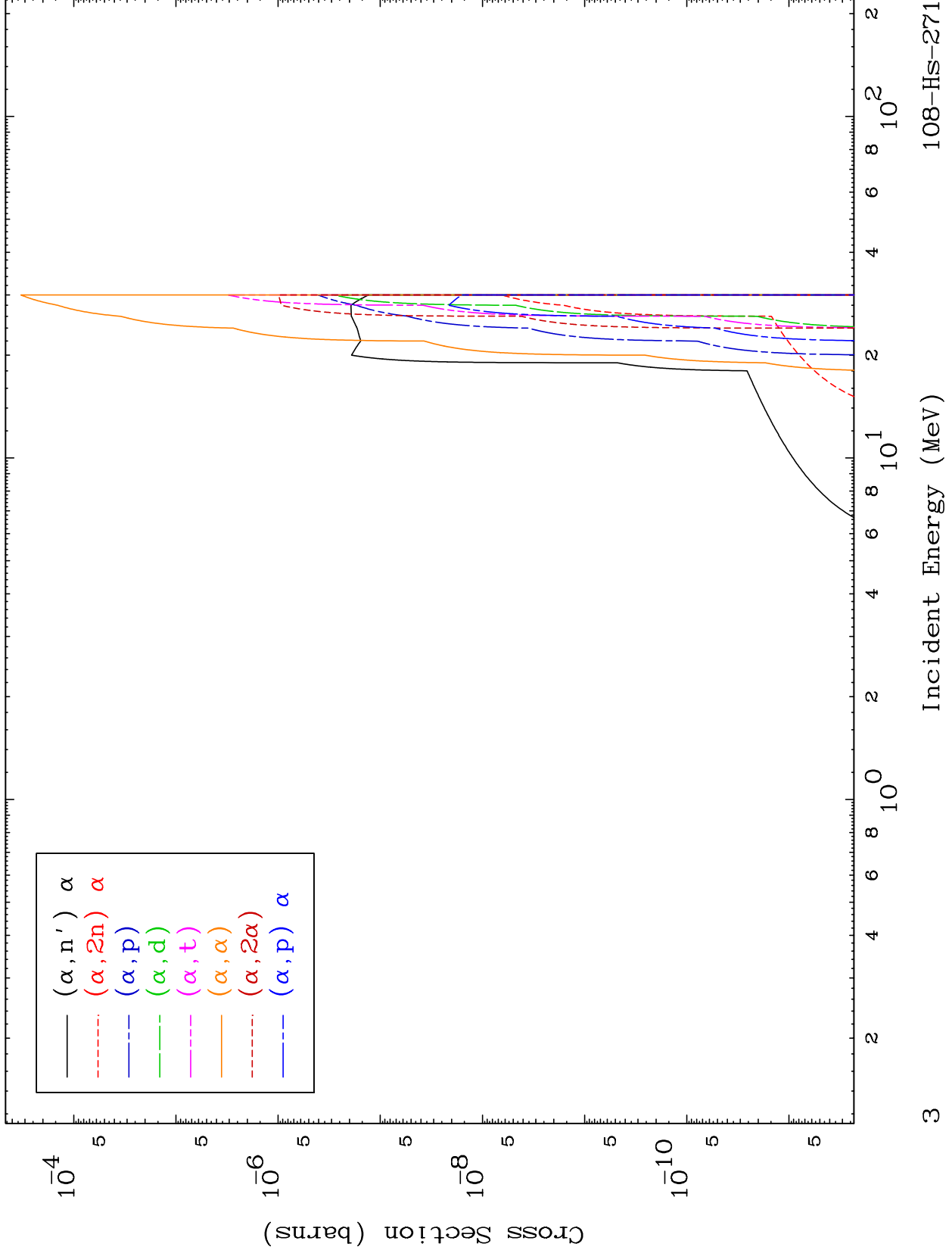
108-Hs-271



2

Incident Energy (MeV)

