

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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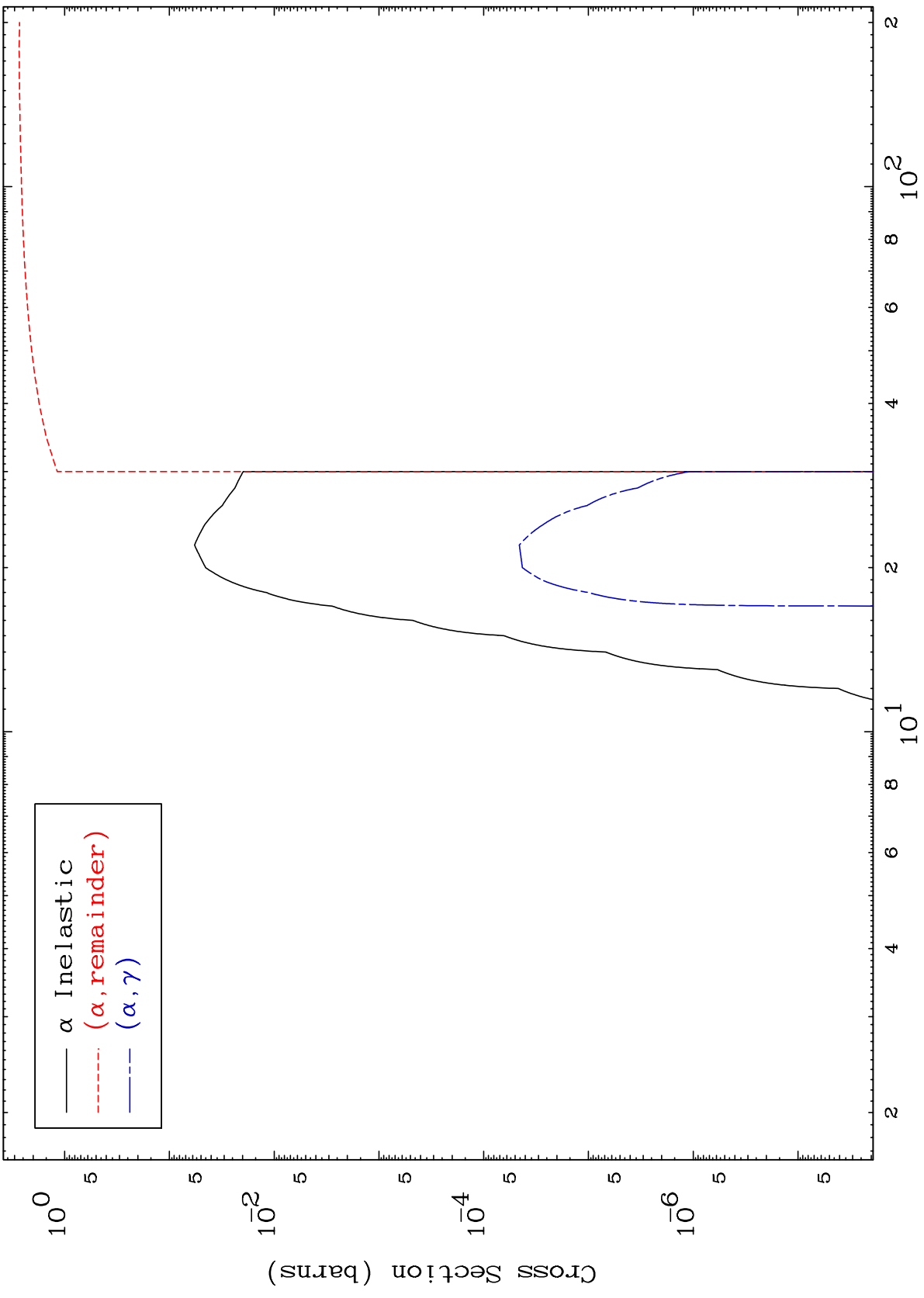
Press Mouse Button to Start

MAT 7725

$\alpha$  Major

77-Ir-191

0 Kelvin Cross Sections



$\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha$ ,  $\gamma$ )

1

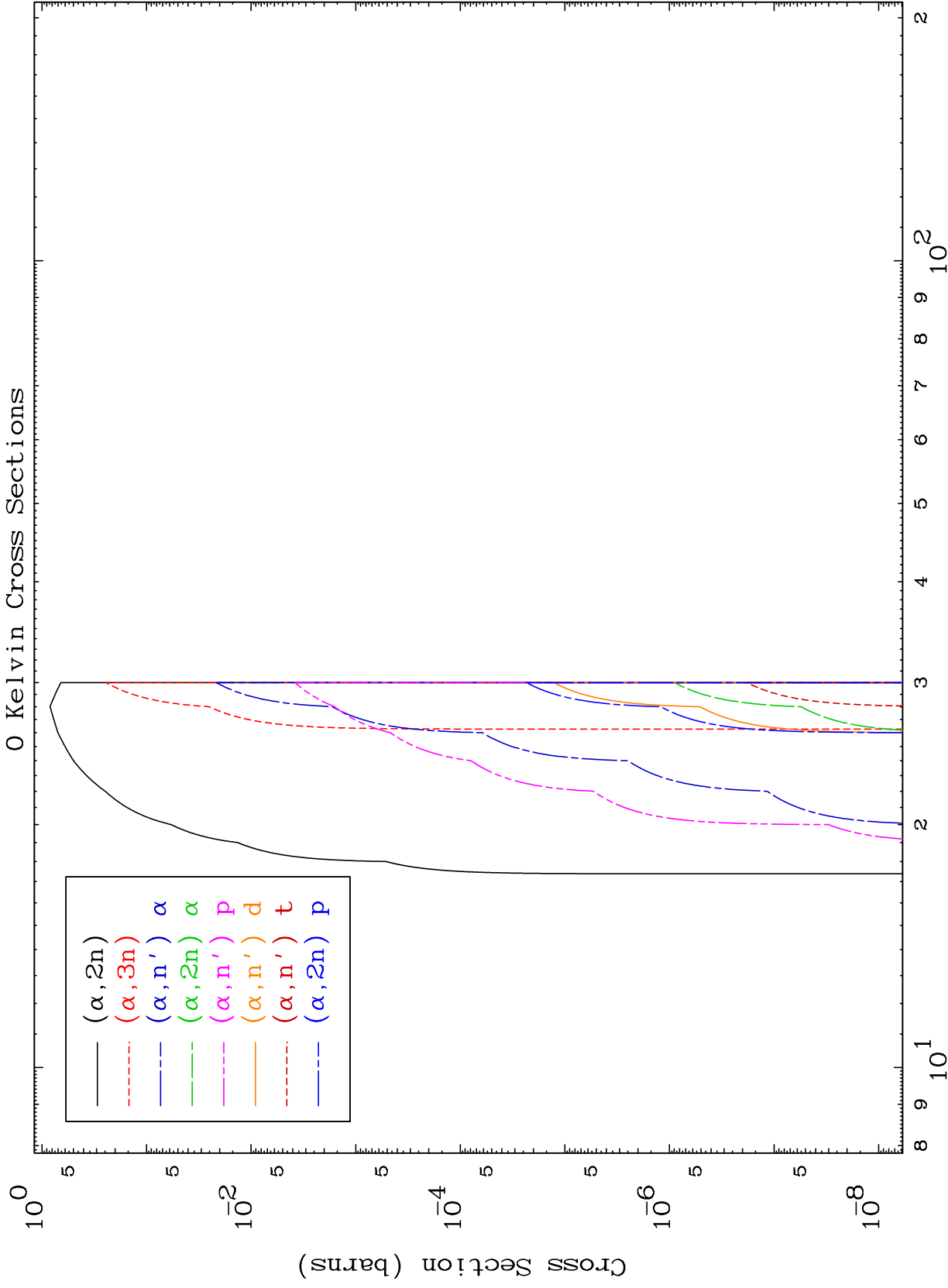
Incident Energy (MeV)

