

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

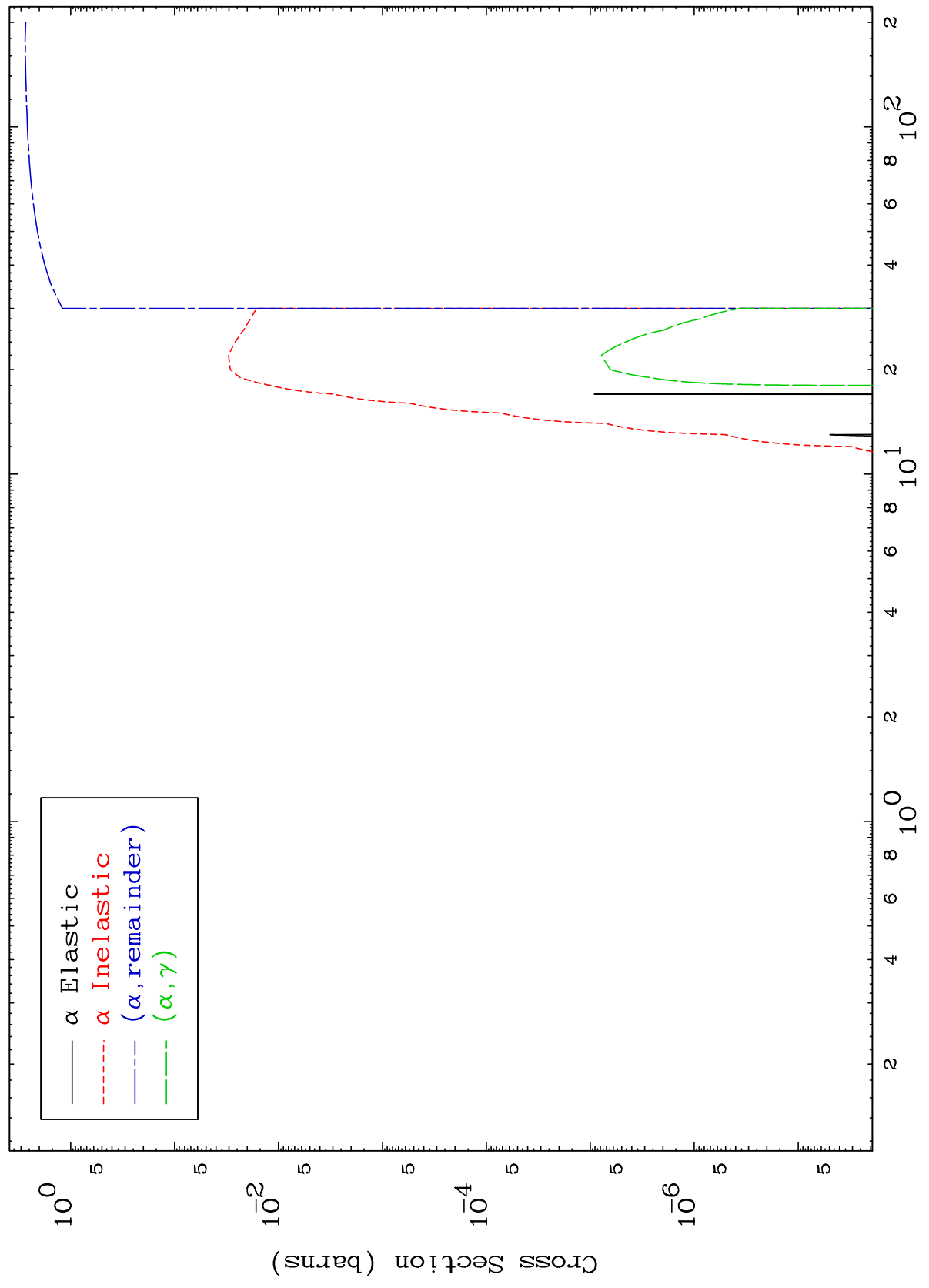
Press Mouse Button to Start

MAT 7735

0 Kelvin

Major Cross Sections

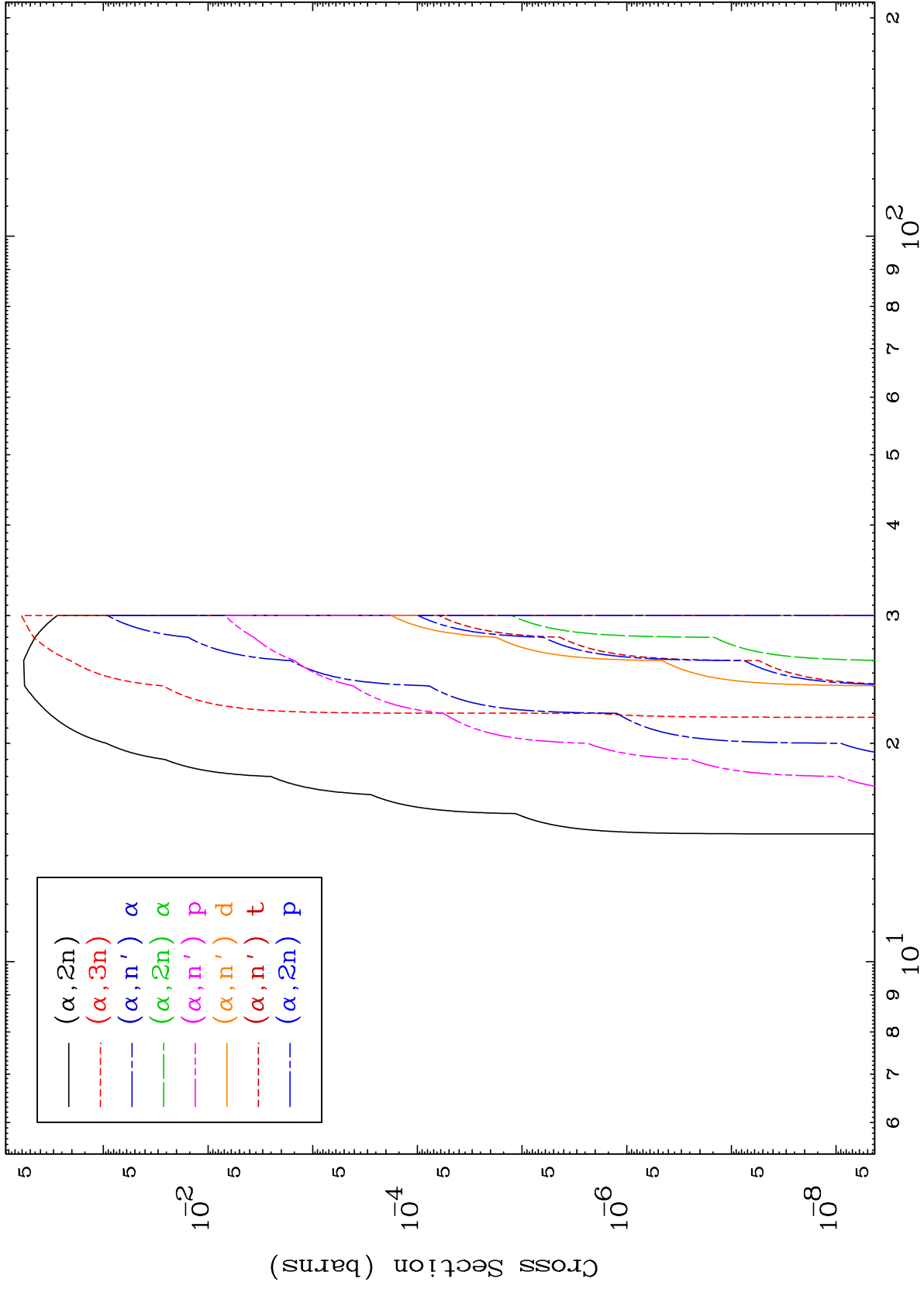
77-Ir-194



MAT 7735

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

77-Ir-194



2

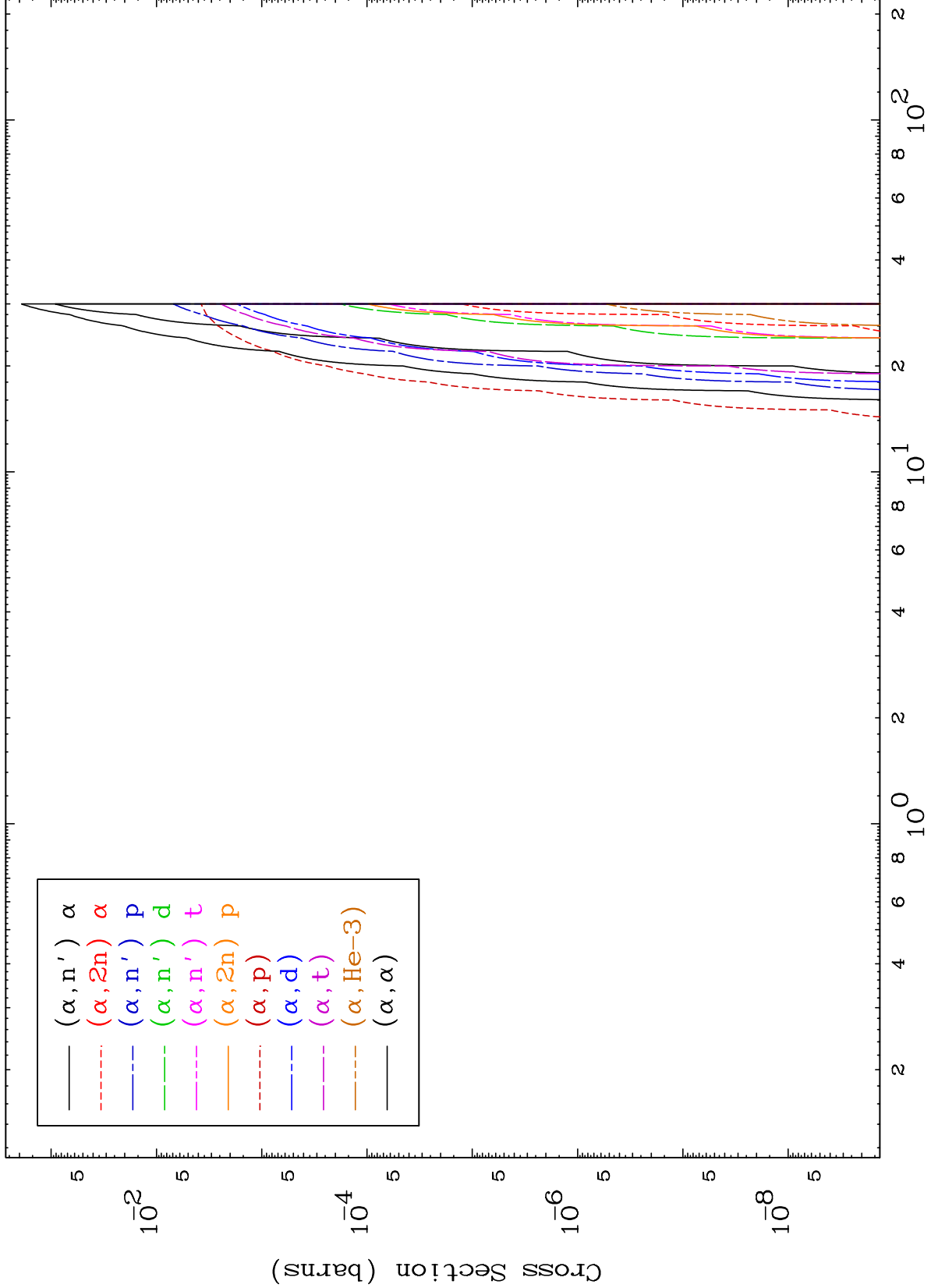
Incident Energy (MeV)