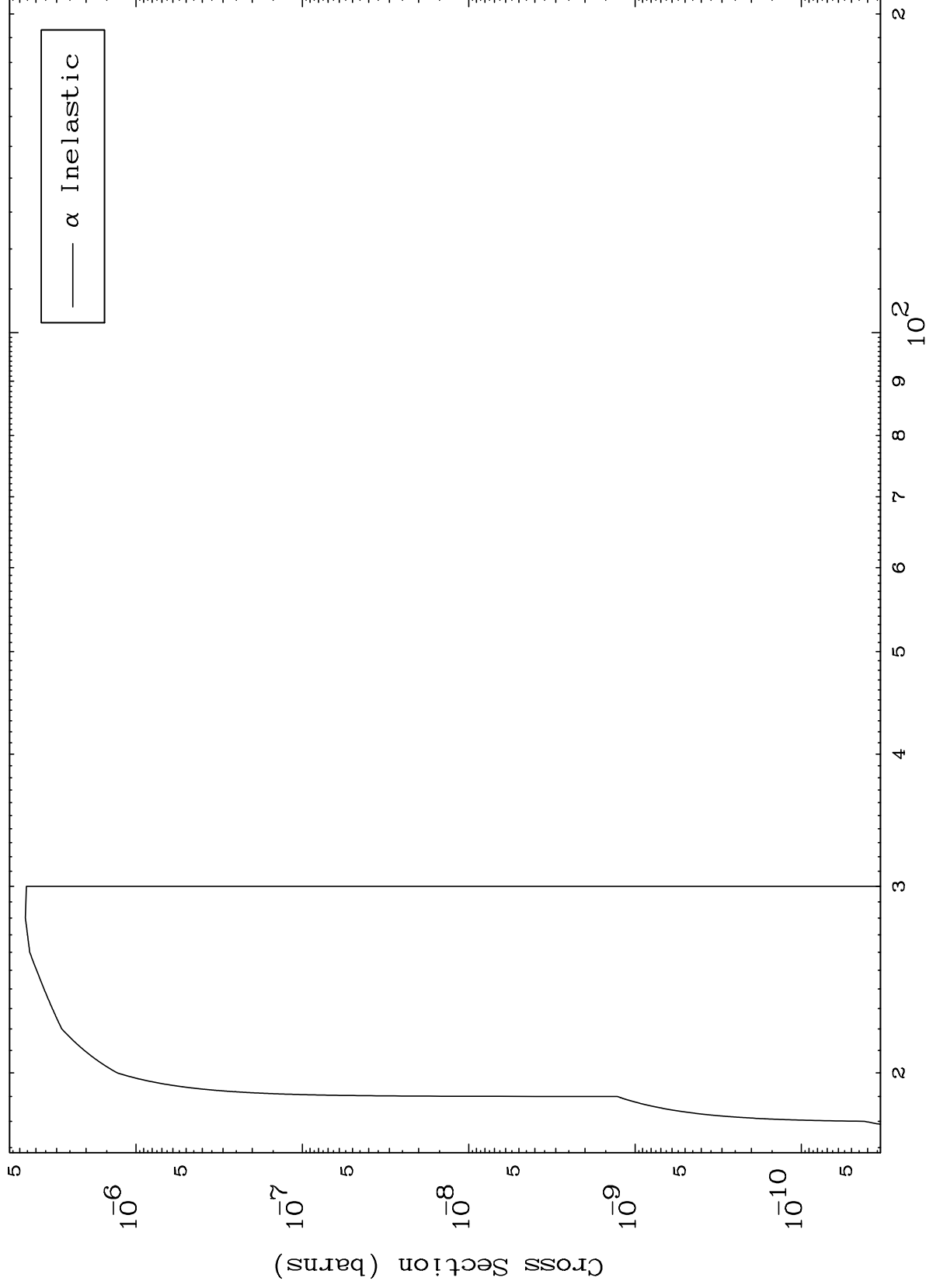
108-Hs-271



MAT 871

108-Hs-271

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