77-Ir-191

MAT 7725

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

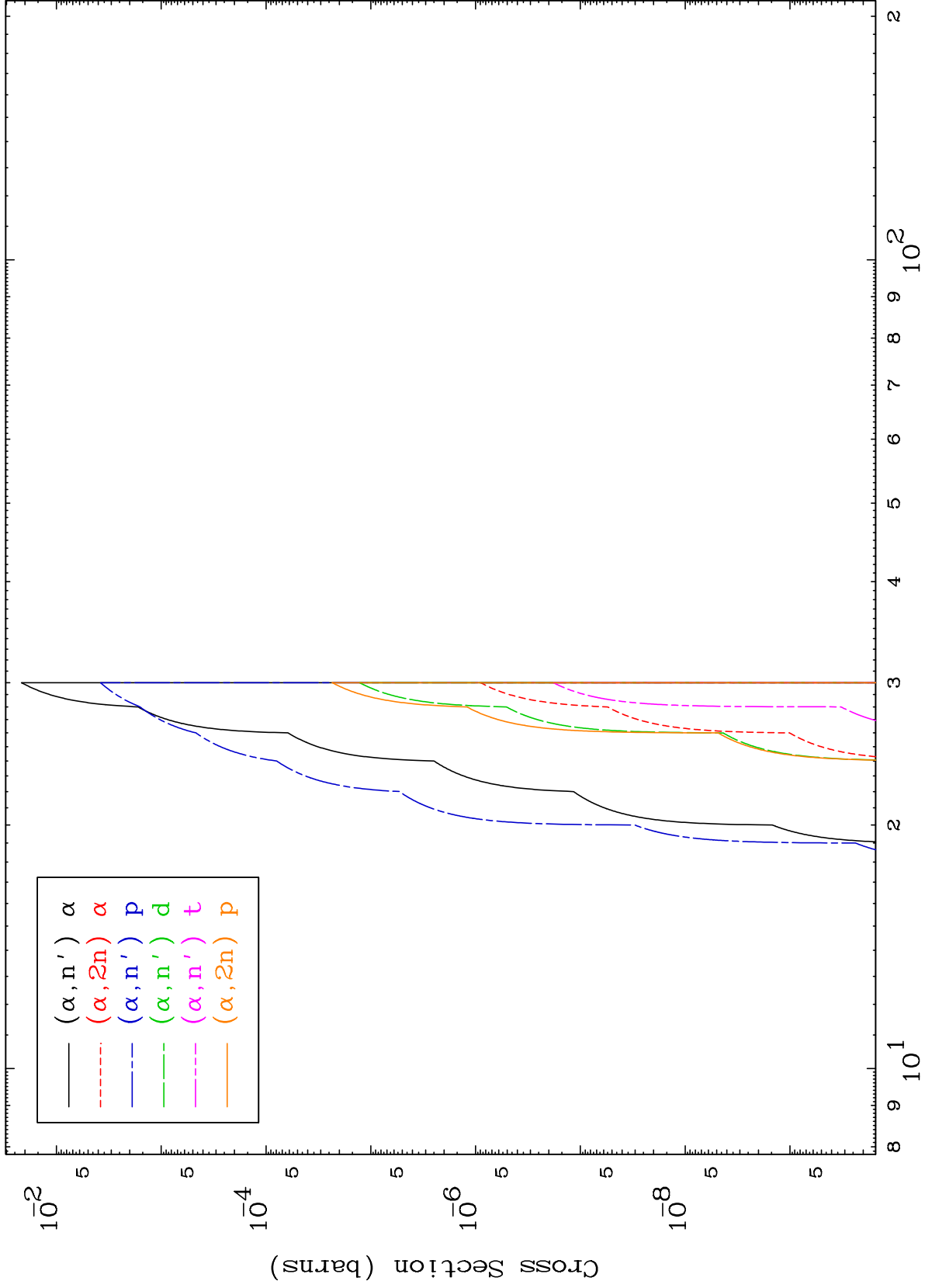
77-Ir-191

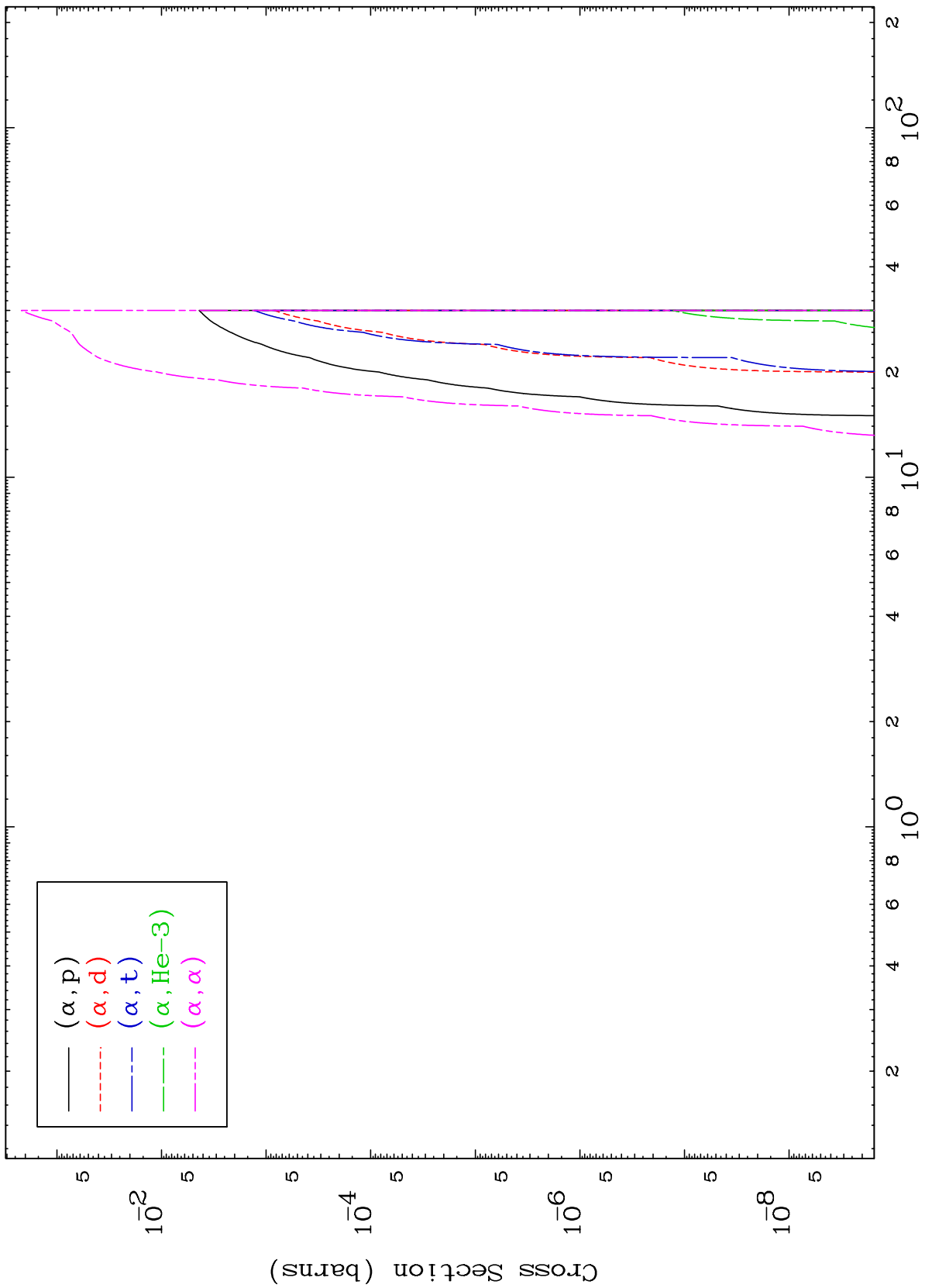


Incident Energy (MeV)