77-Ir-194

MAT 7735

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

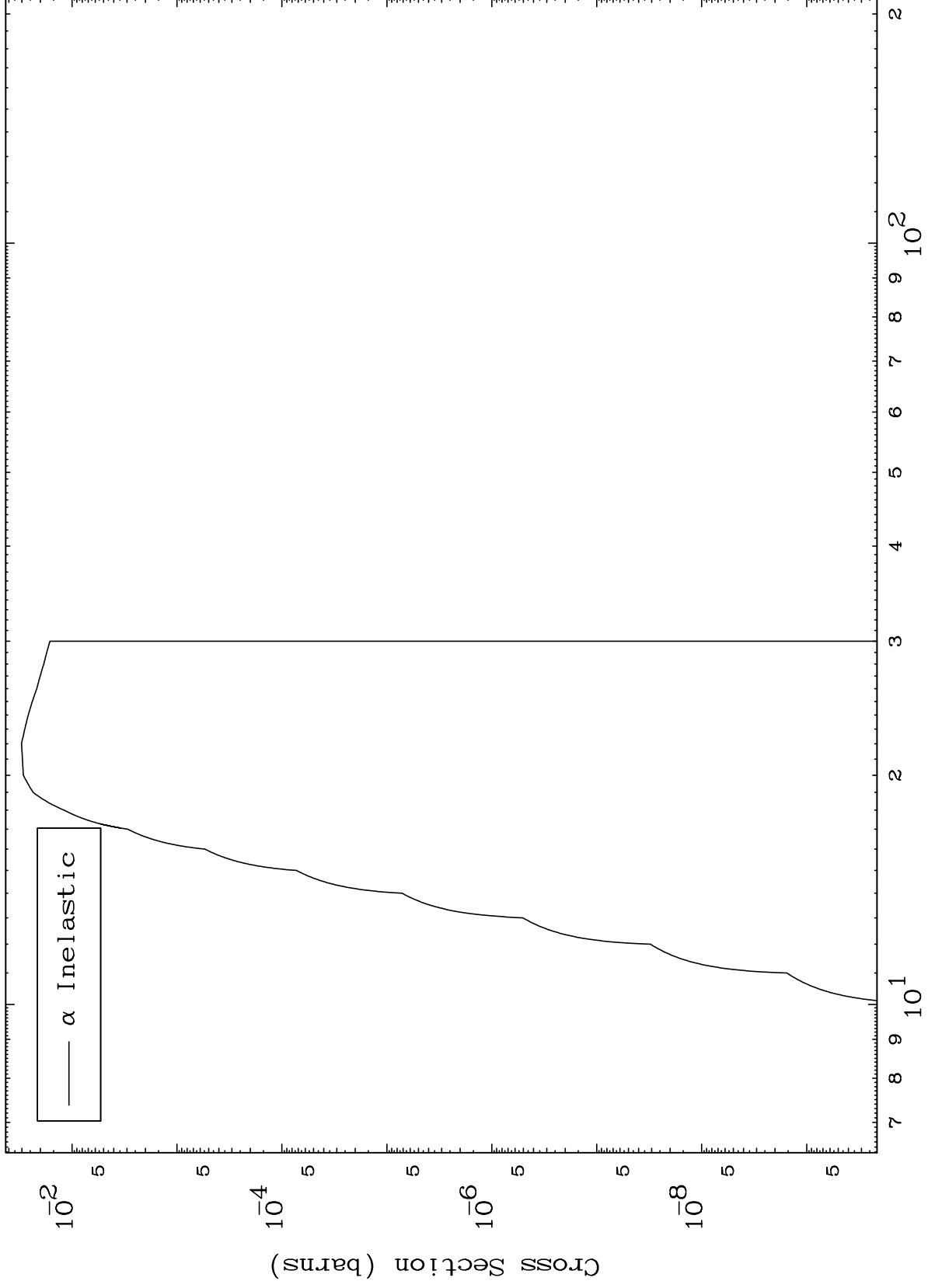
77-Ir-194



MAT 7735

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

77-Ir-194



4

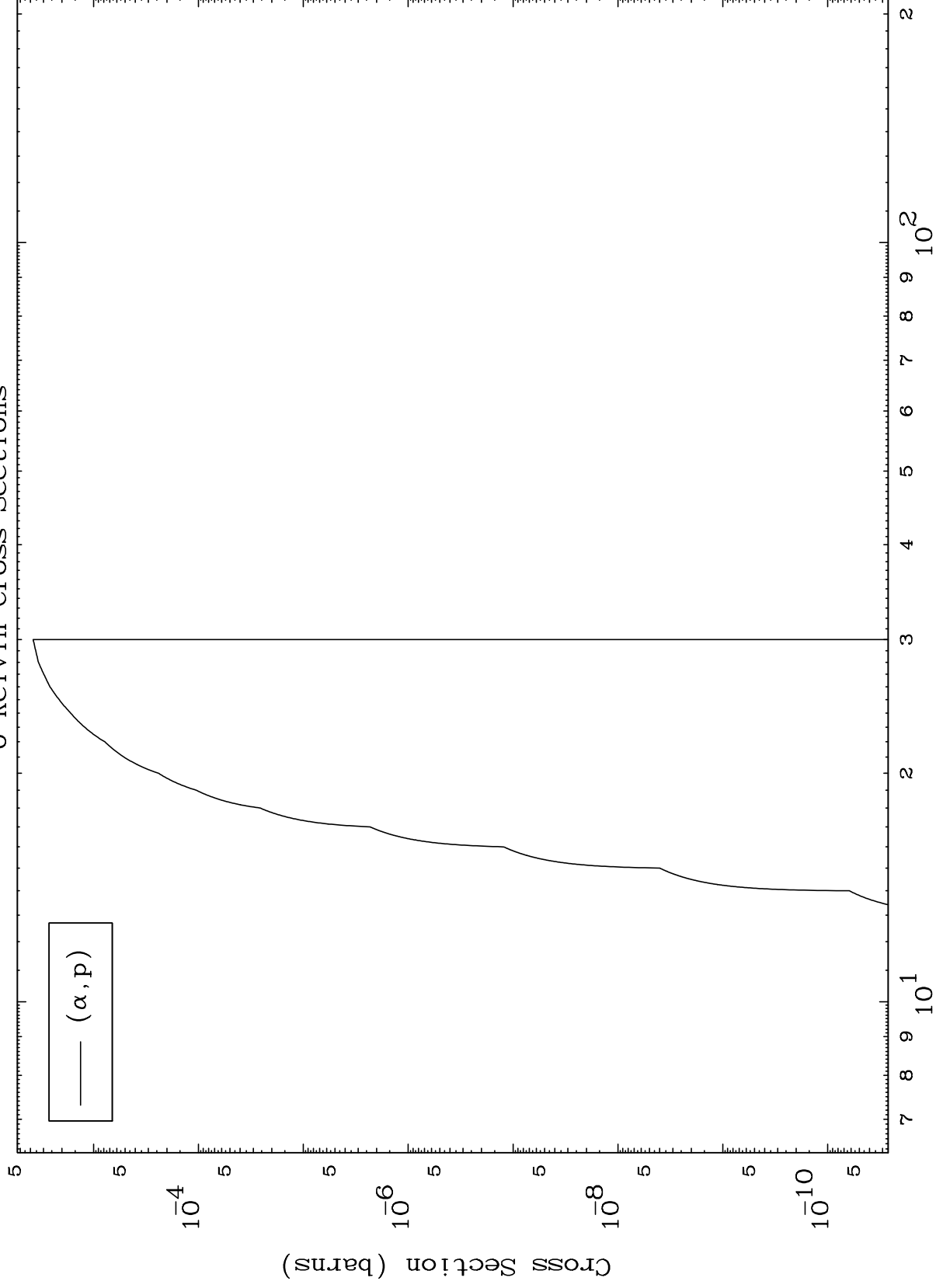
Incident Energy (MeV)

77-Ir-194

MAT 7735

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194



5

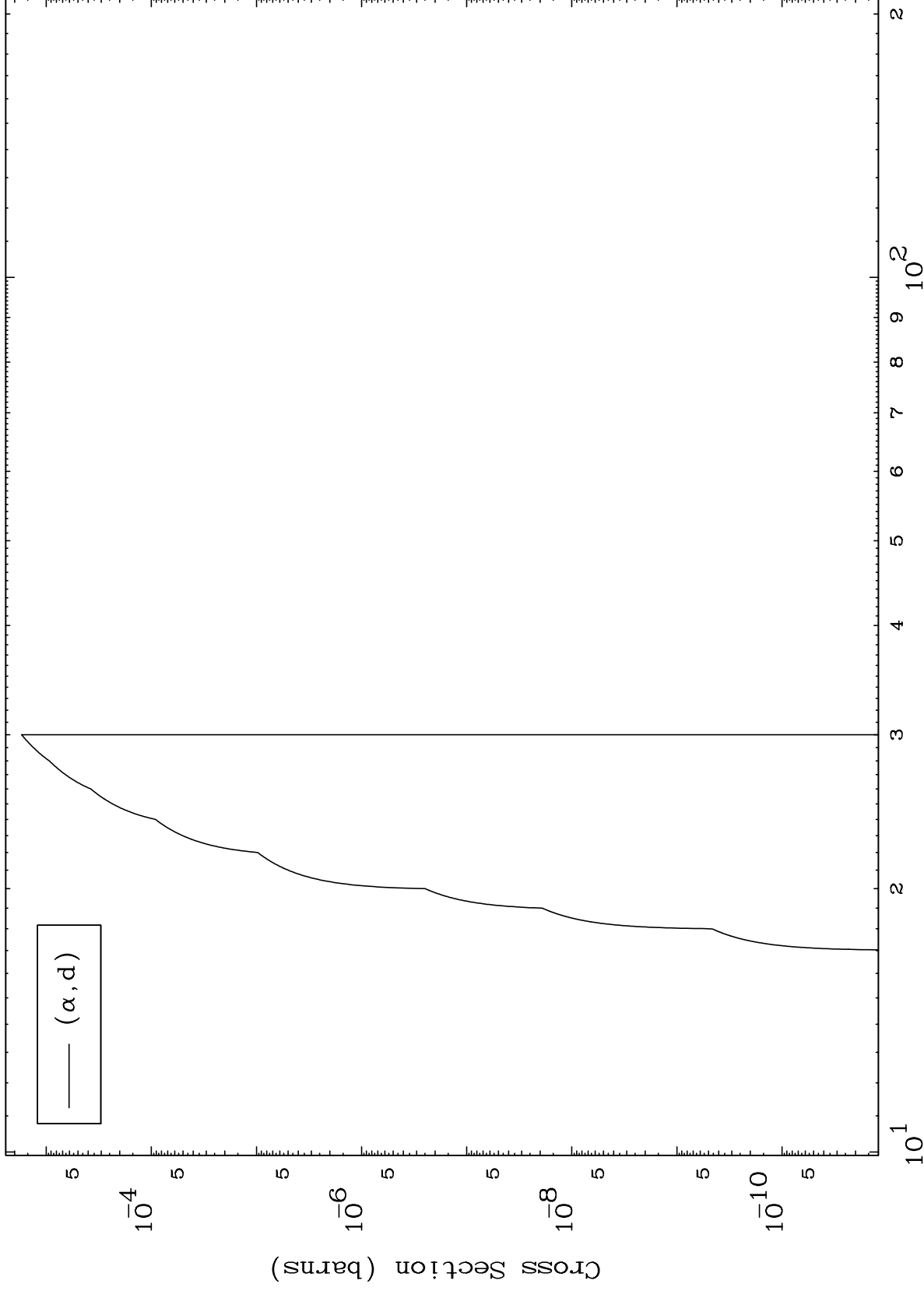
Incident Energy (MeV)

