

Program EVALPLOT  
(Version 2018-1)

by

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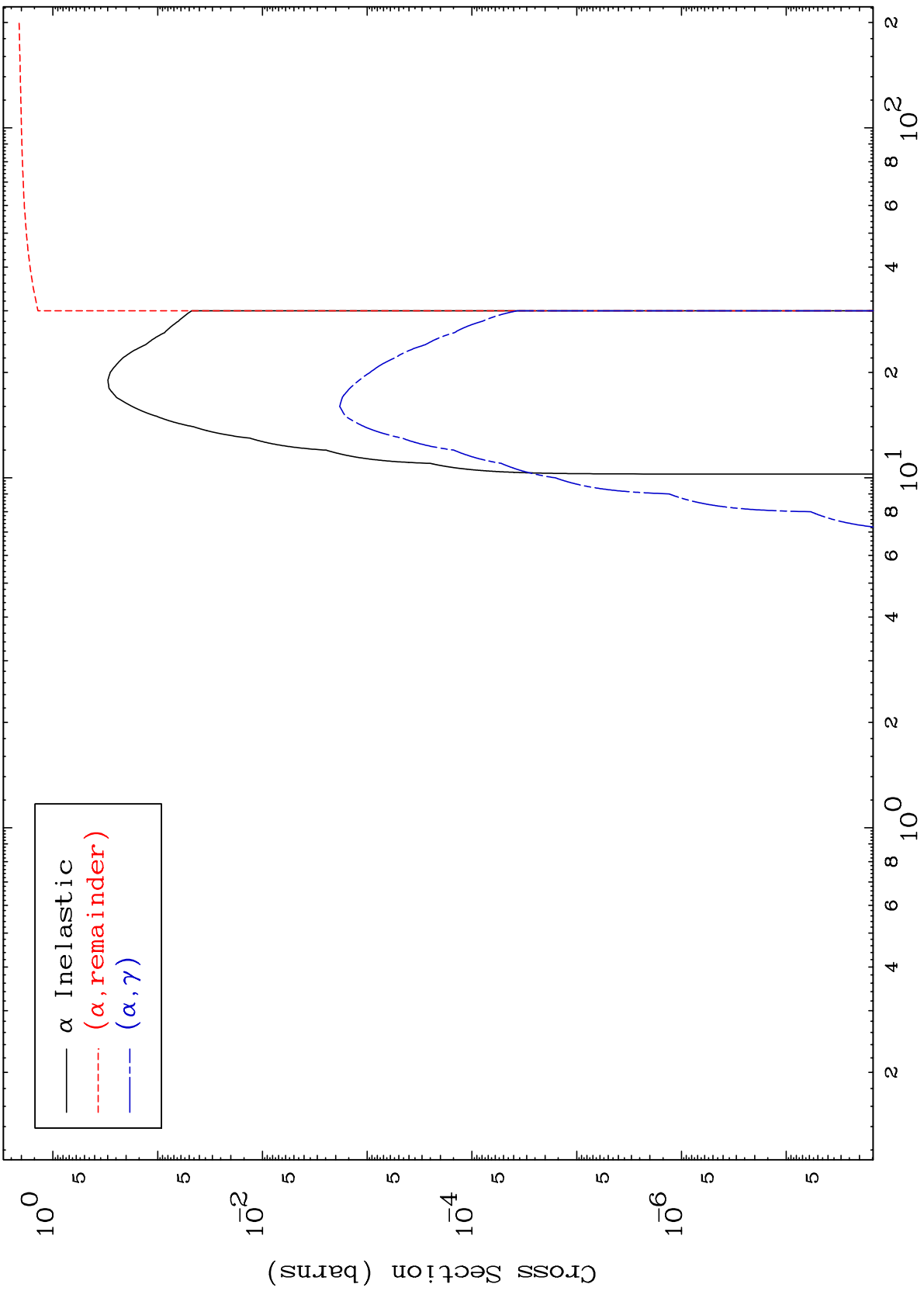
Press Mouse Button to Start

MAT 4914

$\alpha$  Major

49-In-109

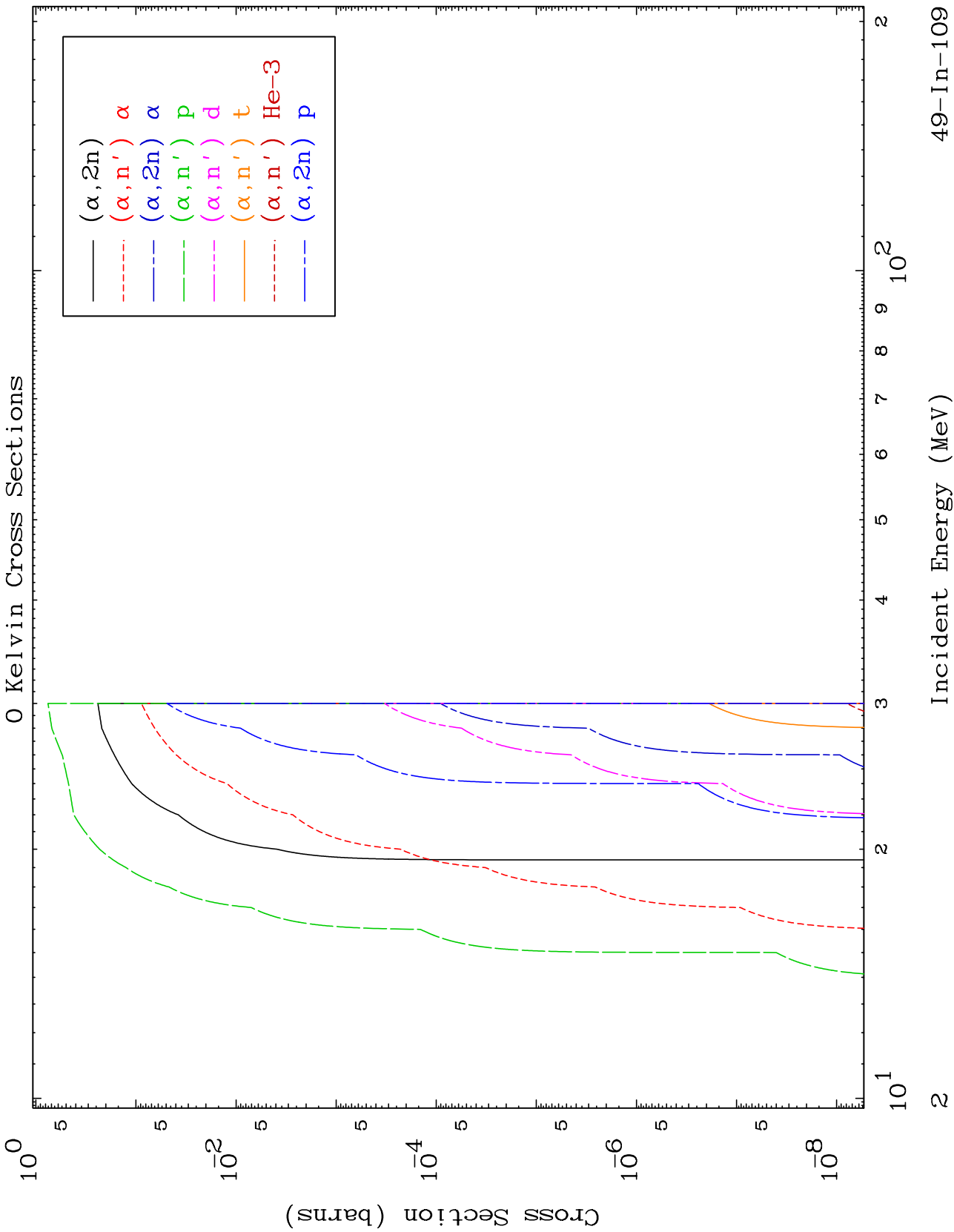
0 Kelvin Cross Sections



MAT 4914

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

49-In-109



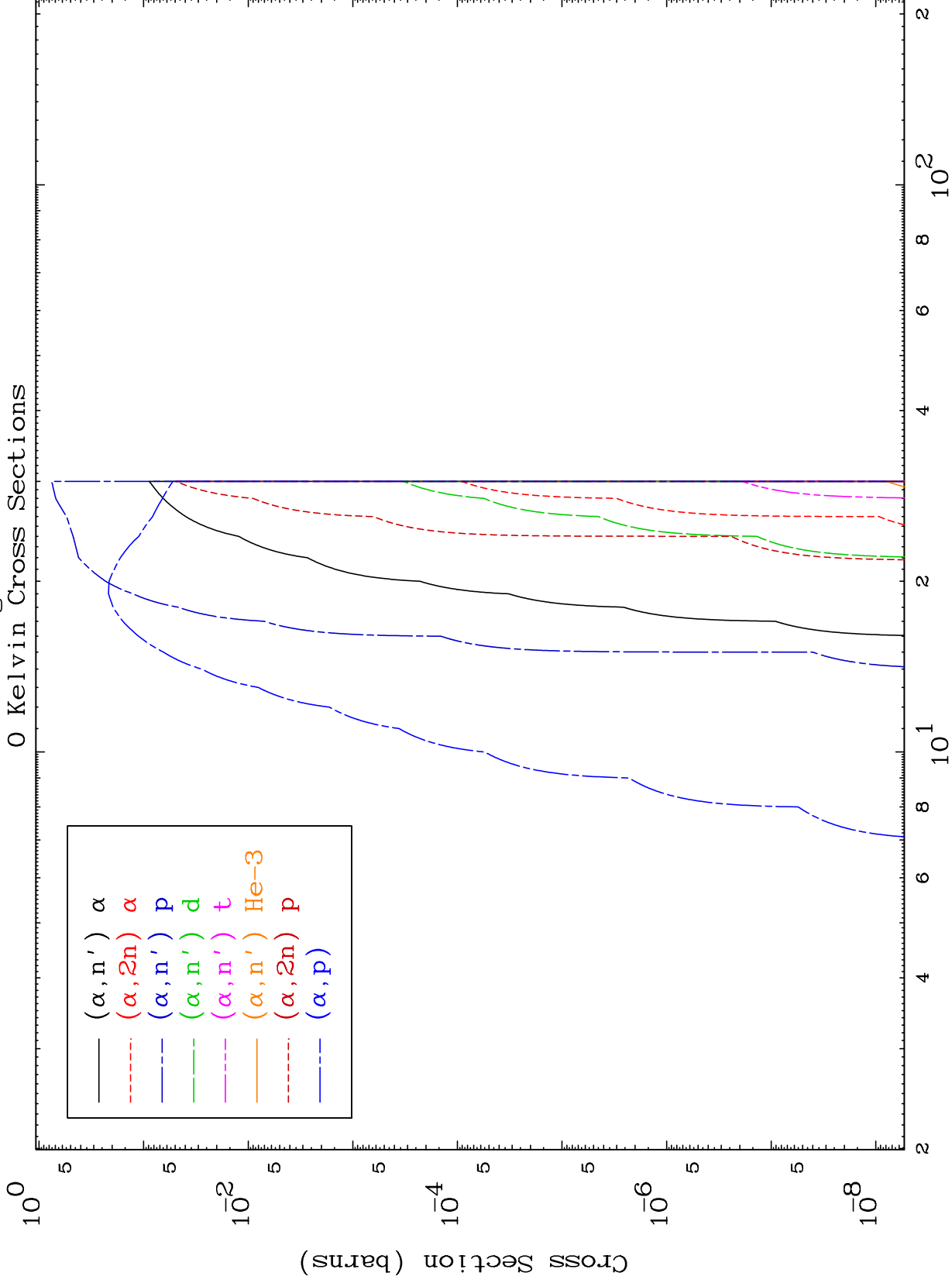
Incident Energy (MeV)

