

Program EVALPLOT  
(Version 2018-1)

by

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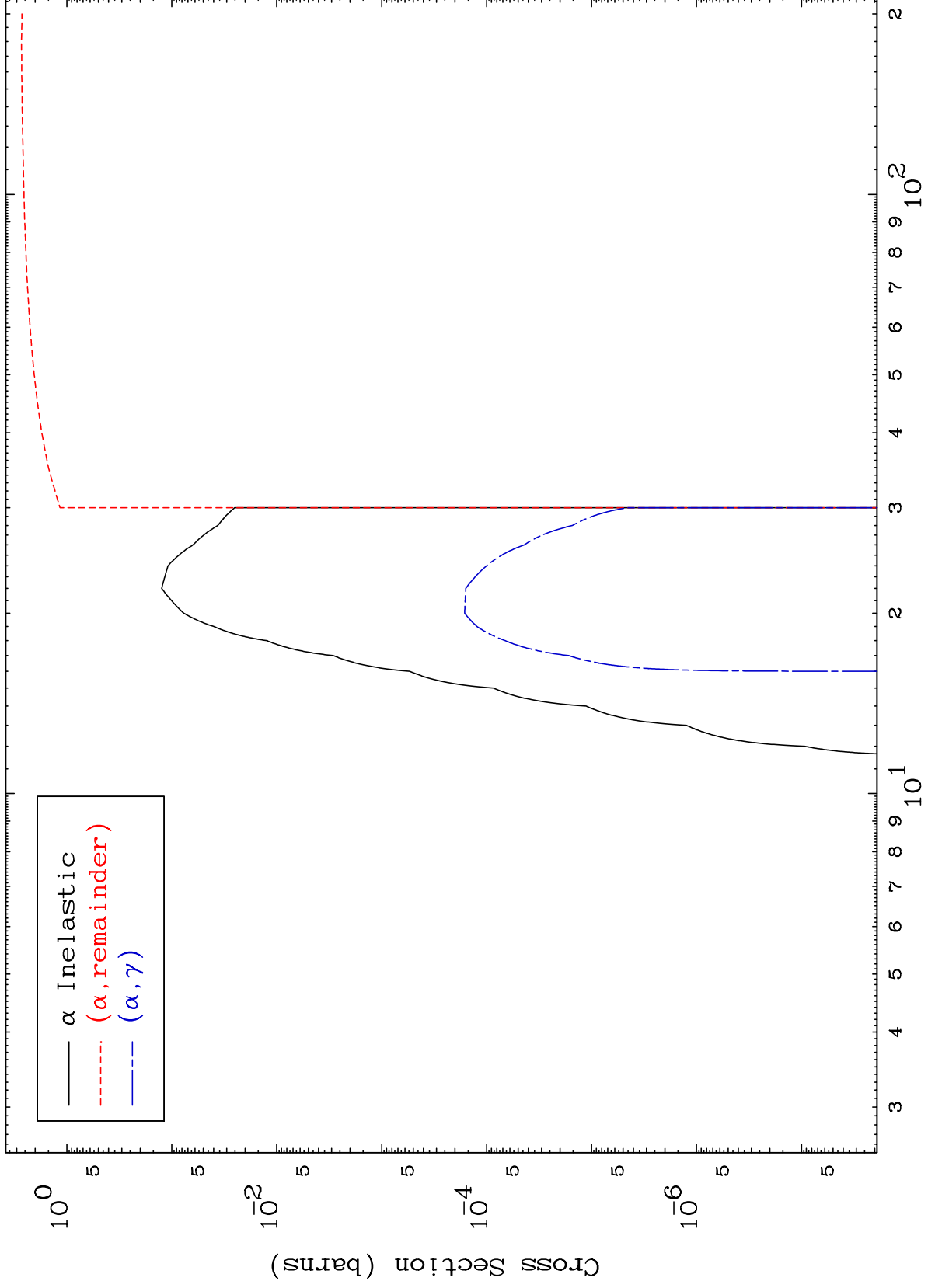
Press Mouse Button to Start

