

Program EVALPLOT  
(Version 2018-1)

by

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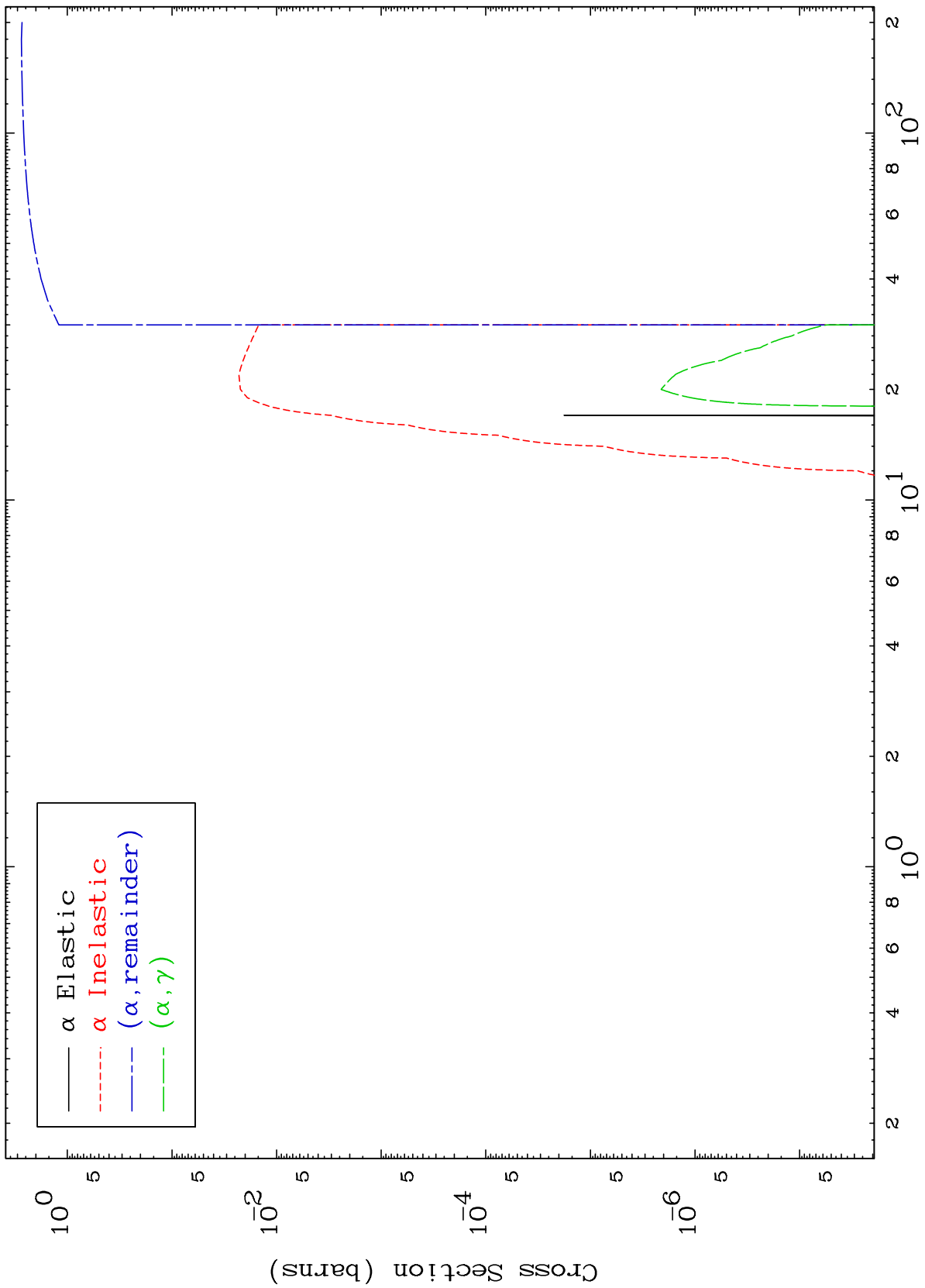
Press Mouse Button to Start