108-Hs-271

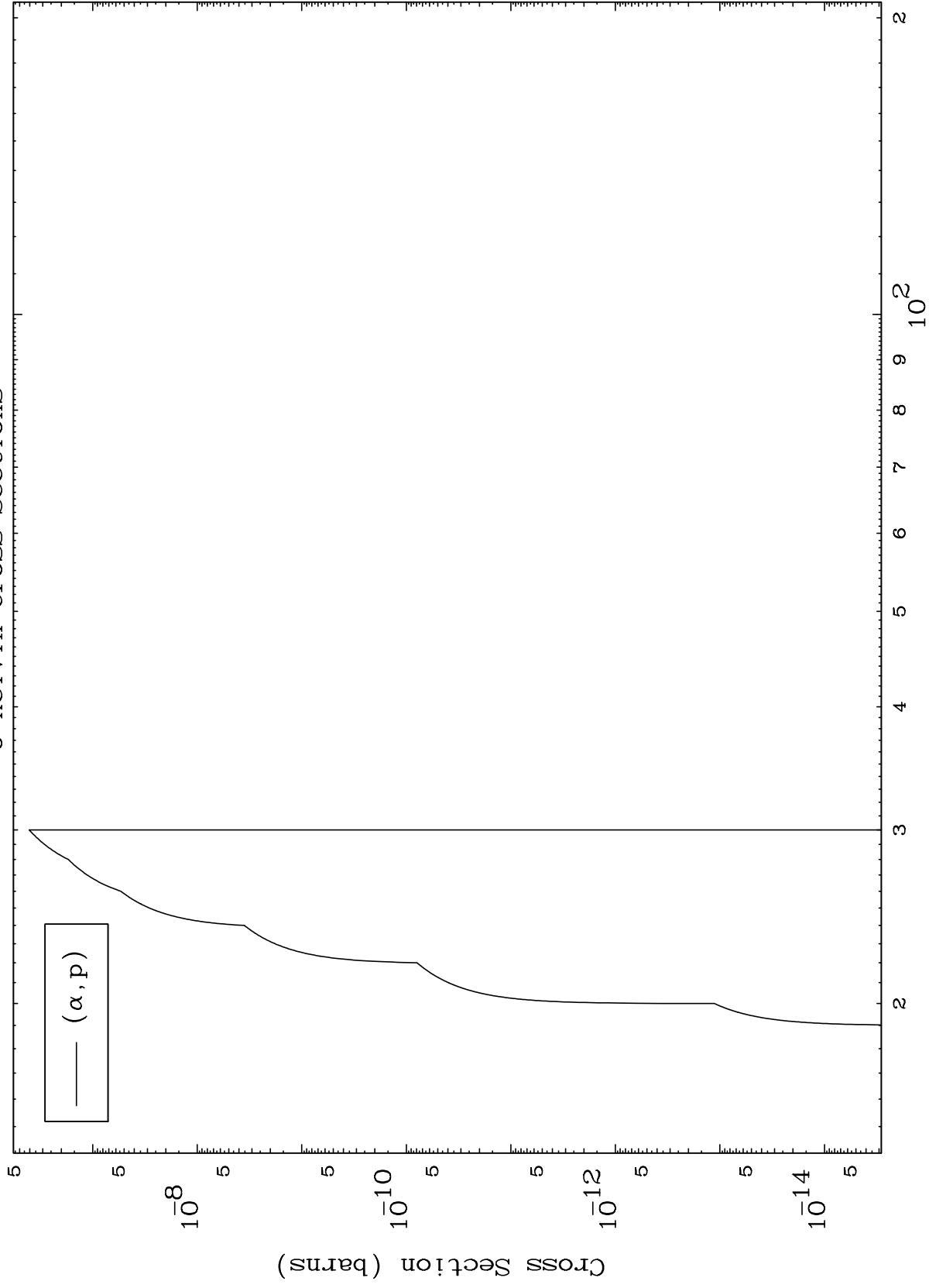
Incident Energy (MeV)