MAT 7719

$\alpha$  Major

77-Ir-189

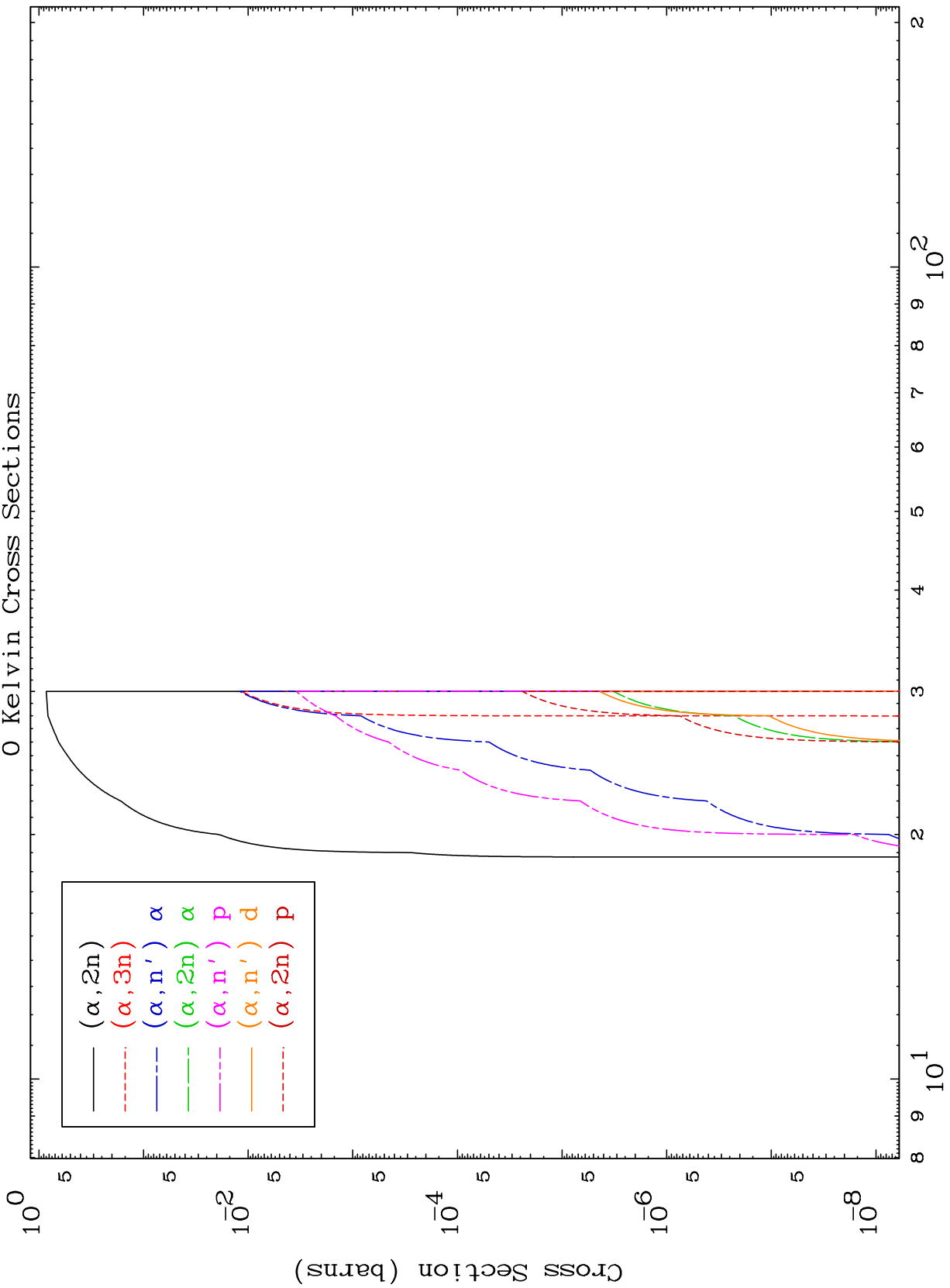
0 Kelvin Cross Sections



MAT 7719

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