49-In-109

MAT 4914

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

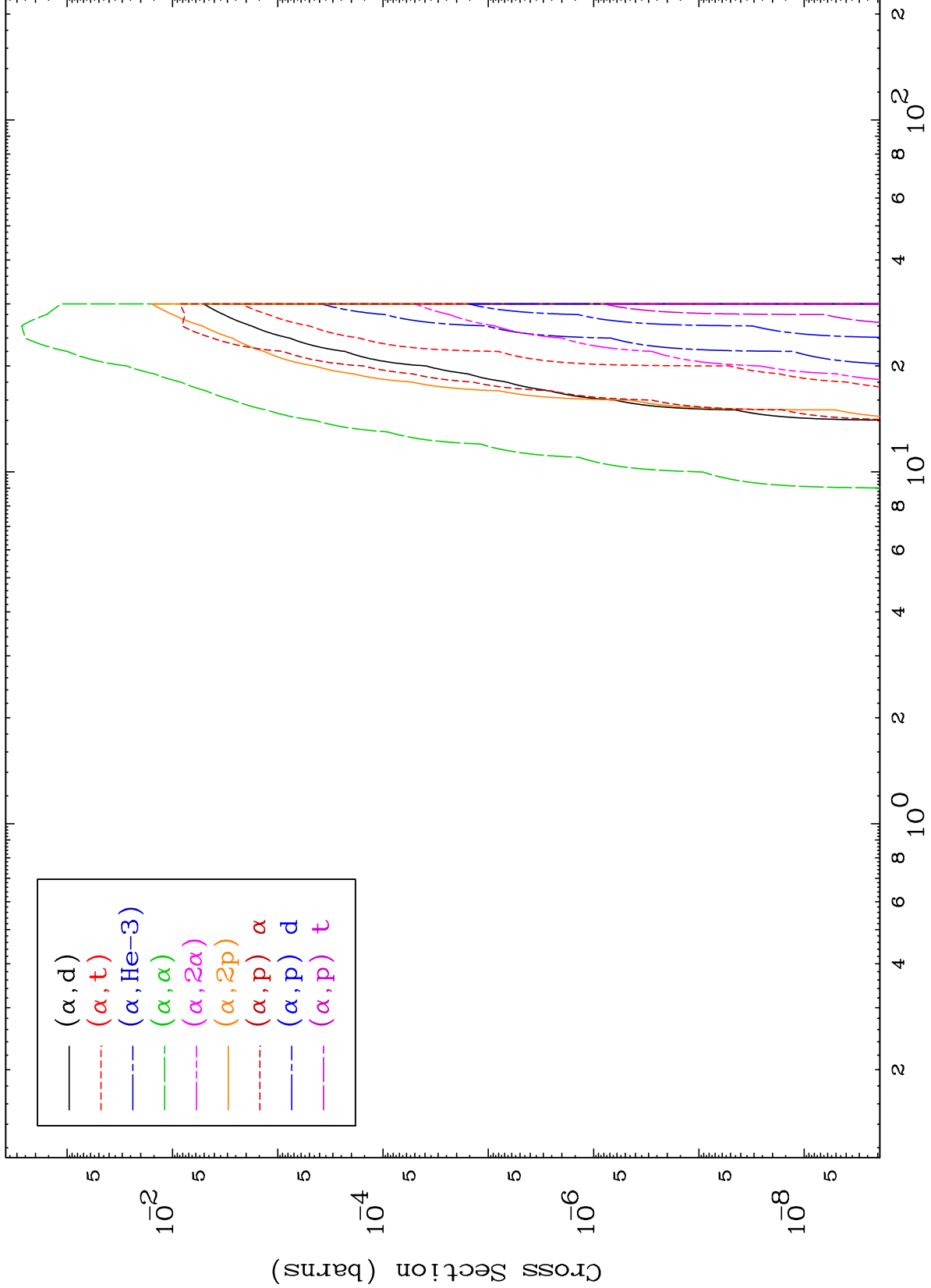
49-In-109



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Incident Energy (MeV)

49-In-109

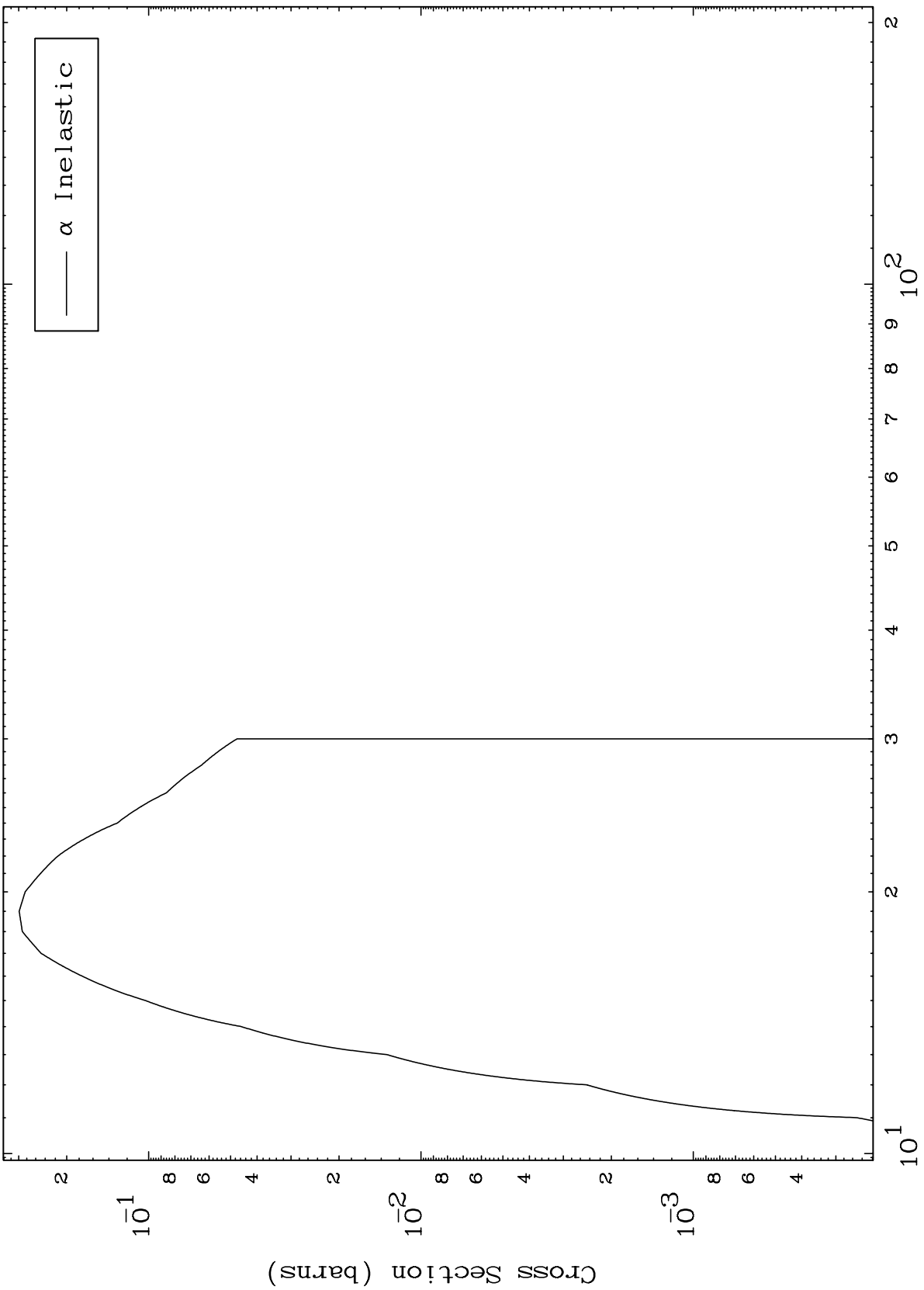


MAT 4914

( $\alpha, n'$ ) Level

49-In-109

0 Kelvin Cross Sections



Incident Energy (MeV)