77-Ir-194

MAT 7735

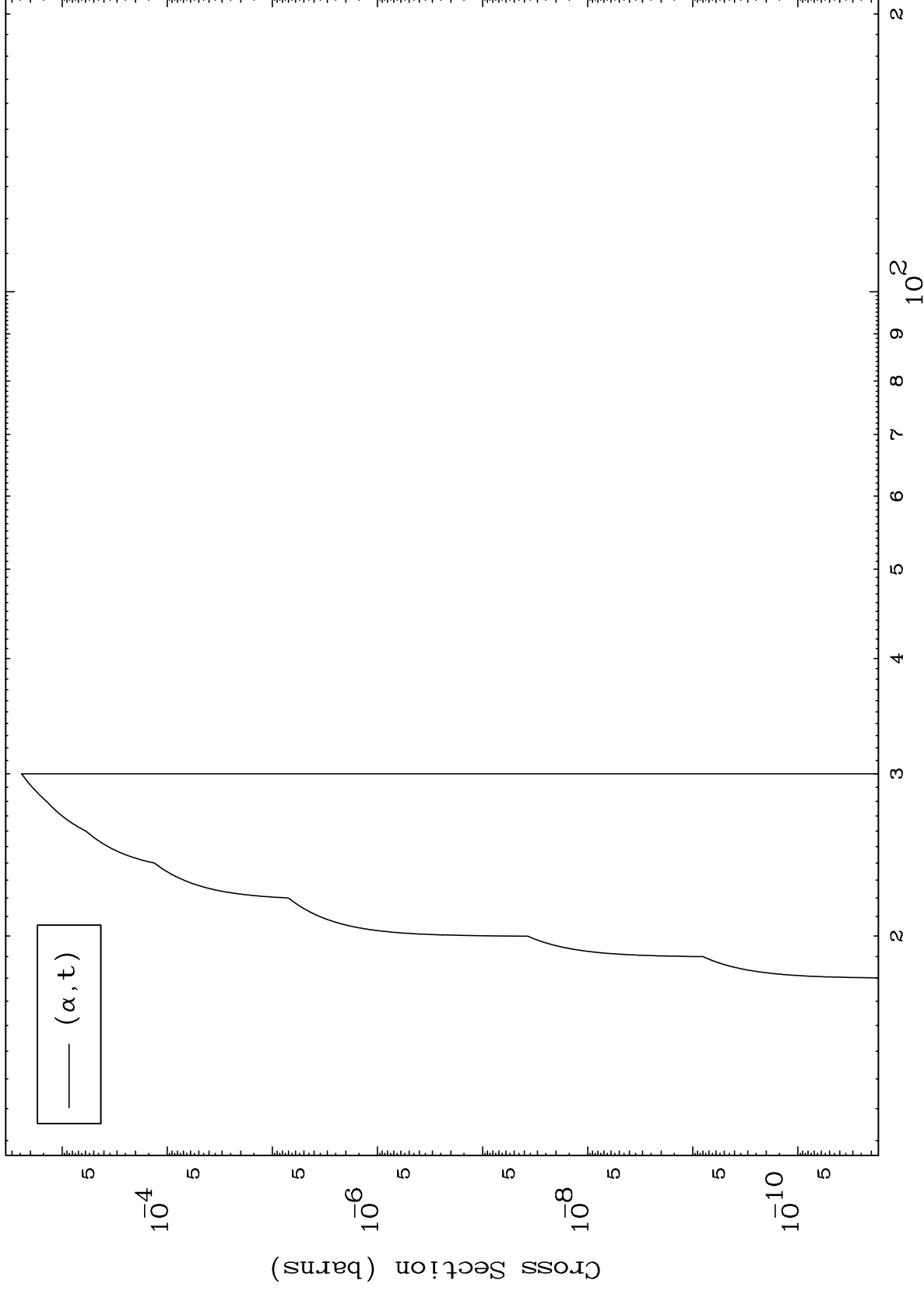
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-194

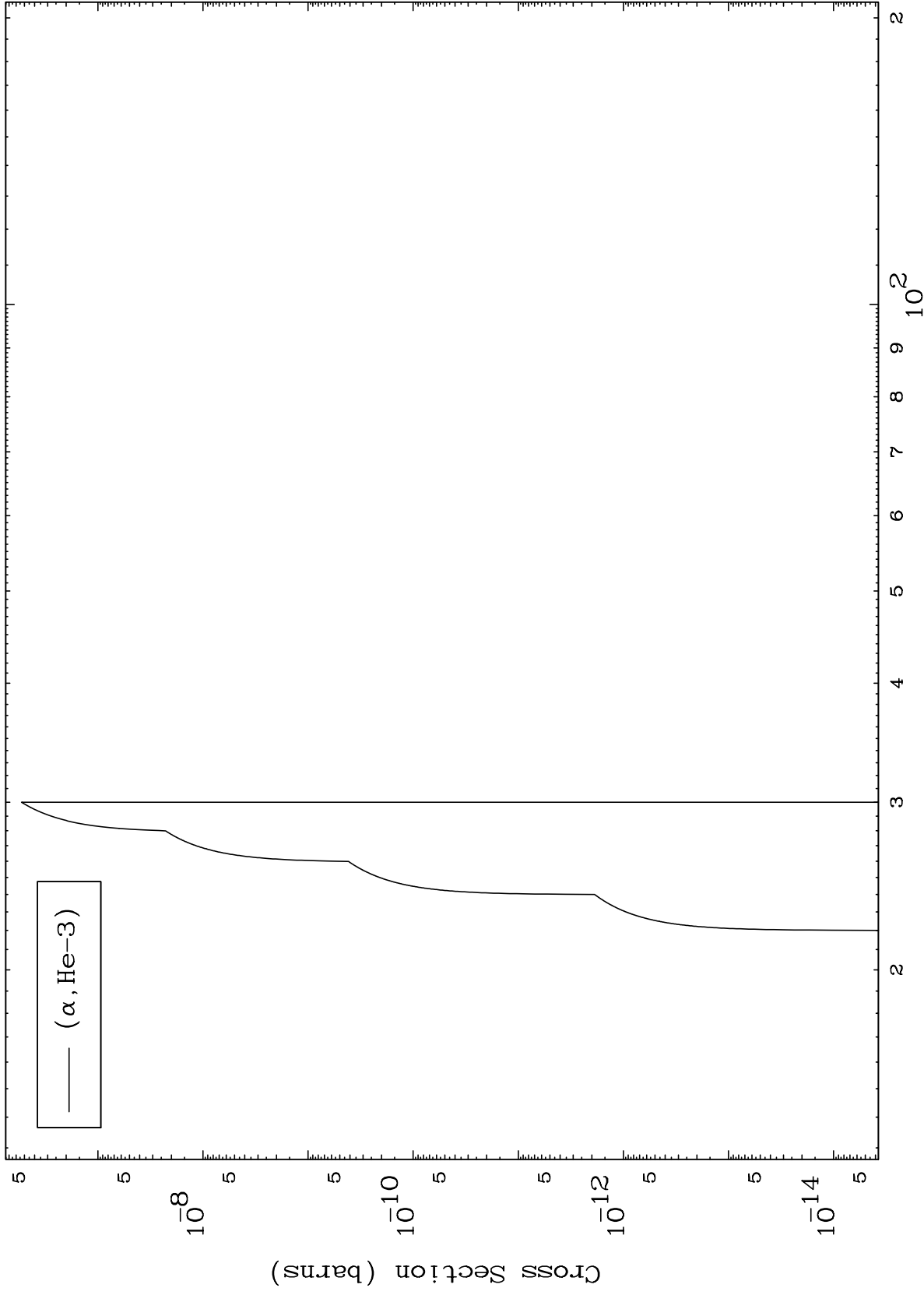


Incident Energy (MeV)