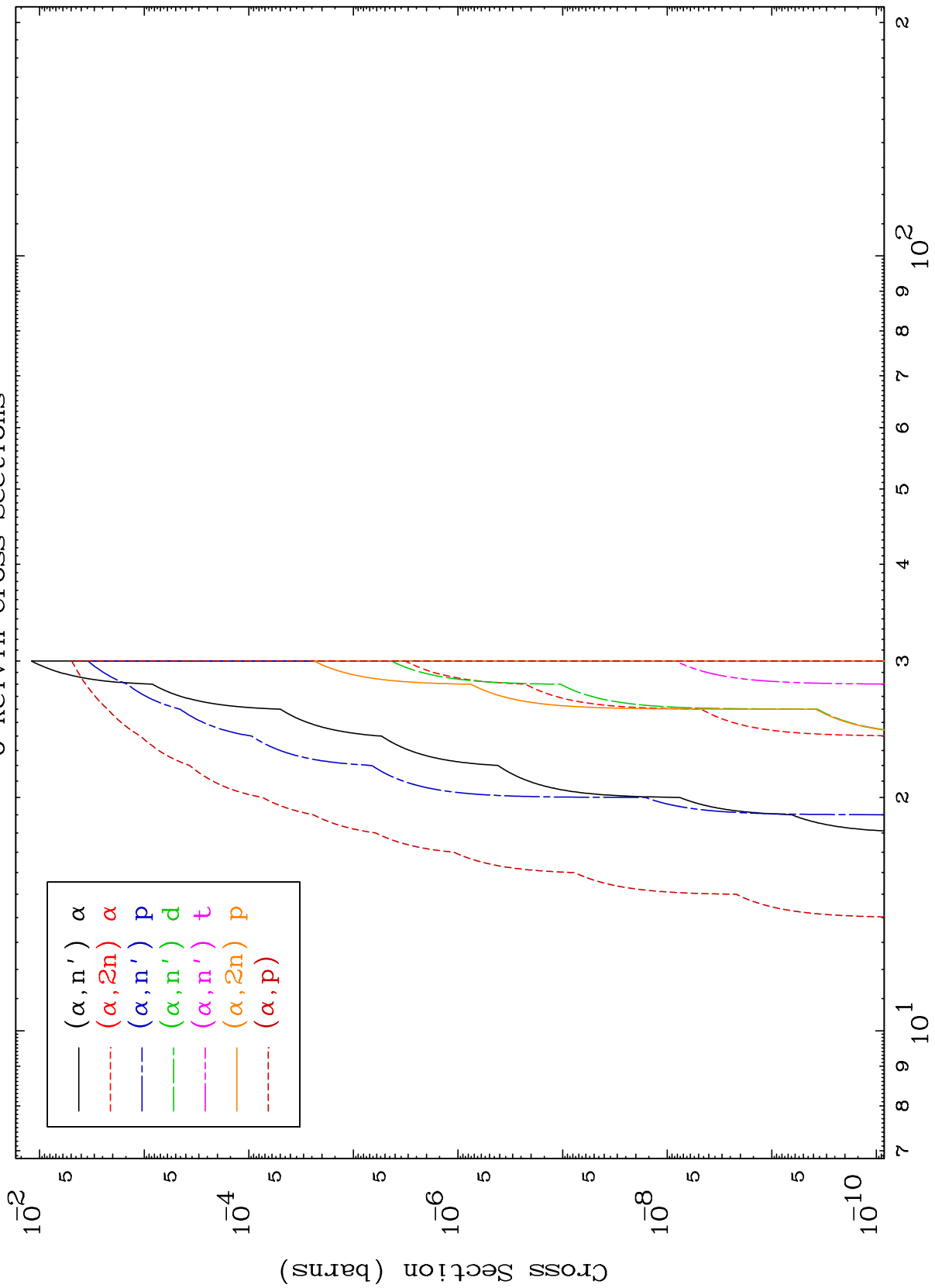
77-Ir-189

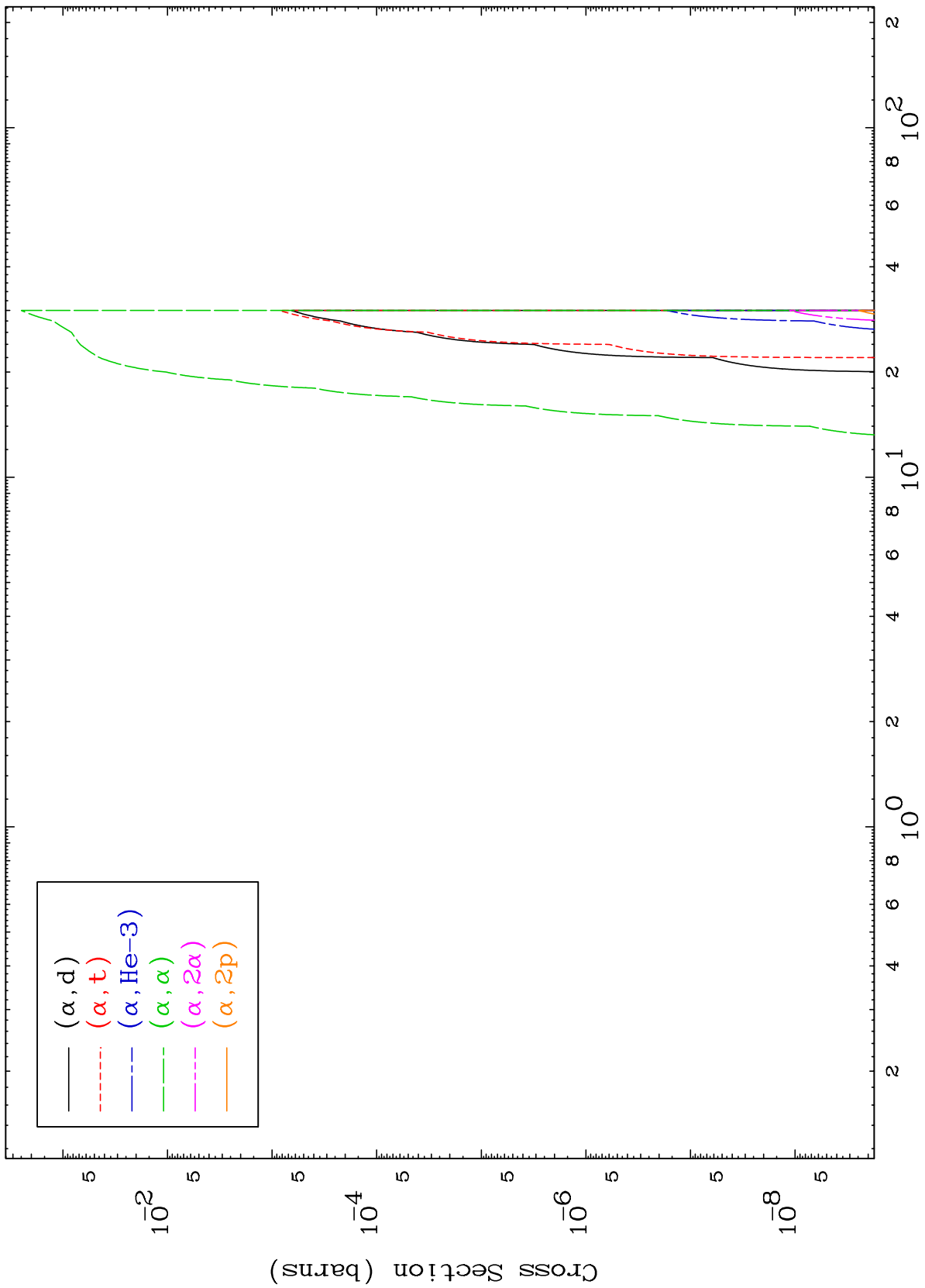


Incident Energy (MeV)

