

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

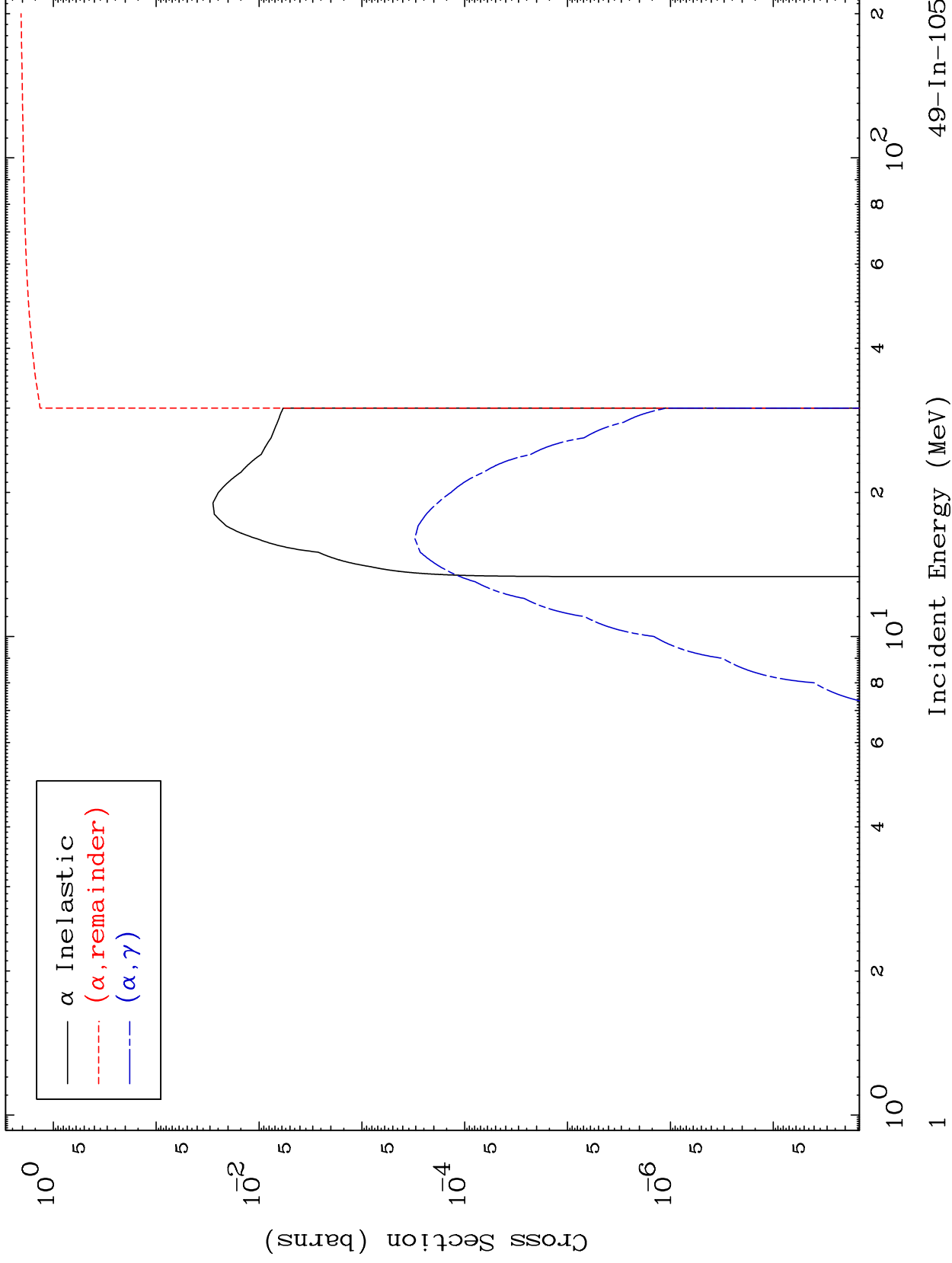
Press Mouse Button to Start

MAT 4901

$\alpha$  Major

49-In-105

0 Kelvin Cross Sections



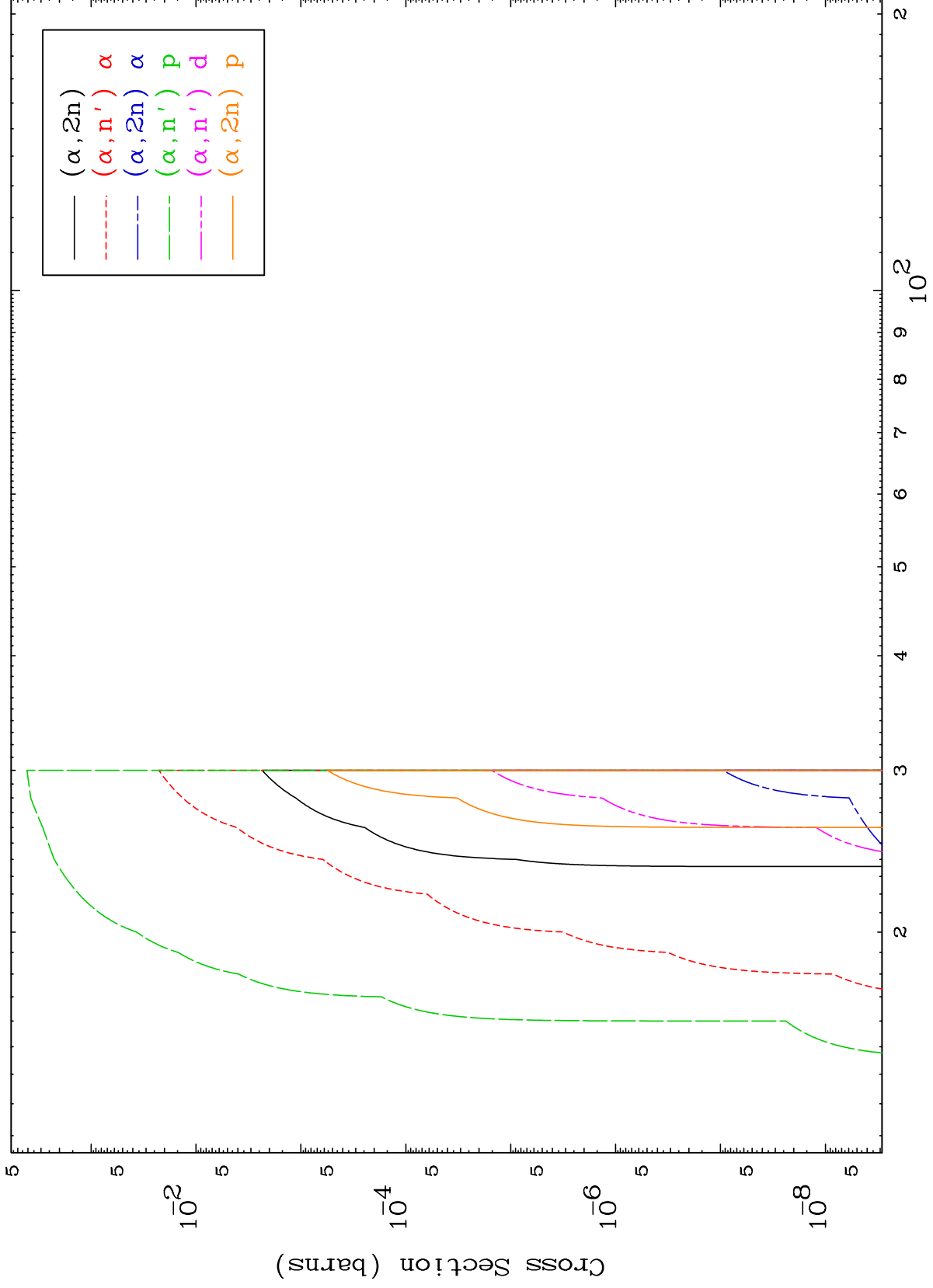
Incident Energy (MeV)