MAT 7737

$\alpha$  Major

77-Ir-195

0 Kelvin Cross Sections



1

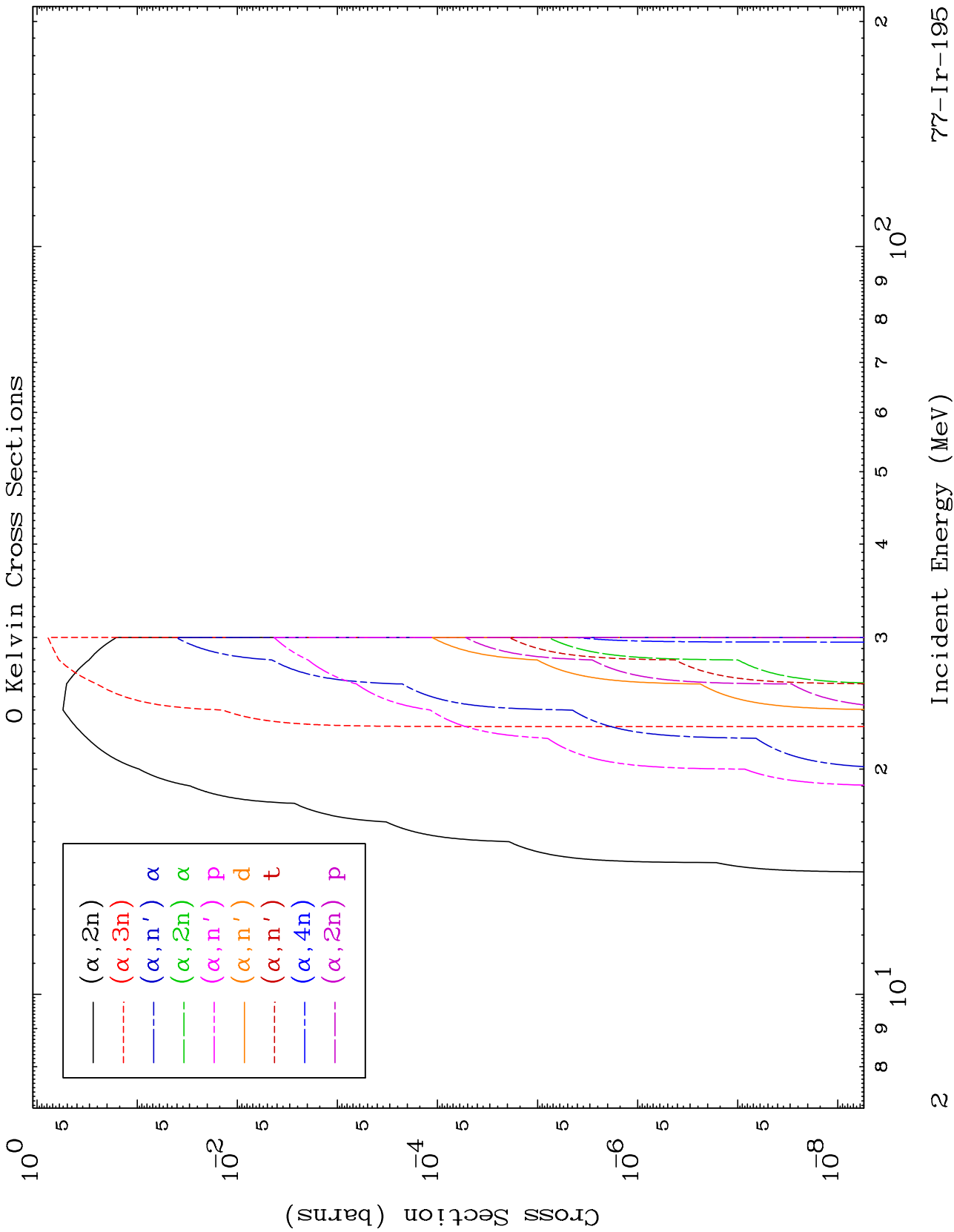
Incident Energy (MeV)