77-Ir-189

2



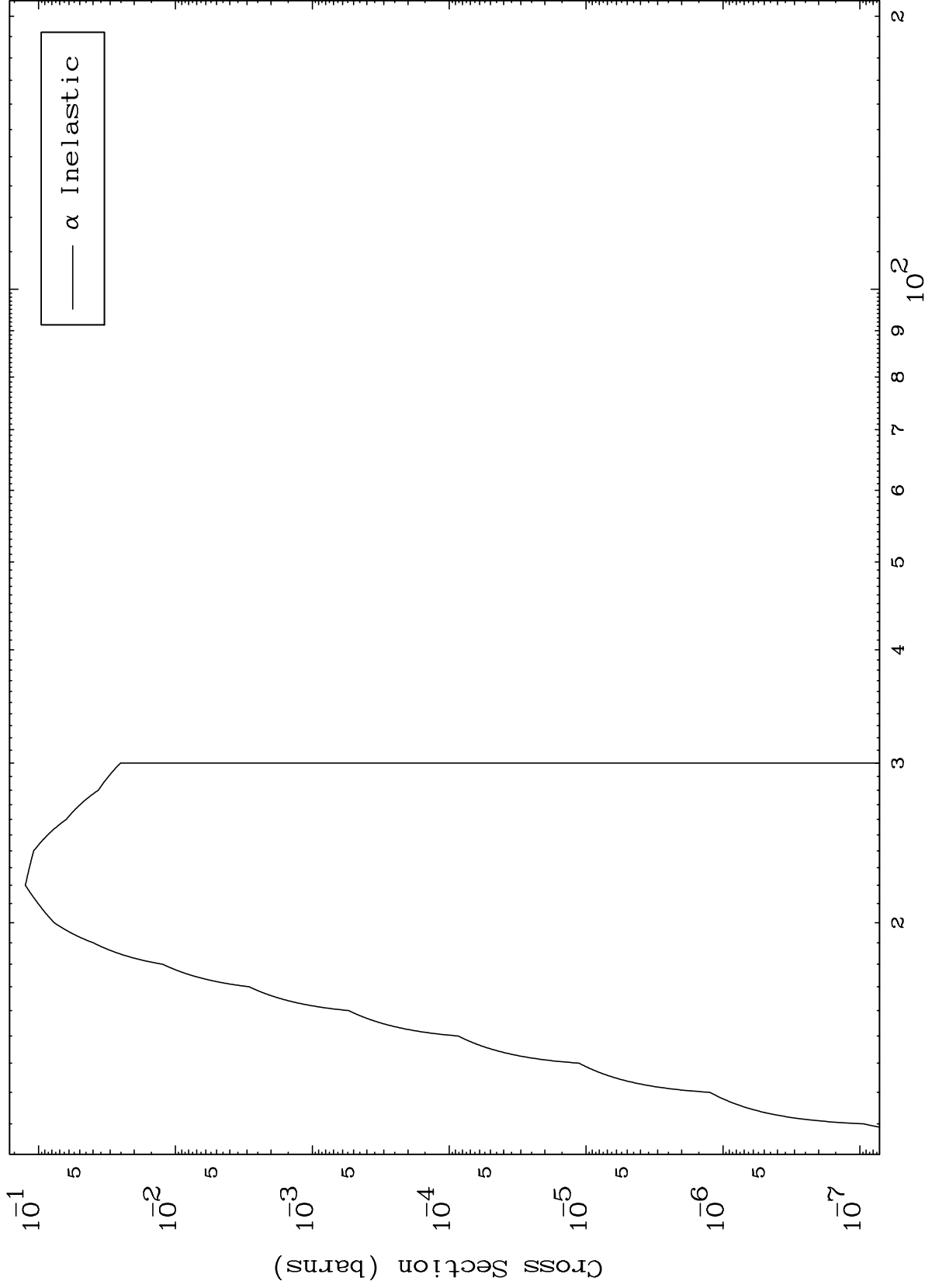


MAT 7719

( $\alpha, n'$ ) Level

77-Ir-189

0 Kelvin Cross Sections



Incident Energy (MeV)