49-In-105

MAT 4901

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

49-In-105



2

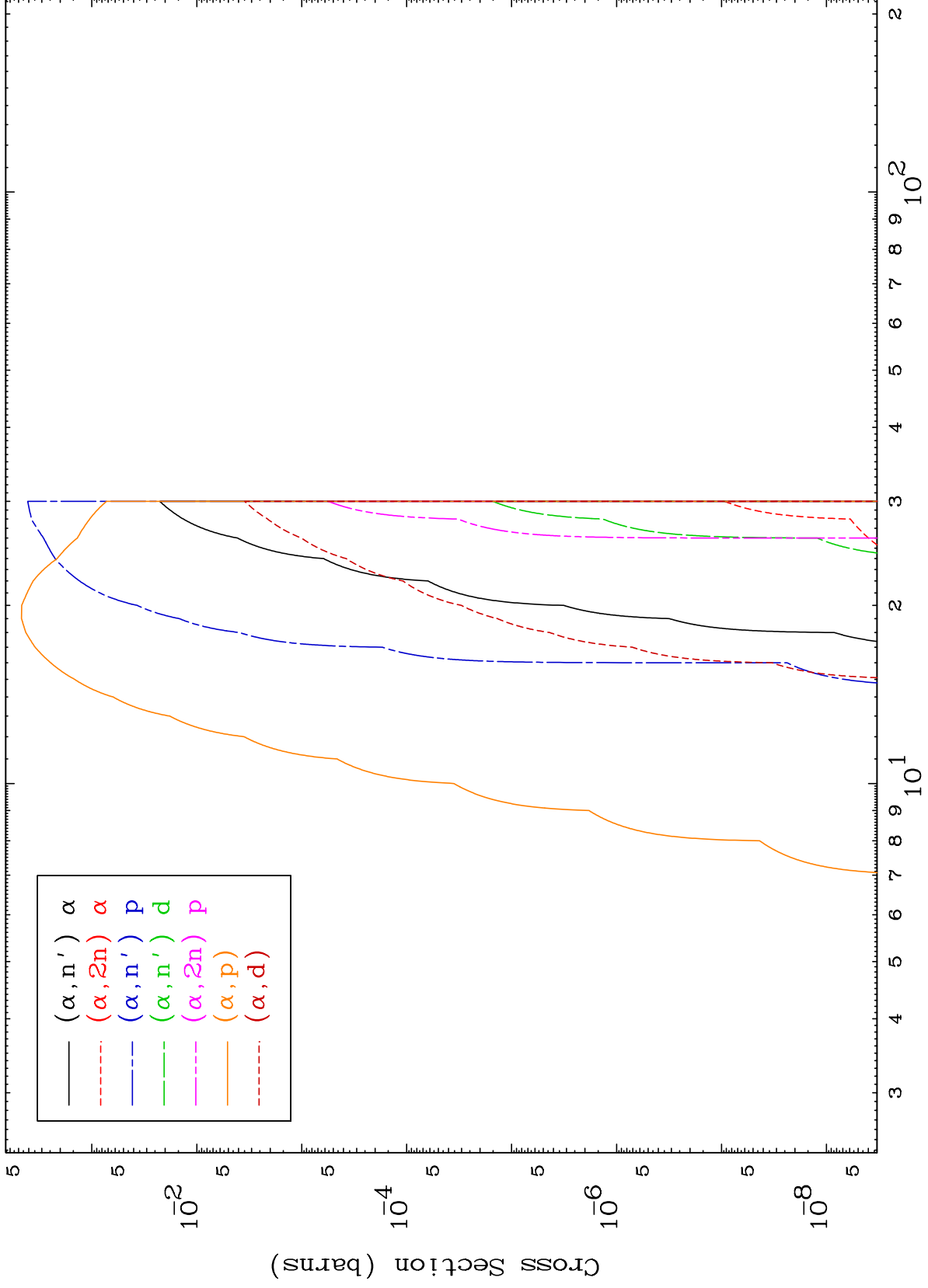
Incident Energy (MeV)

49-In-105

MAT 4901

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

49-In-105



Incident Energy (MeV)

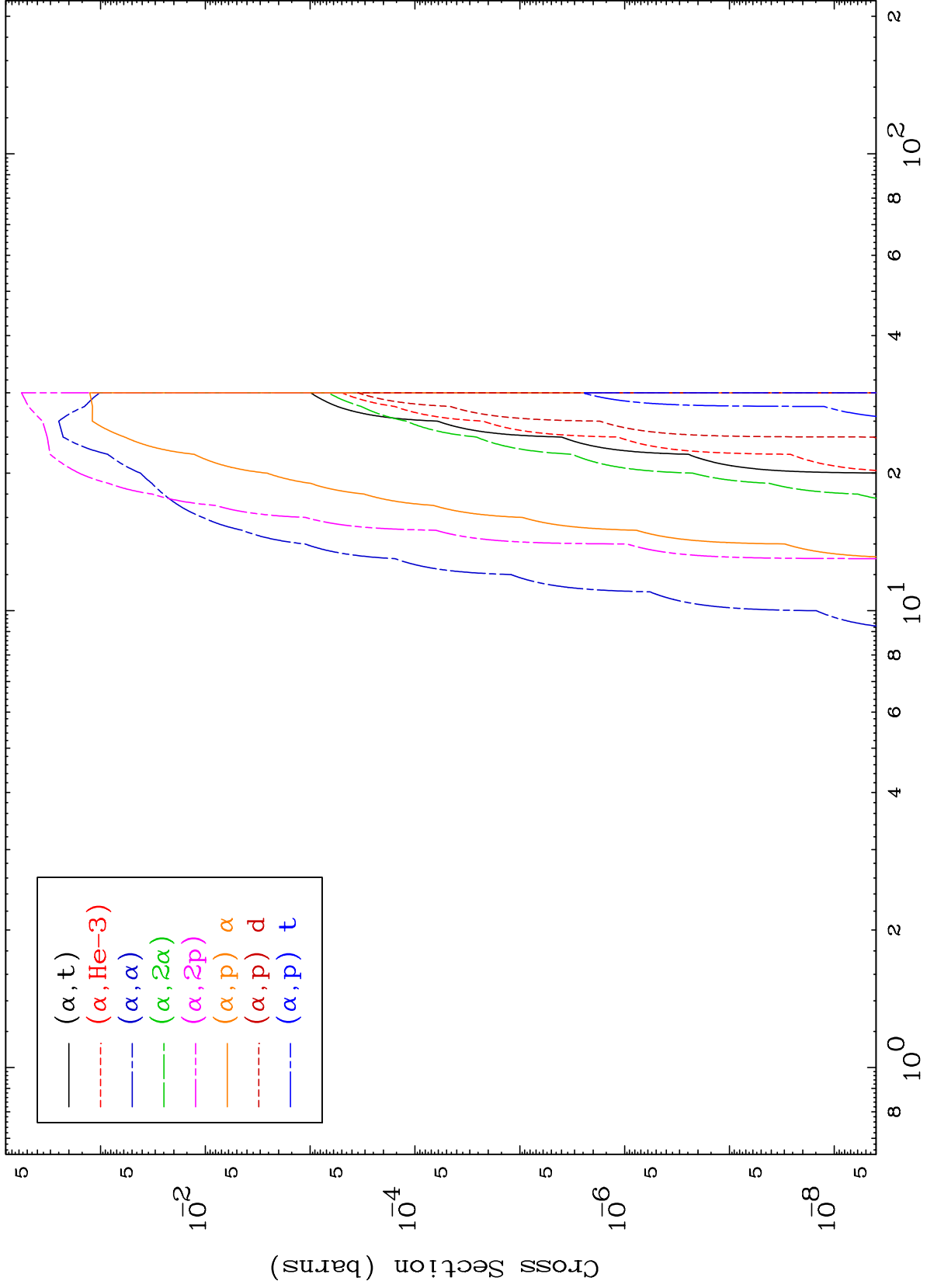
49-In-105

3

MAT 4901

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

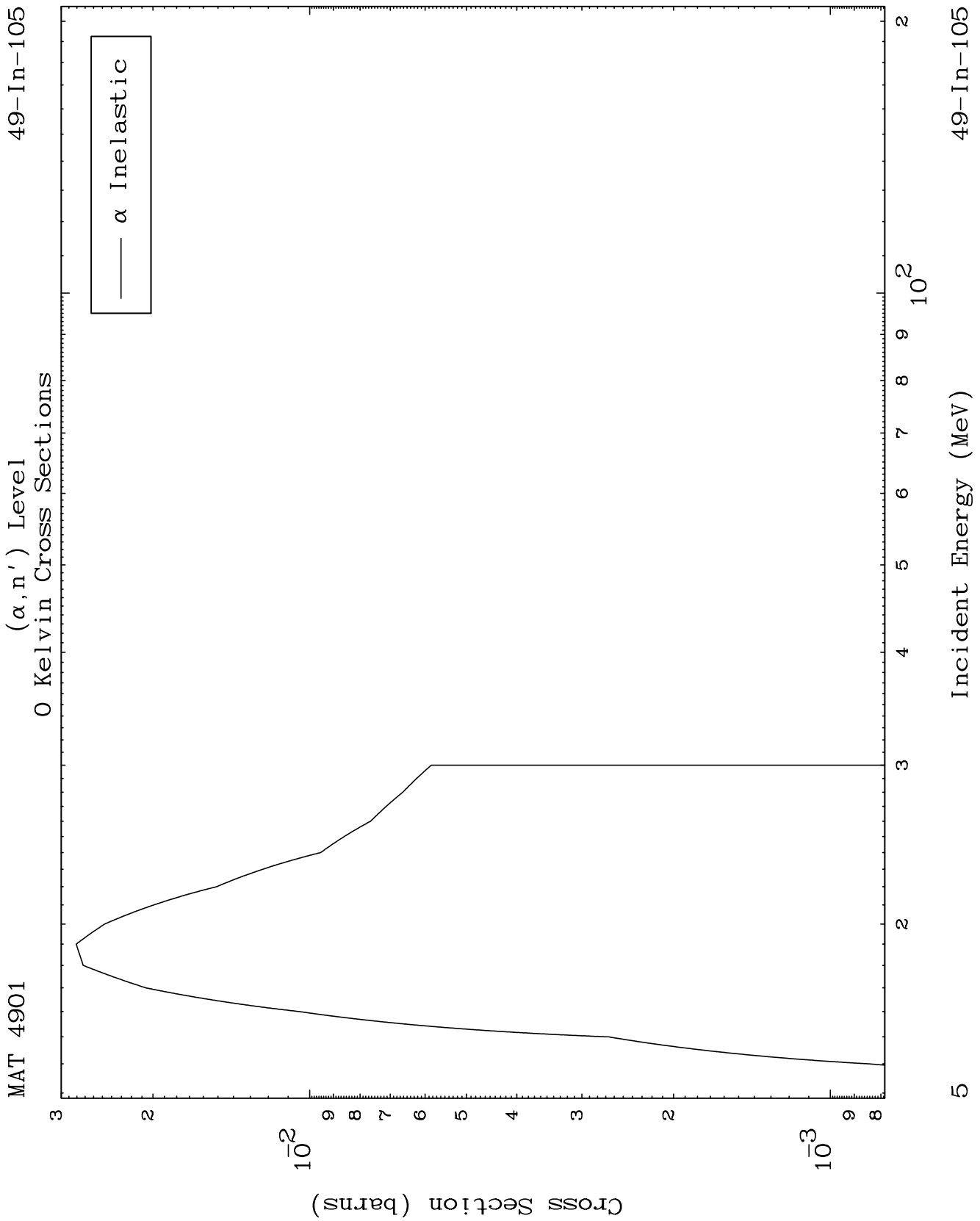
49-In-105



4

Incident Energy (MeV)