77-Ir-191

2



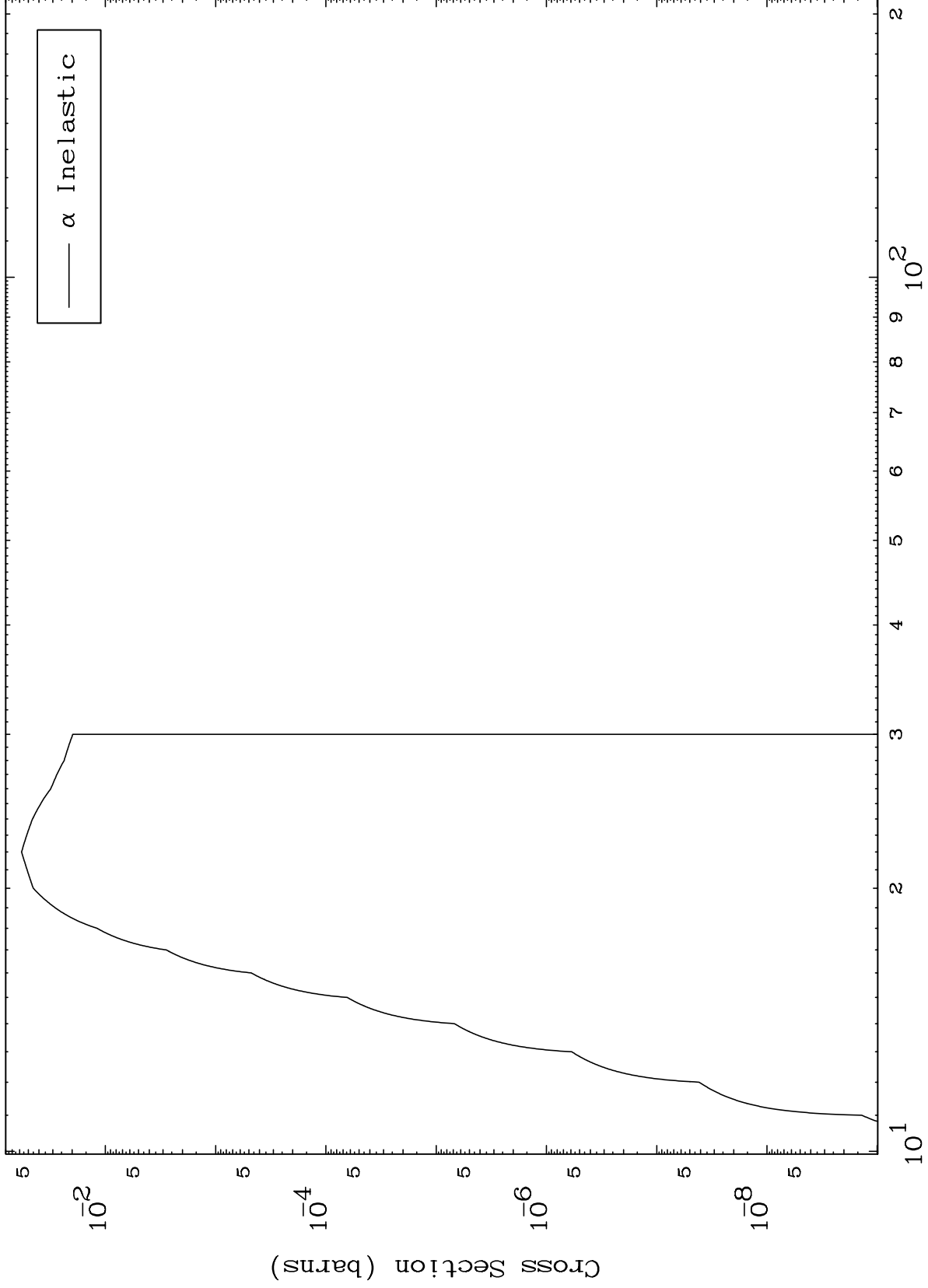


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( $\alpha, n'$ ) Level

77-Ir-191

0 Kelvin Cross Sections



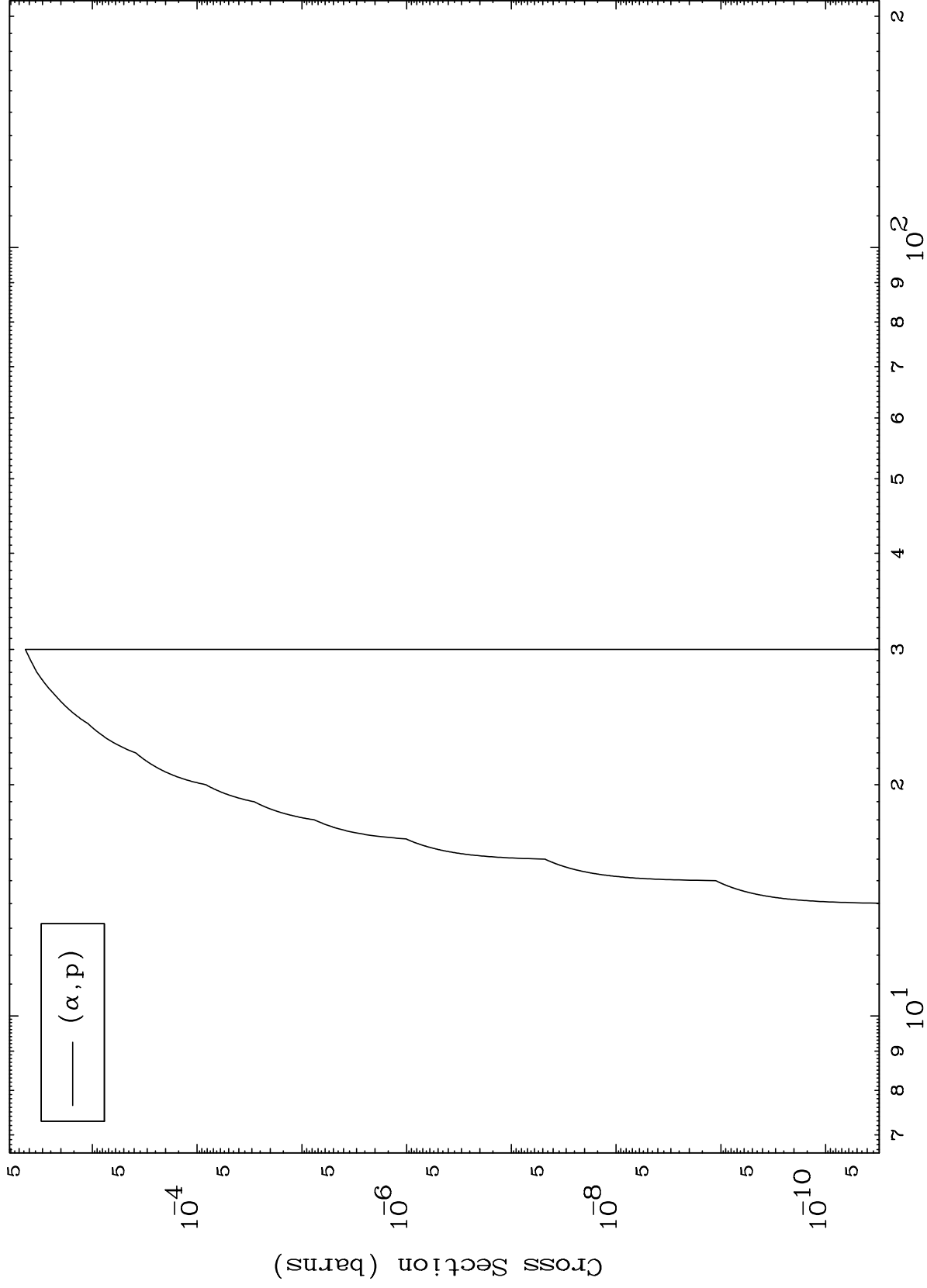
Incident Energy (MeV)

77-Ir-191

MAT 7725

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-191



6

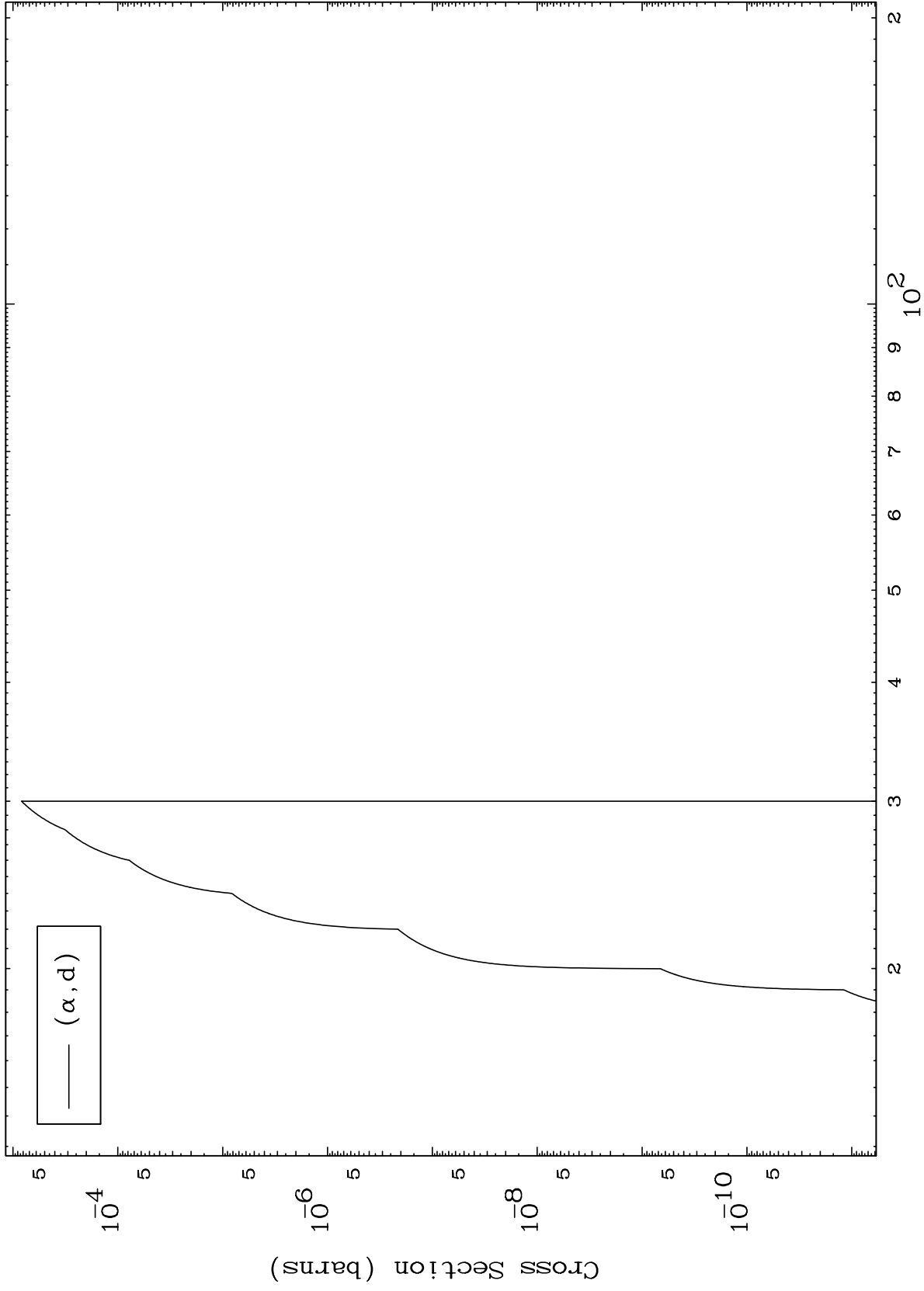
Incident Energy (MeV)

77-Ir-191

MAT 7725

77-Ir-191

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



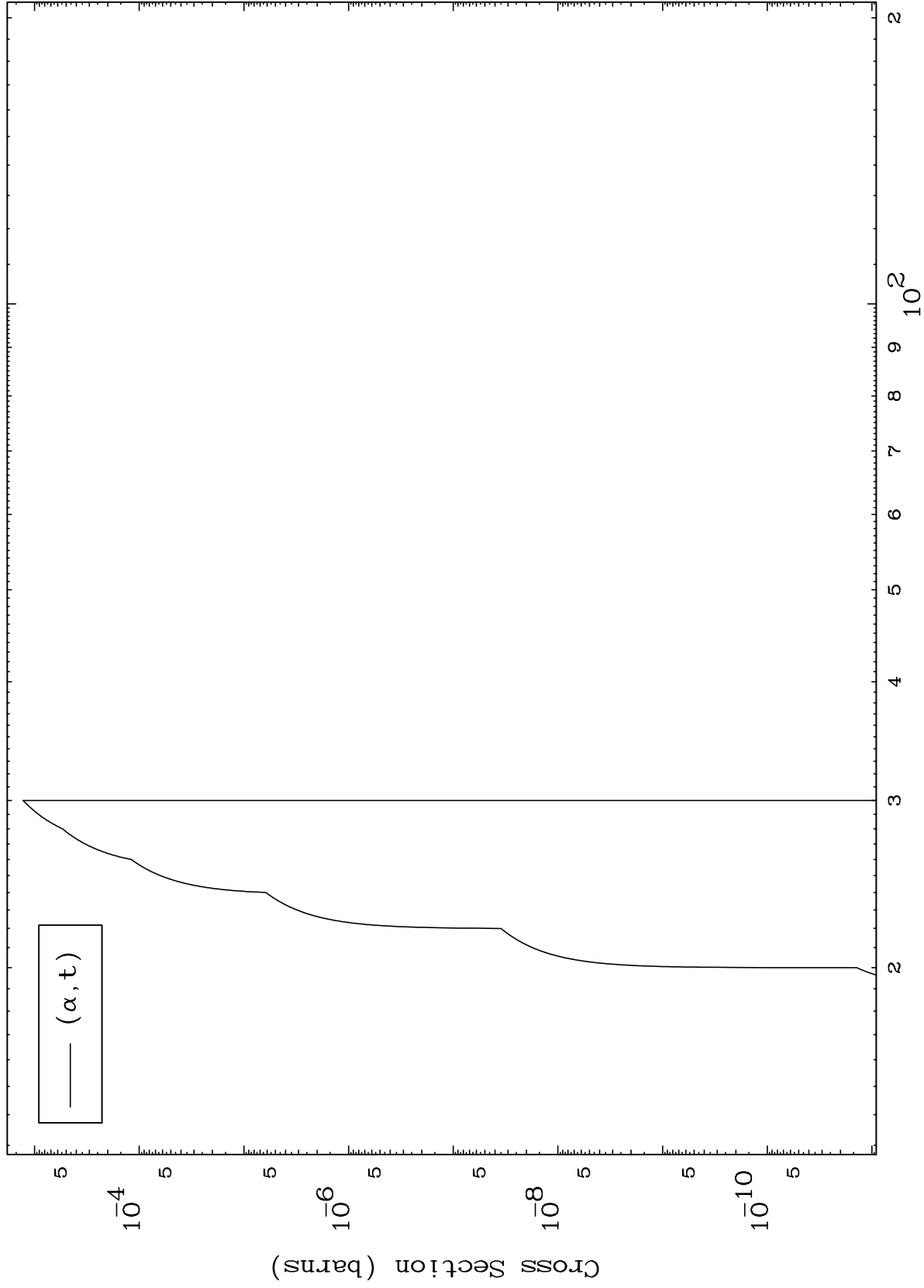
77-Ir-191

Incident Energy (MeV)