77-Ir-195

MAT 7737

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

77-Ir-195



77-Ir-195

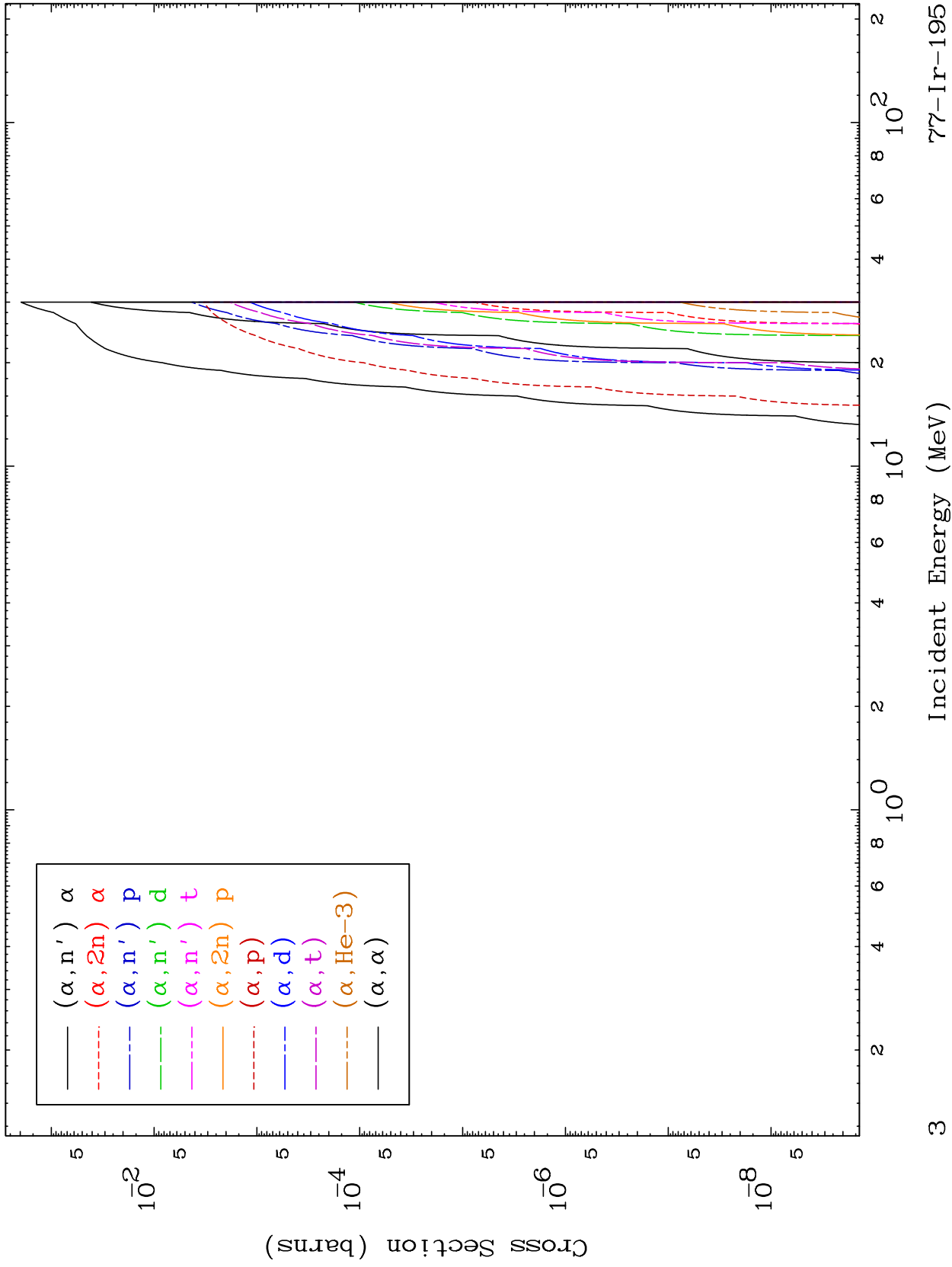
Incident Energy (MeV)