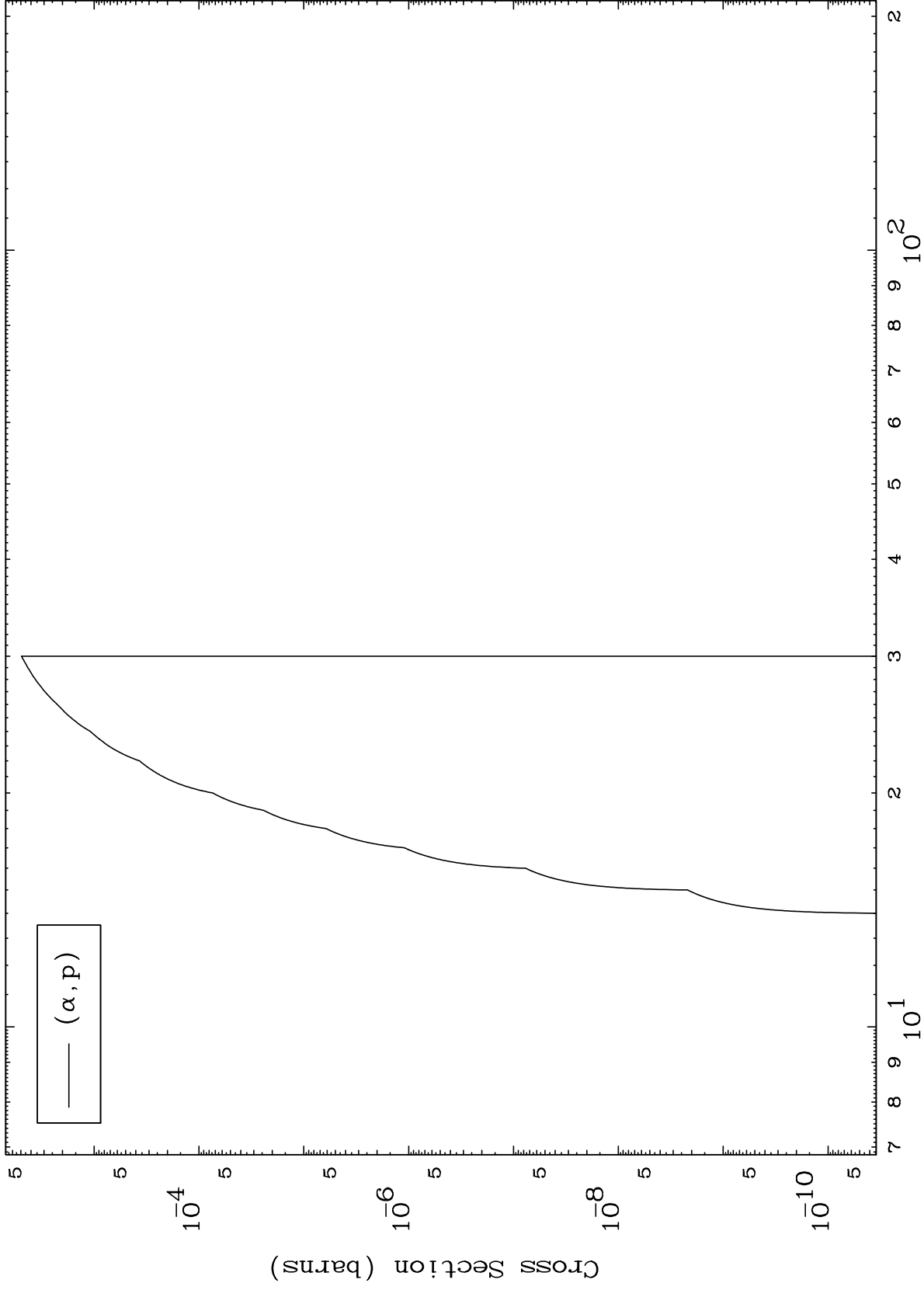
77-Ir-189

5

MAT 7719

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

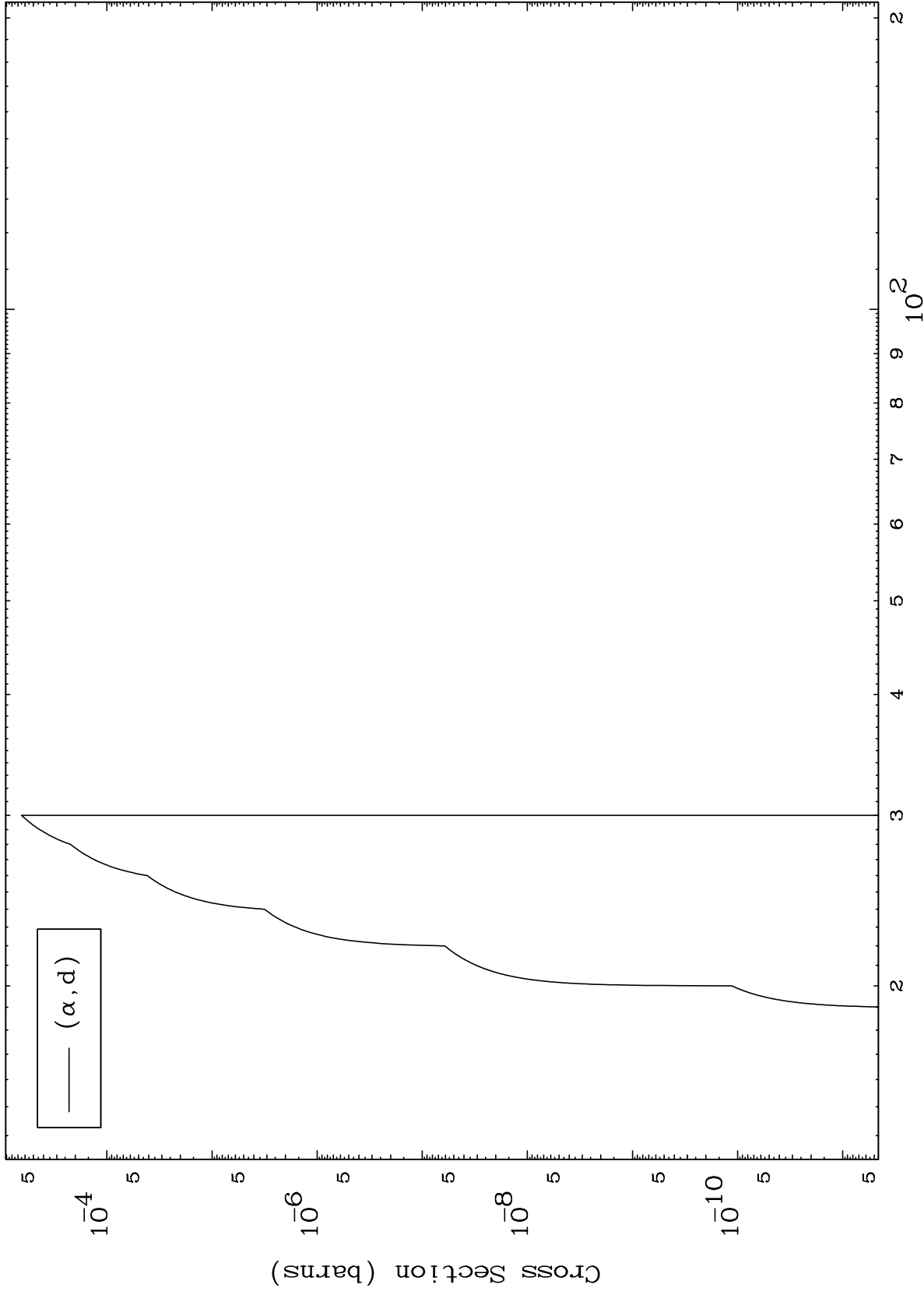
77-Ir-189



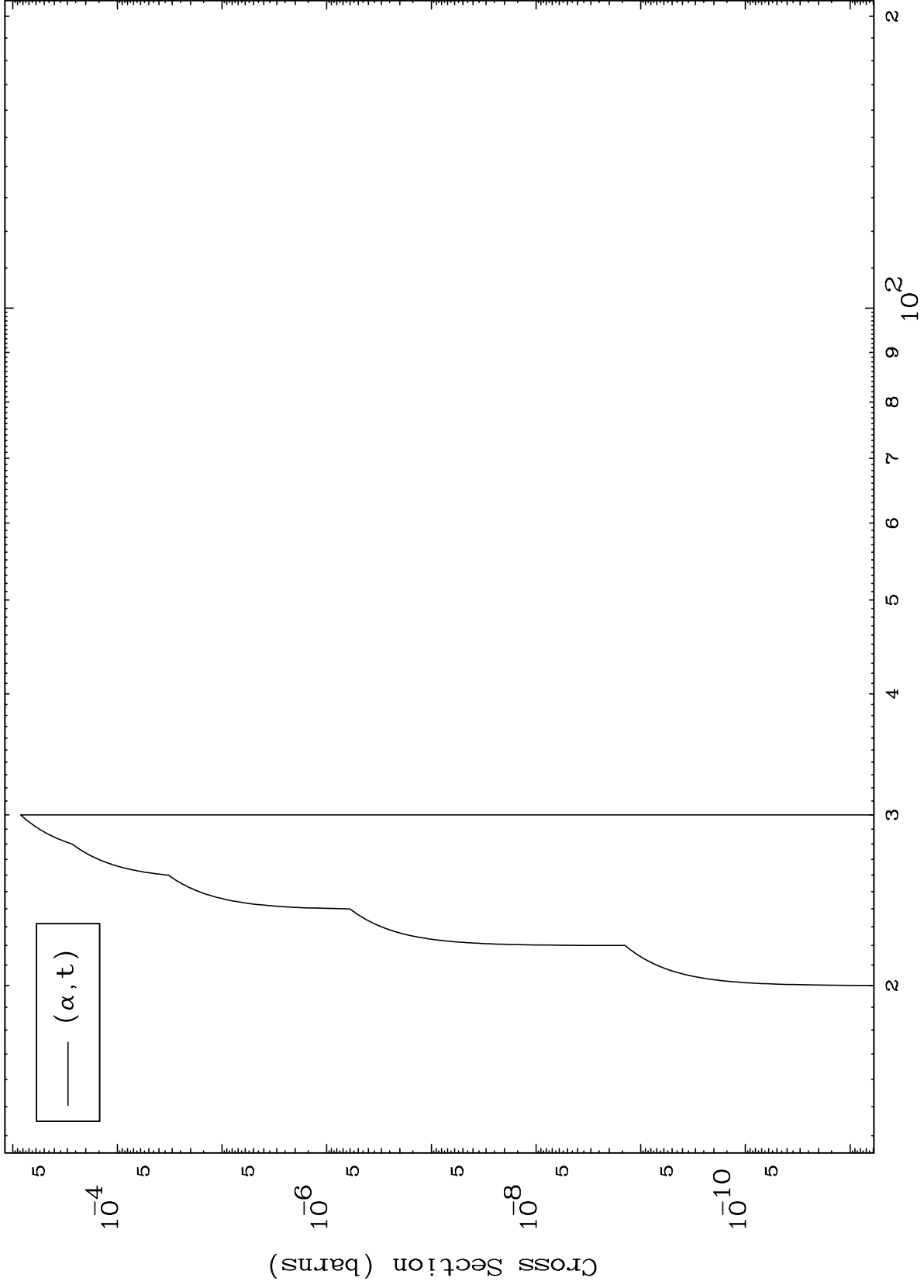
6

Incident Energy (MeV)