77-Ir-194





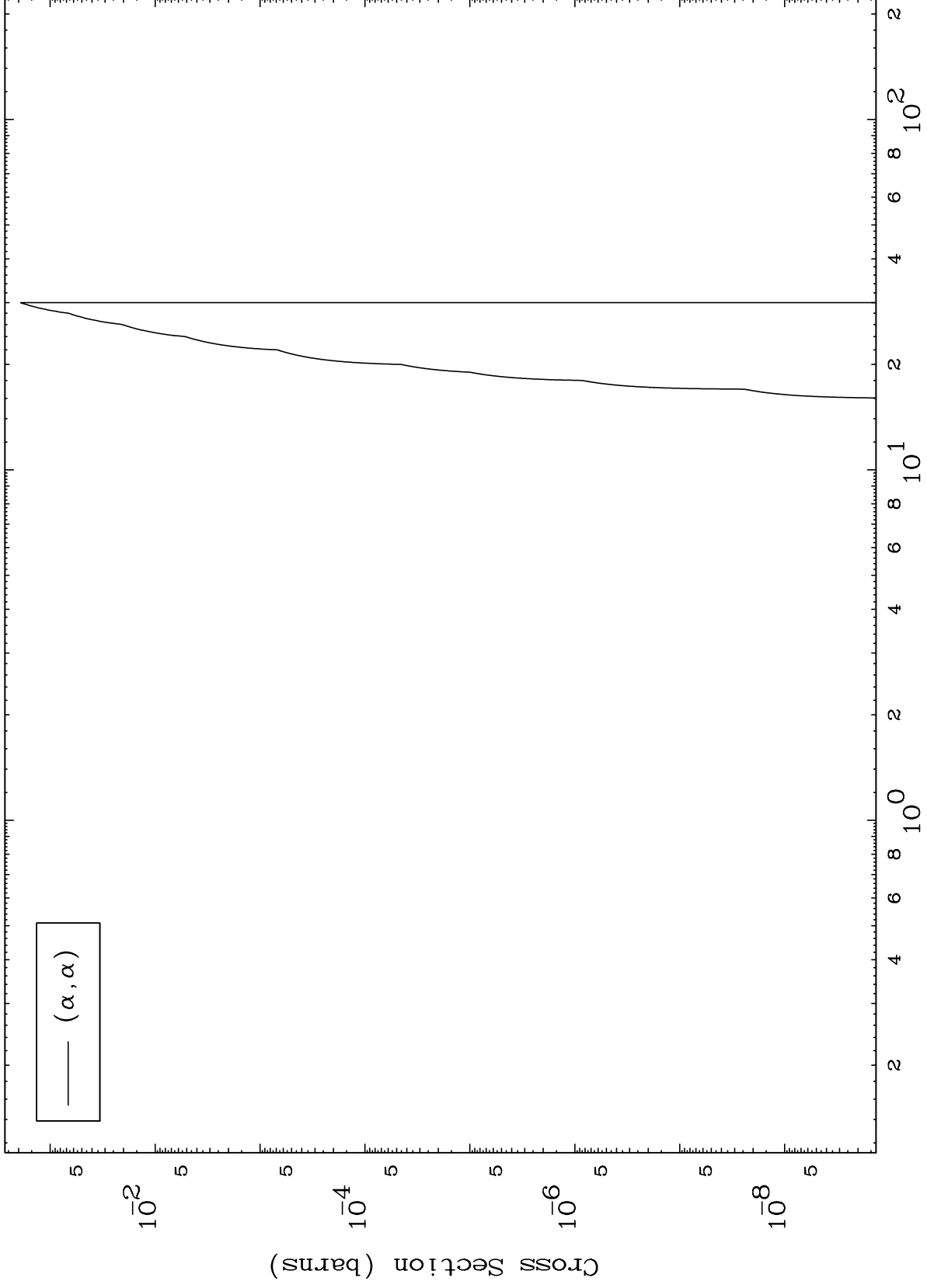


MAT 7735

( $\alpha, \alpha$ ) Levels

77-Ir-194

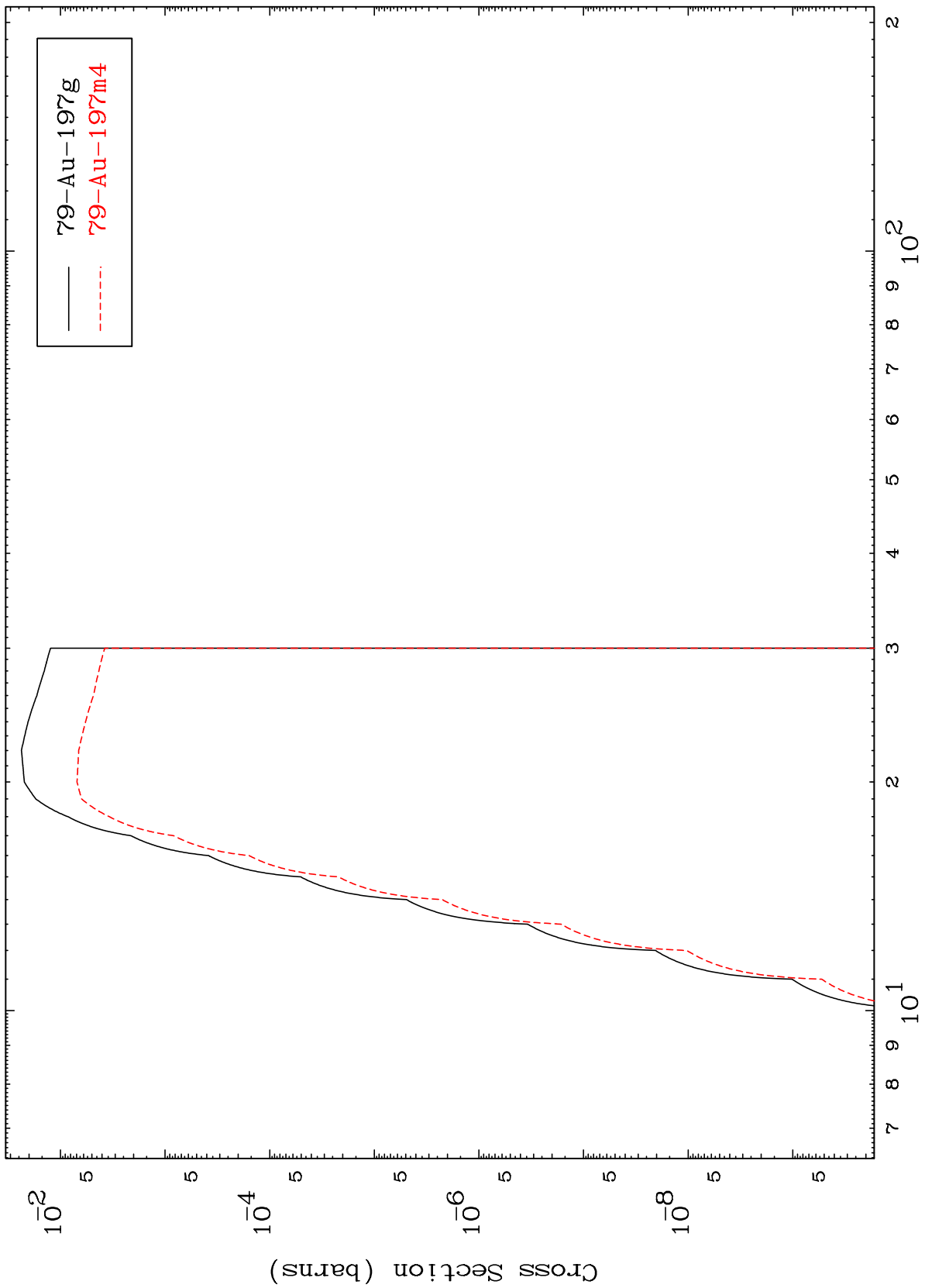
0 Kelvin Cross Sections



MAT 7735

77-Ir-194

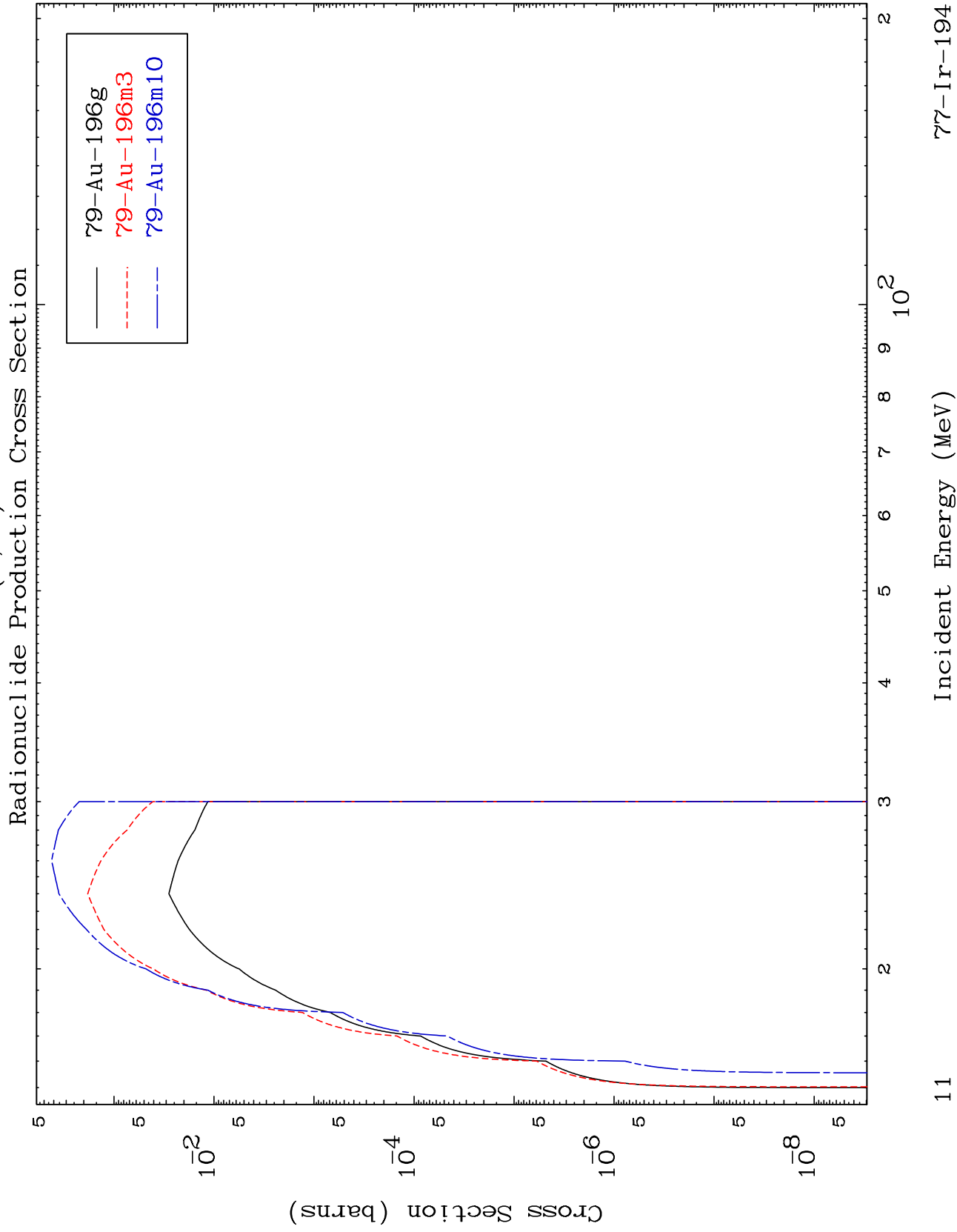
$\alpha$  Inelastic  
Radionuclide Production Cross Section