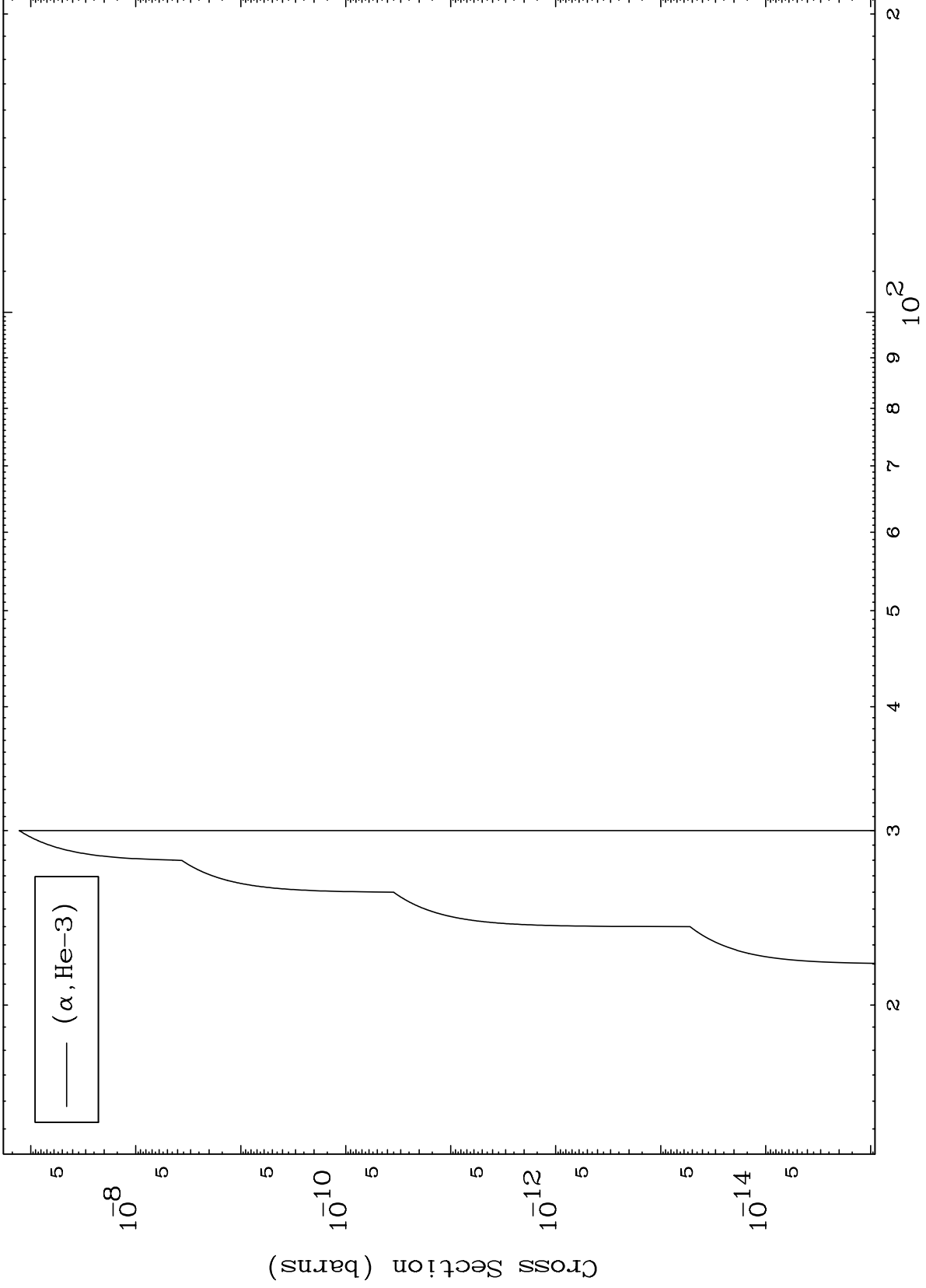
7



( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

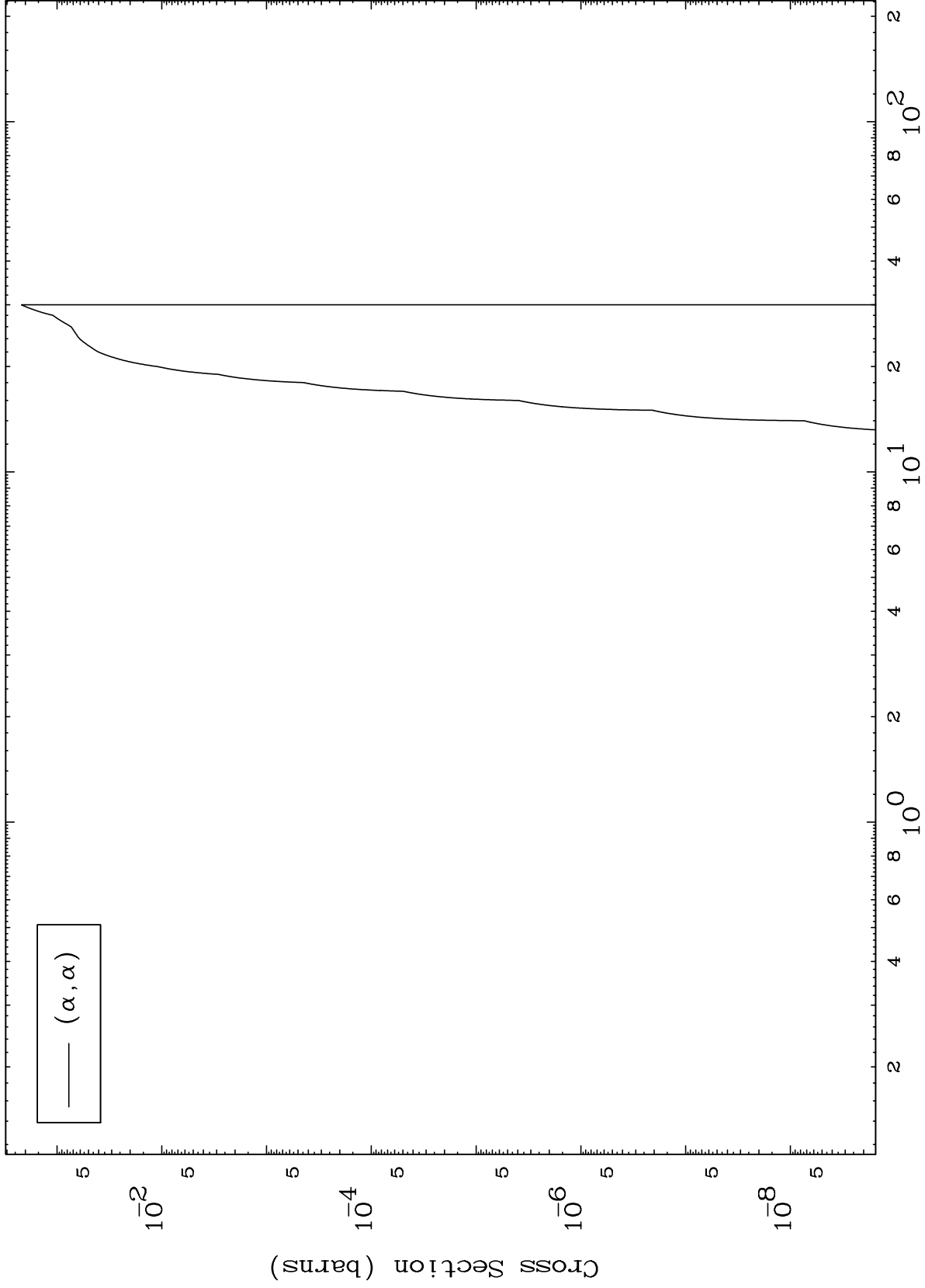


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( $\alpha, \alpha$ ) Levels

77-Ir-191

0 Kelvin Cross Sections



10

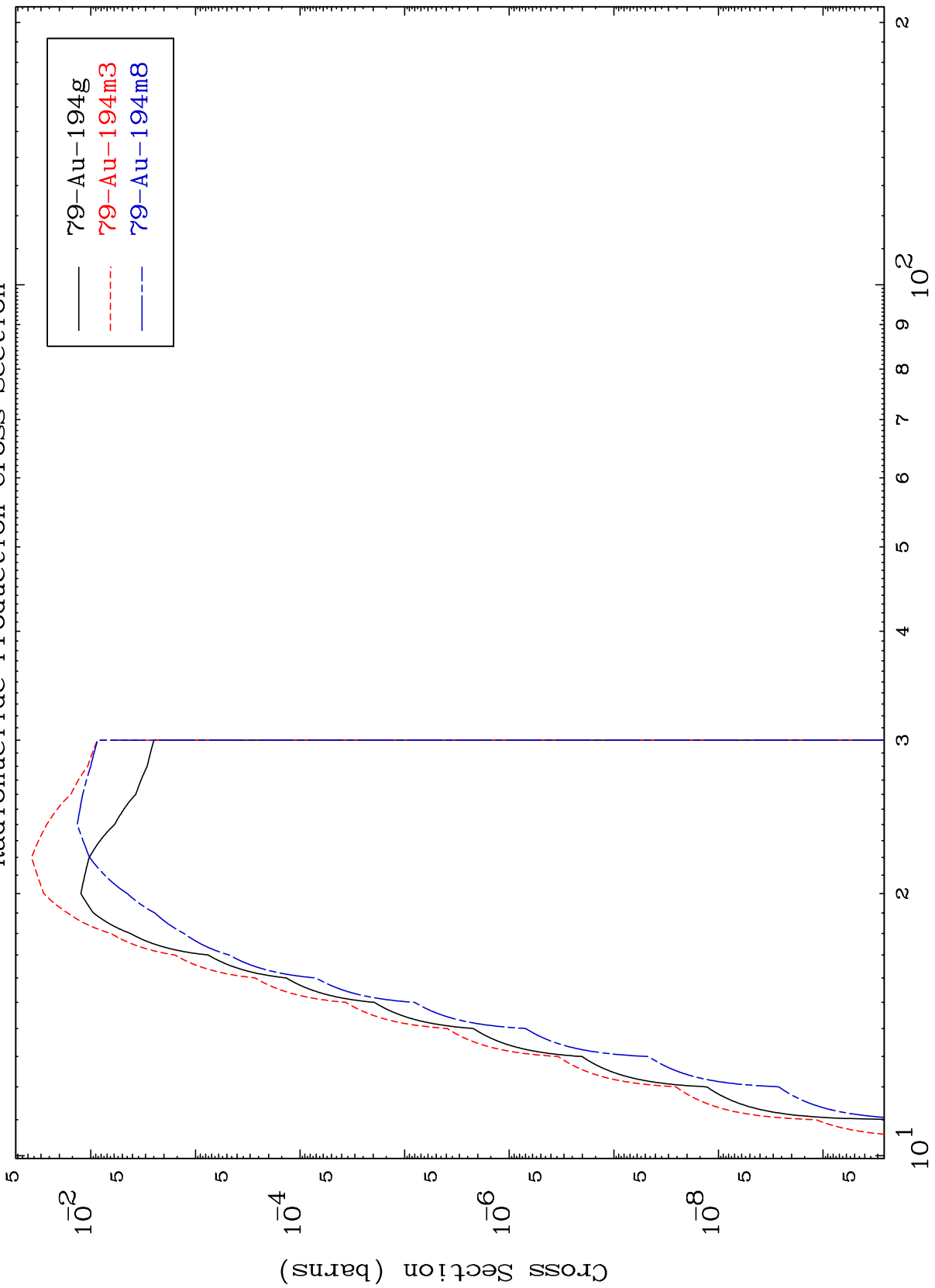
Incident Energy (MeV)