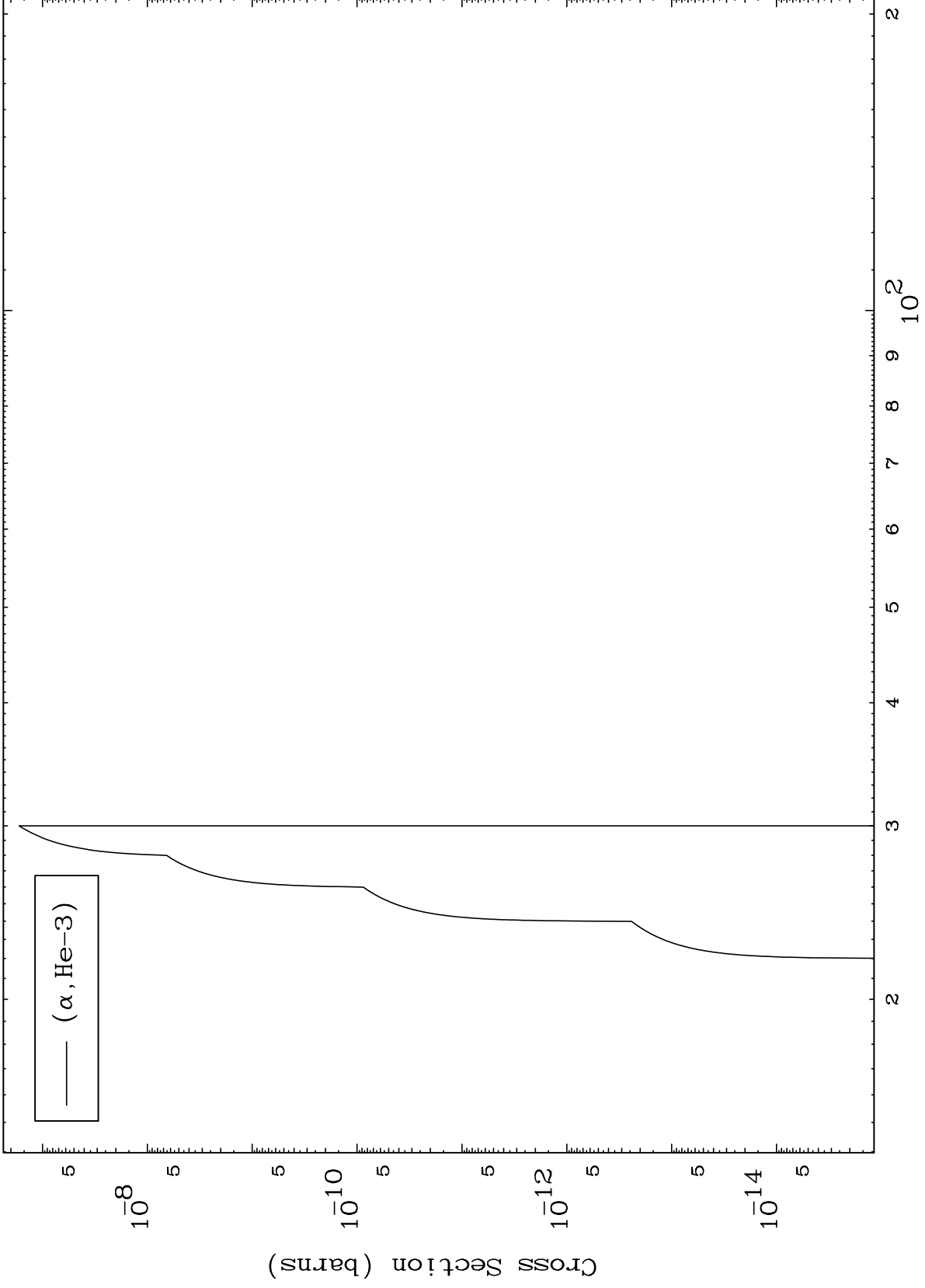
77-Ir-189







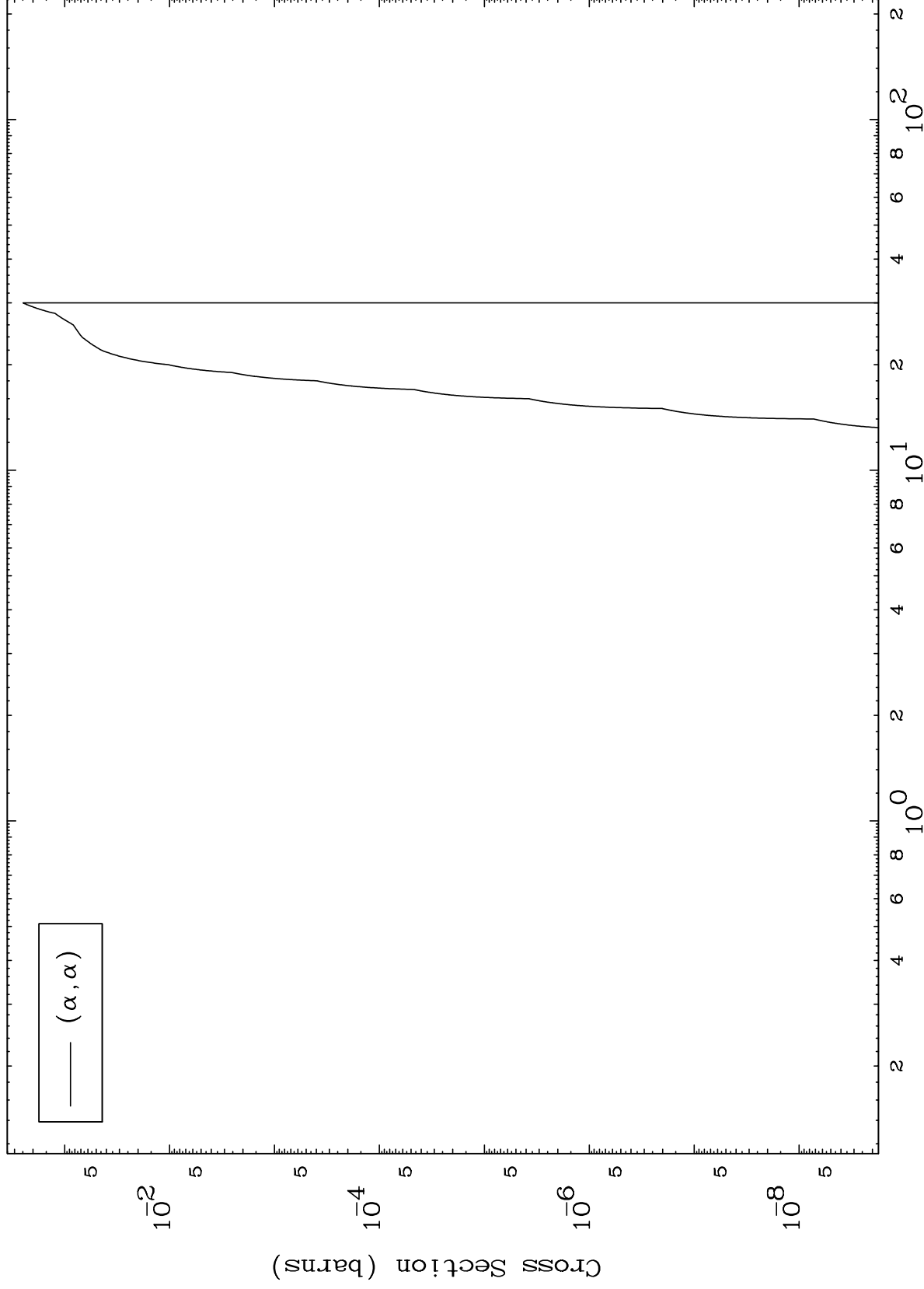
0 Kelvin Cross Sections



MAT 7719

77-Ir-189

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections



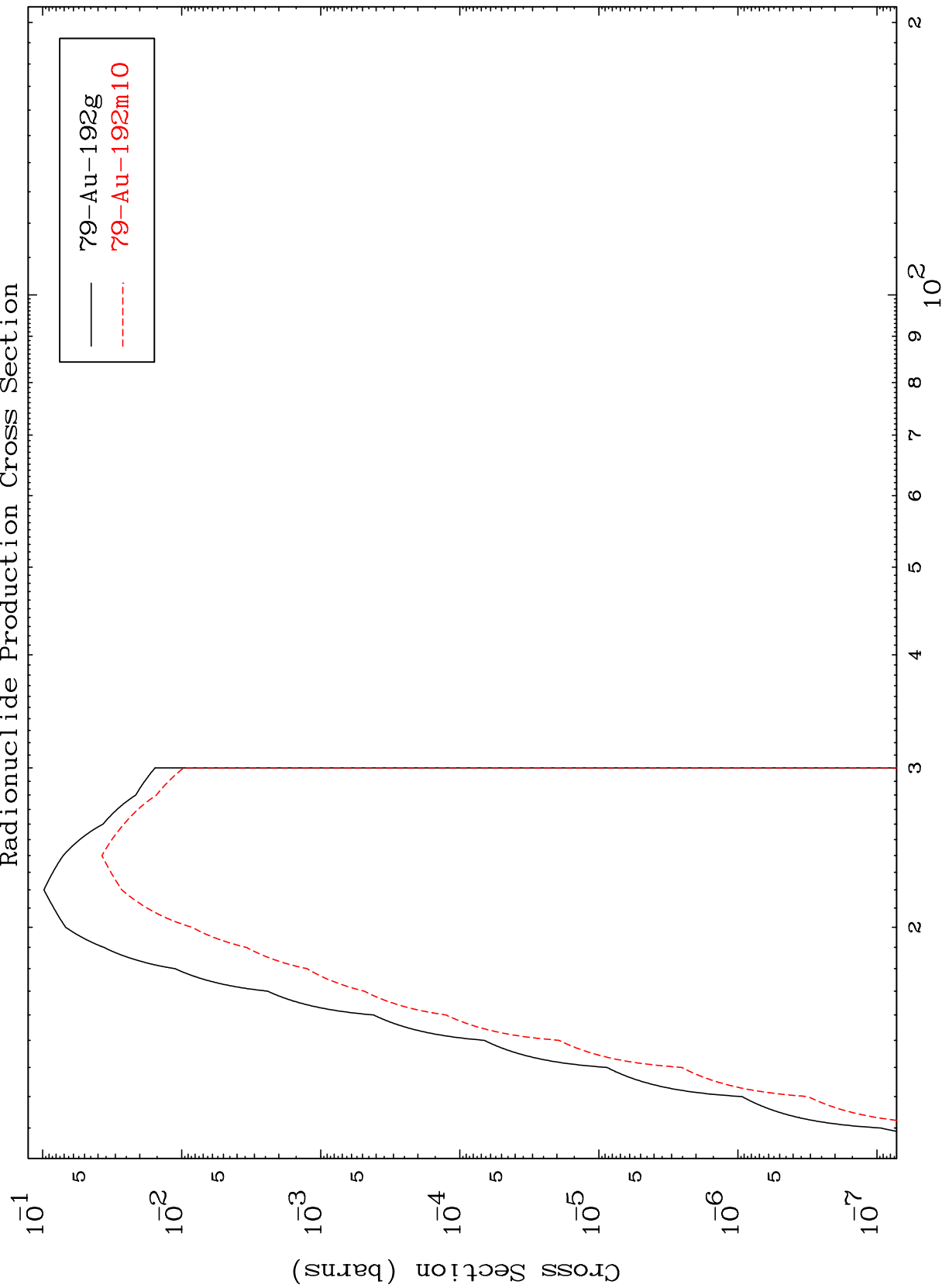
10

Incident Energy (MeV)