2

MAT 7737

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

77-Ir-195

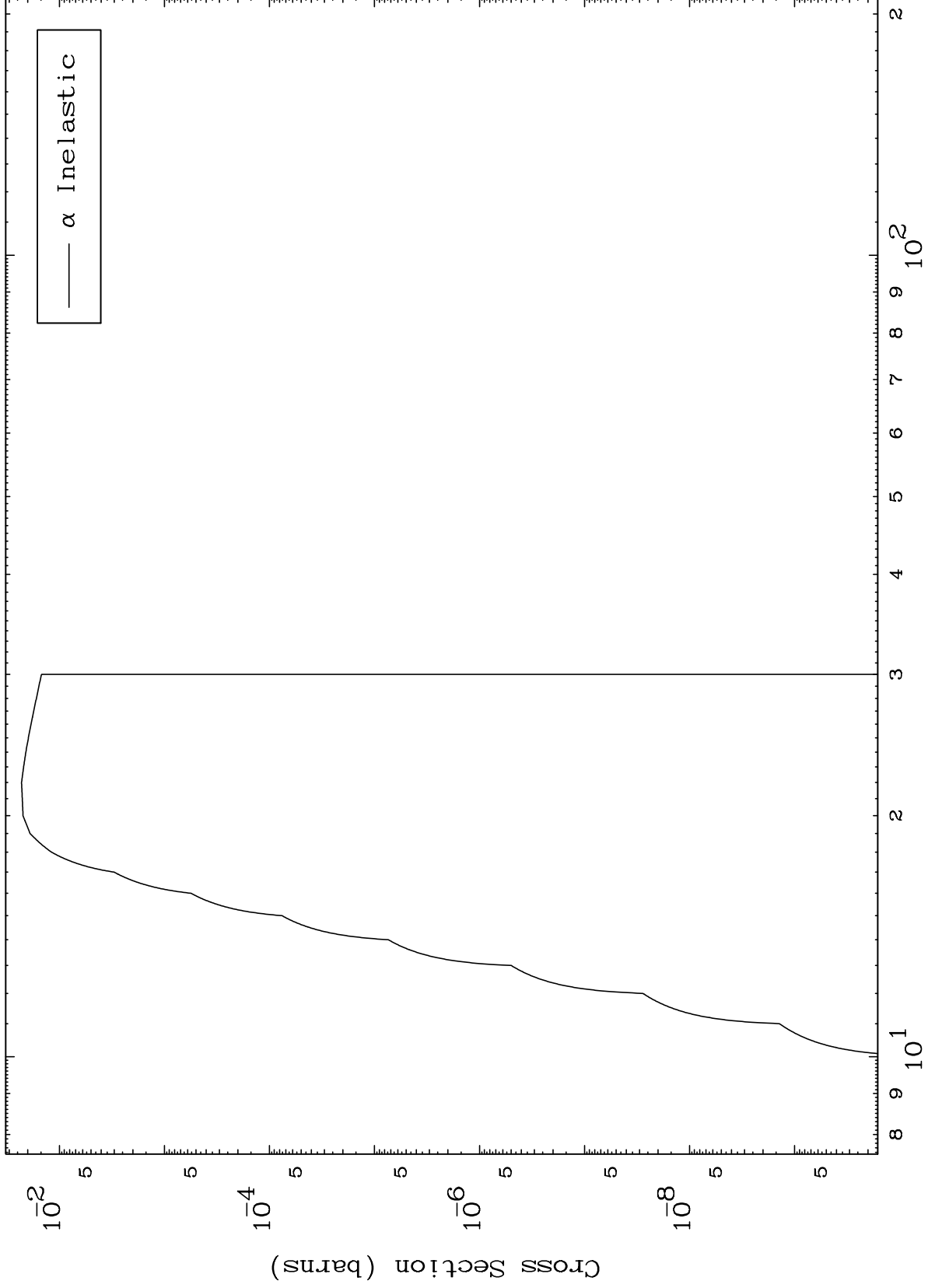


MAT 7737