77-Ir-191

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77-Ir-191

Radionuclide Production Cross Section

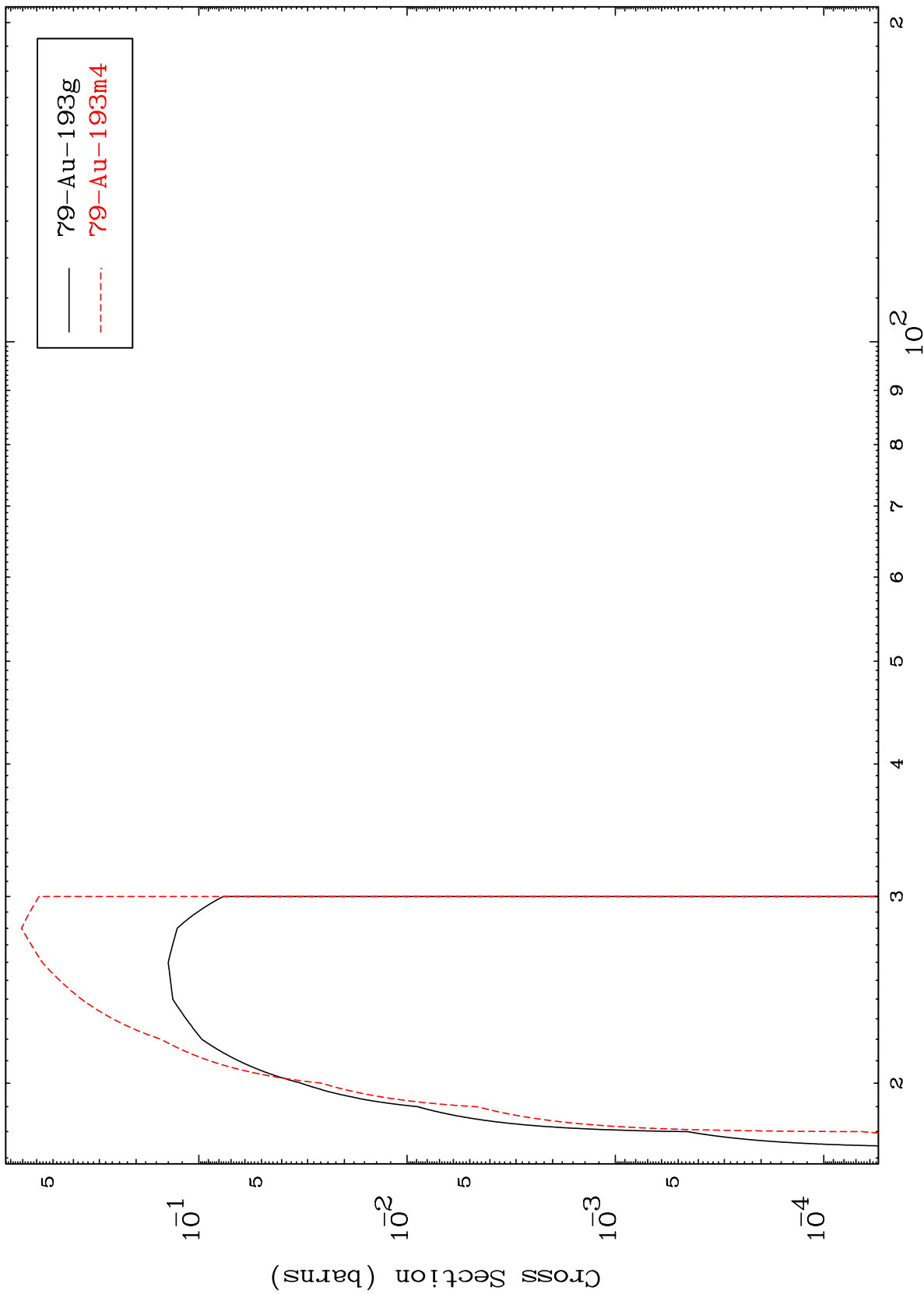


77-Ir-191

Incident Energy (MeV)

11

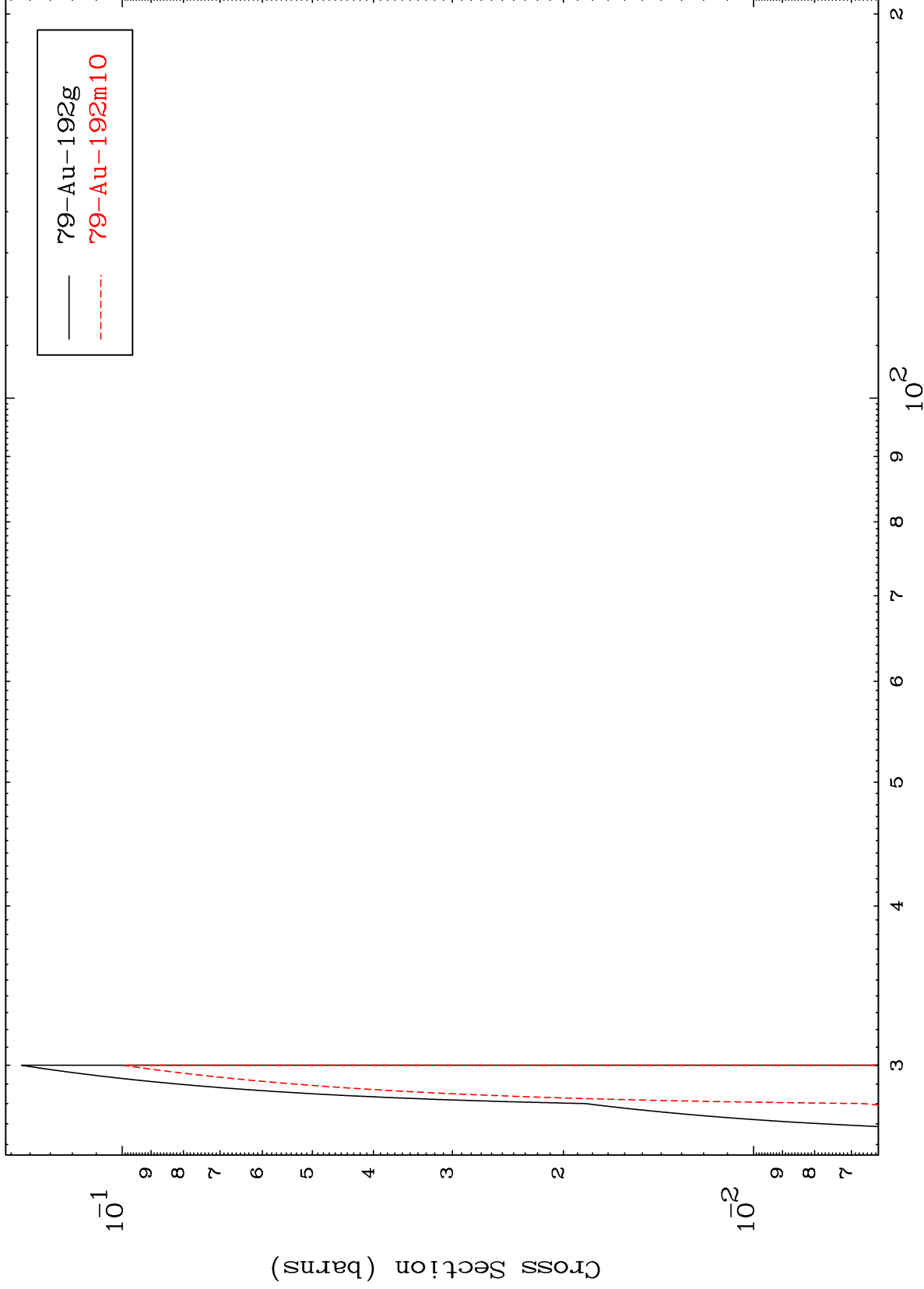
Radionuclide Production Cross Section  
( $\alpha, 2n$ )