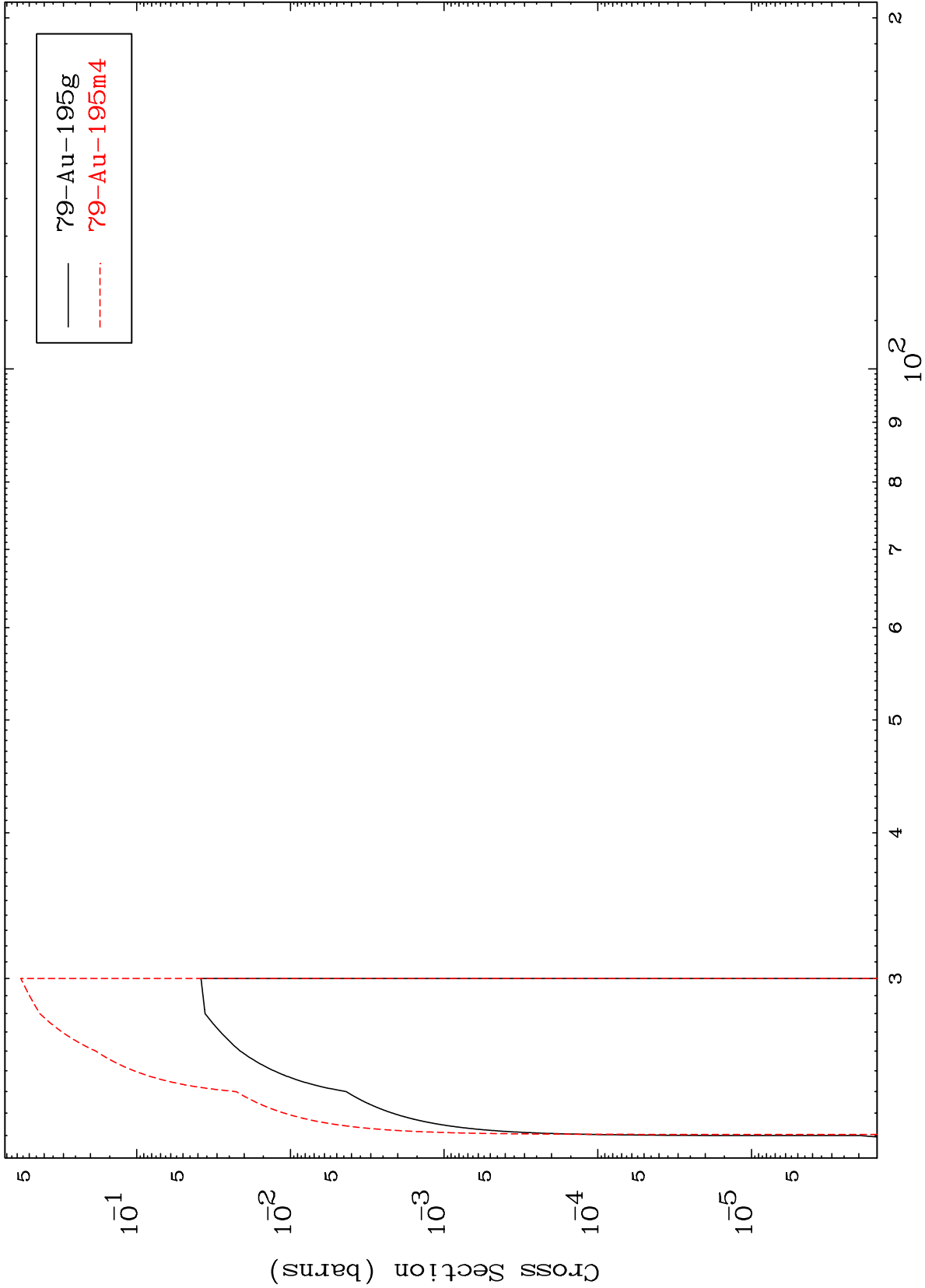
10

Incident Energy (MeV)

77-Ir-194



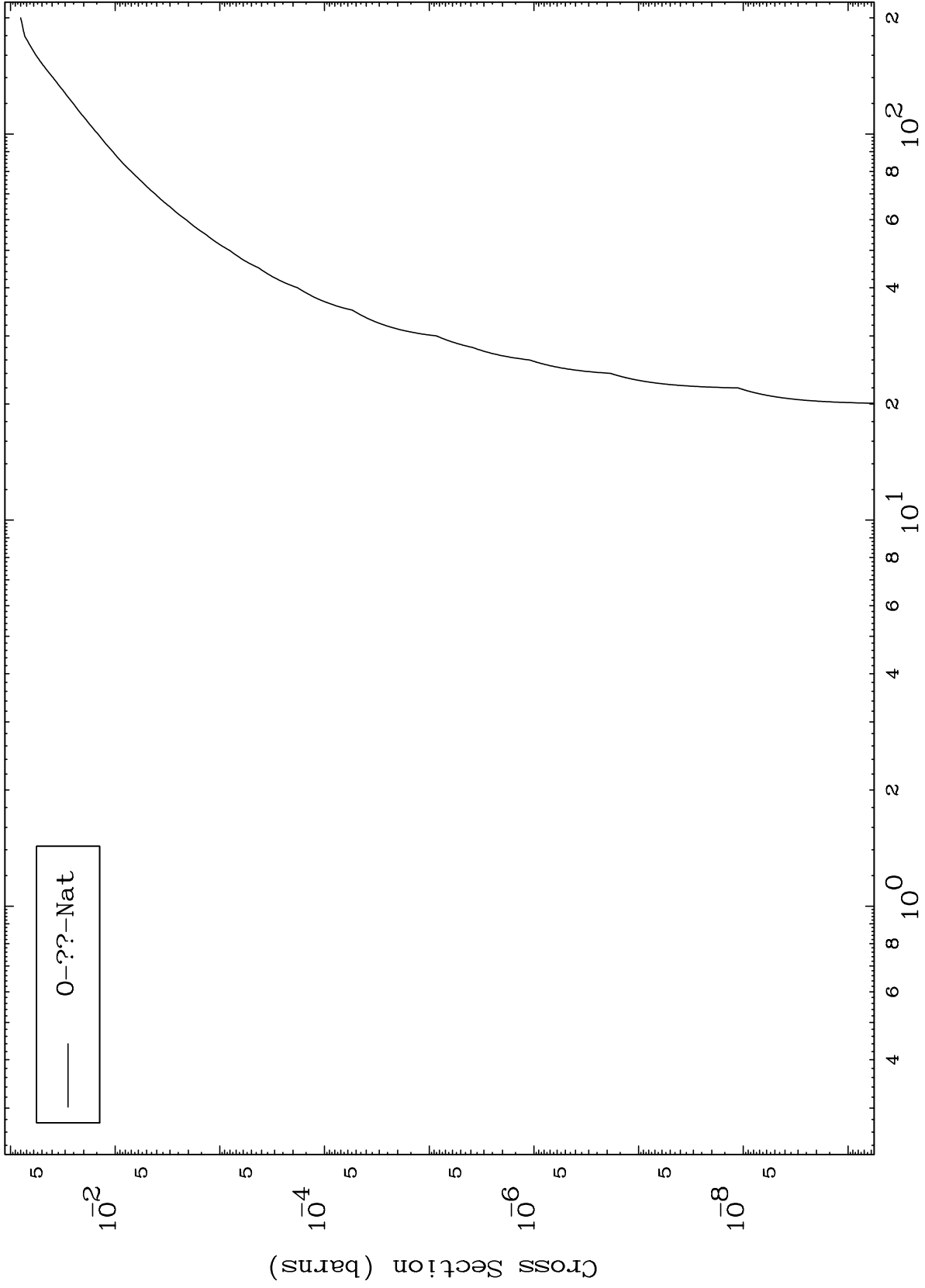
Radionuclide Production Cross Section



MAT 7735

77-Ir-194

$\alpha$  Fission  
Radionuclide Production Cross Section



13

77-Ir-194

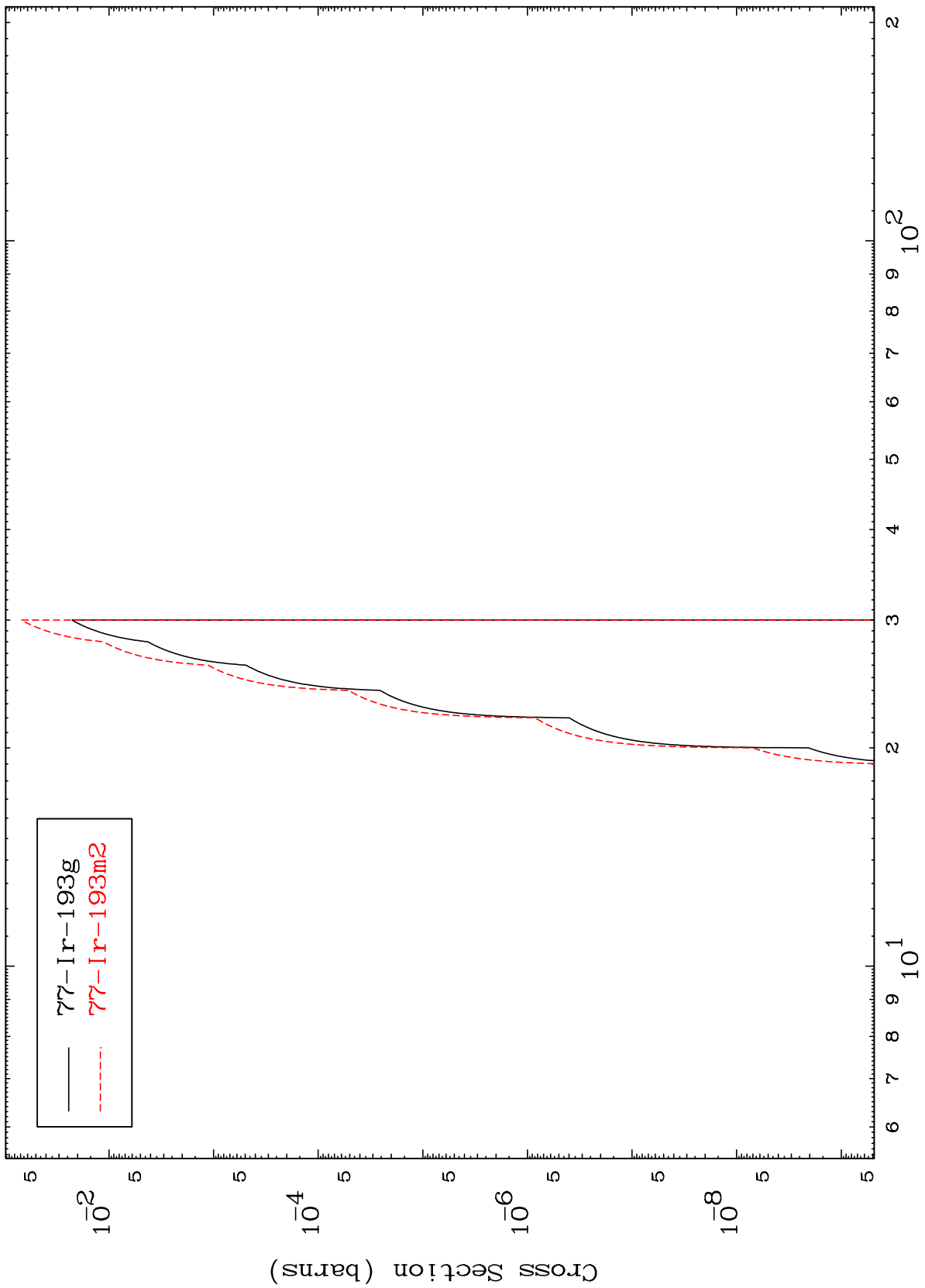
Incident Energy (MeV)