4

MAT 871

108-Hs-271

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



108-Hs-271

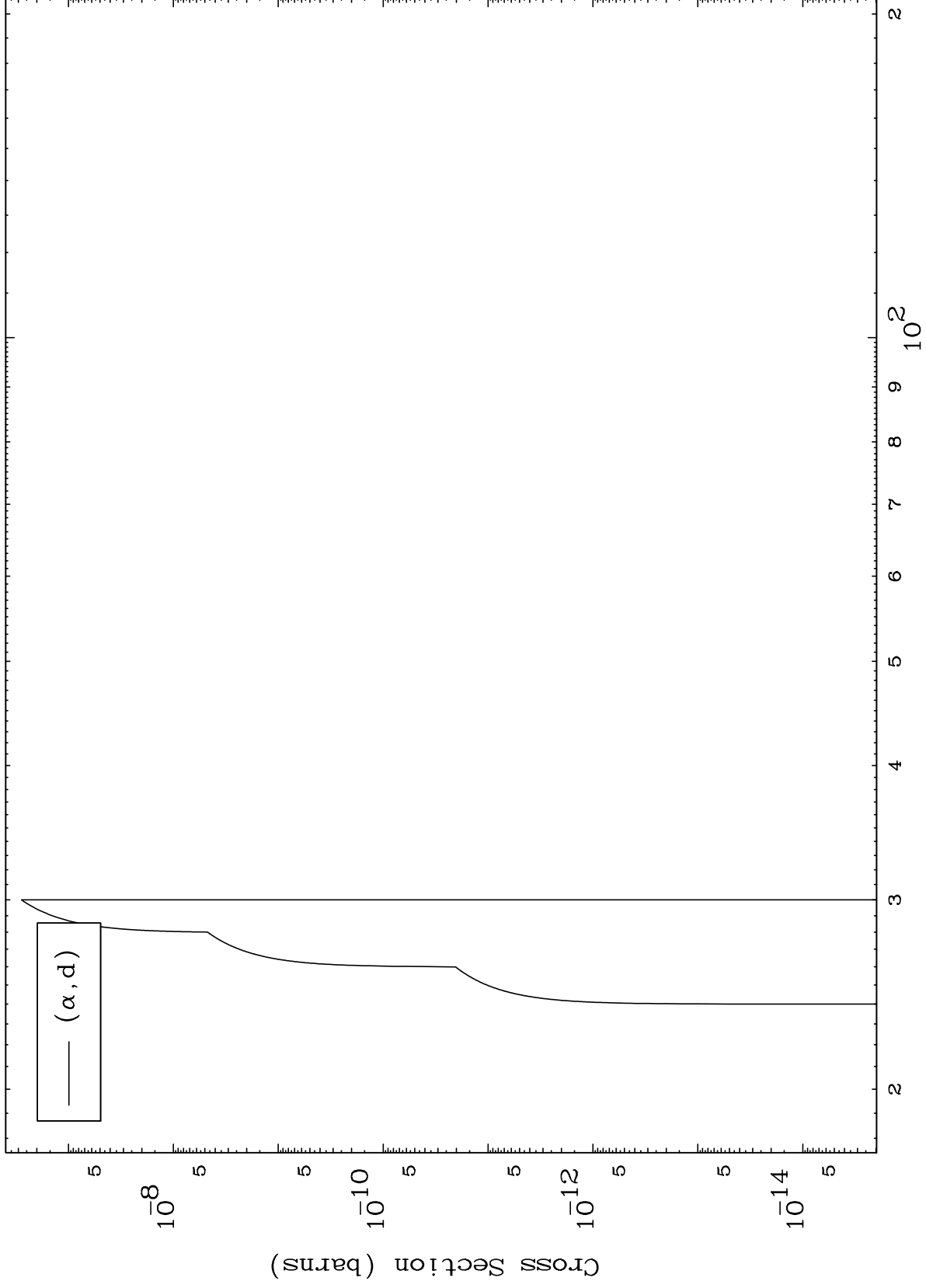
Incident Energy (MeV)