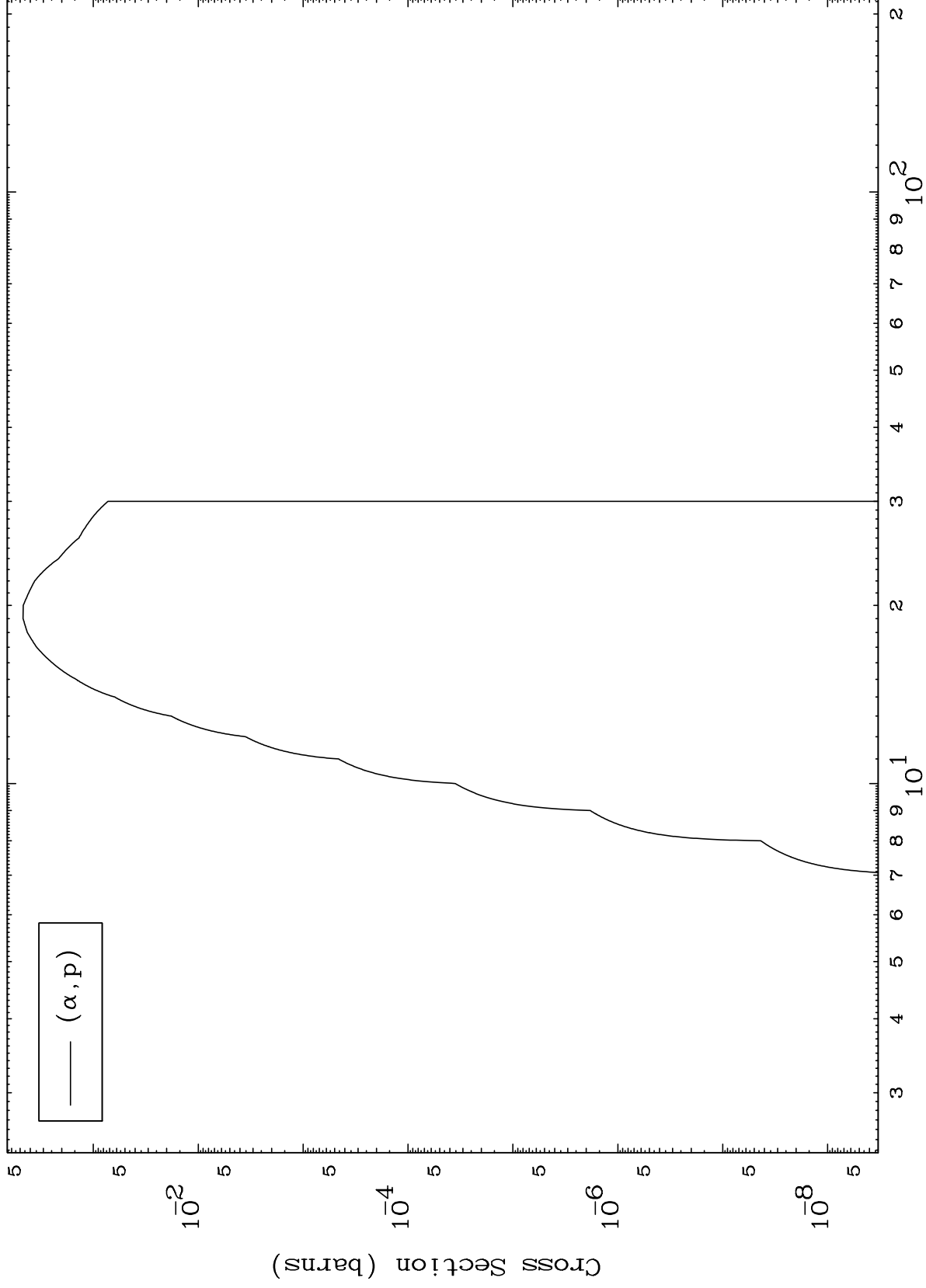
49-In-105



MAT 4901

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

49-In-105



6

Incident Energy (MeV)

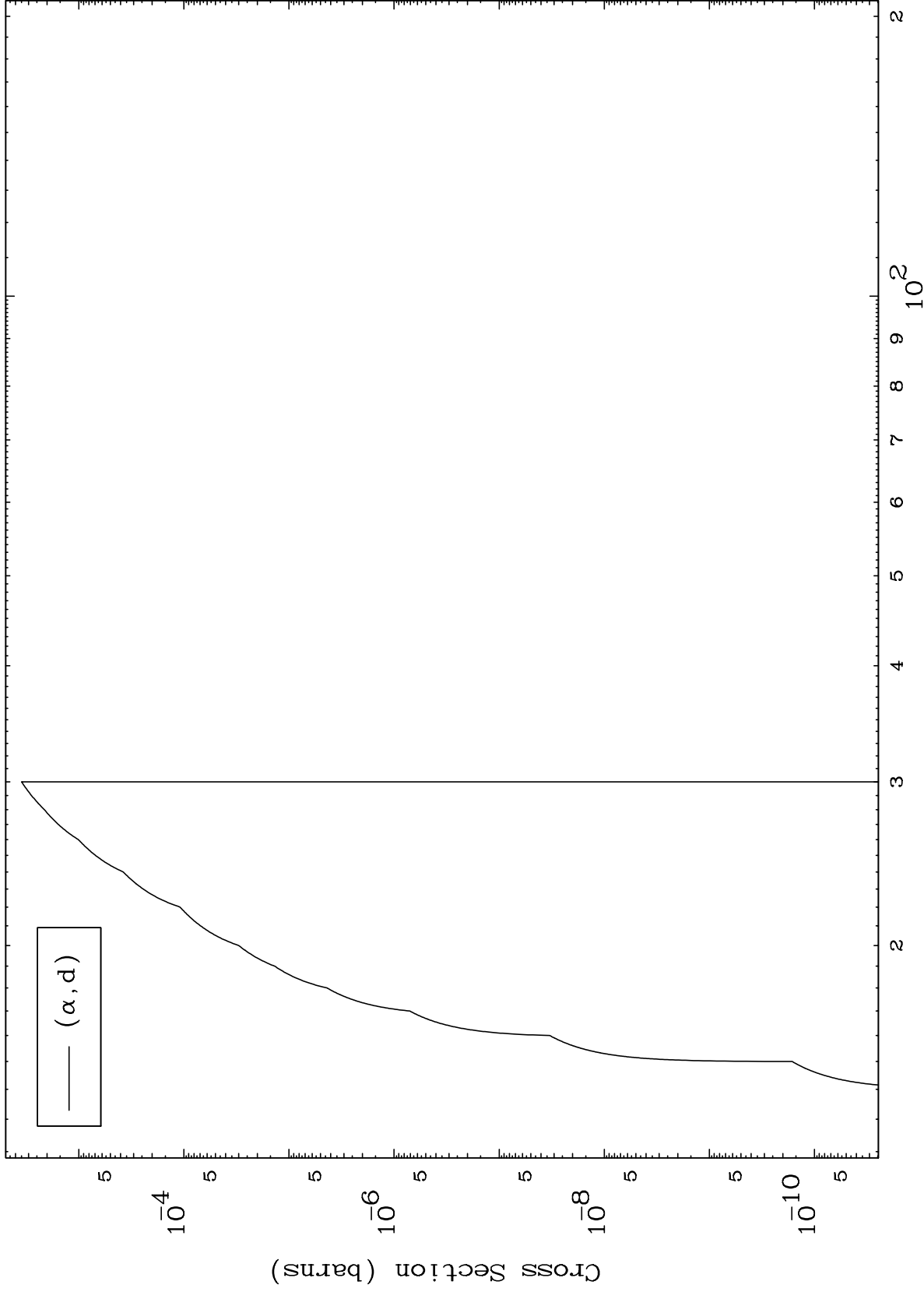
49-In-105

MAT 4901

( $\alpha, d$ ) Levels

49-In-105

0 Kelvin Cross Sections



7

Incident Energy (MeV)