77-Ir-189

Radionuclide Production Cross Section

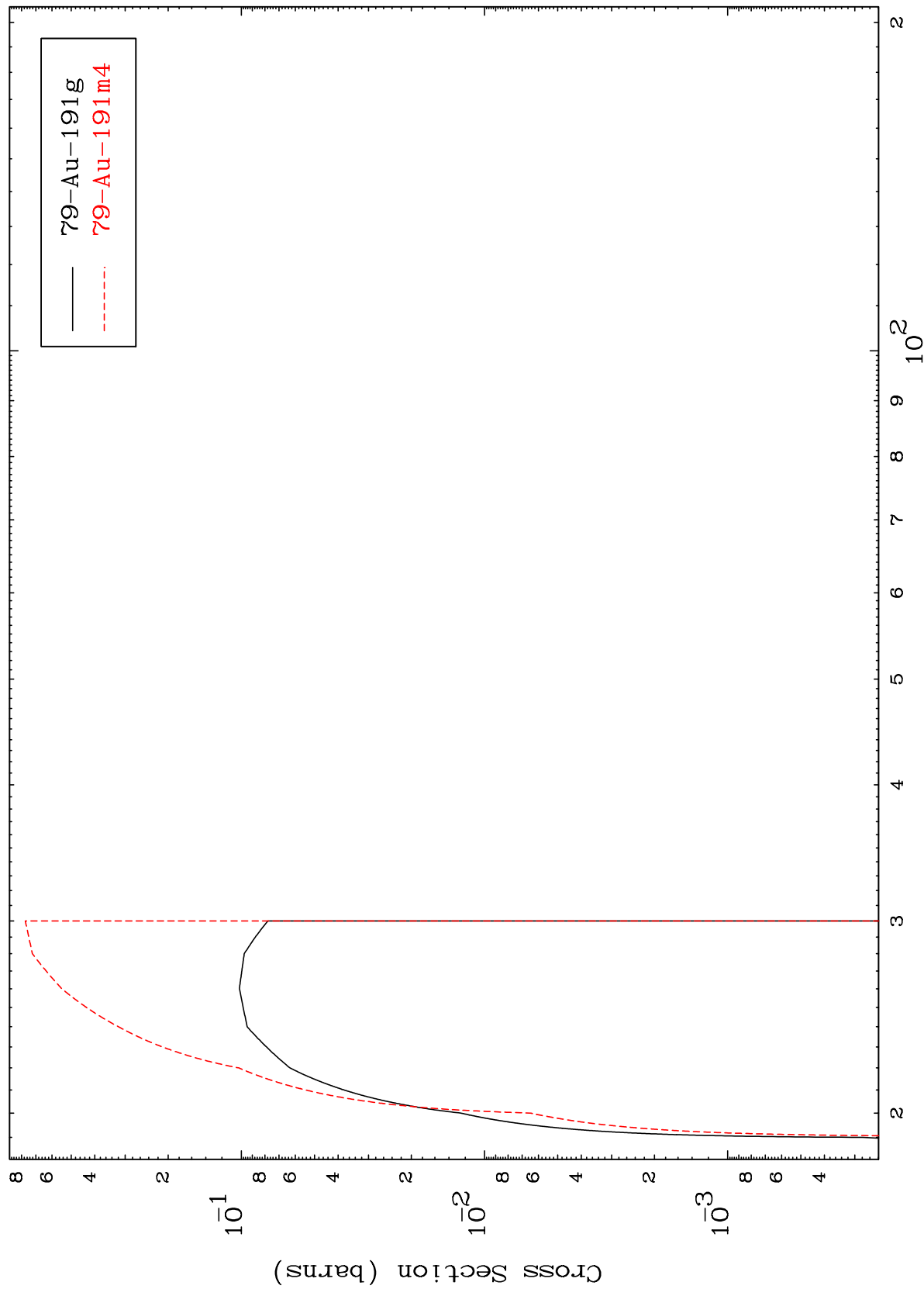
$\alpha$  Inelastic



MAT 7719

77-Ir-189

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



12

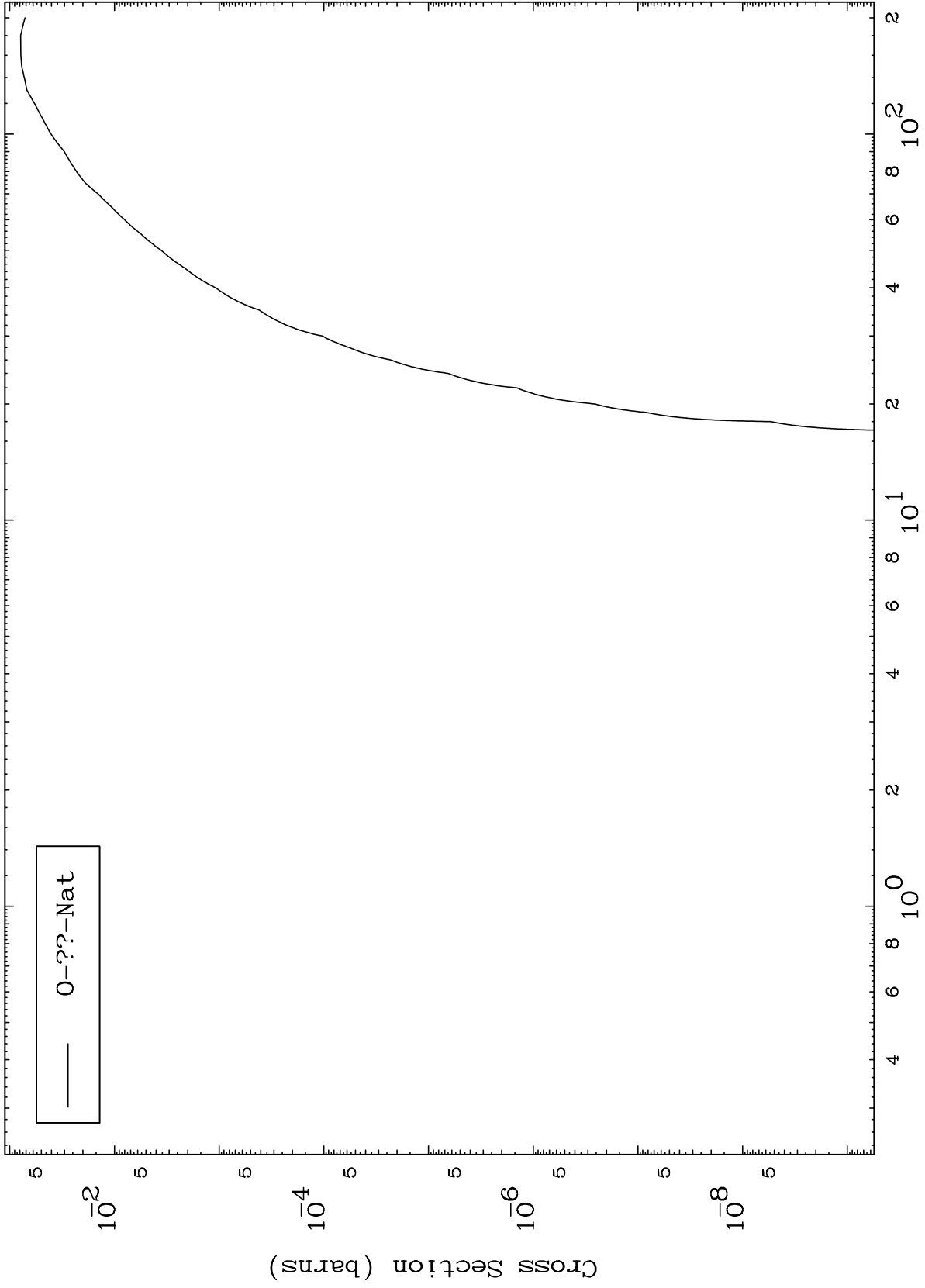
Incident Energy (MeV)