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77-Ir-191

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



13

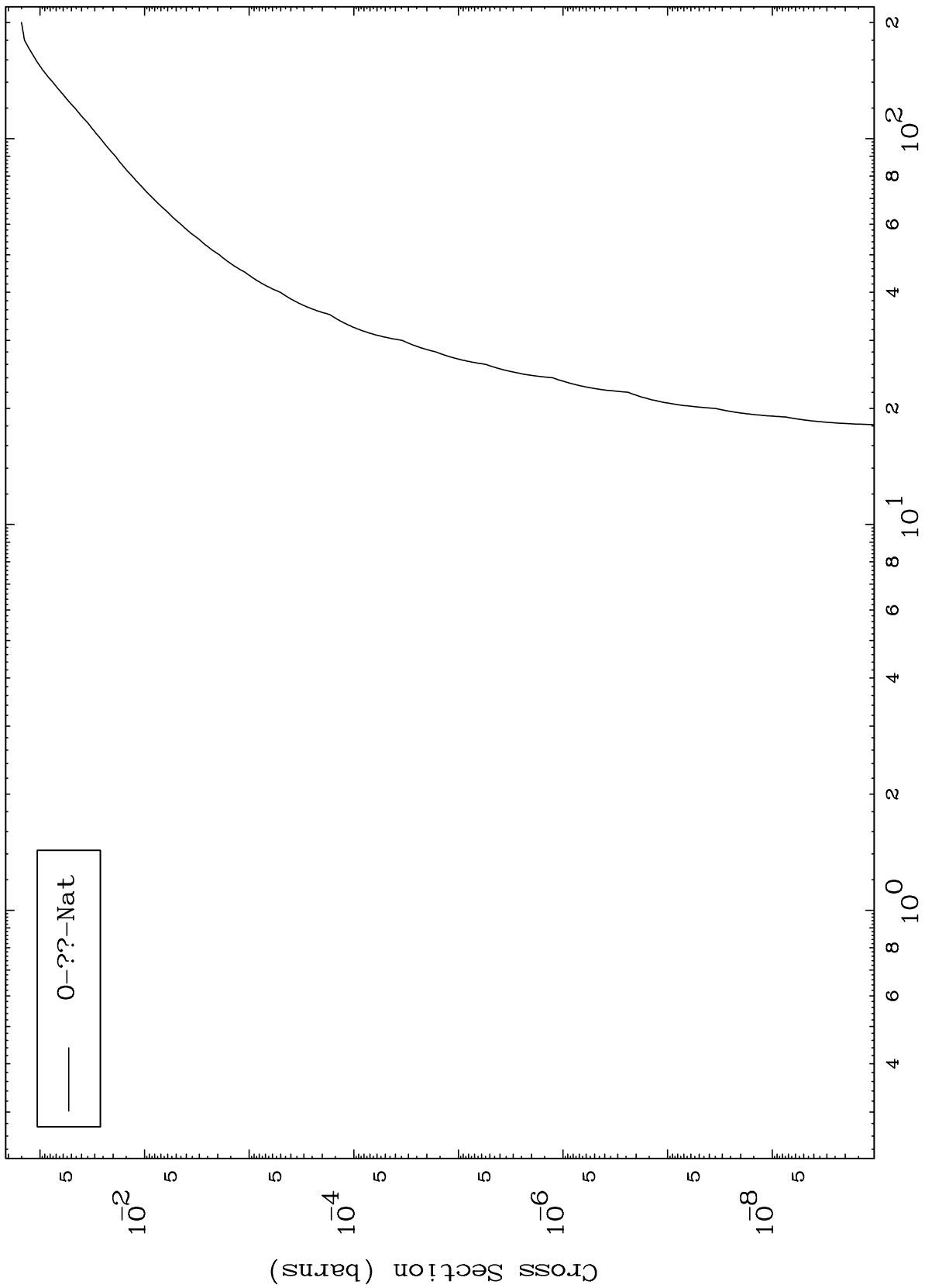
77-Ir-191

Incident Energy (MeV)

MAT 7725

<sup>77</sup>Ir-191

<sup>α</sup> Fission  
Radionuclide Production Cross Section



14

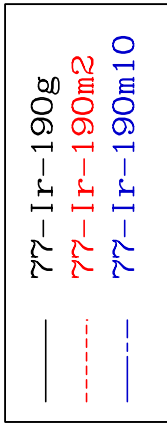
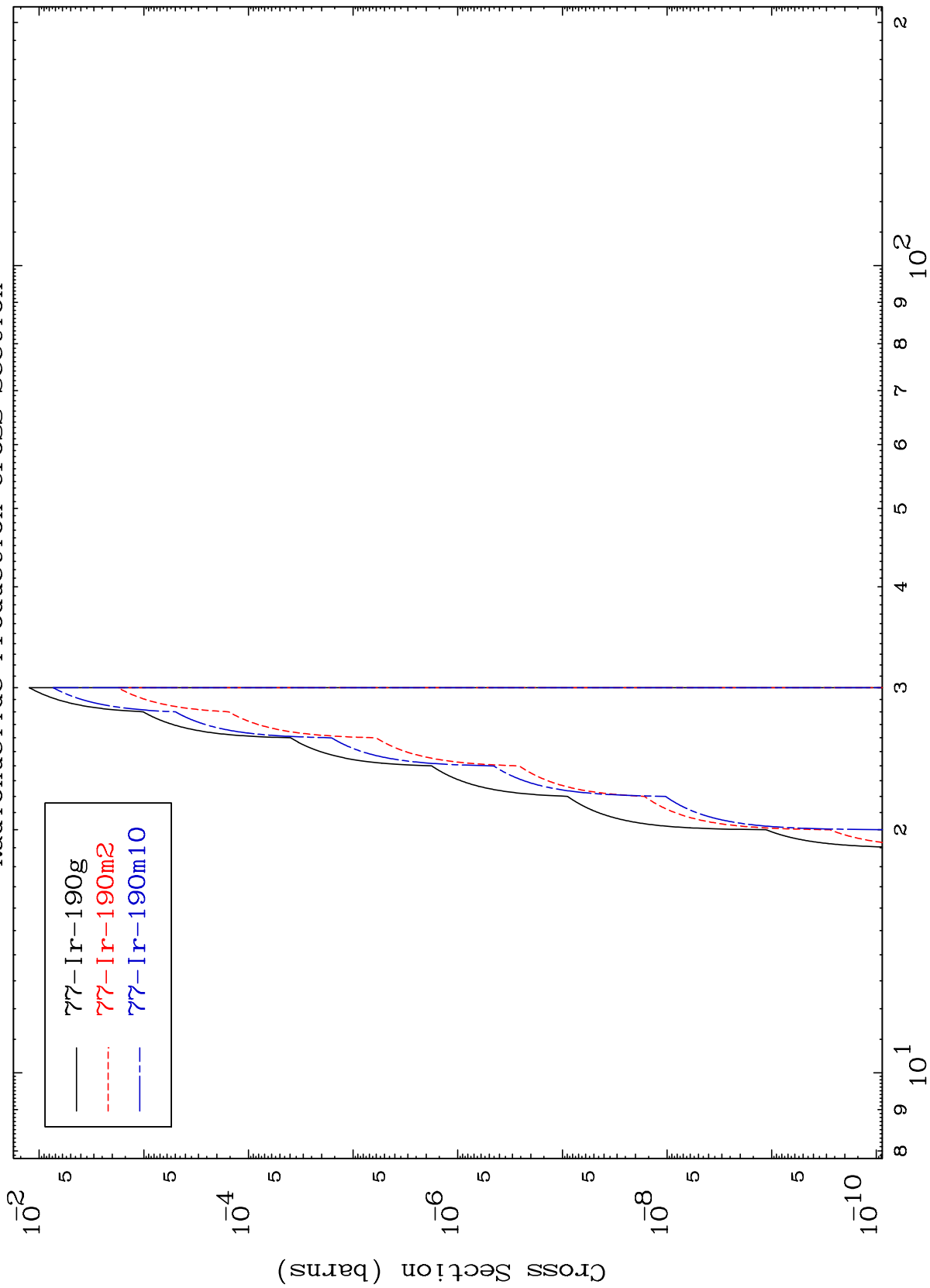
<sup>77</sup>Ir-191

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$(\alpha, n')$   $\alpha$

$^{77}\text{Ir-191}$

Radionuclide Production Cross Section



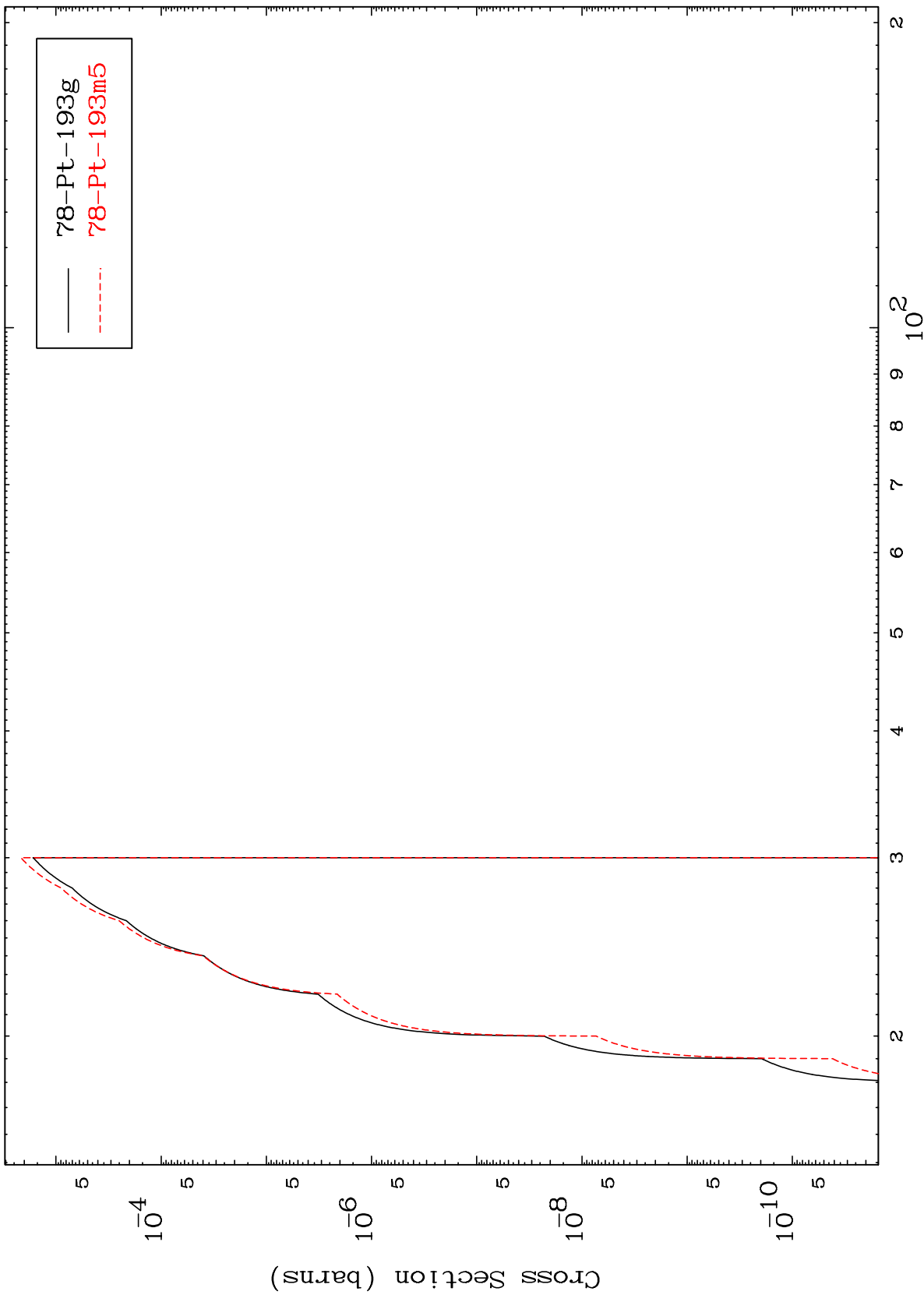
Incident Energy (MeV)