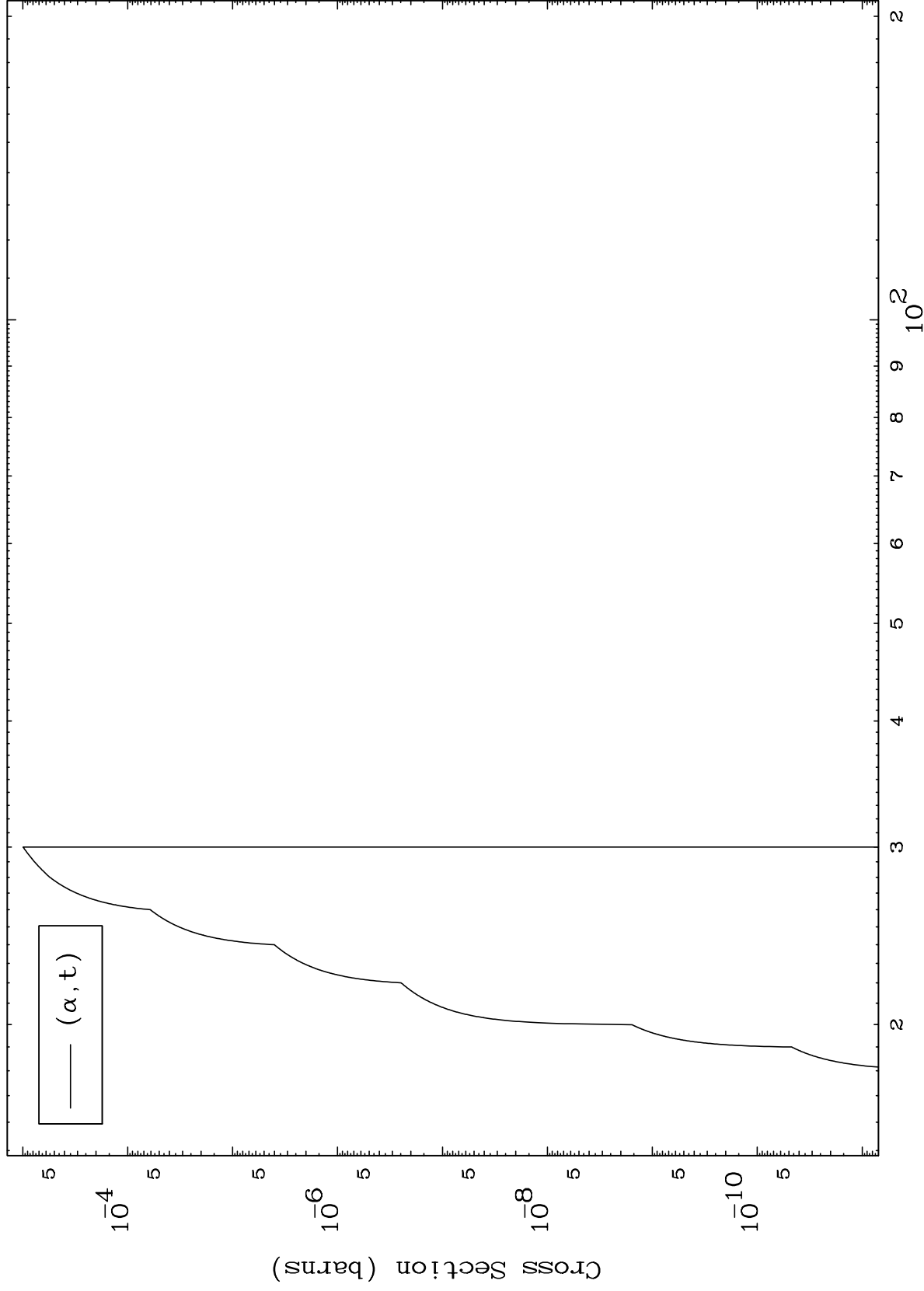
49-In-105



MAT 4901

49-In-105

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



8

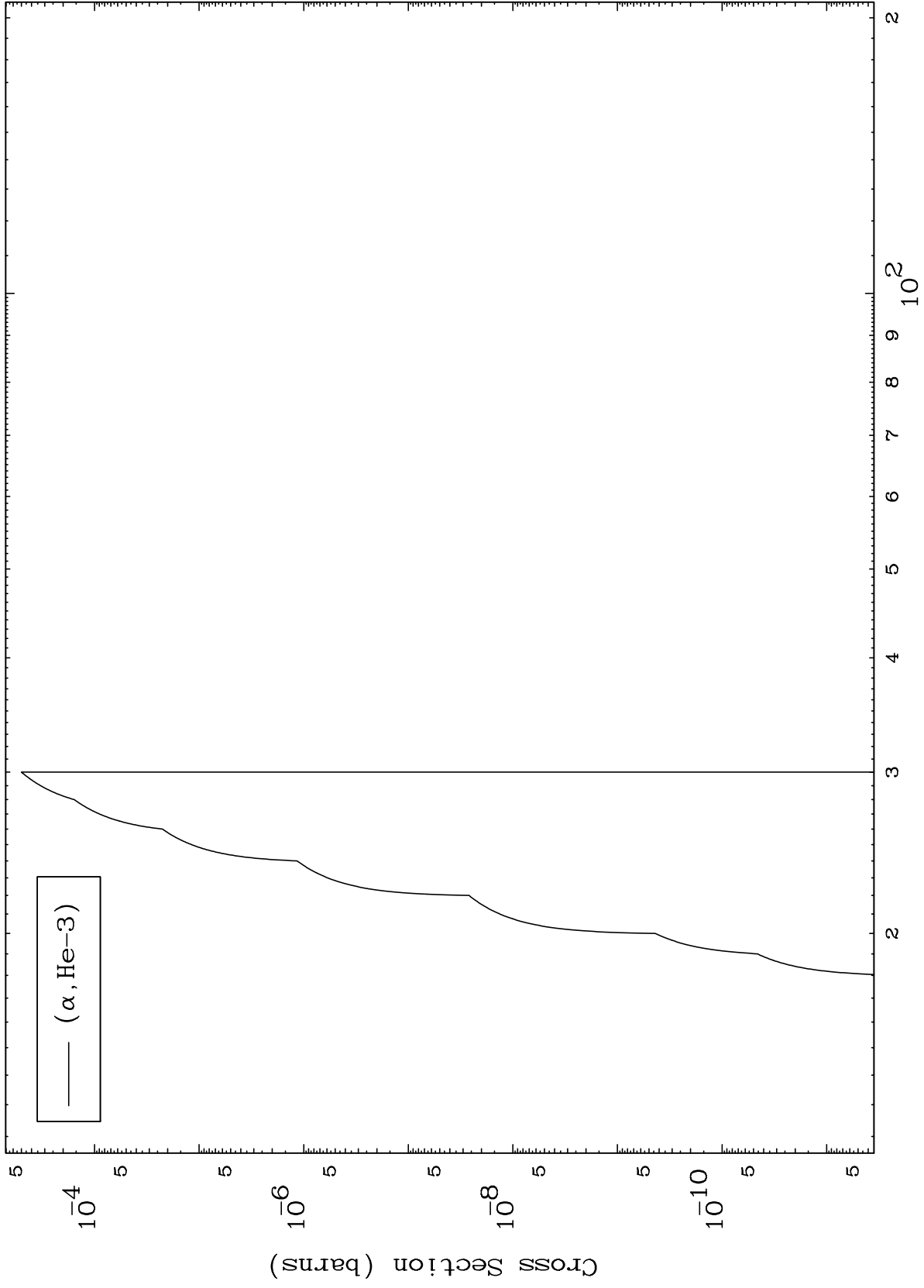
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

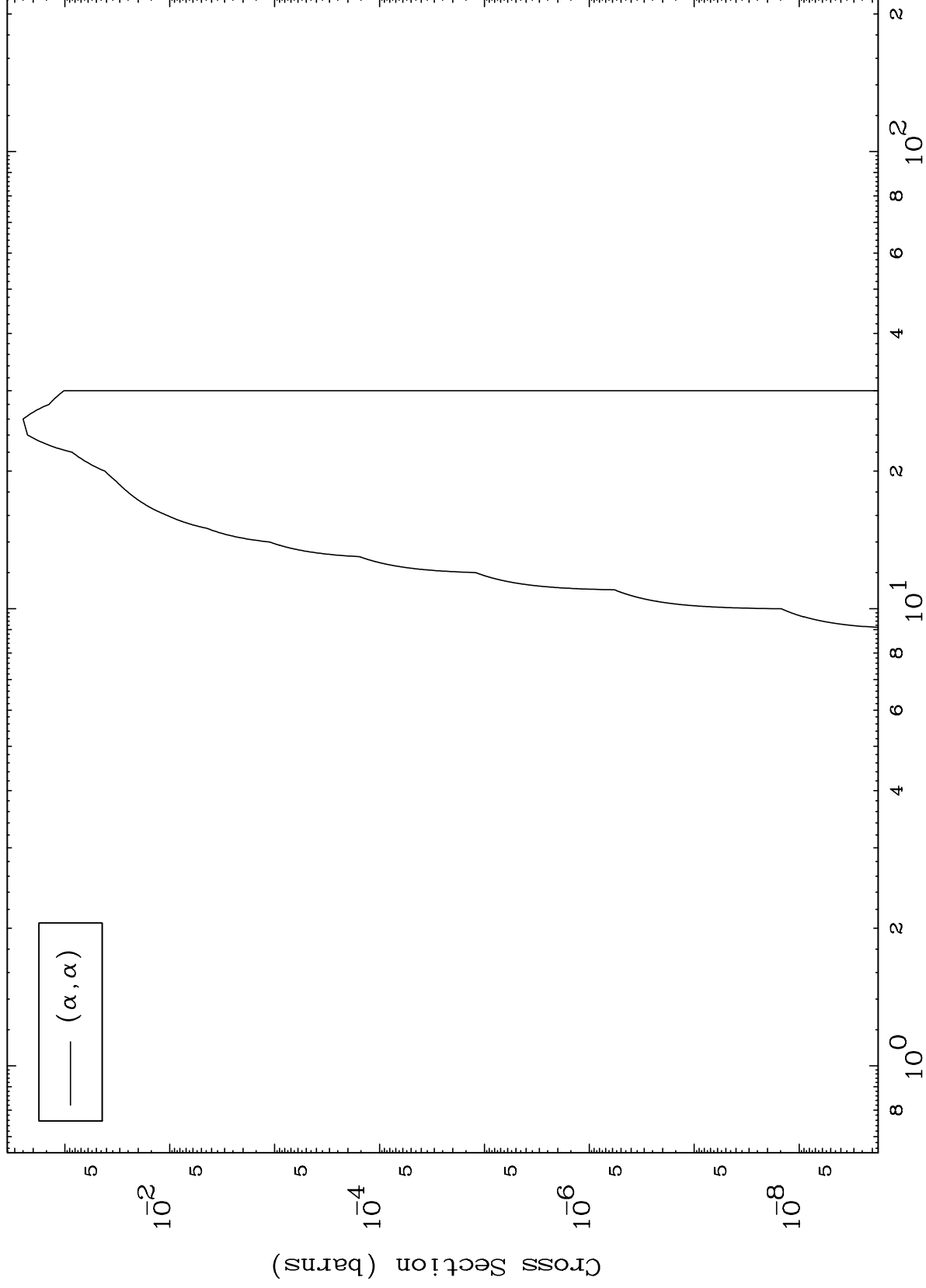


MAT 4901

( $\alpha, \alpha$ ) Levels

49-In-105

0 Kelvin Cross Sections



10

Incident Energy (MeV)