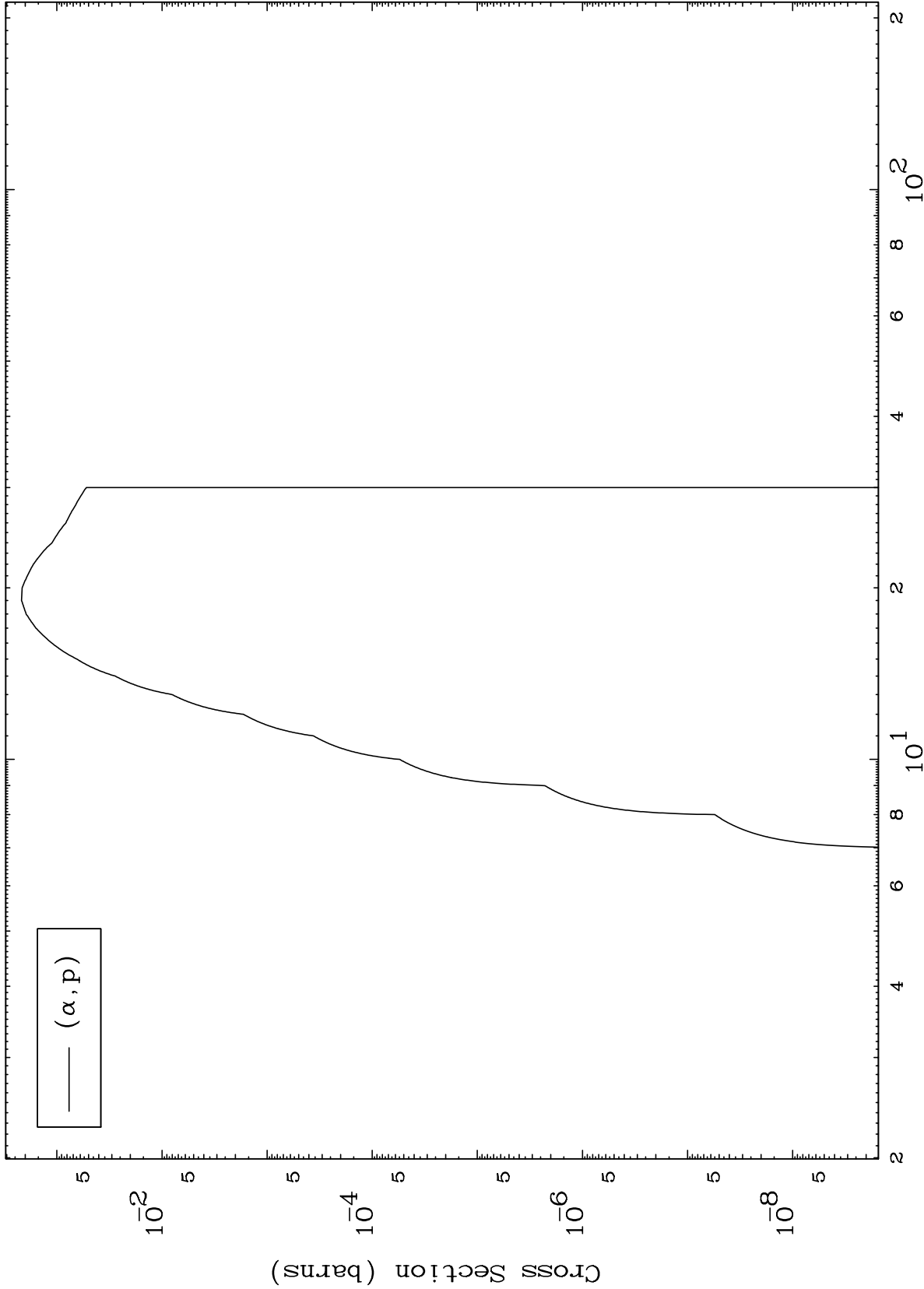
49-In-109

MAT 4914

( $\alpha, p$ ) Levels

49-In-109

0 Kelvin Cross Sections

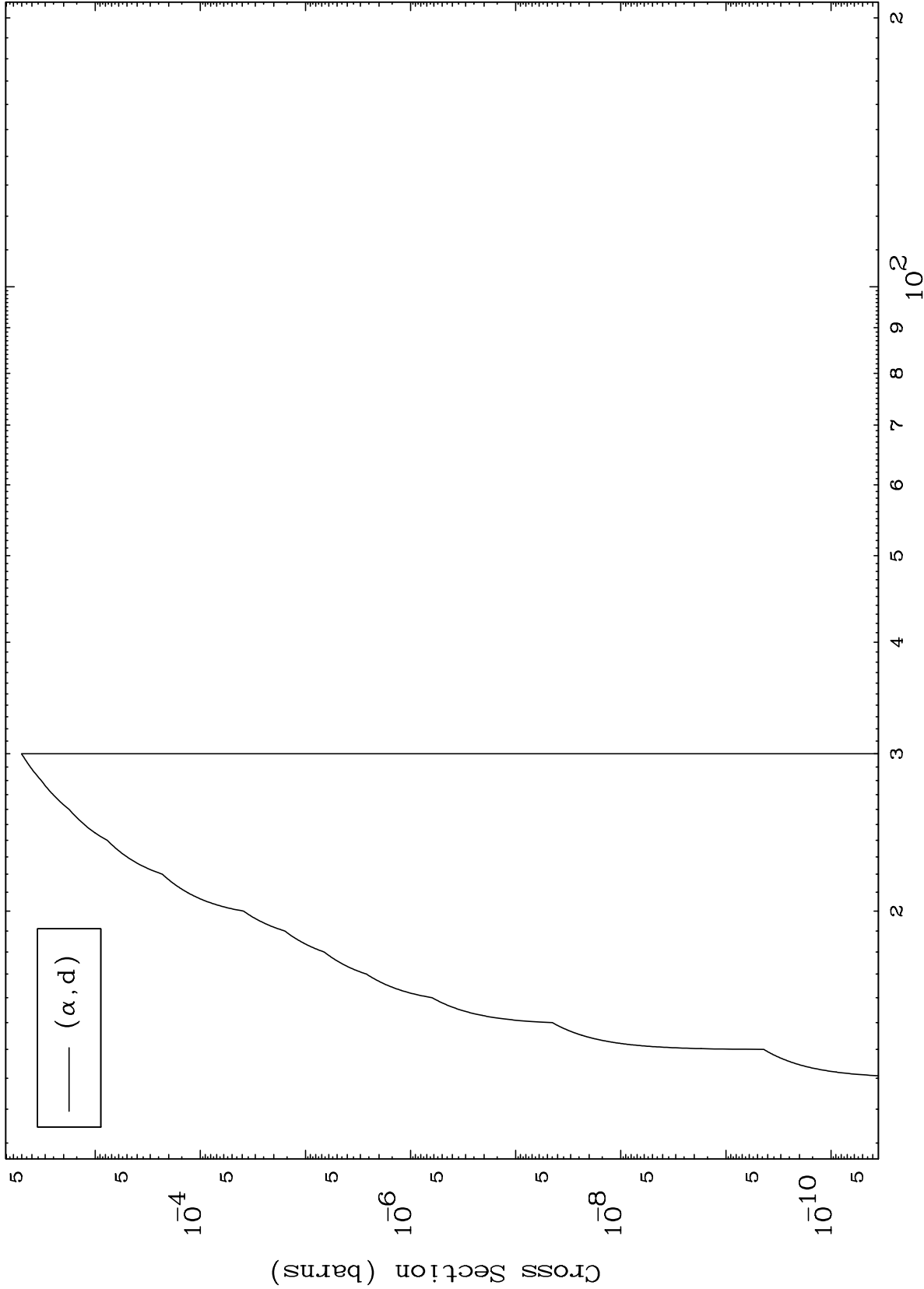


MAT 4914

( $\alpha, d$ ) Levels

49-In-109

0 Kelvin Cross Sections

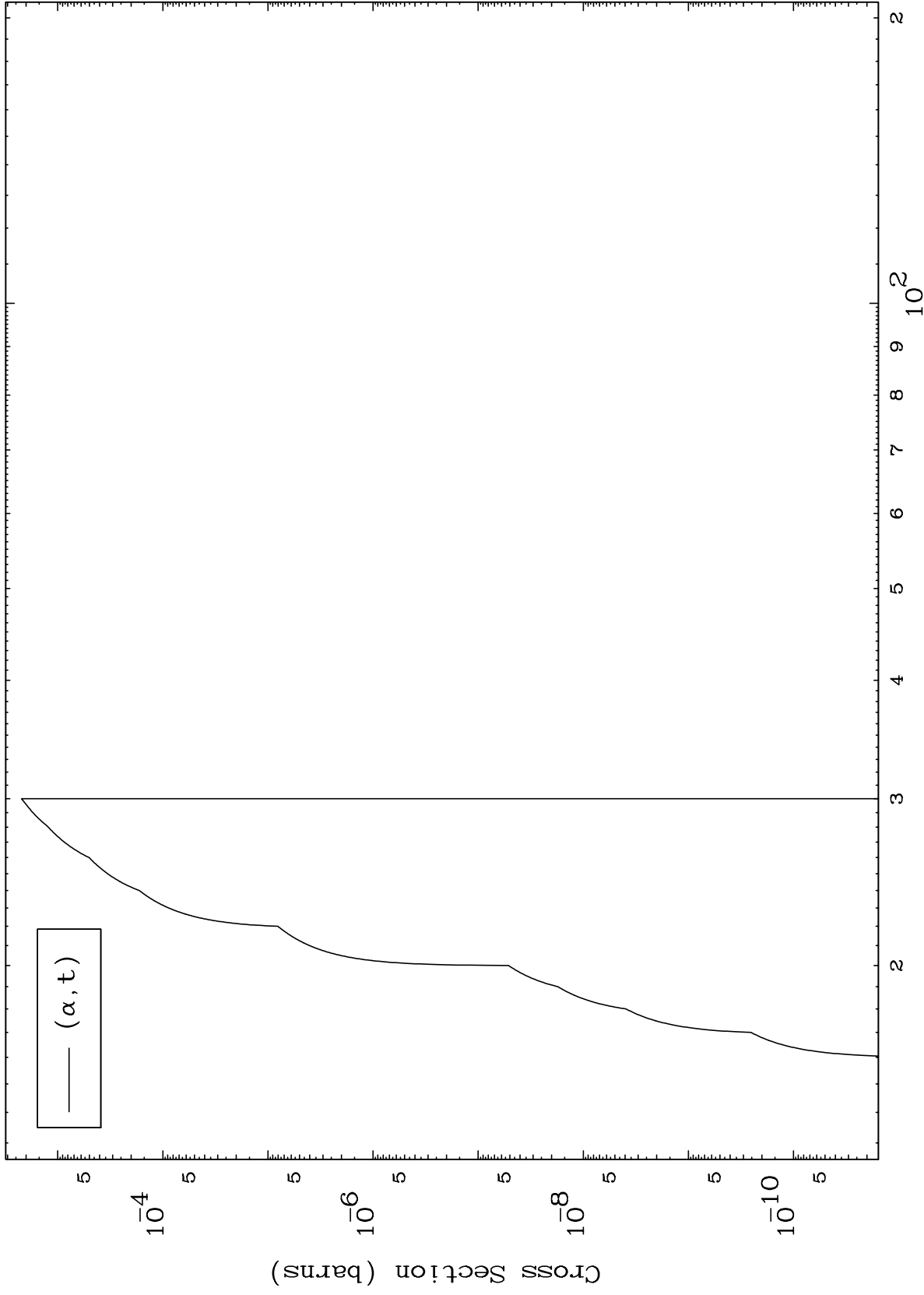