$^{77}\text{Ir-191}$

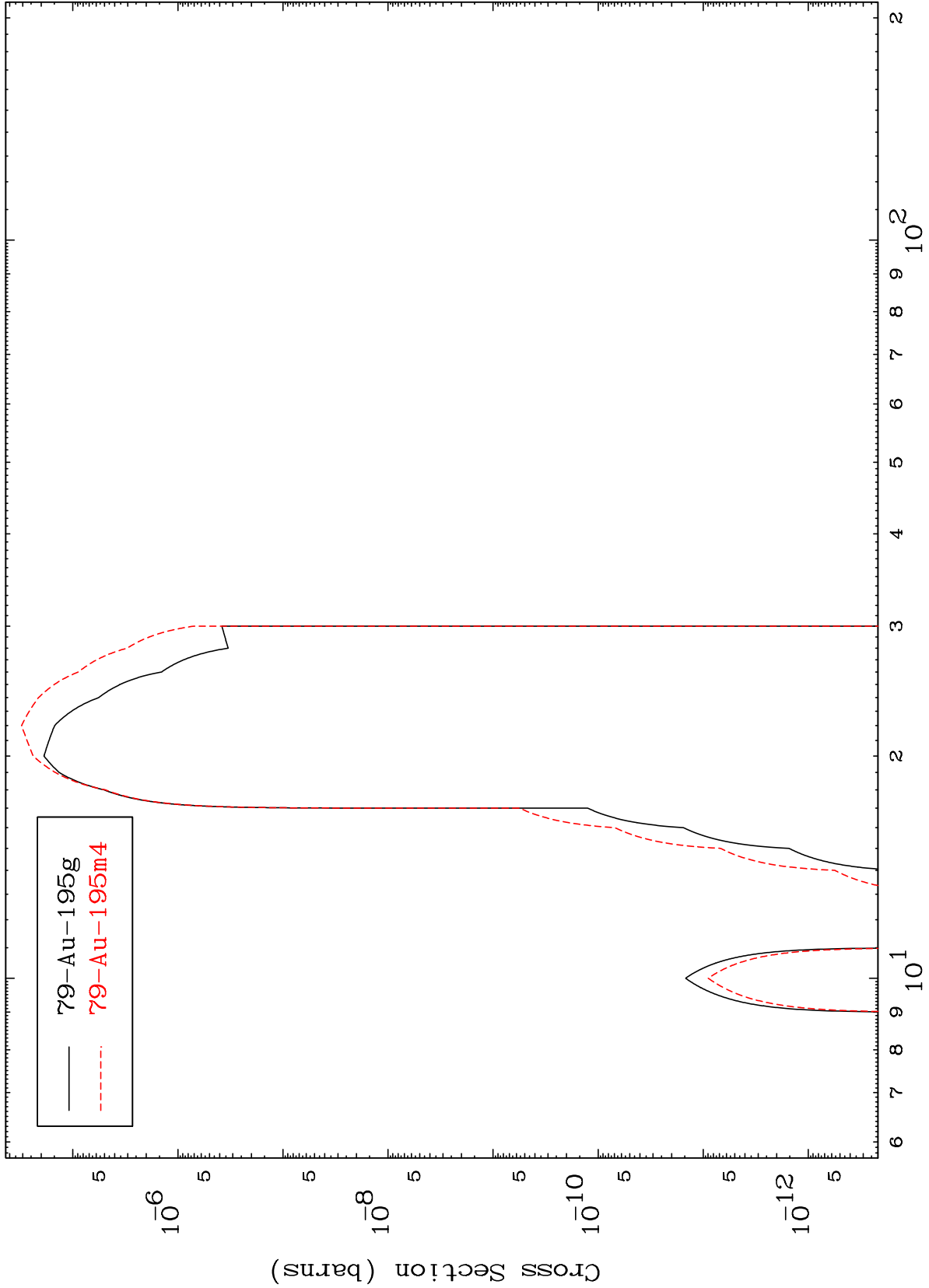
15



Radionuclide Production Cross Section



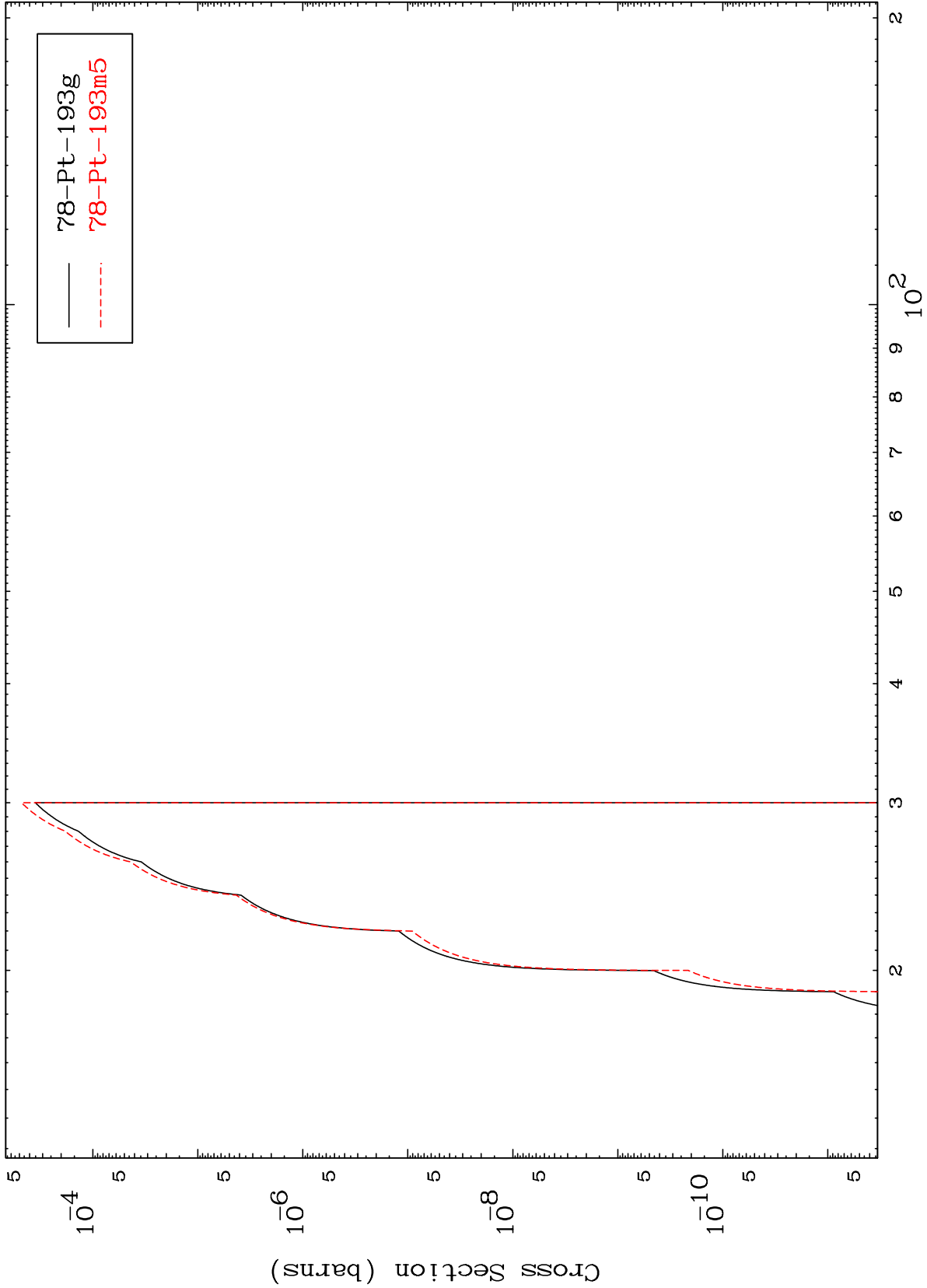
( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