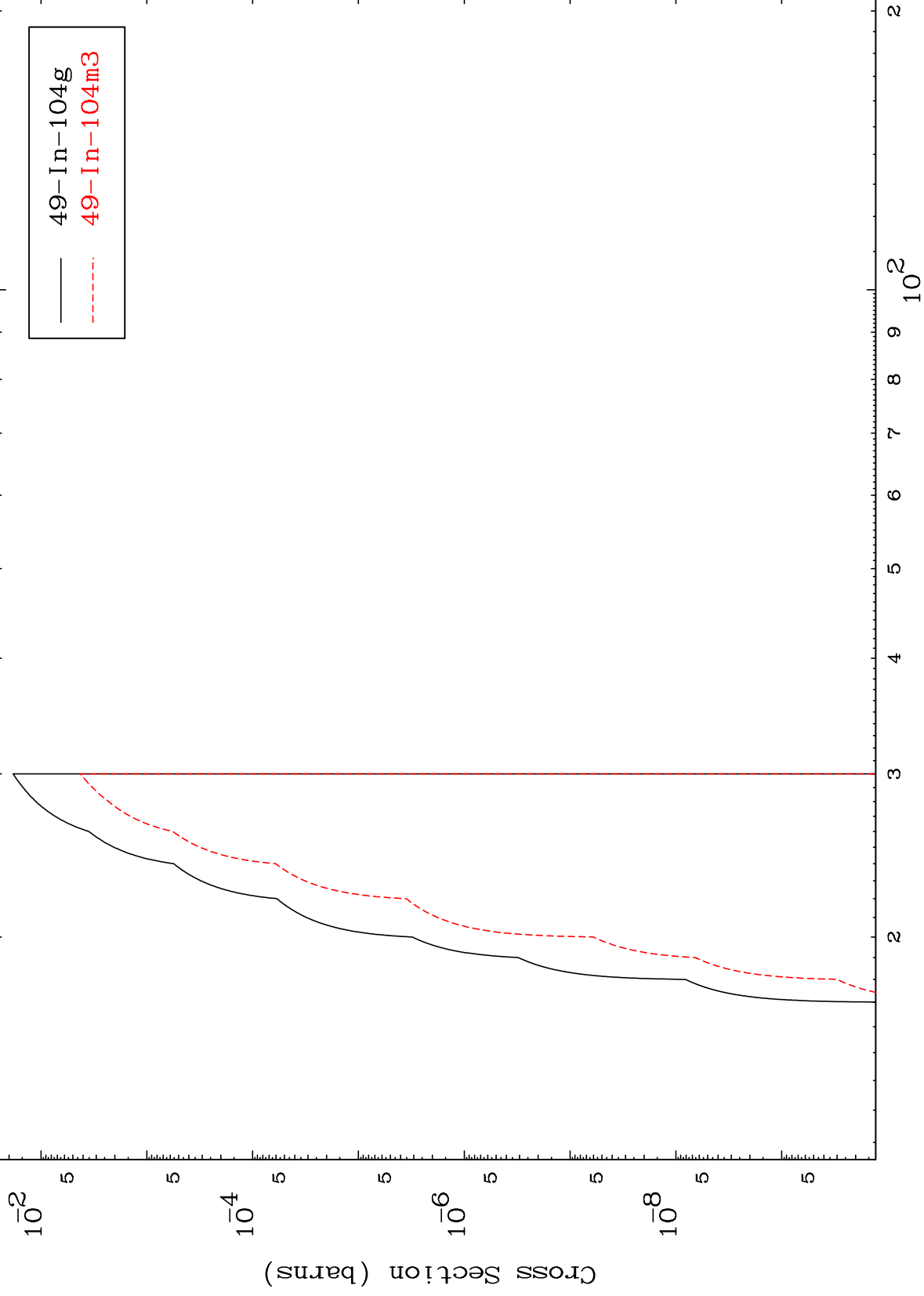
49-In-105

MAT 4901

$(\alpha, n')$   $\alpha$

49-In-105

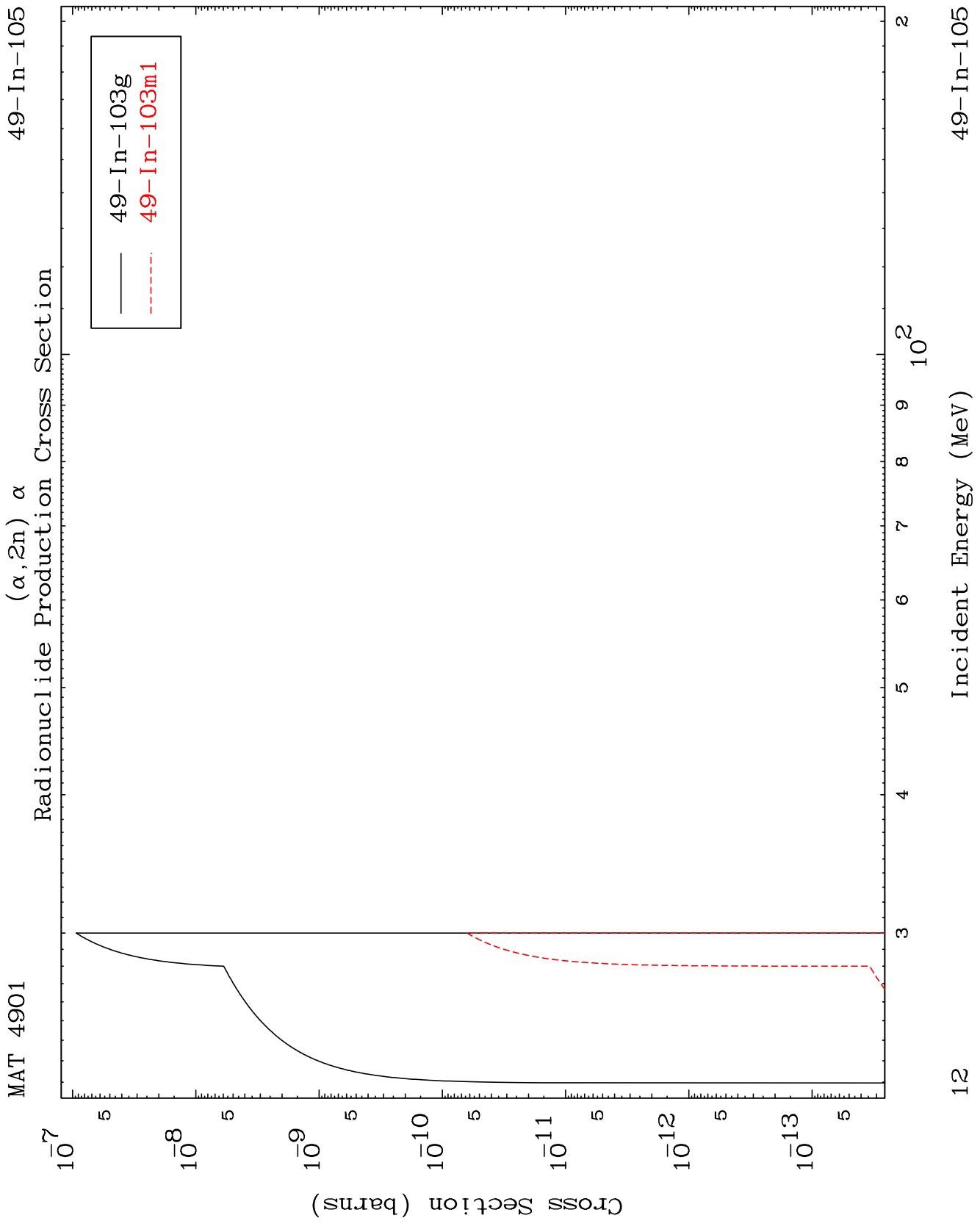
Radionuclide Production Cross Section



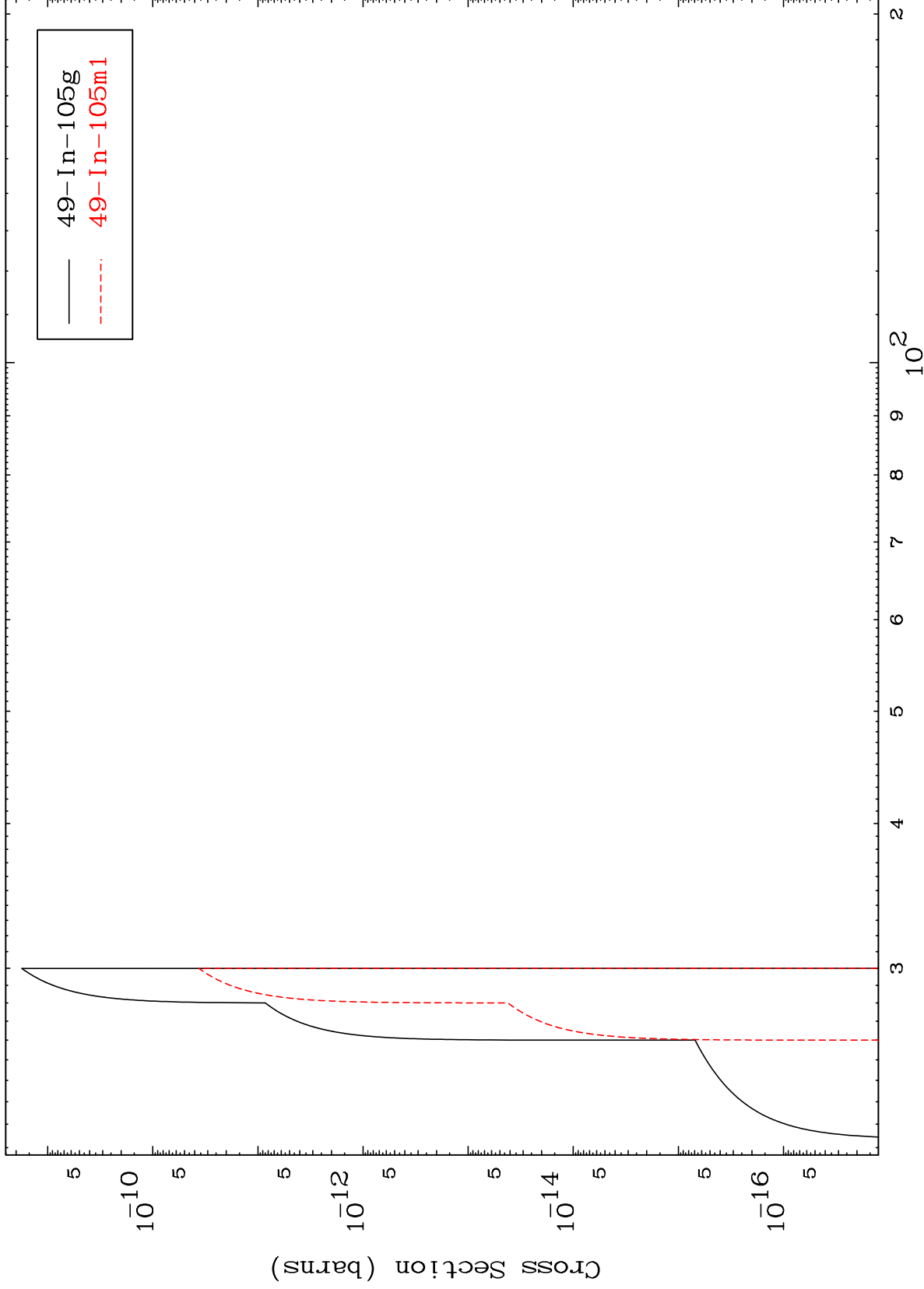
11

Incident Energy (MeV)

49-In-105