MAT 7735

$(\alpha, n')$   $\alpha$

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

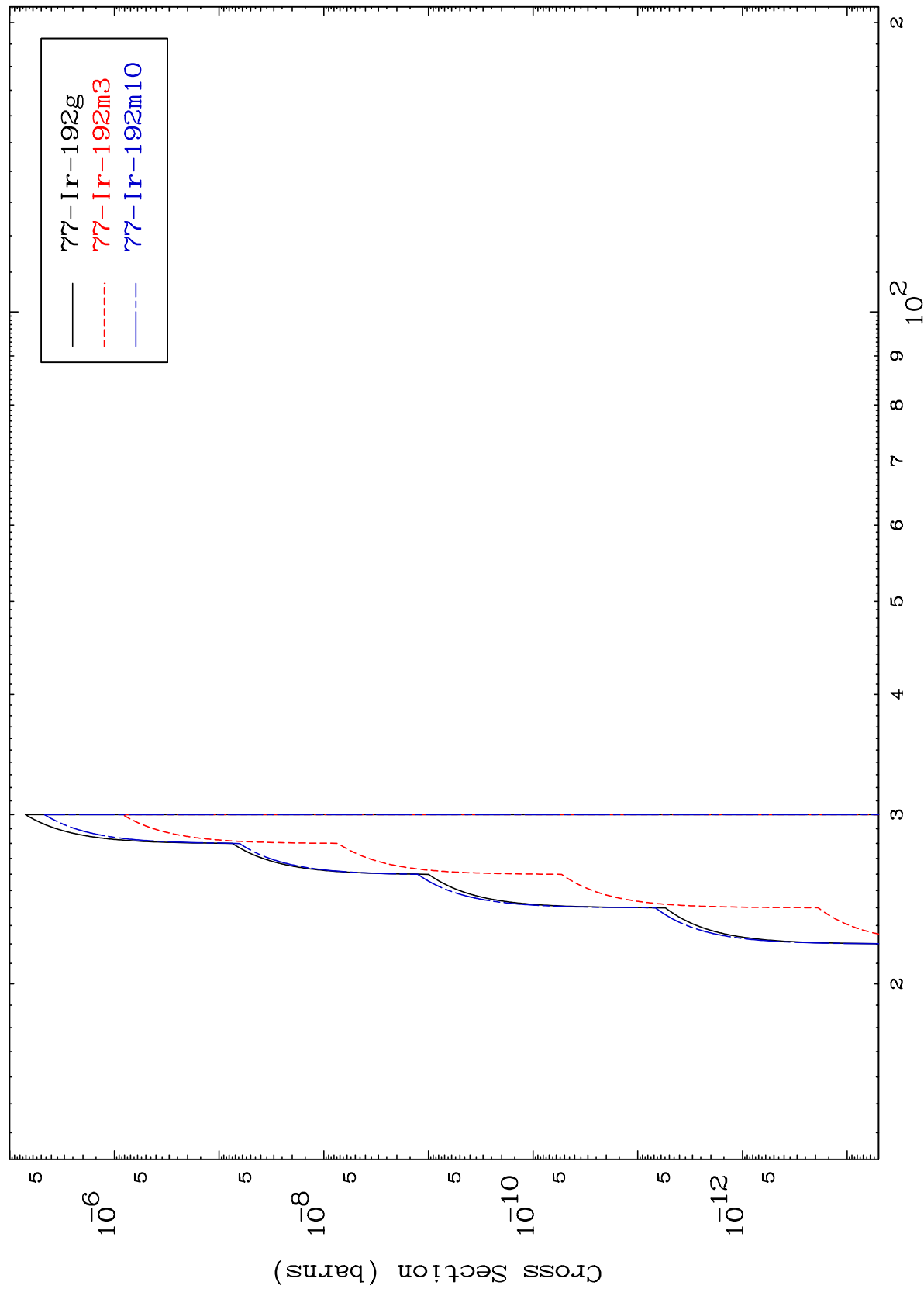
$^{77}\text{Ir-194}$

MAT 7735

$(\alpha, 2n)$   $\alpha$

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

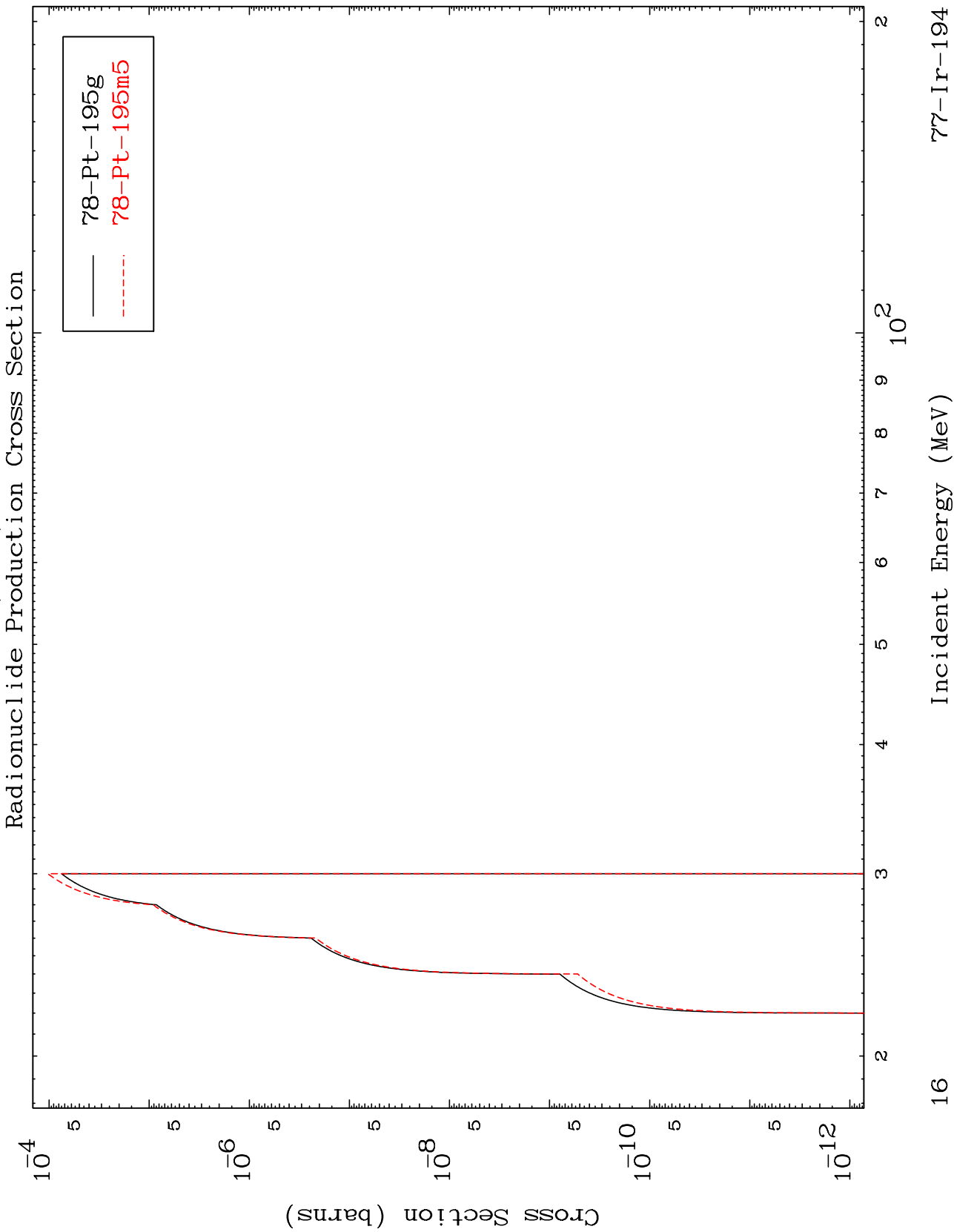
$^{77}\text{Ir-194}$



MAT 7735