7

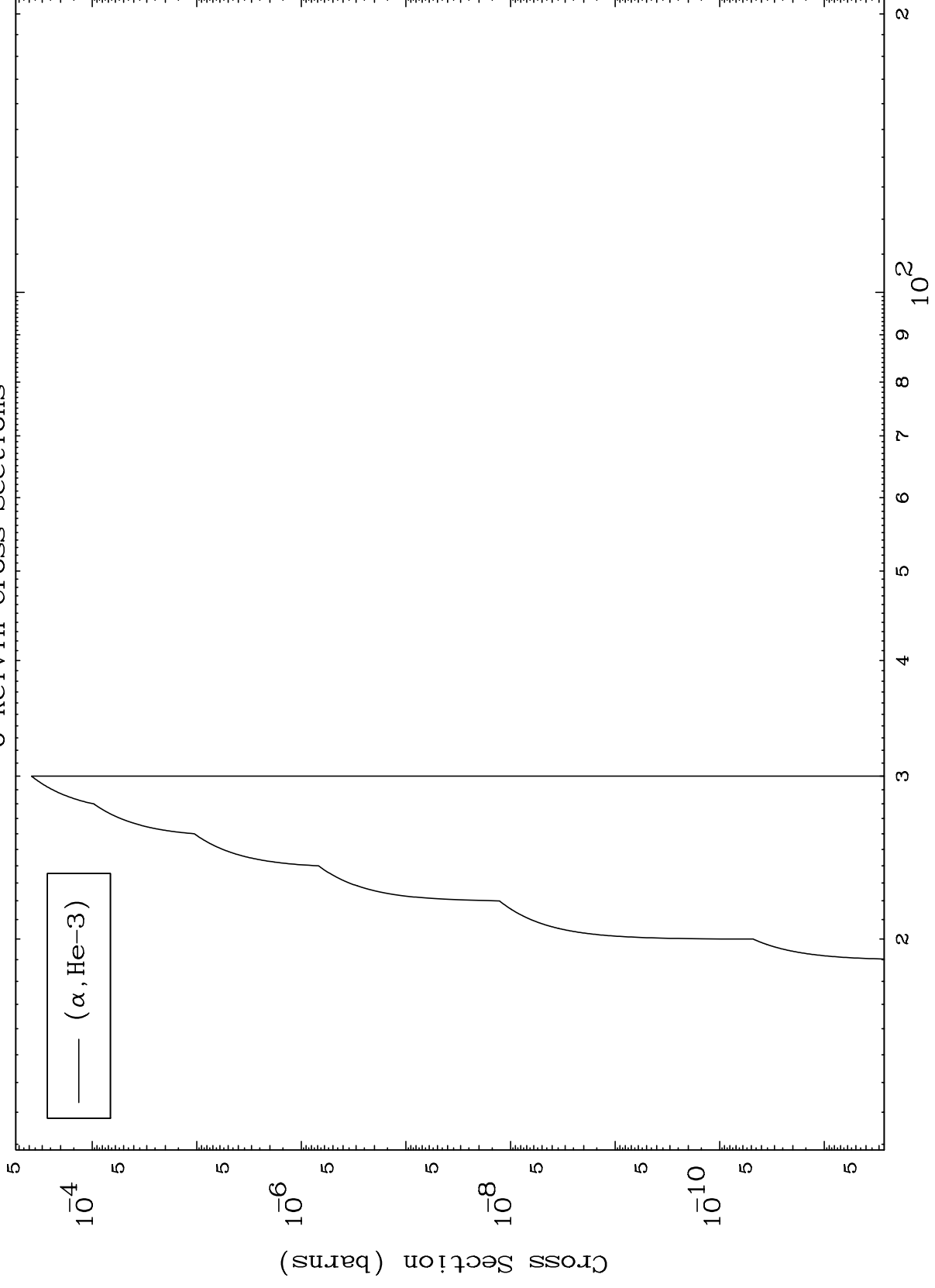
Incident Energy (MeV)

49-In-109





0 Kelvin Cross Sections

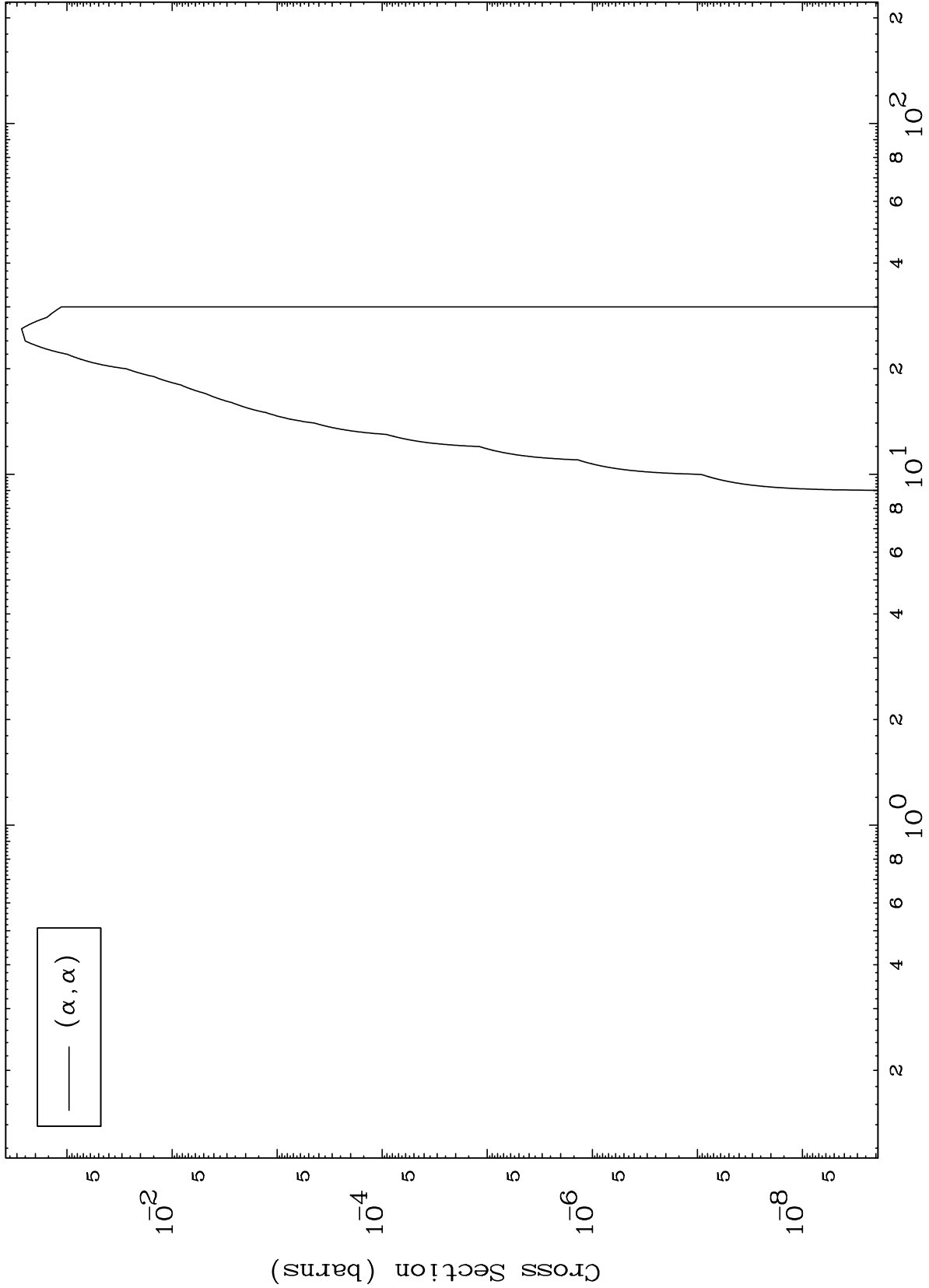


MAT 4914

( $\alpha, \alpha$ ) Levels

49-In-109

0 Kelvin Cross Sections



10

Incident Energy (MeV)