( $\alpha, n'$ ) Level

77-Ir-195

0 Kelvin Cross Sections



4

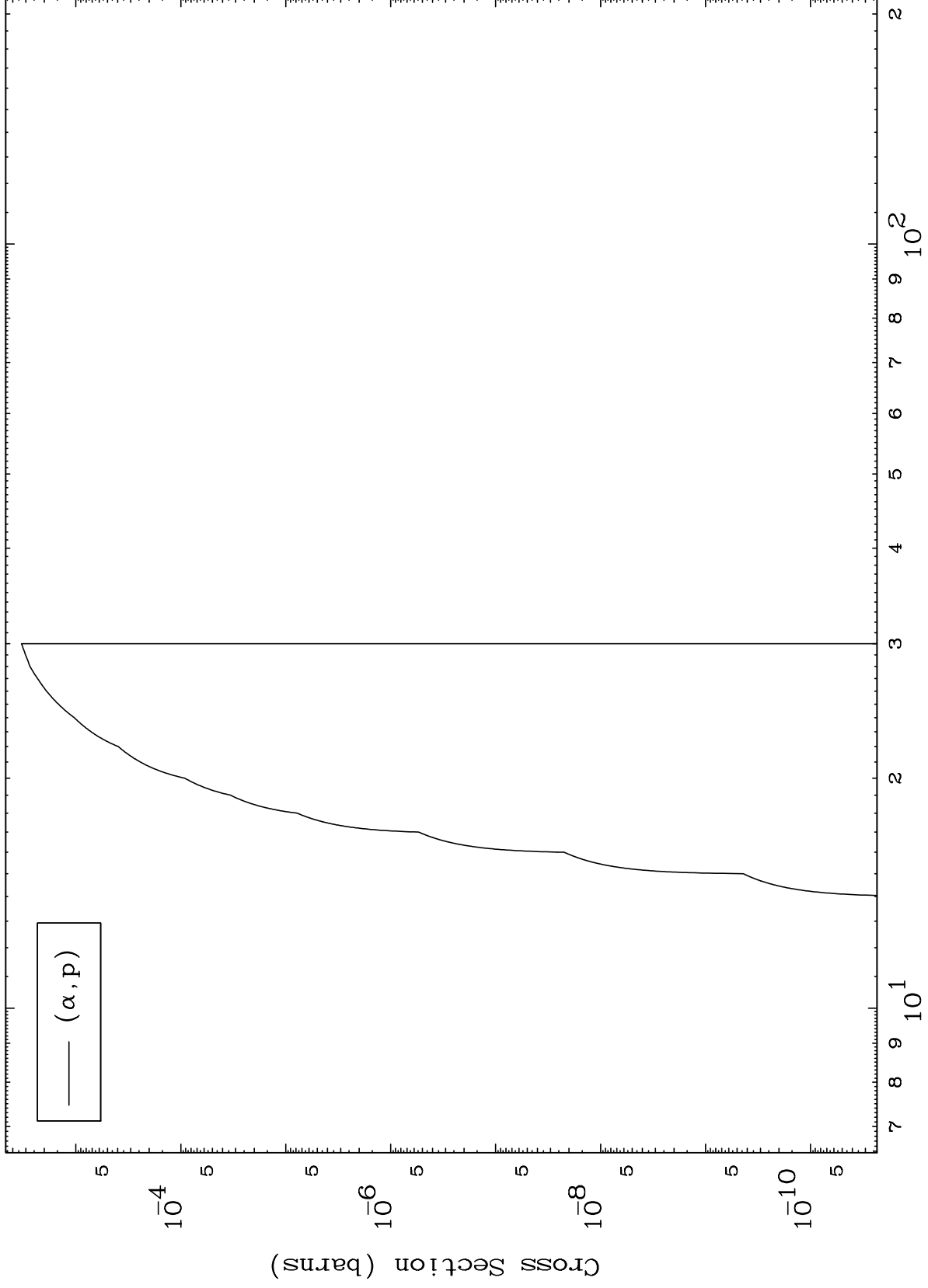
Incident Energy (MeV)