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77-Ir-191

( $\alpha, d$ )  
Radionuclide Production Cross Section

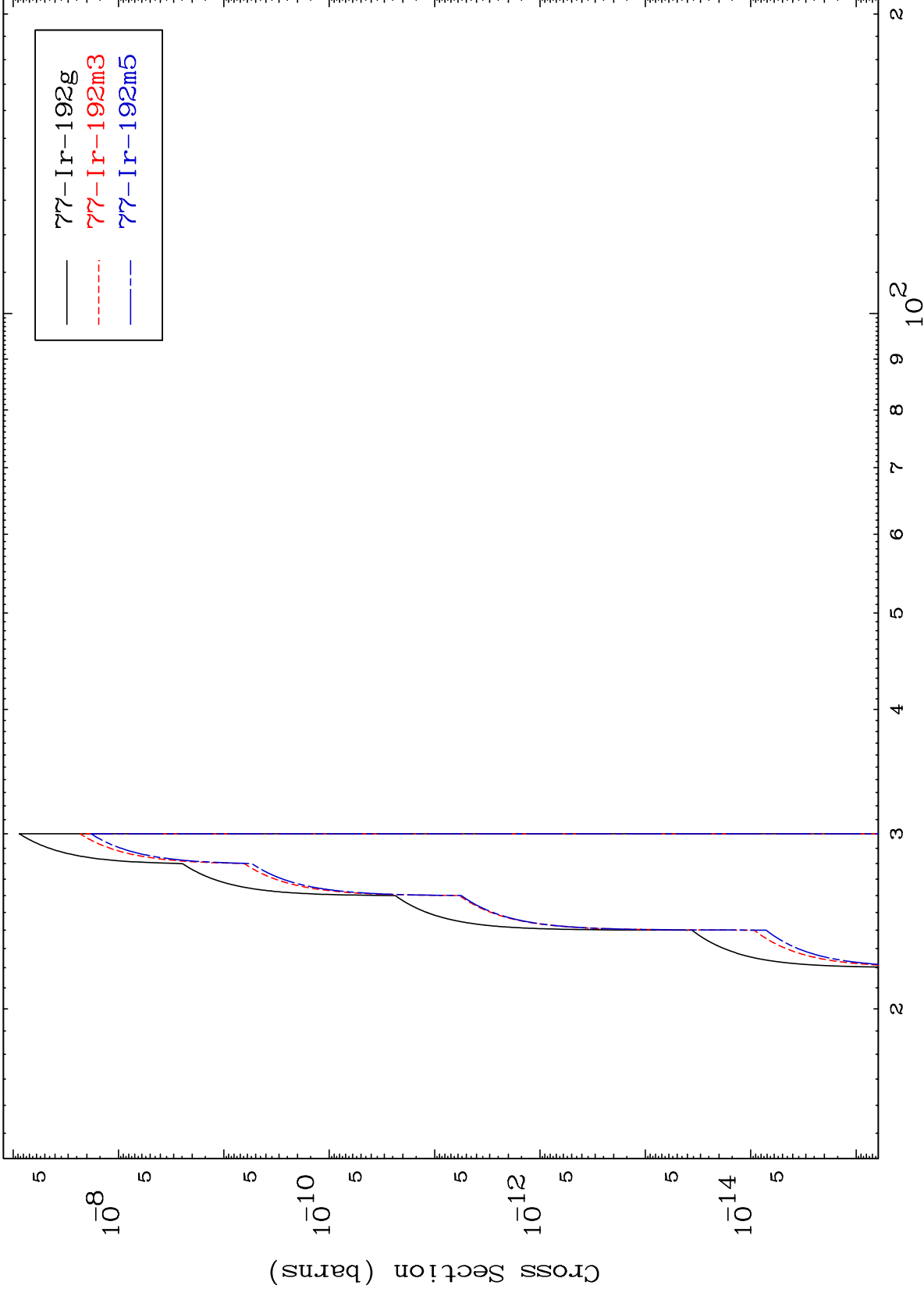


18

Incident Energy (MeV)

77-Ir-191

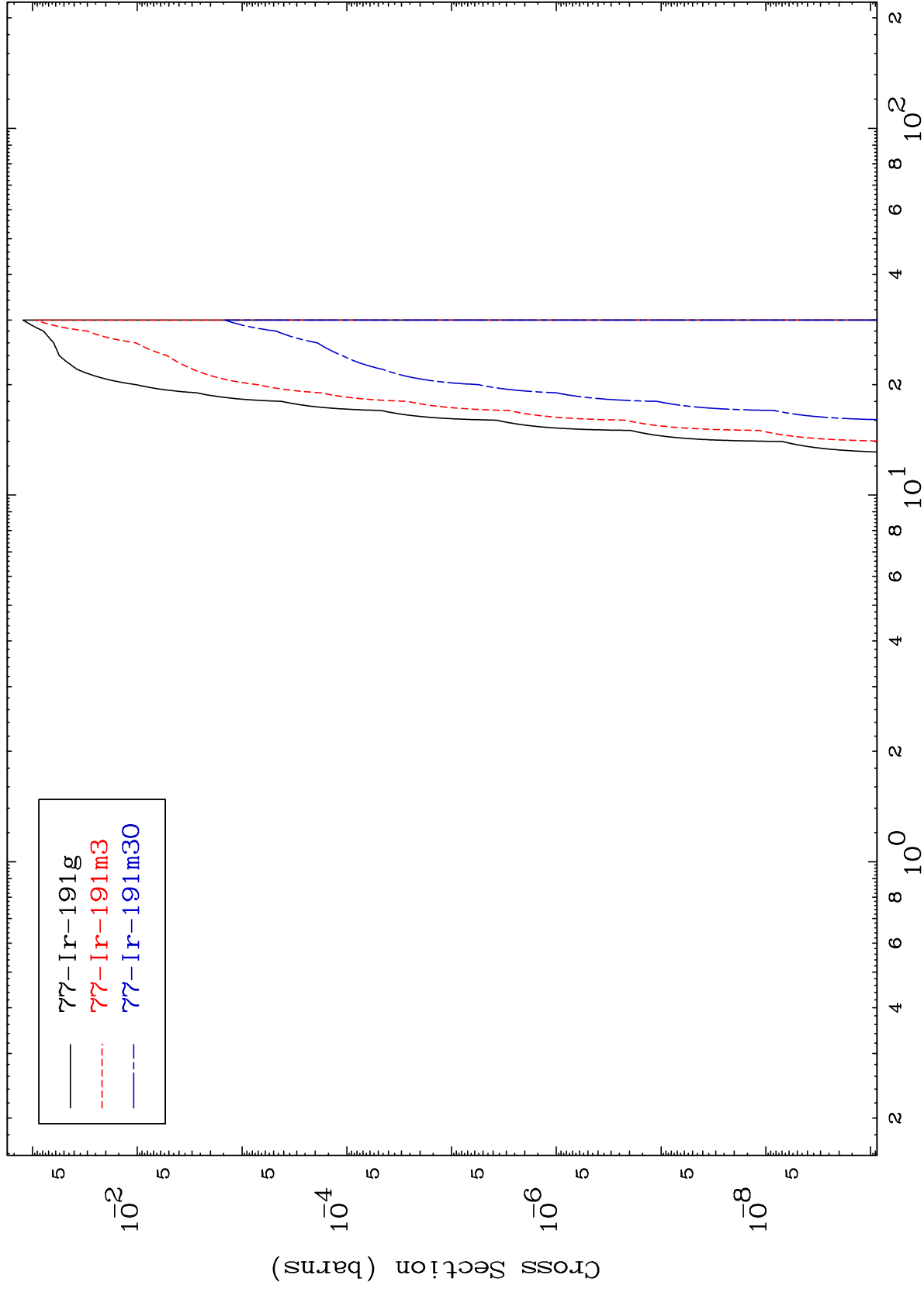
Radionuclide Production Cross Section



MAT 7725

<sup>77</sup>Ir-191

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



<sup>77</sup>Ir-191

Incident Energy (MeV)

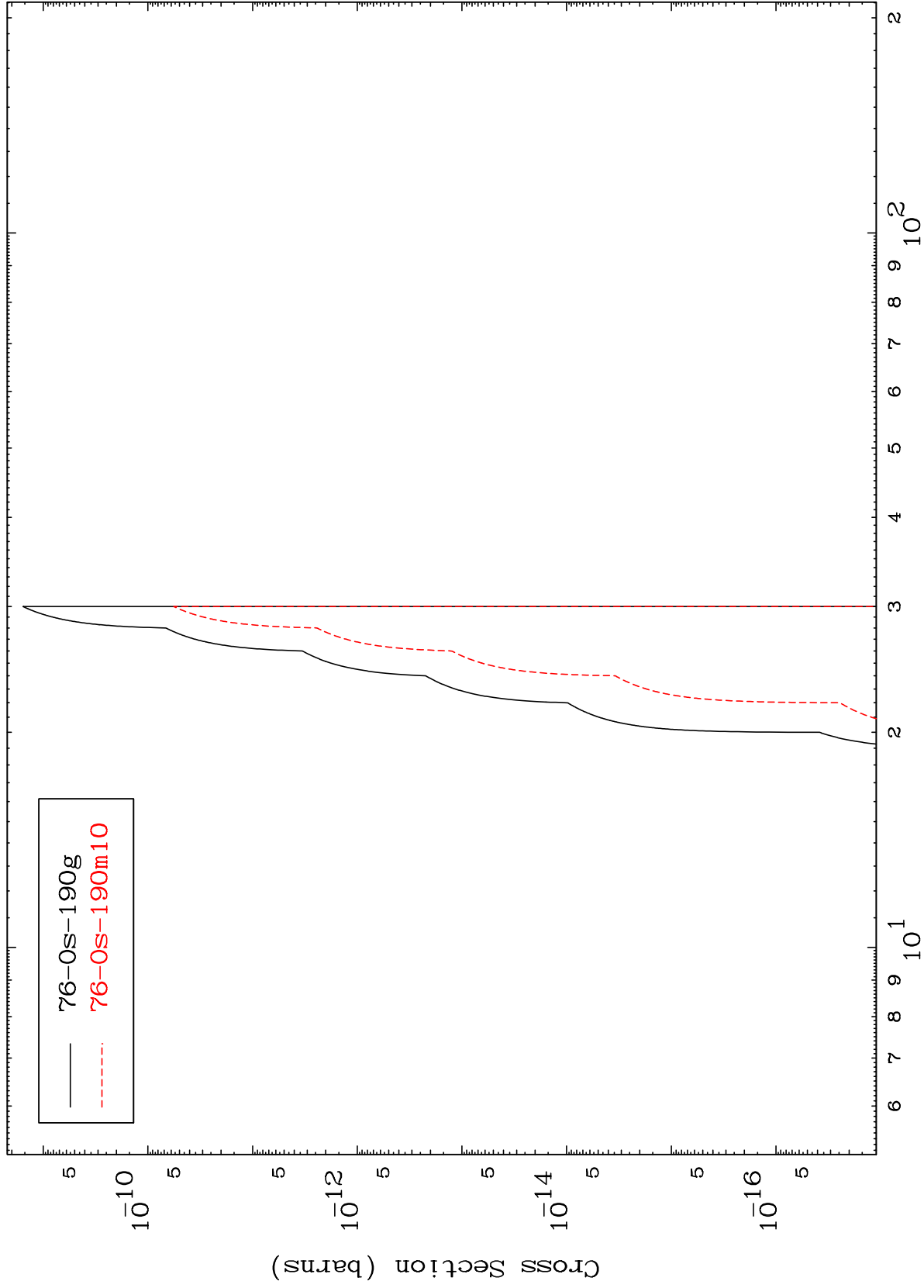
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$^{77}\text{Ir-191}$   $\alpha$

$^{77}\text{Ir-191}$

Radionuclide Production Cross Section



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Incident Energy (MeV)

$^{77}\text{Ir-191}$