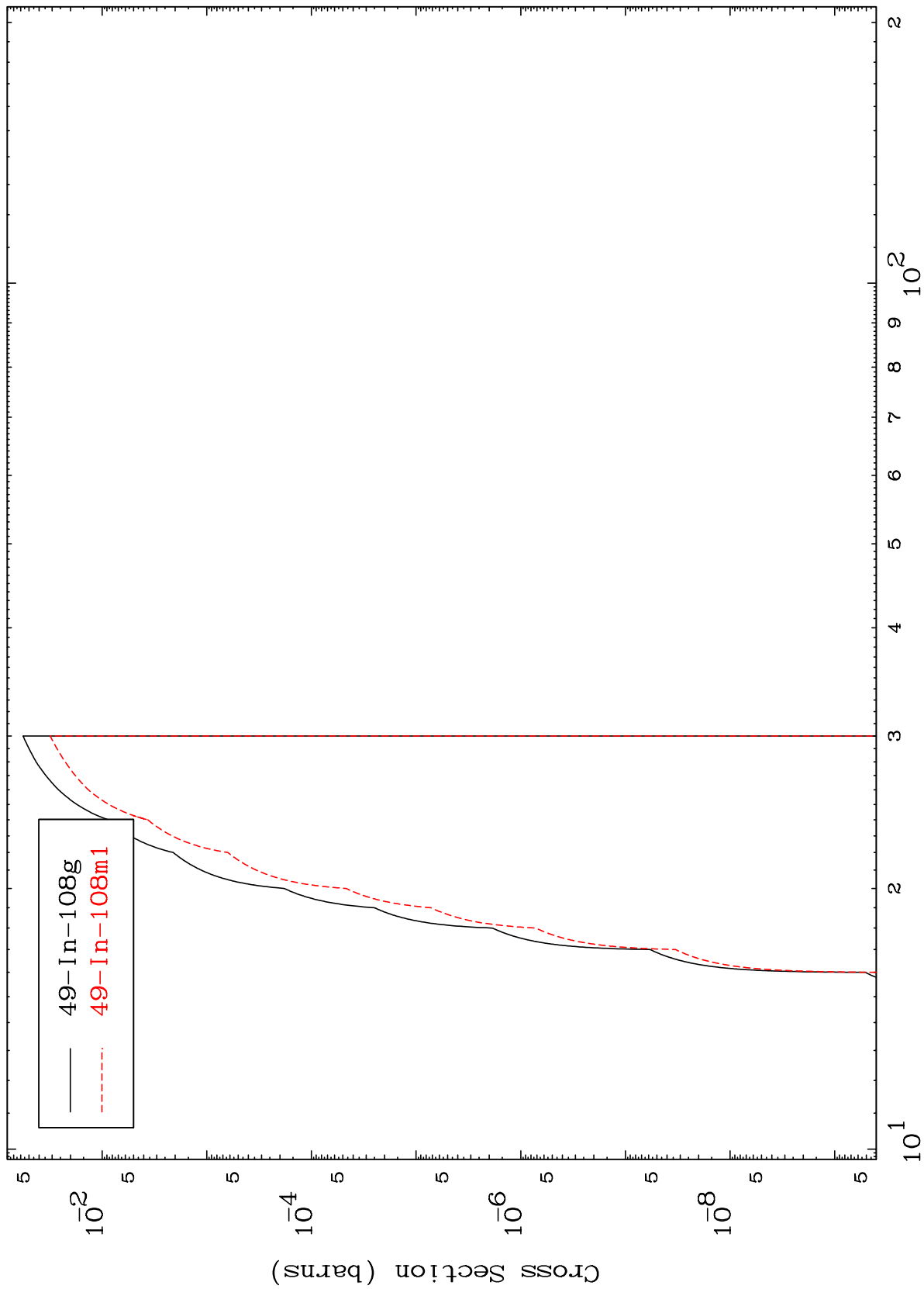
49-In-109

MAT 4914

( $\alpha, n'$ )  $\alpha$

49-In-109

Radionuclide Production Cross Section



11

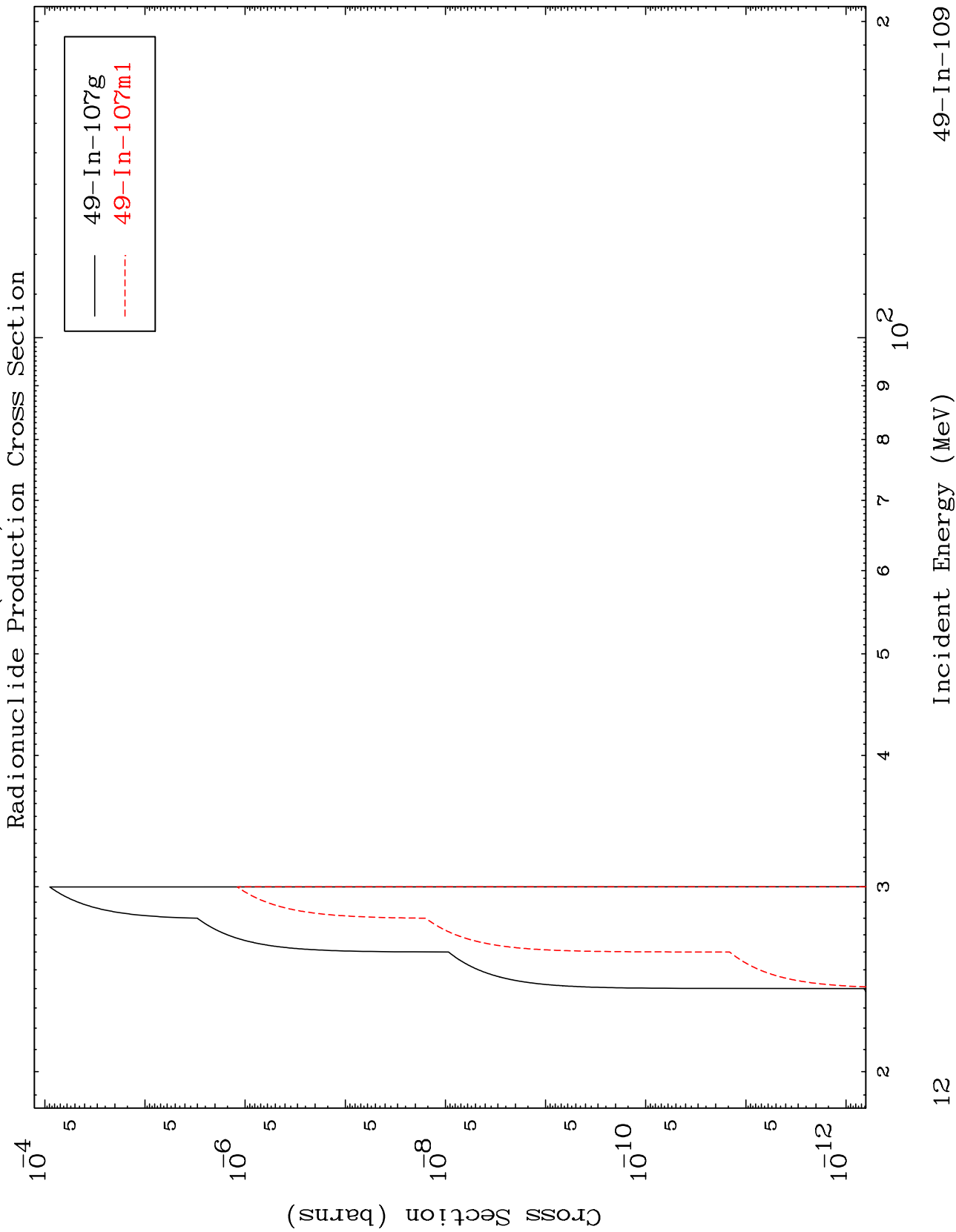
Incident Energy (MeV)

49-In-109

MAT 4914

( $\alpha, 2n$ )  $\alpha$

49-In-109



12

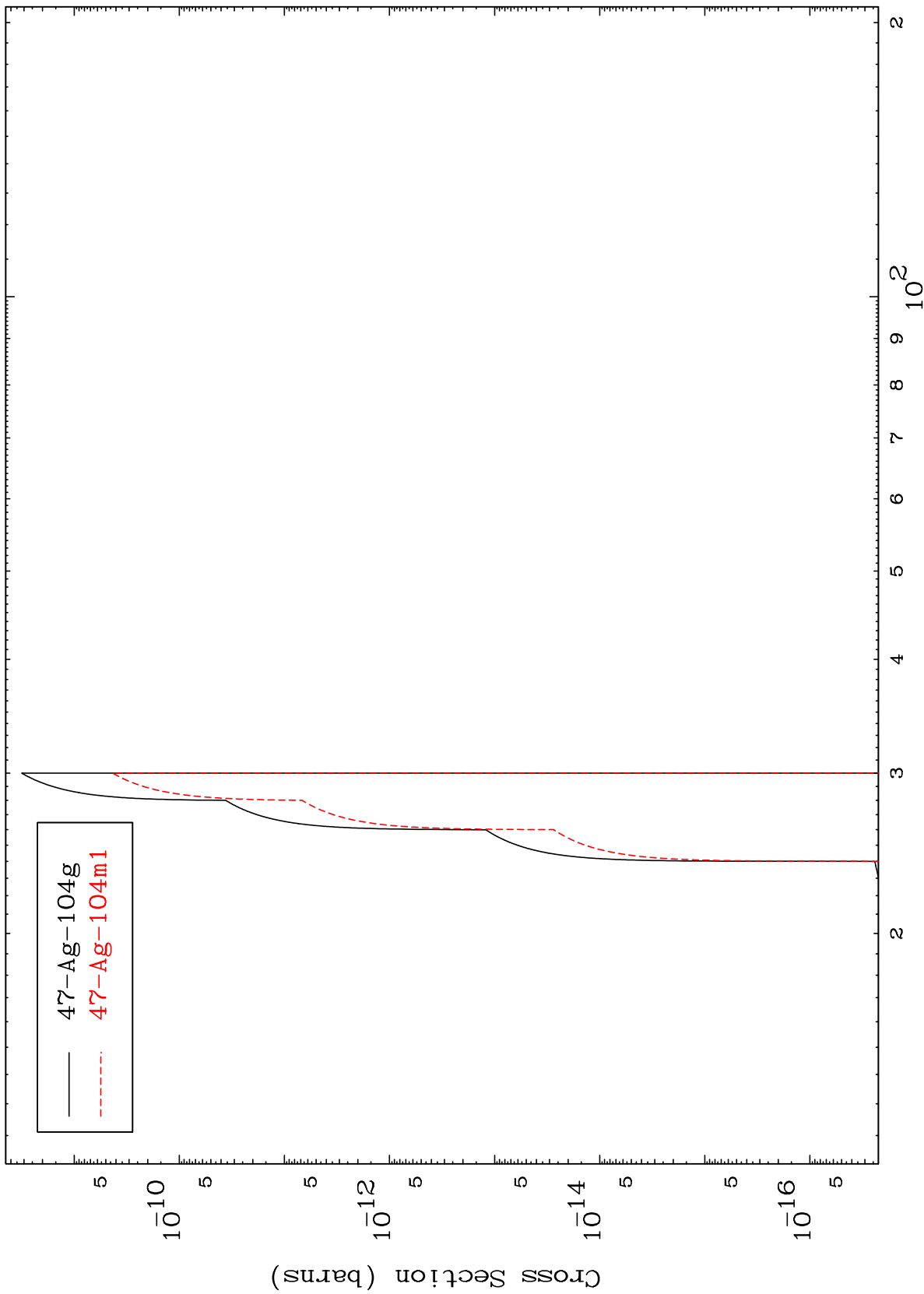
Incident Energy (MeV)