$(\alpha, n')$  d

77-Ir-194

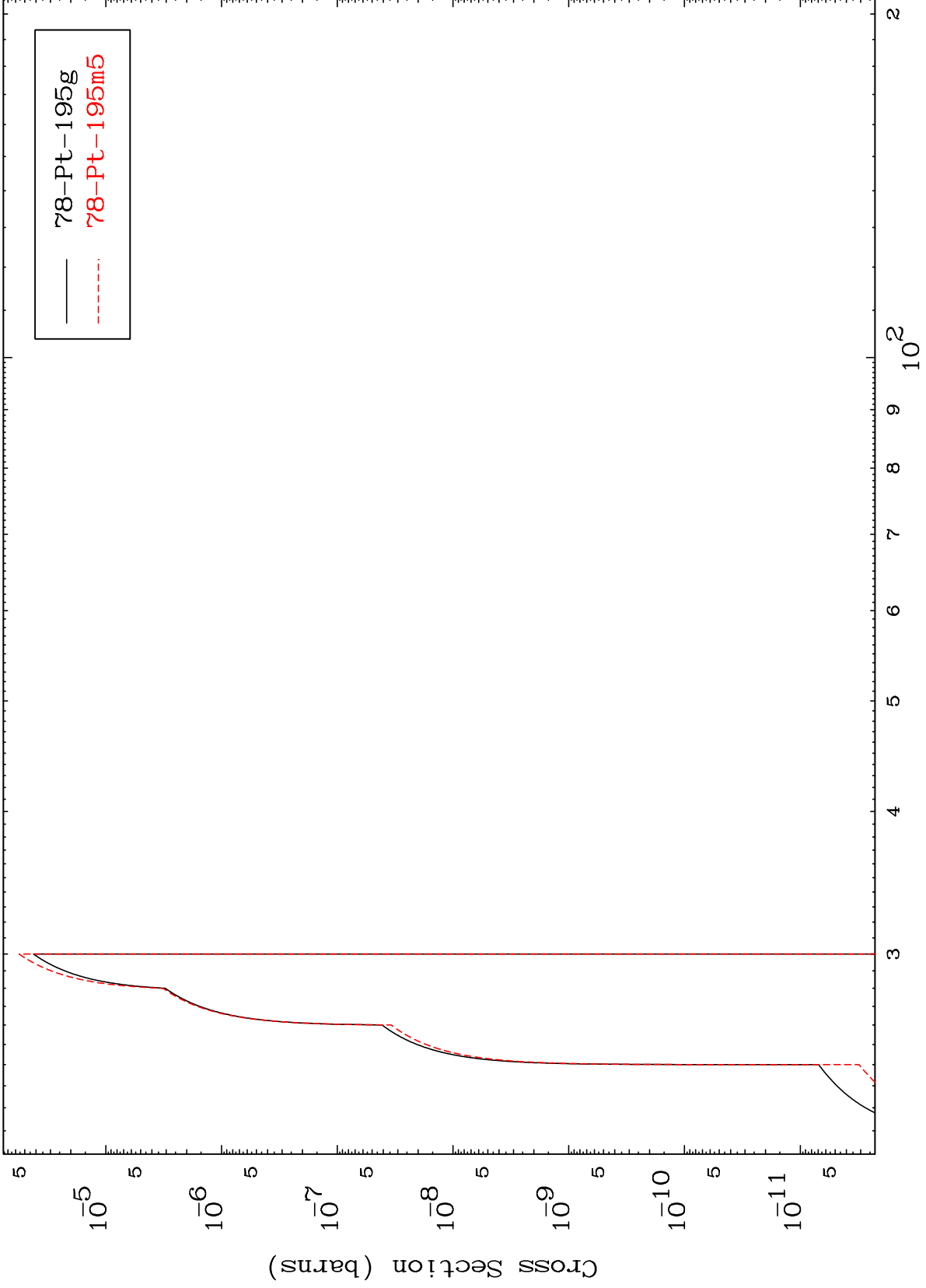


16

Incident Energy (MeV)

77-Ir-194

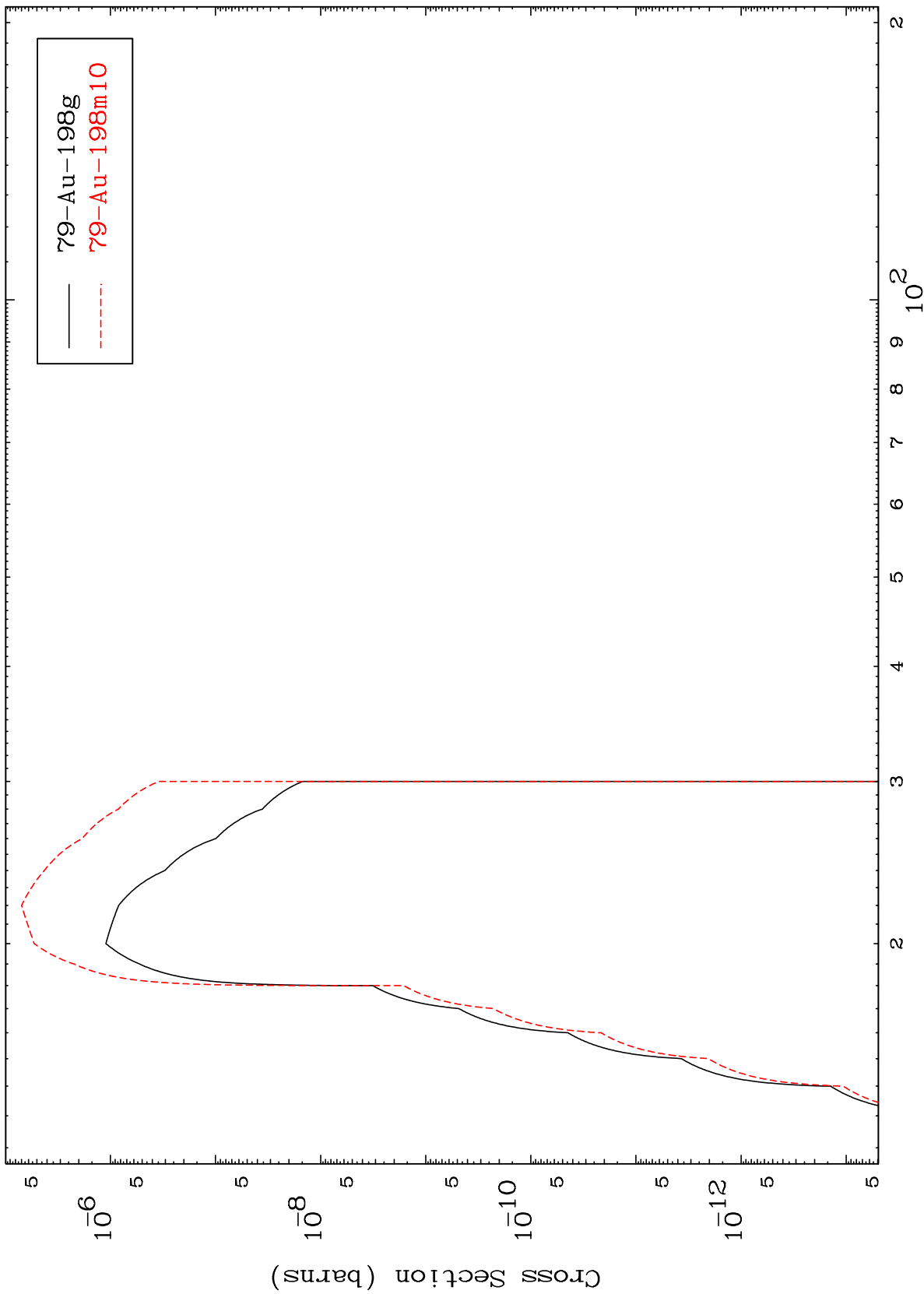
Radionuclide Production Cross Section



MAT 7735

77-Ir-194

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



18

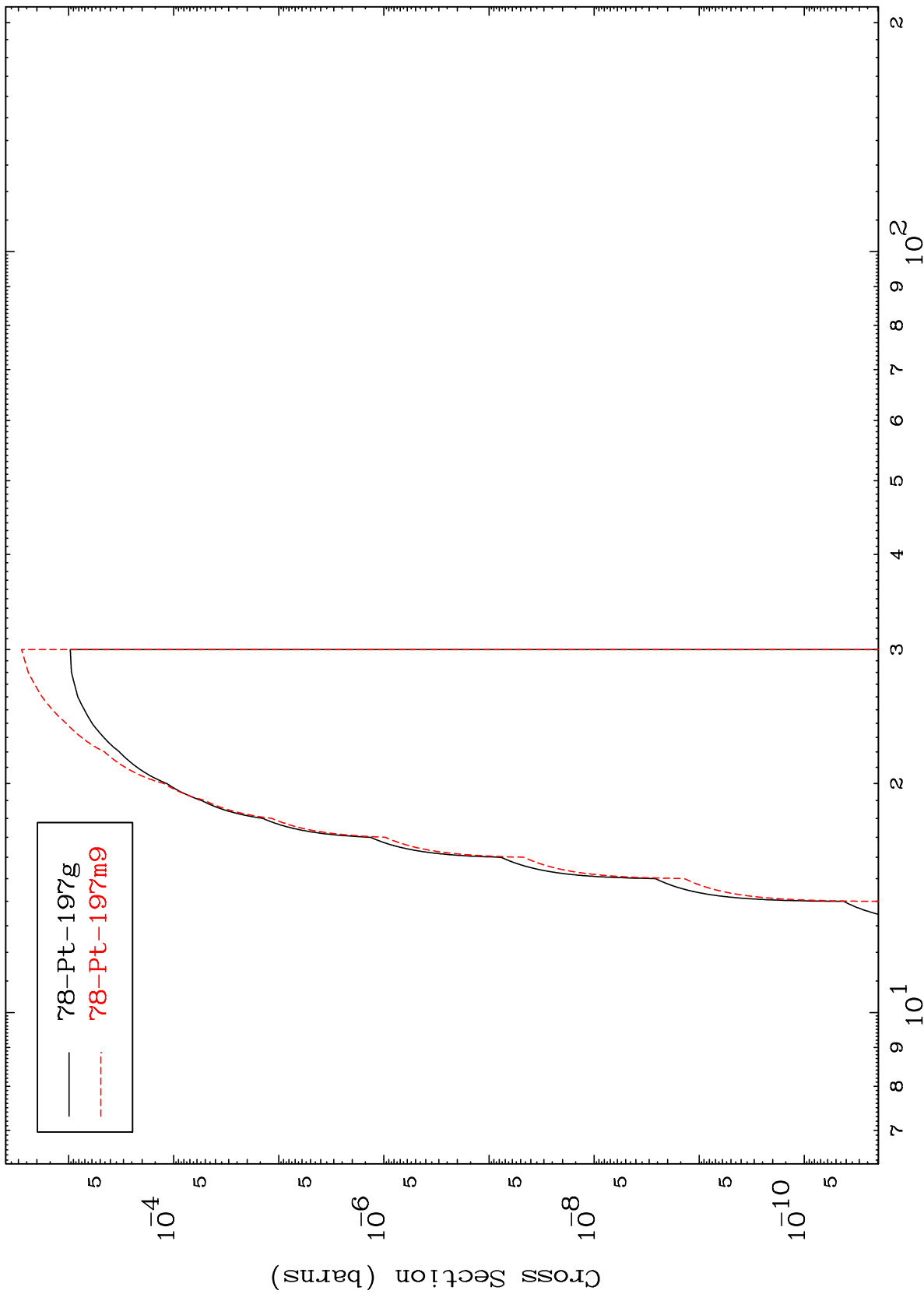
Incident Energy (MeV)