5

MAT 871

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

108-Hs-271



6

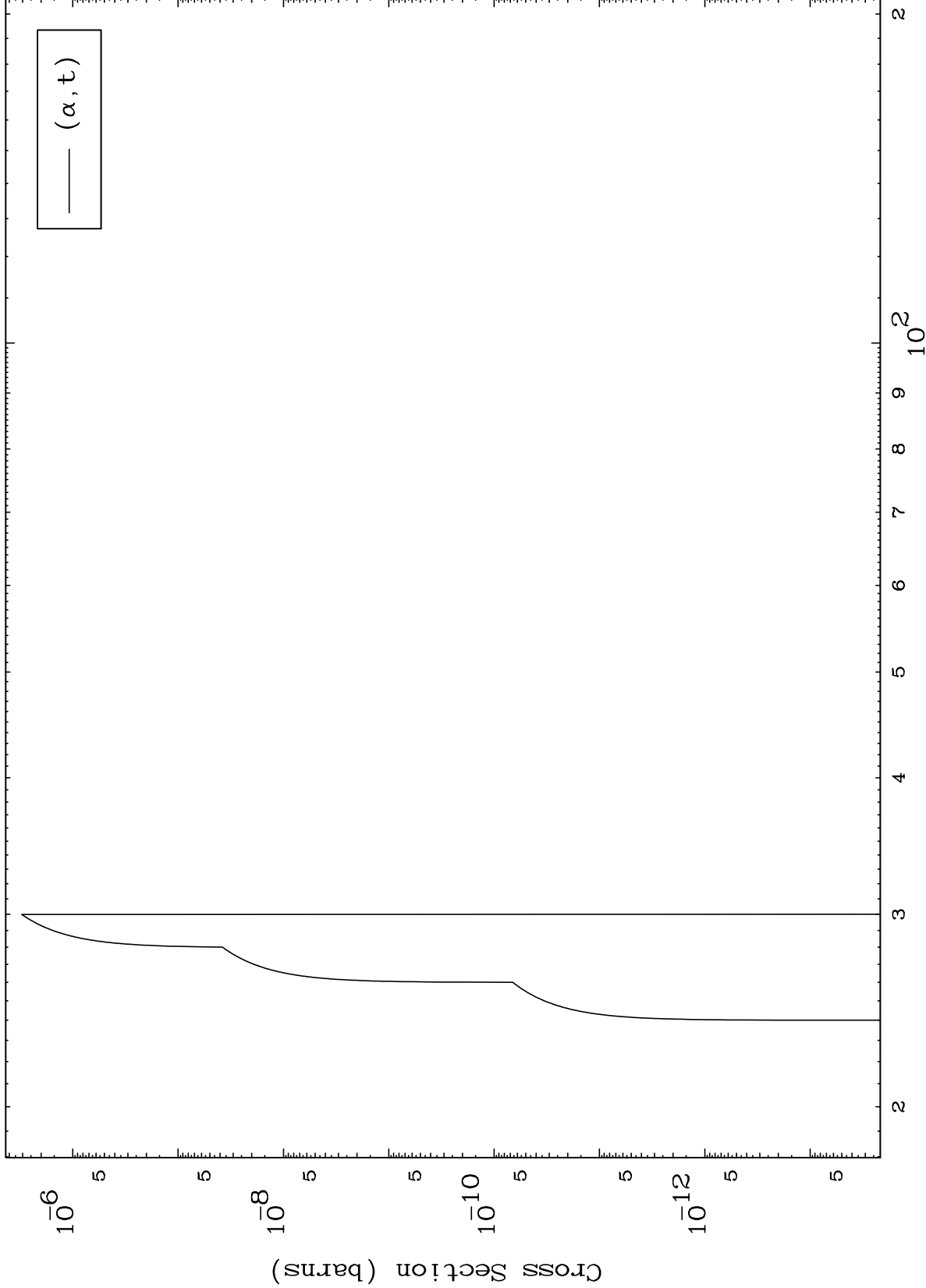
Incident Energy (MeV)

108-Hs-271

MAT 871

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

108-Hs-271



7

Incident Energy (MeV)