49-In-109

MAT 4914

49-In-109

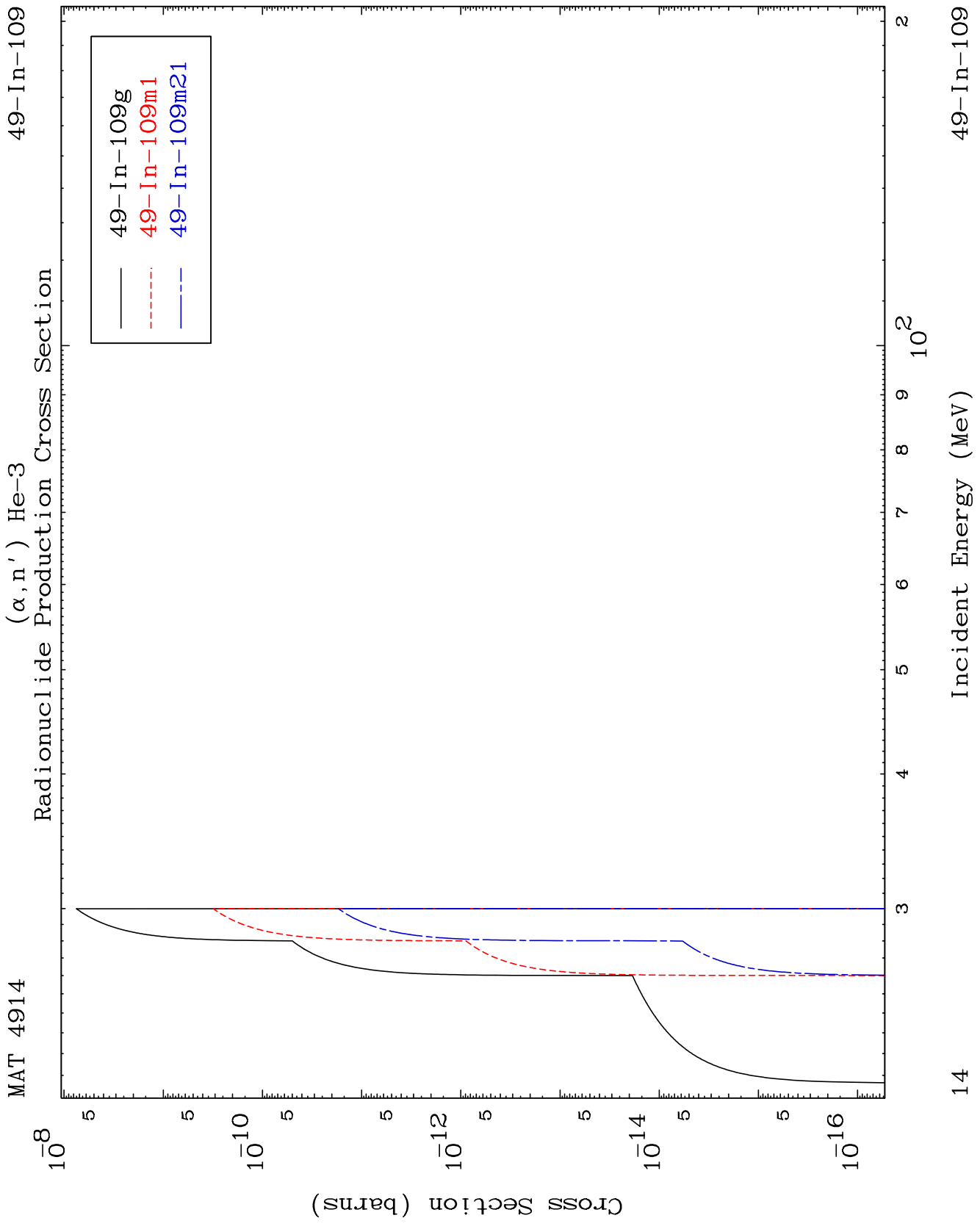
$(\alpha, n')$   $2\alpha$   
Radionuclide Production Cross Section



13

Incident Energy (MeV)

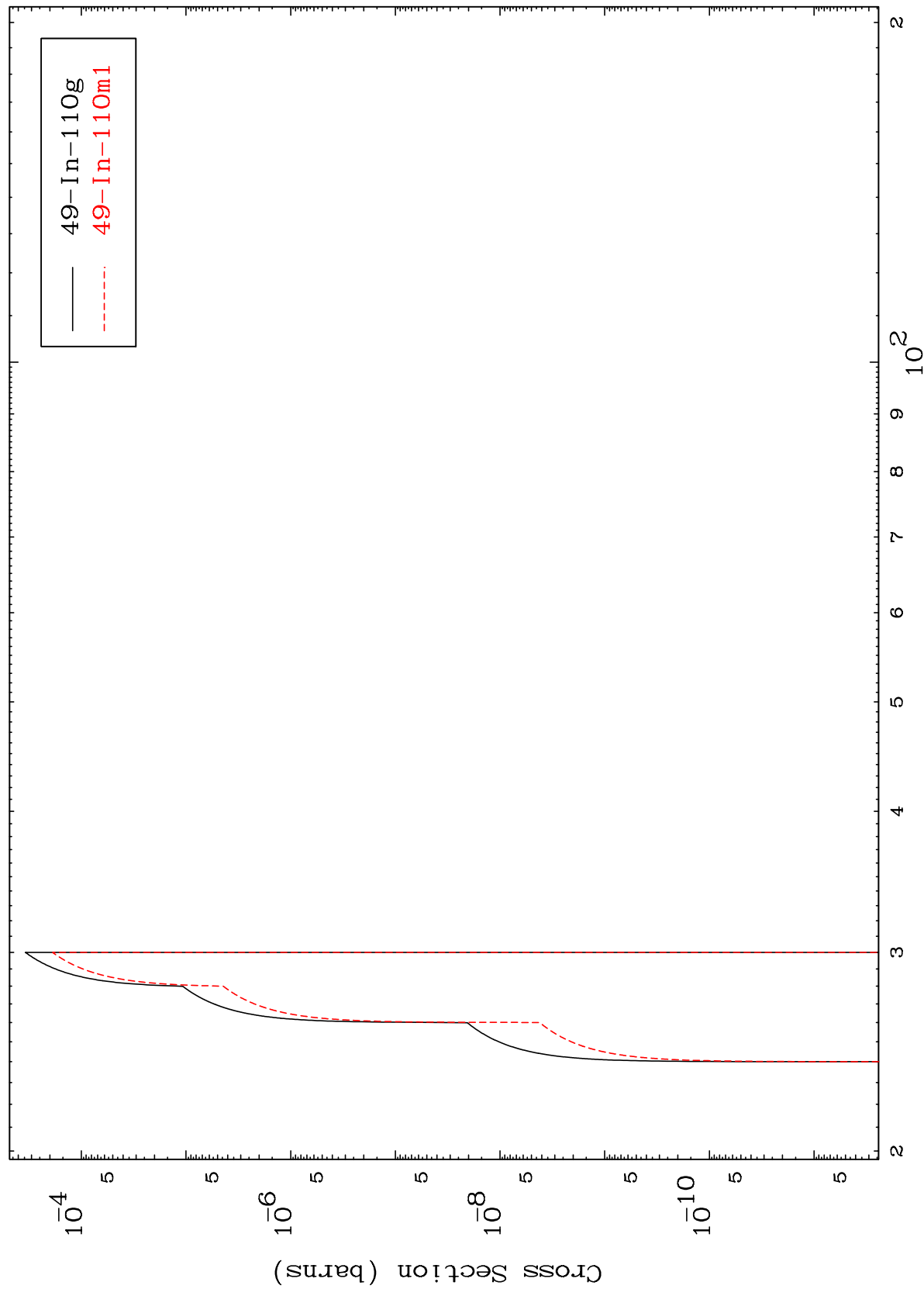
49-In-109



MAT 4914

49-In-109

$(\alpha, 2n)$  p  
Radionuclide Production Cross Section



15

49-In-109

Incident Energy (MeV)