Radionuclide Production Cross Section

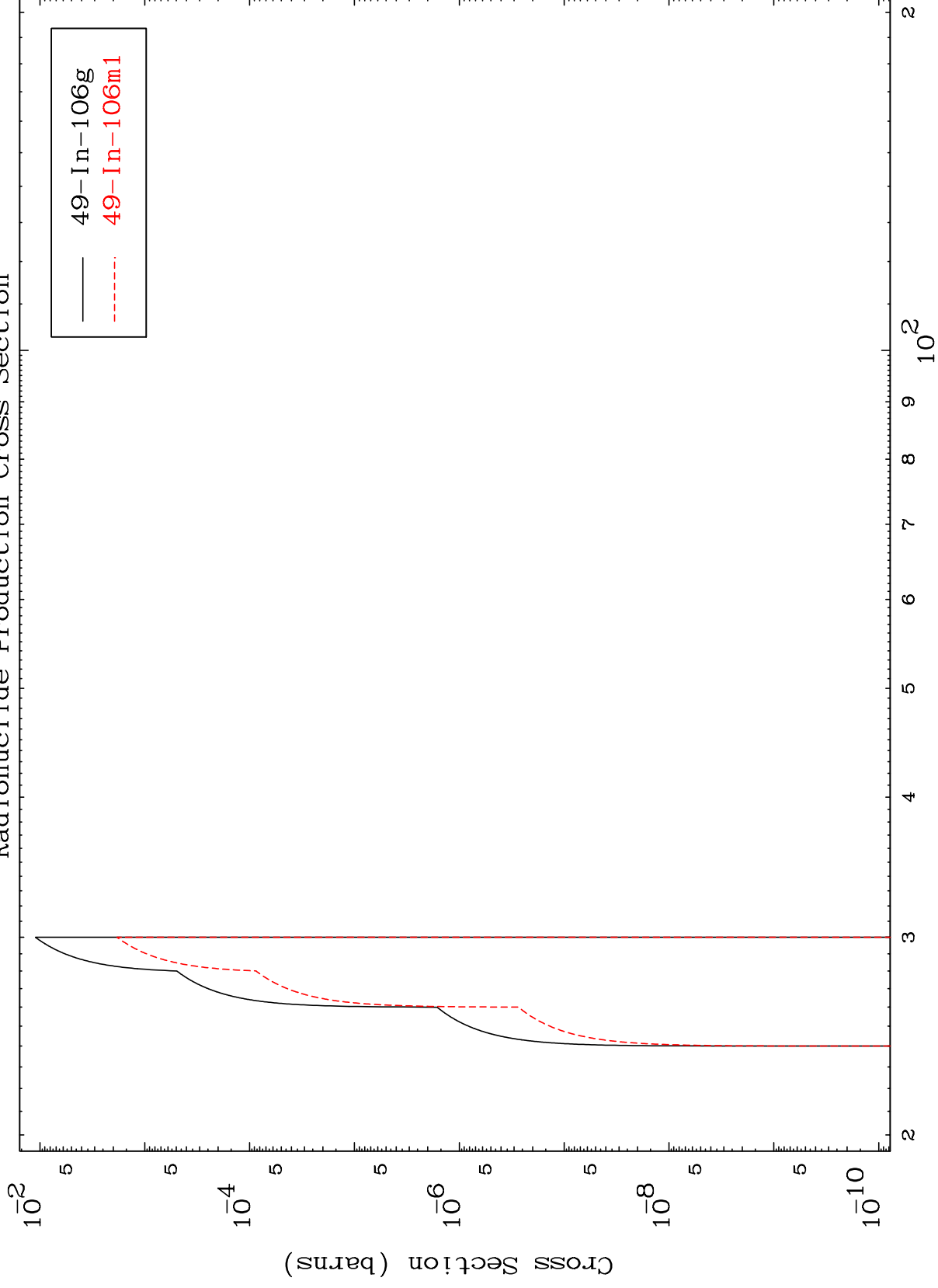


MAT 4901

$(\alpha, 2n)$  p

49-In-105

Radionuclide Production Cross Section



14

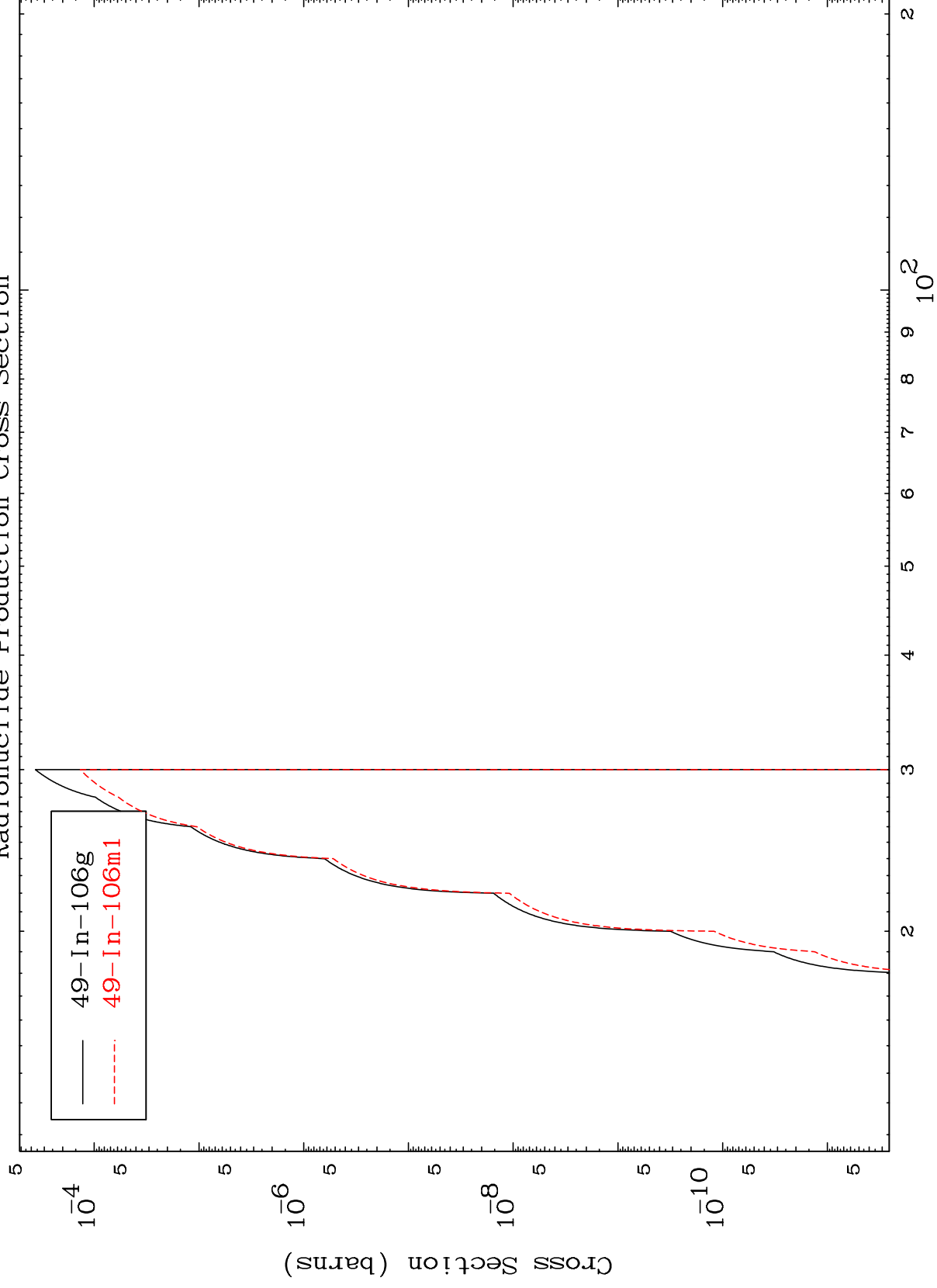
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