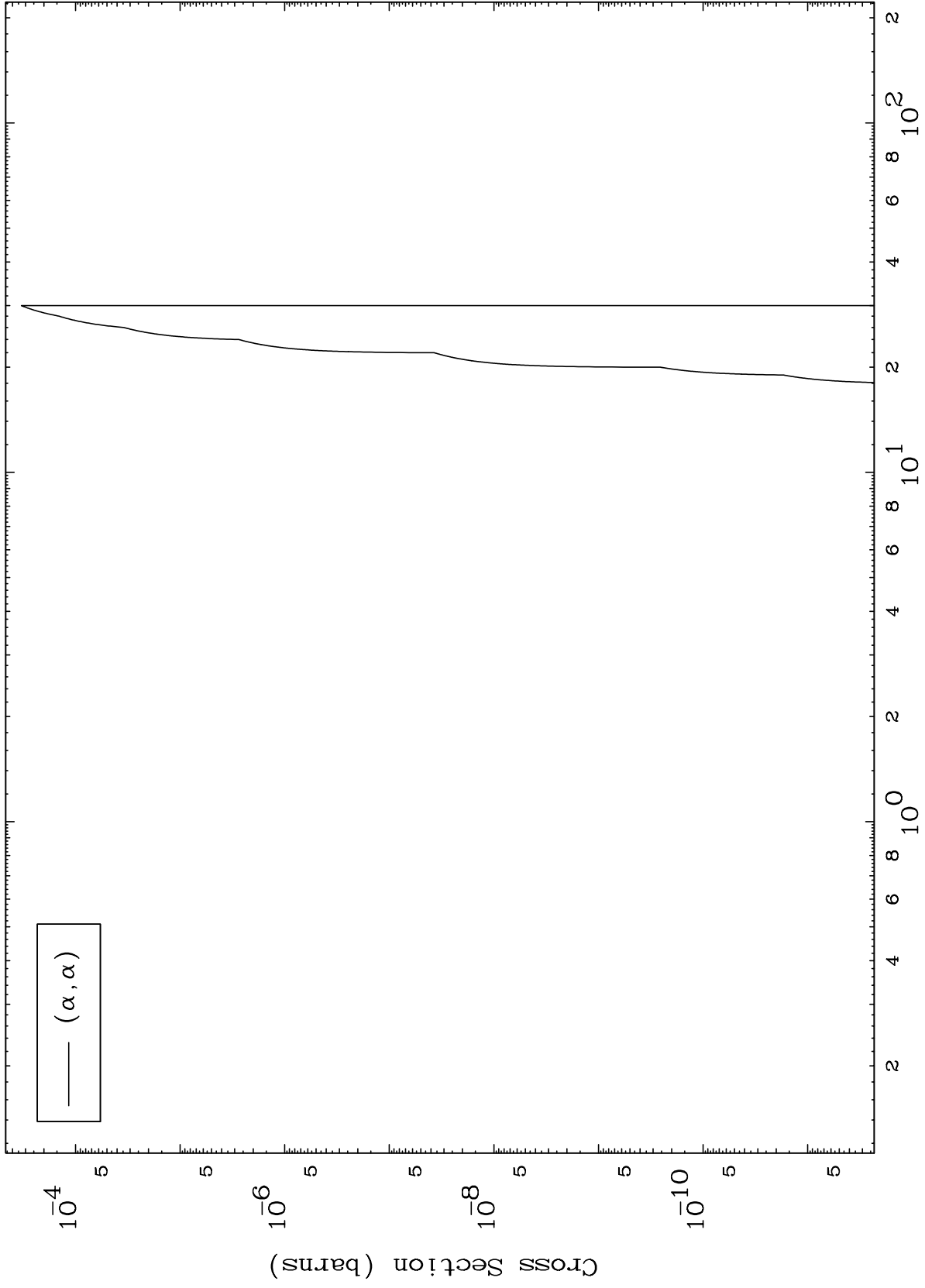
108-Hs-271



MAT 871

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

108-Hs-271



MAT 871

108-Hs-271

$\alpha$  Fission

Radionuclide Production Cross Section