77-Ir-194

MAT 7735

77-Ir-194

( $\alpha, p$ )  
Radionuclide Production Cross Section

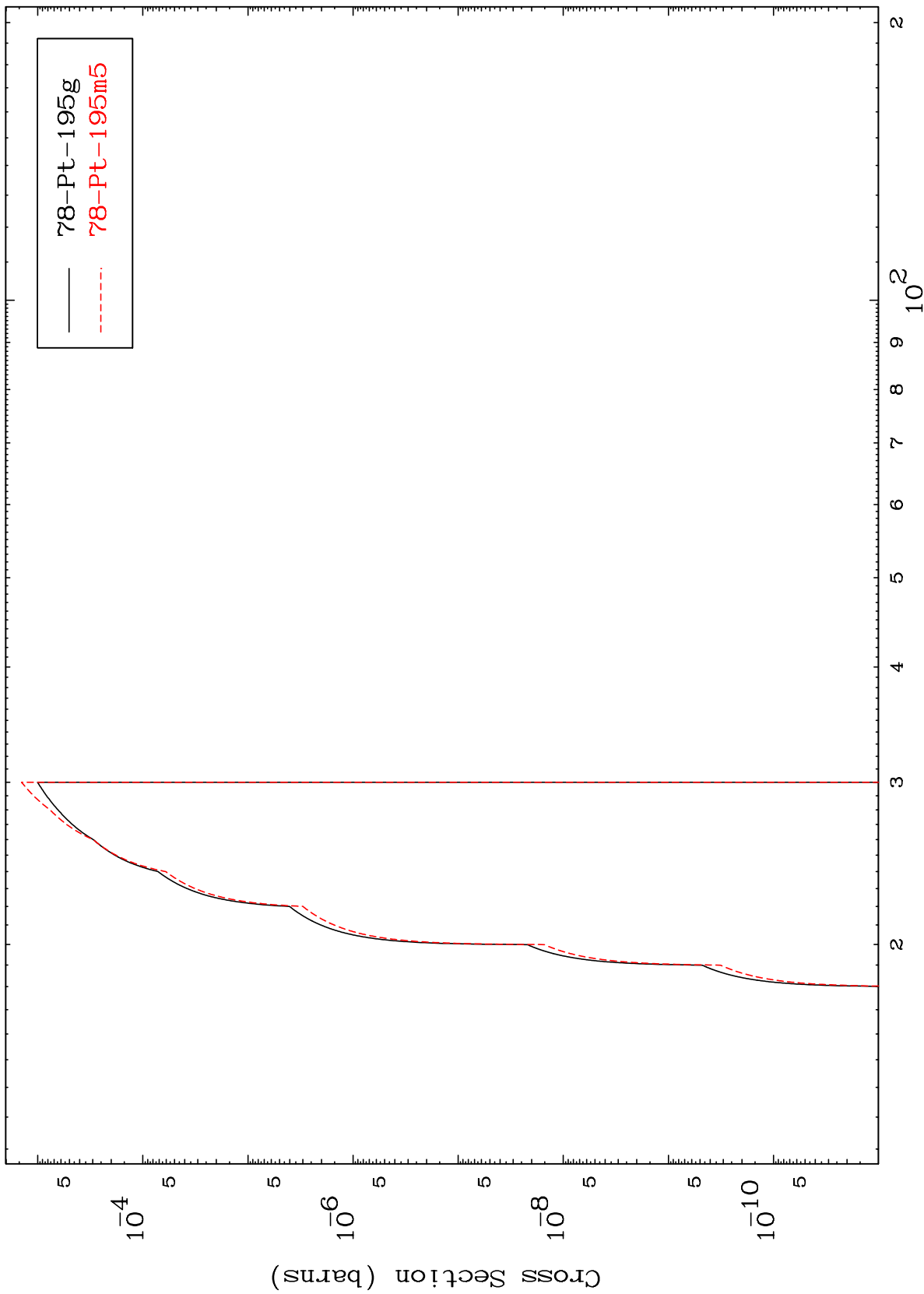


19

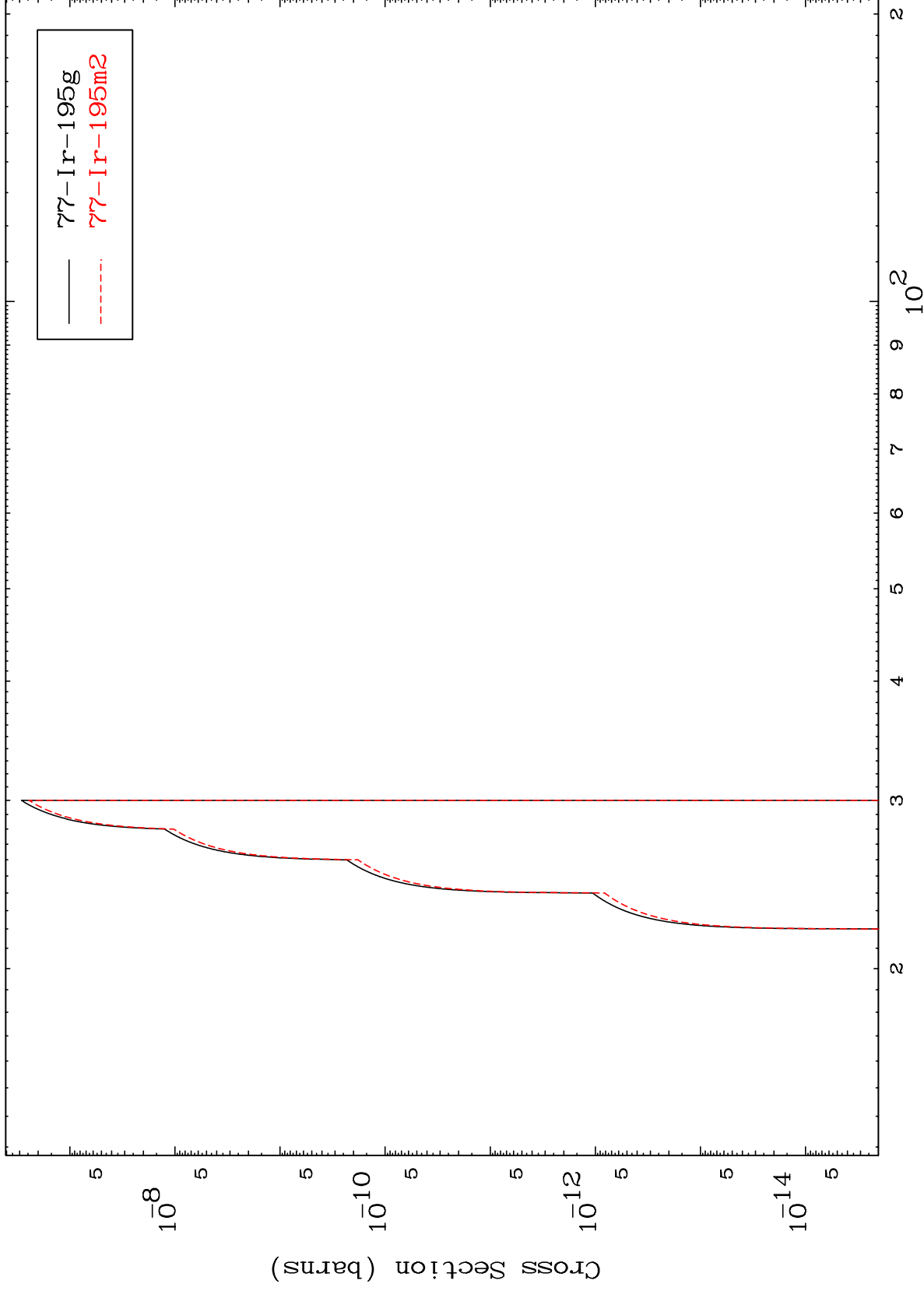
Incident Energy (MeV)

77-Ir-194

( $\alpha, t$ )  
Radionuclide Production Cross Section



Radionuclide Production Cross Section

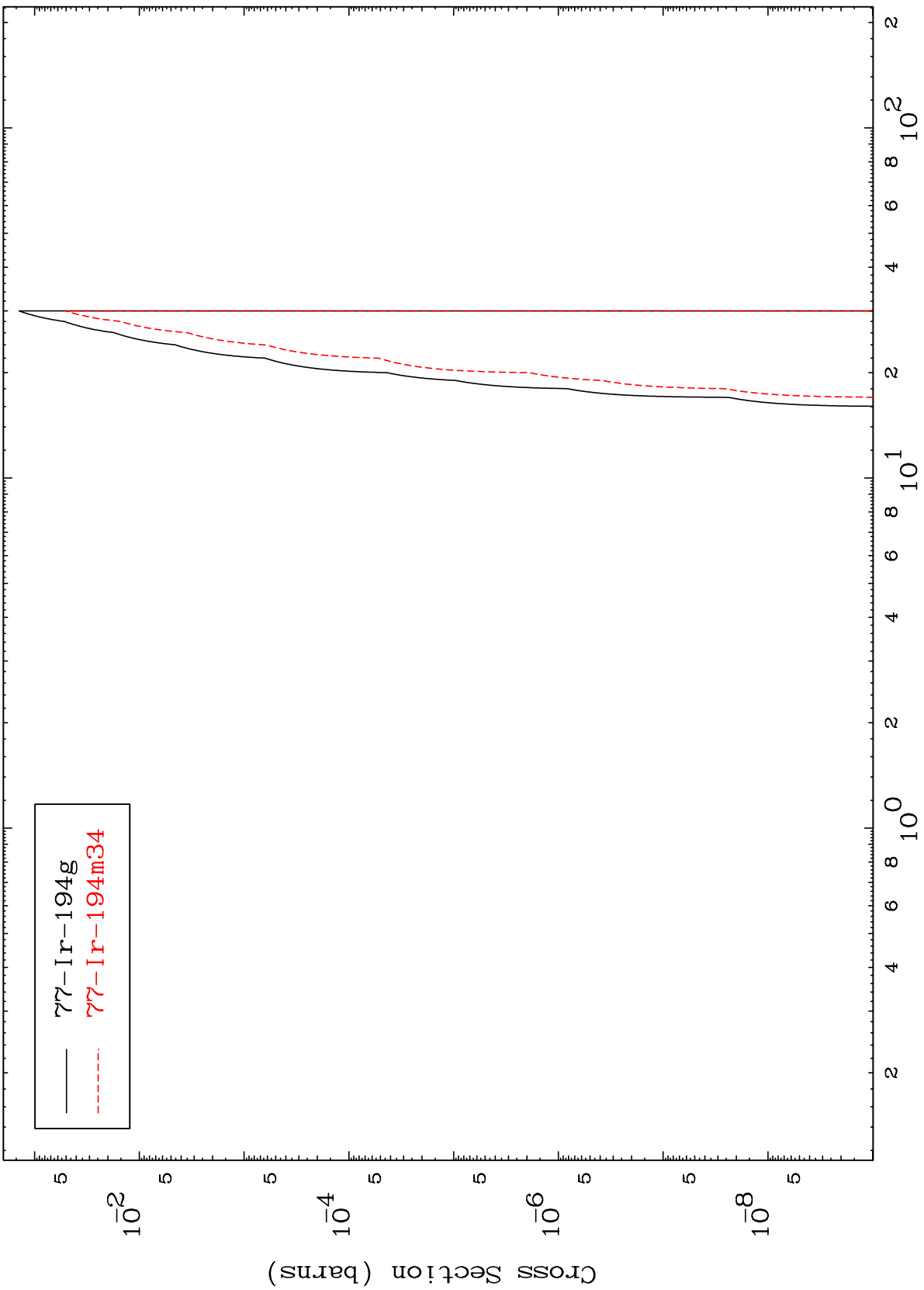


MAT 7735

( $\alpha, \alpha$ )

<sup>77</sup>Ir-194

Radionuclide Production Cross Section



— <sup>77</sup>Ir-194g  
- - - <sup>77</sup>Ir-194m34