( $\alpha, \text{He-3}$ )  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

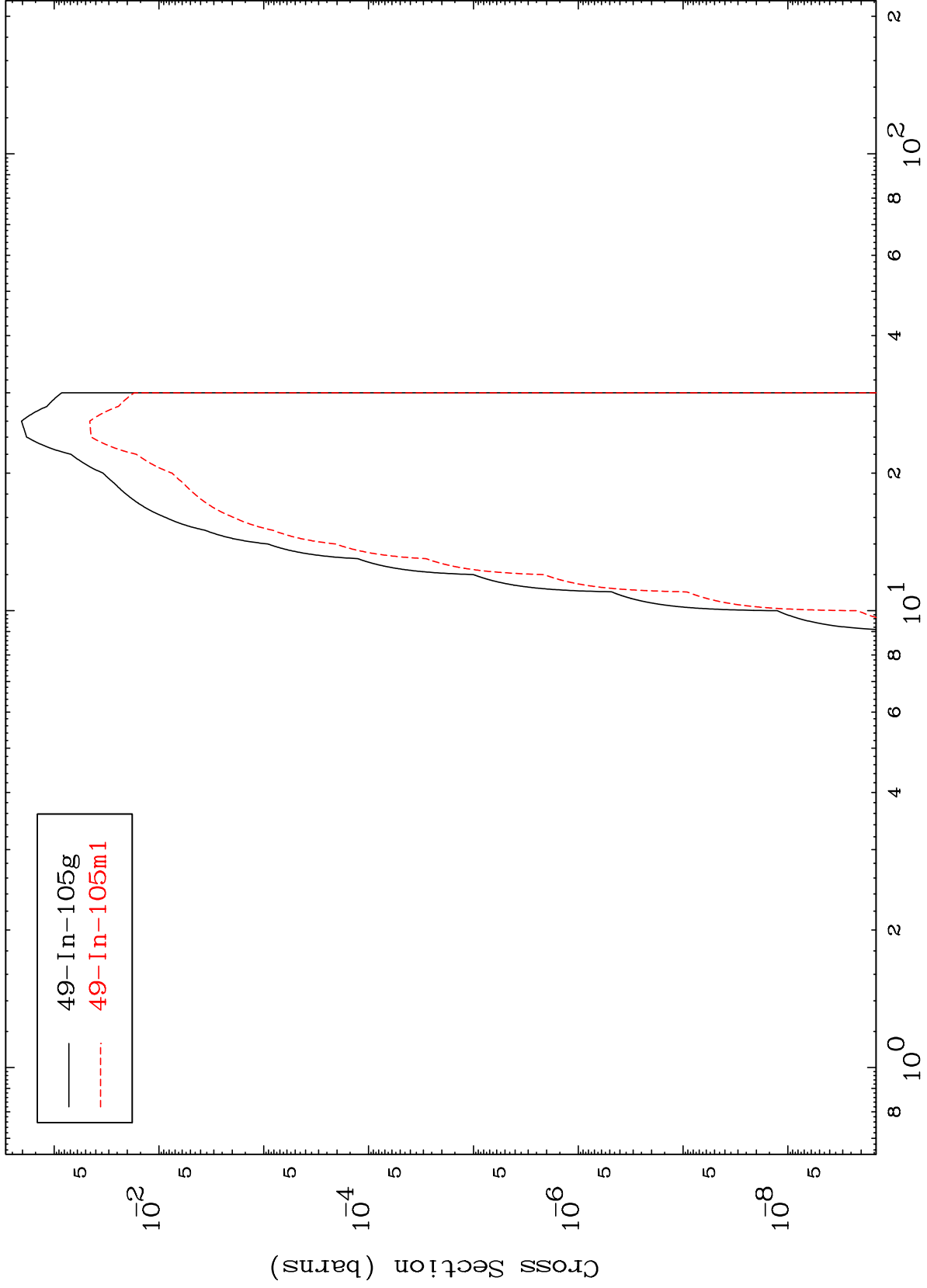
49-In-105



MAT 4901

49-In-105

Radionuclide Production Cross Section  
( $\alpha, \alpha$ )



16

49-In-105

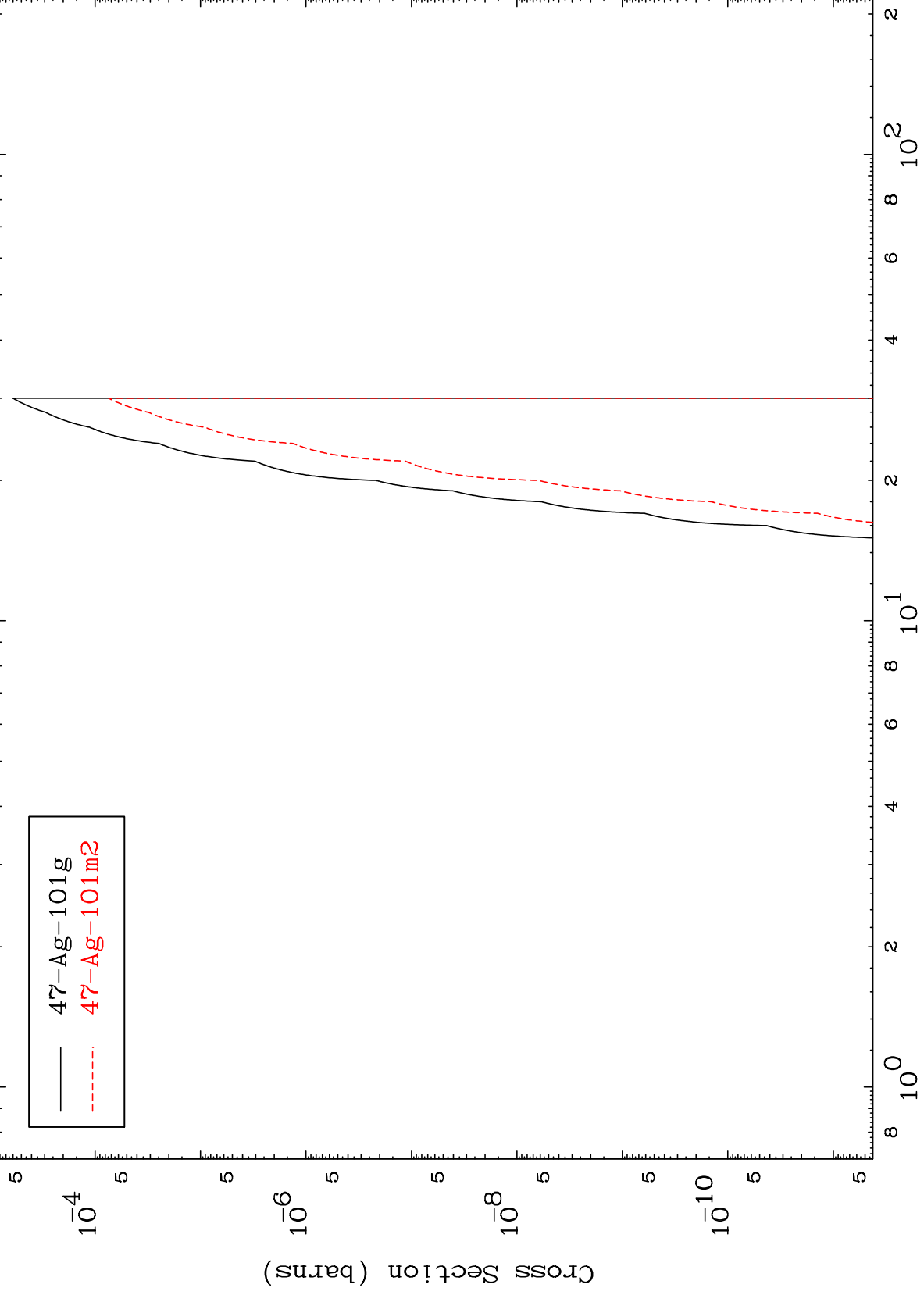
Incident Energy (MeV)

MAT 4901

( $\alpha, 2\alpha$ )

49-In-105

Radionuclide Production Cross Section



17

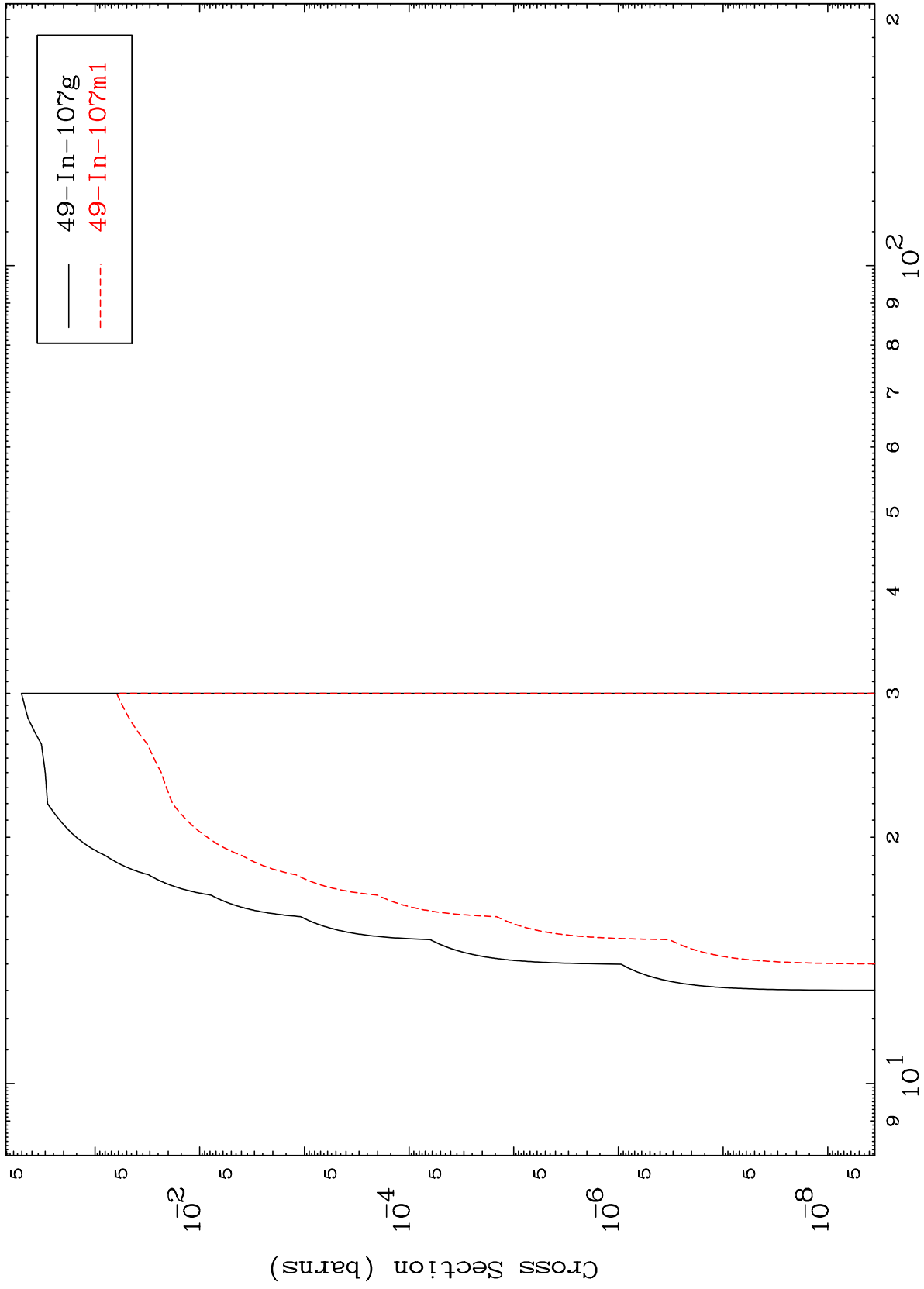
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