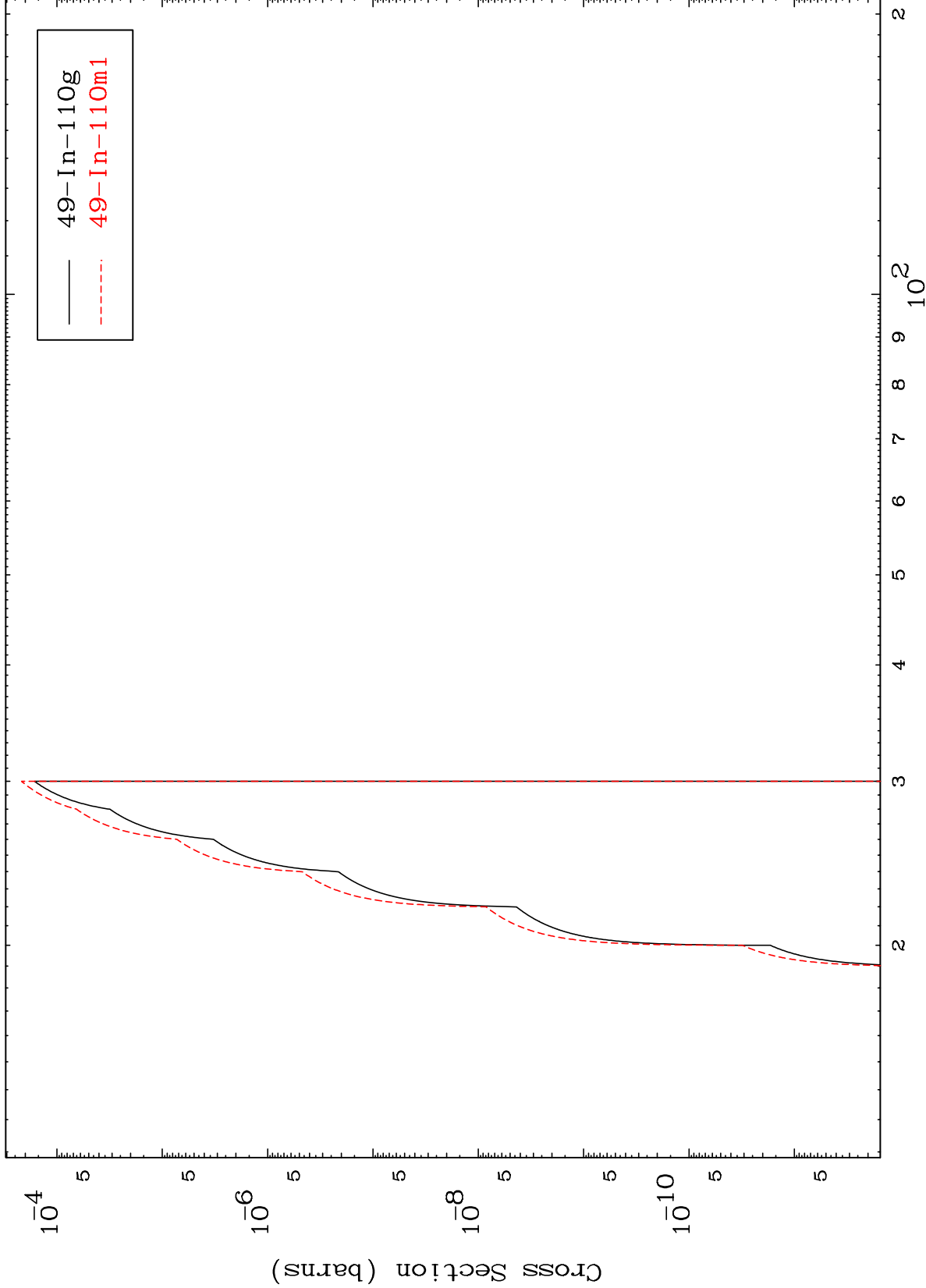


MAT 4914

( $\alpha, \text{He-3}$ )

49-In-109

Radionuclide Production Cross Section



16

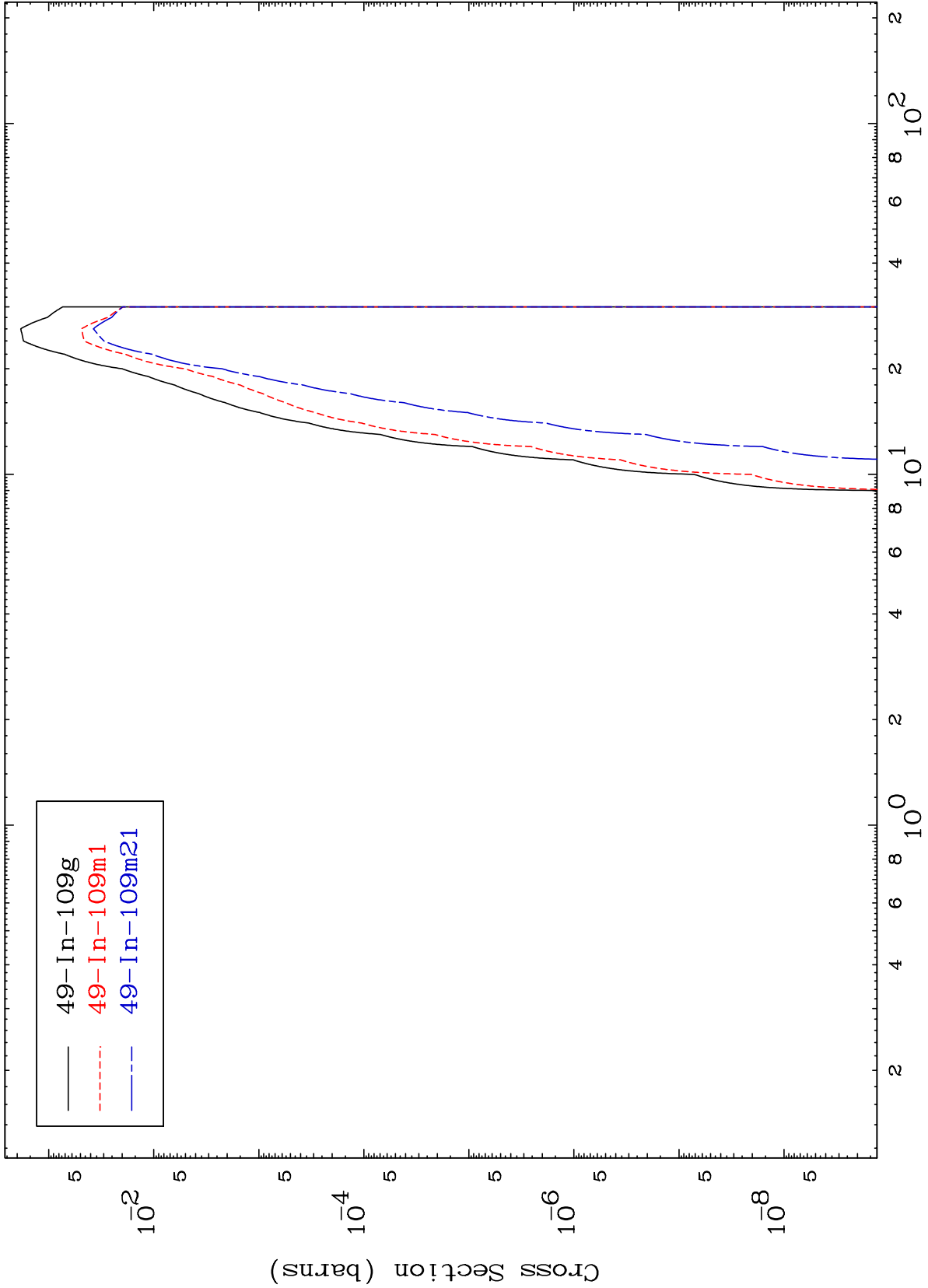
Incident Energy (MeV)

49-In-109

MAT 4914

49-In-109

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



17

49-In-109

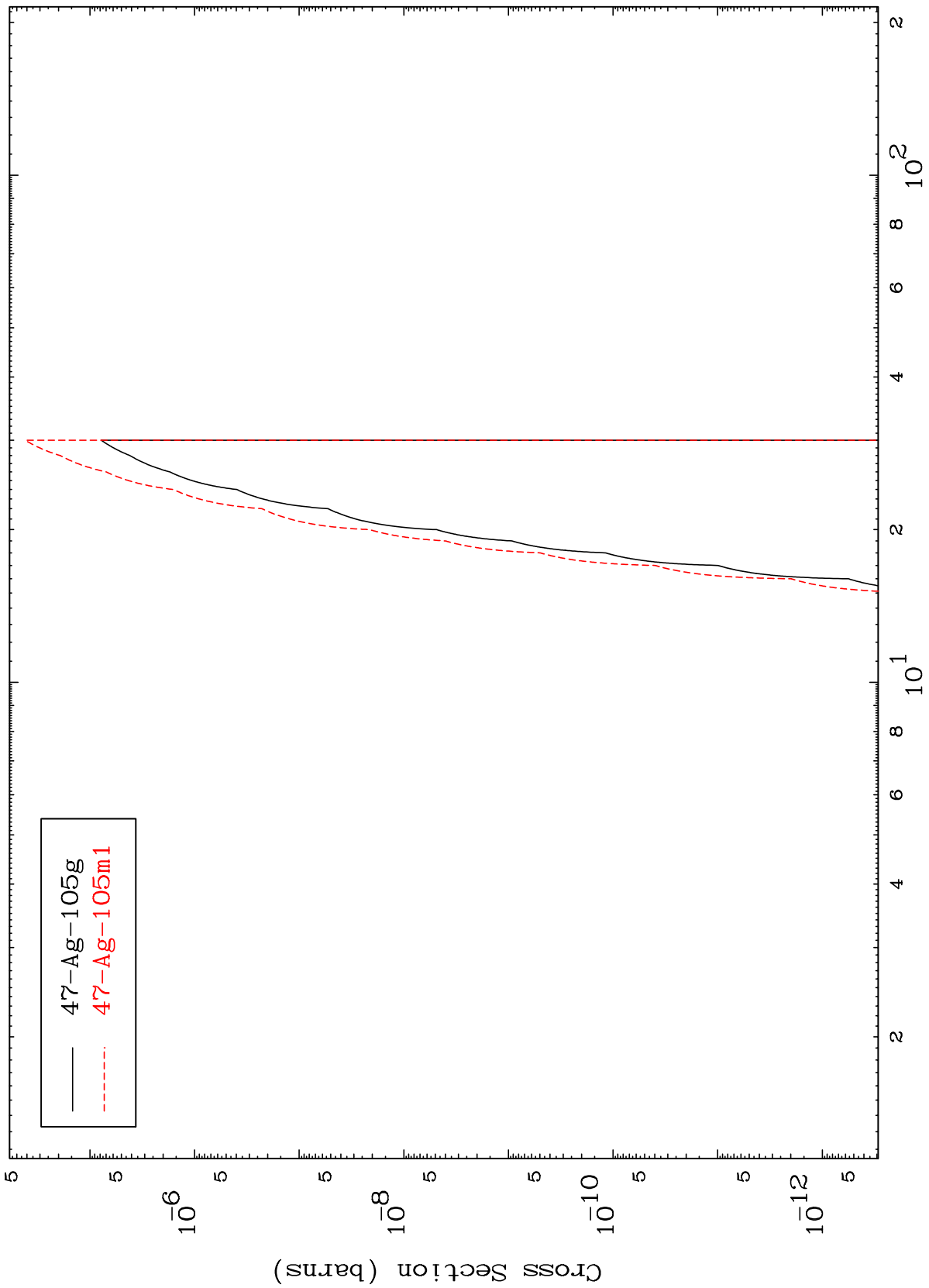
Incident Energy (MeV)