77-Ir-189

MAT 7719

77-Ir-189

$\alpha$  Fission  
Radionuclide Production Cross Section



13

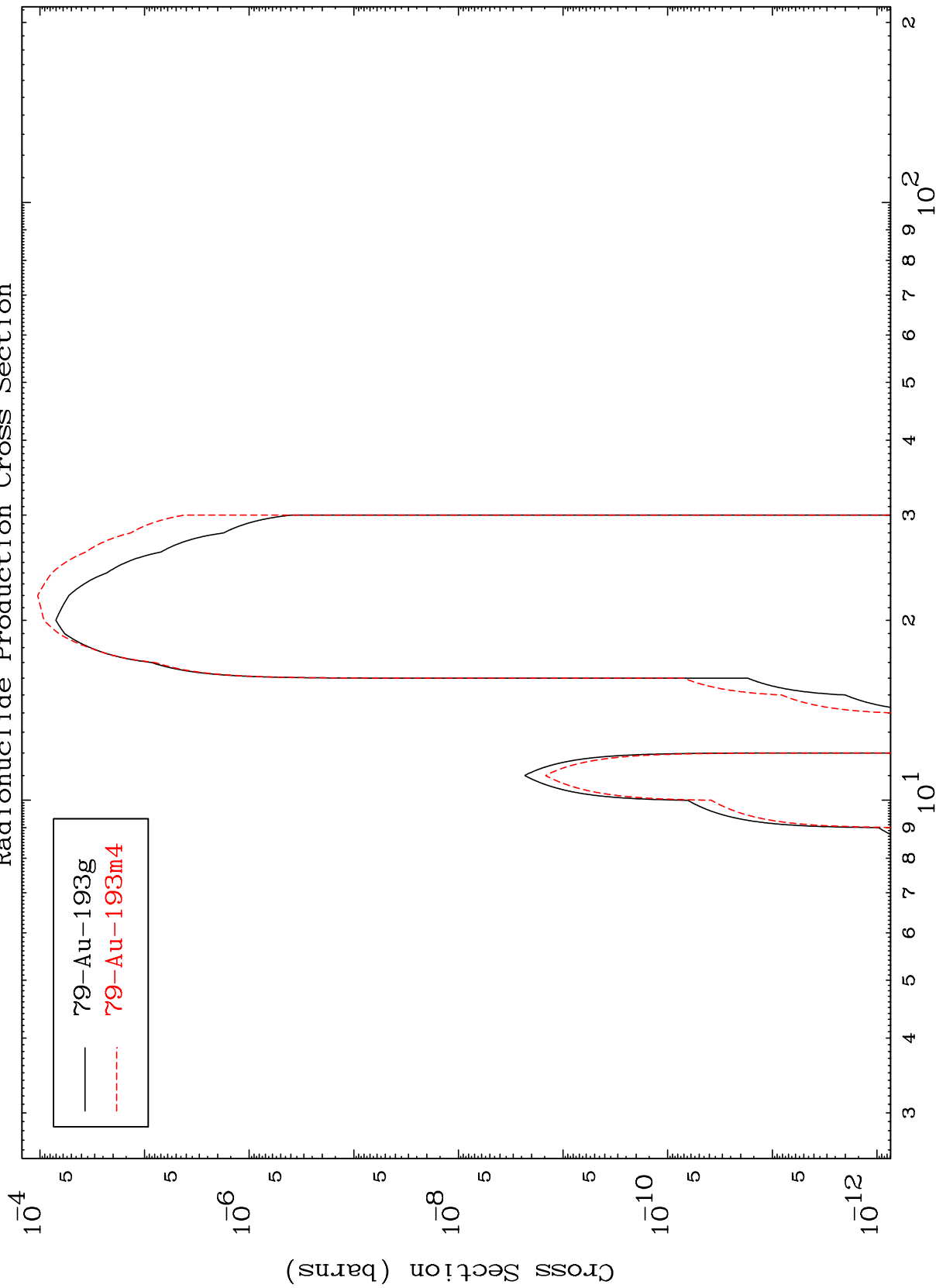
77-Ir-189

Incident Energy (MeV)

MAT 7719

<sup>77</sup>Ir-189

Radionuclide Production Cross Section



— 79-Au-193g  
- - - 79-Au-193m4

<sup>77</sup>Ir-189

Incident Energy (MeV)

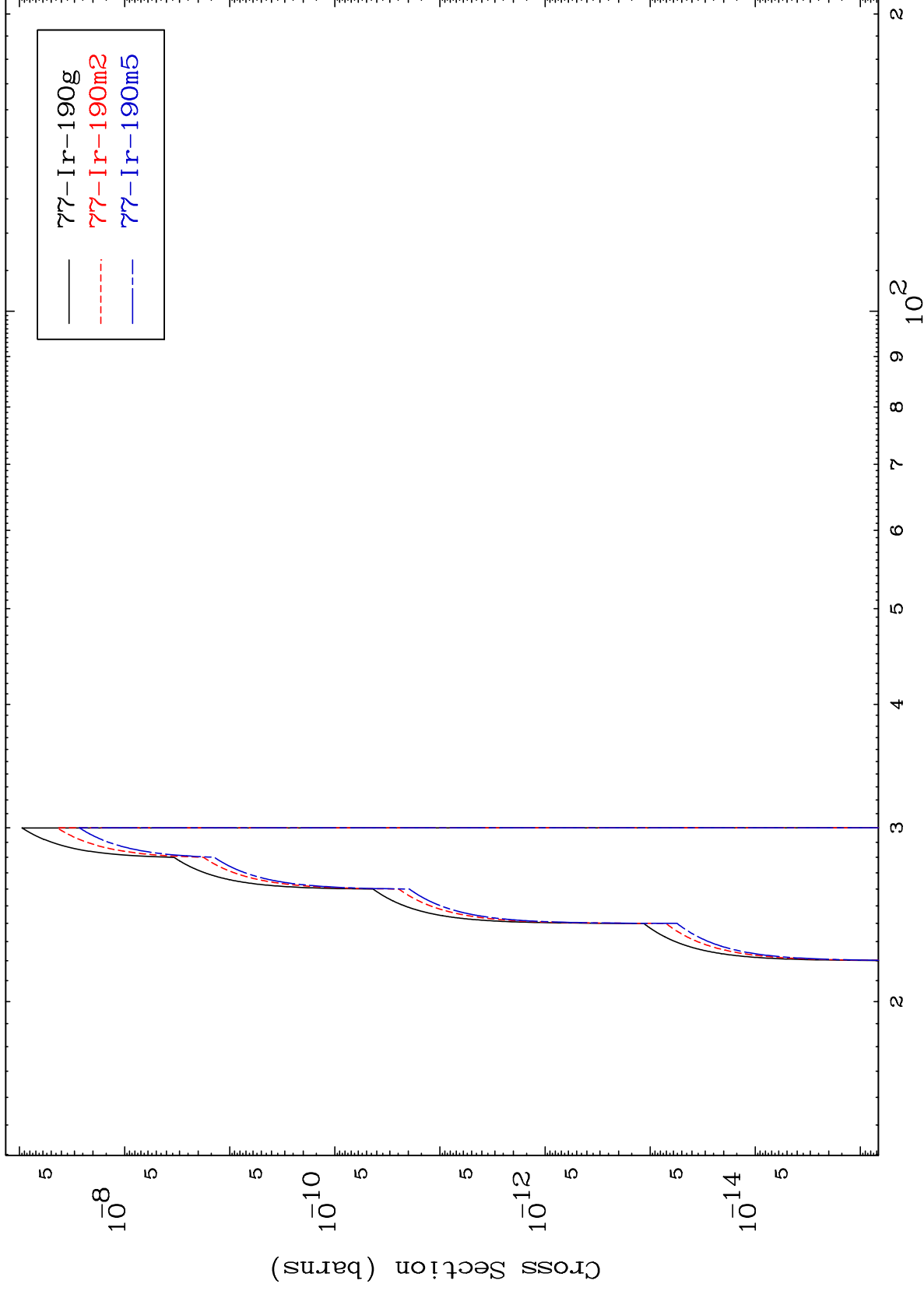
14

MAT 7719

( $\alpha, \text{He-3}$ )

77-Ir-189

Radionuclide Production Cross Section



15

Incident Energy (MeV)

77-Ir-189



( $\alpha, 2p$ )  
Radionuclide Production Cross Section