( $\alpha, 2p$ )  
Radionuclide Production Cross Section

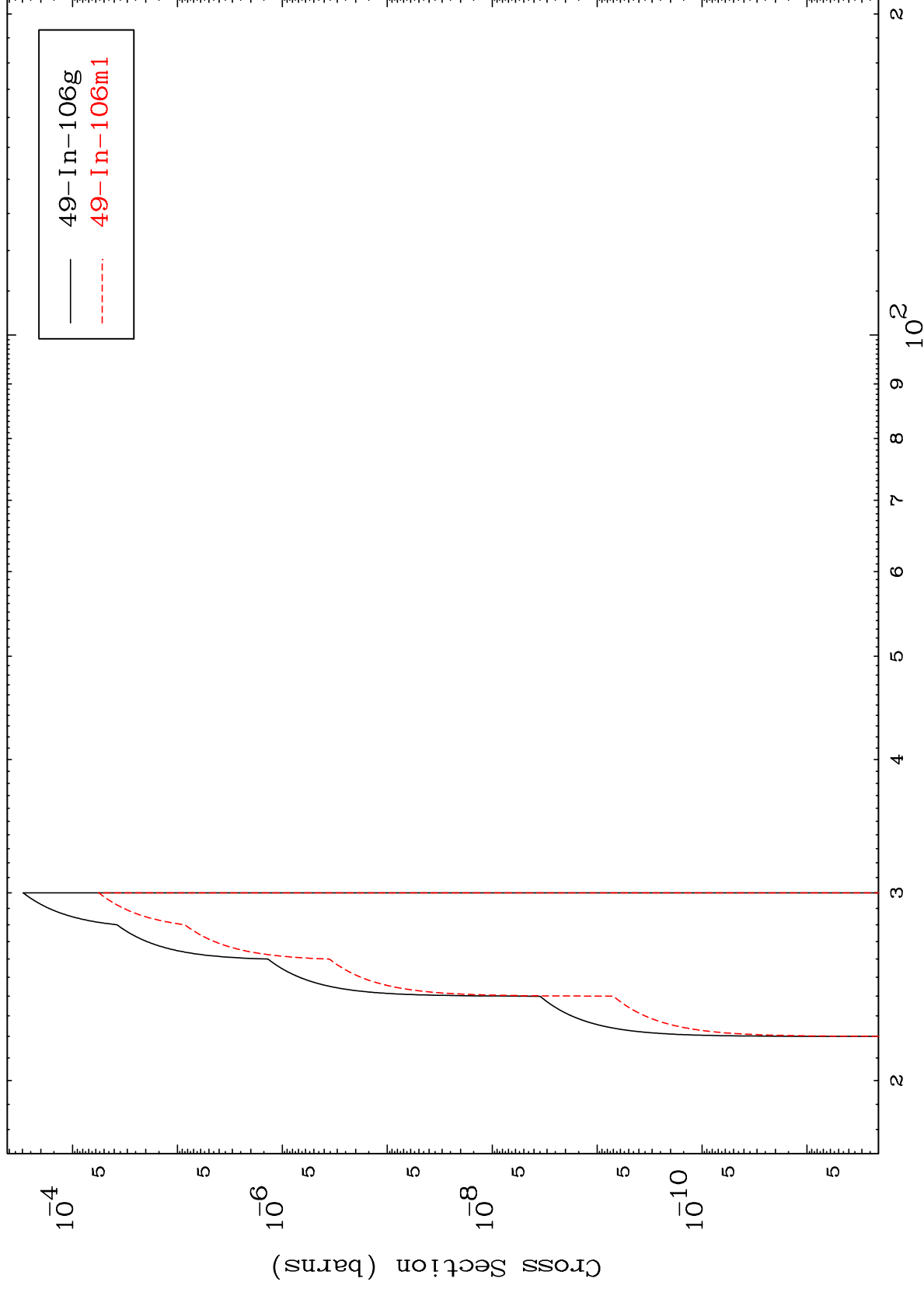


49-In-105

Incident Energy (MeV)

18

Radionuclide Production Cross Section

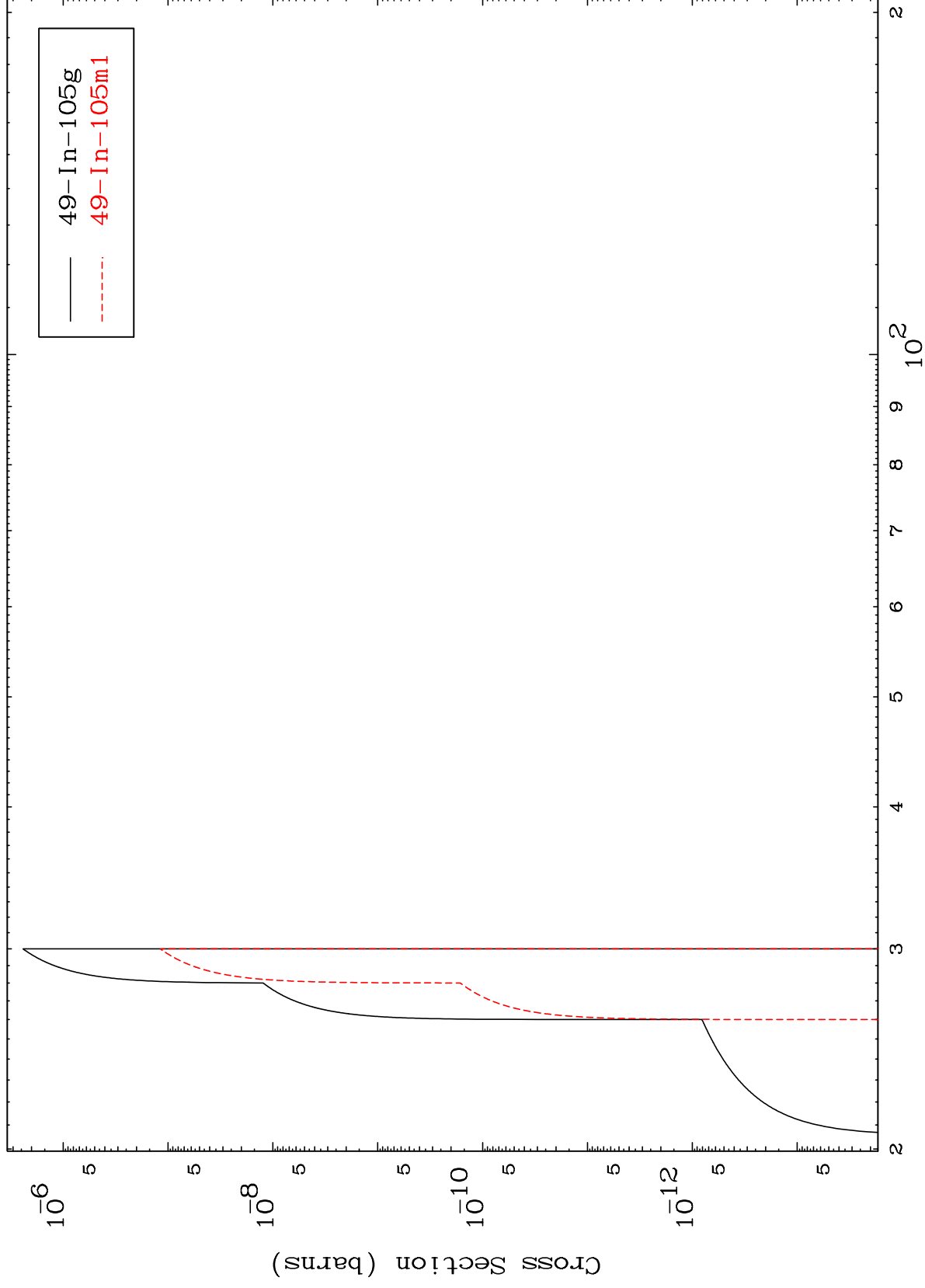


MAT 4901

( $\alpha, p$ ) t

49-In-105

Radionuclide Production Cross Section



20

Incident Energy (MeV)

49-In-105