MAT 4914

( $\alpha, 2\alpha$ )

49-In-109

Radionuclide Production Cross Section

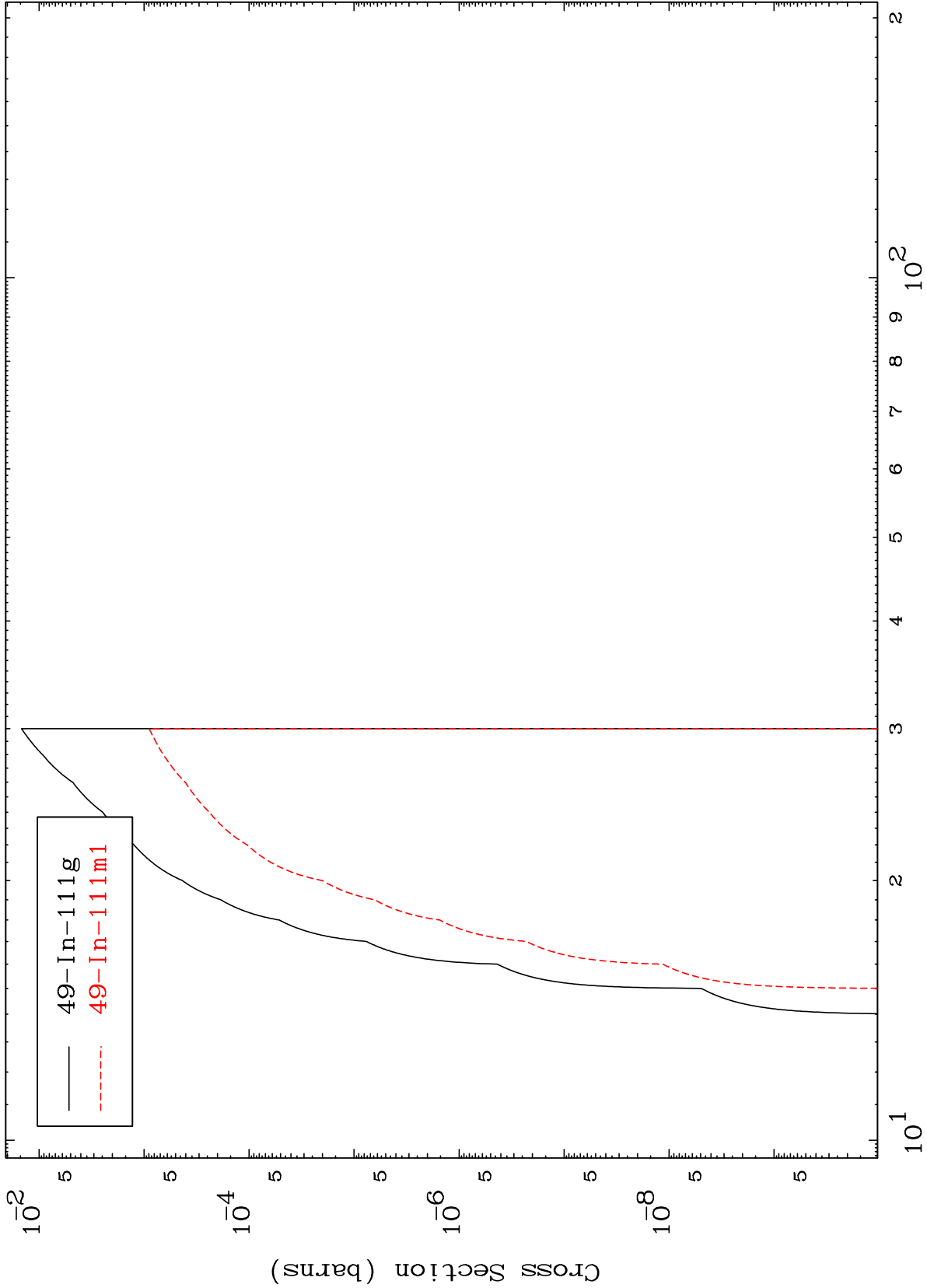


— 47-Ag-105g  
- - - 47-Ag-105m1

MAT 4914

49-In-109

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



49-In-109

Incident Energy (MeV)

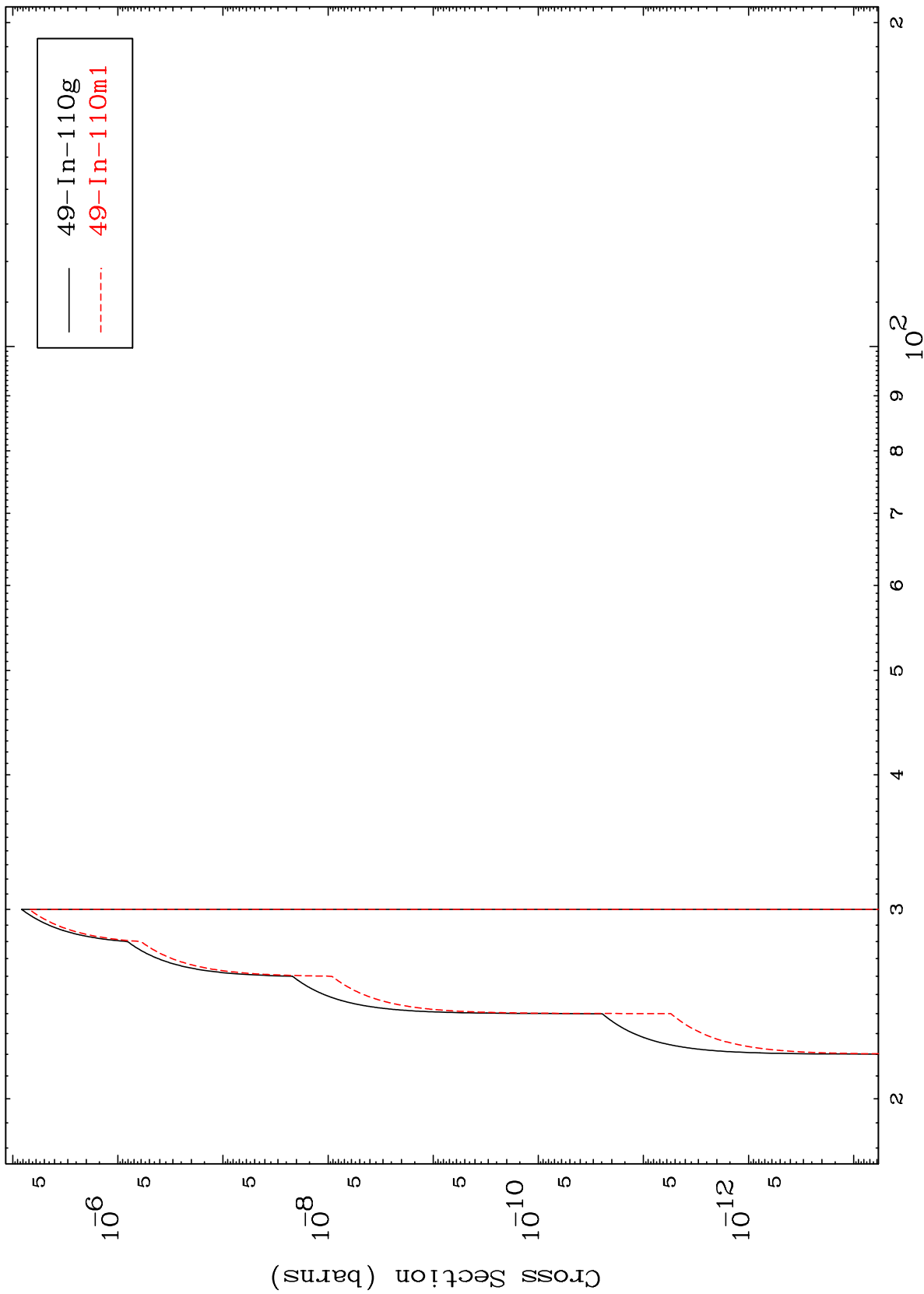
19

MAT 4914

( $\alpha, p$ ) d

49-In-109

Radionuclide Production Cross Section



20

Incident Energy (MeV)

49-In-109