77-Ir-195

MAT 7737

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

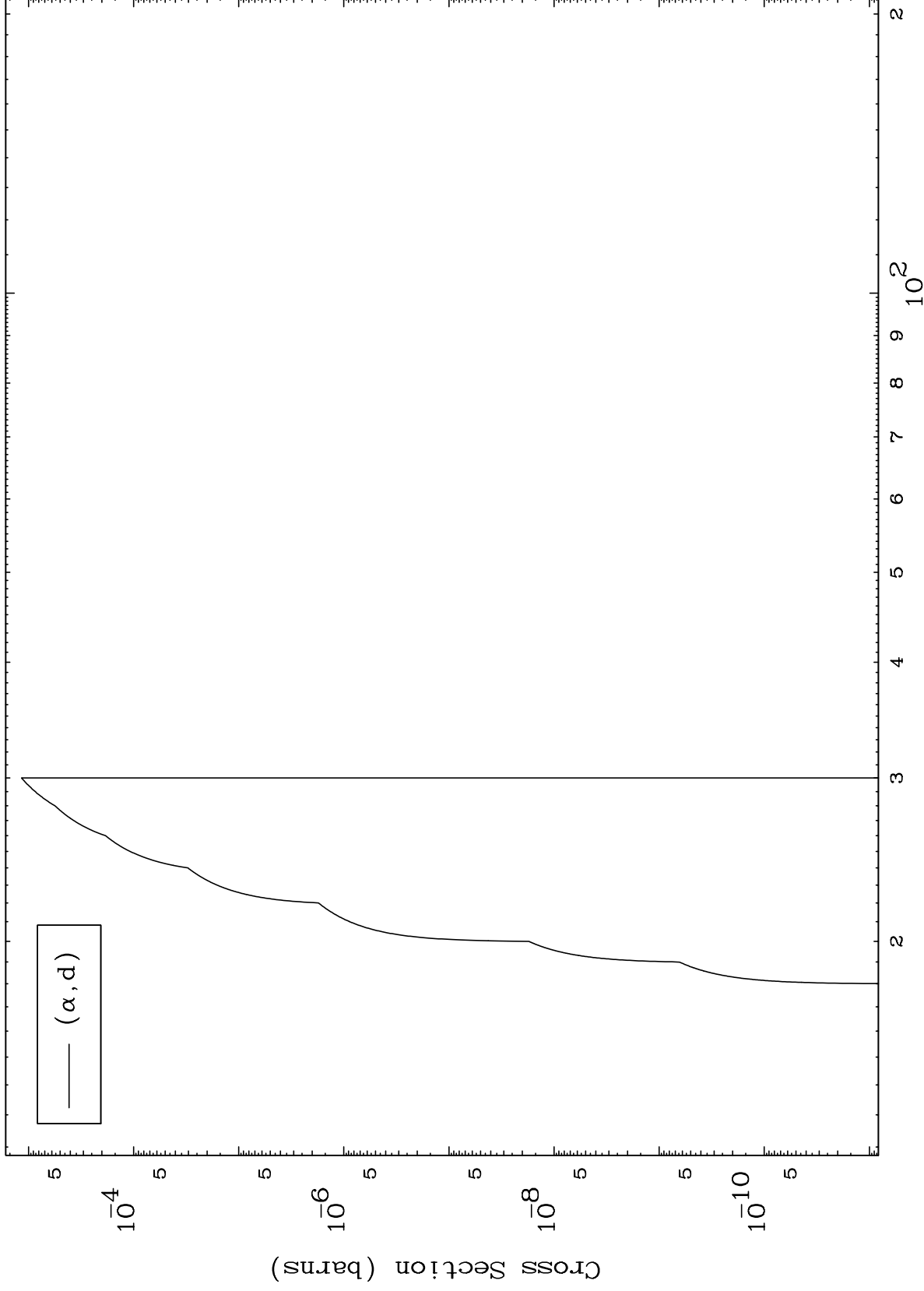
77-Ir-195



5

Incident Energy (MeV)

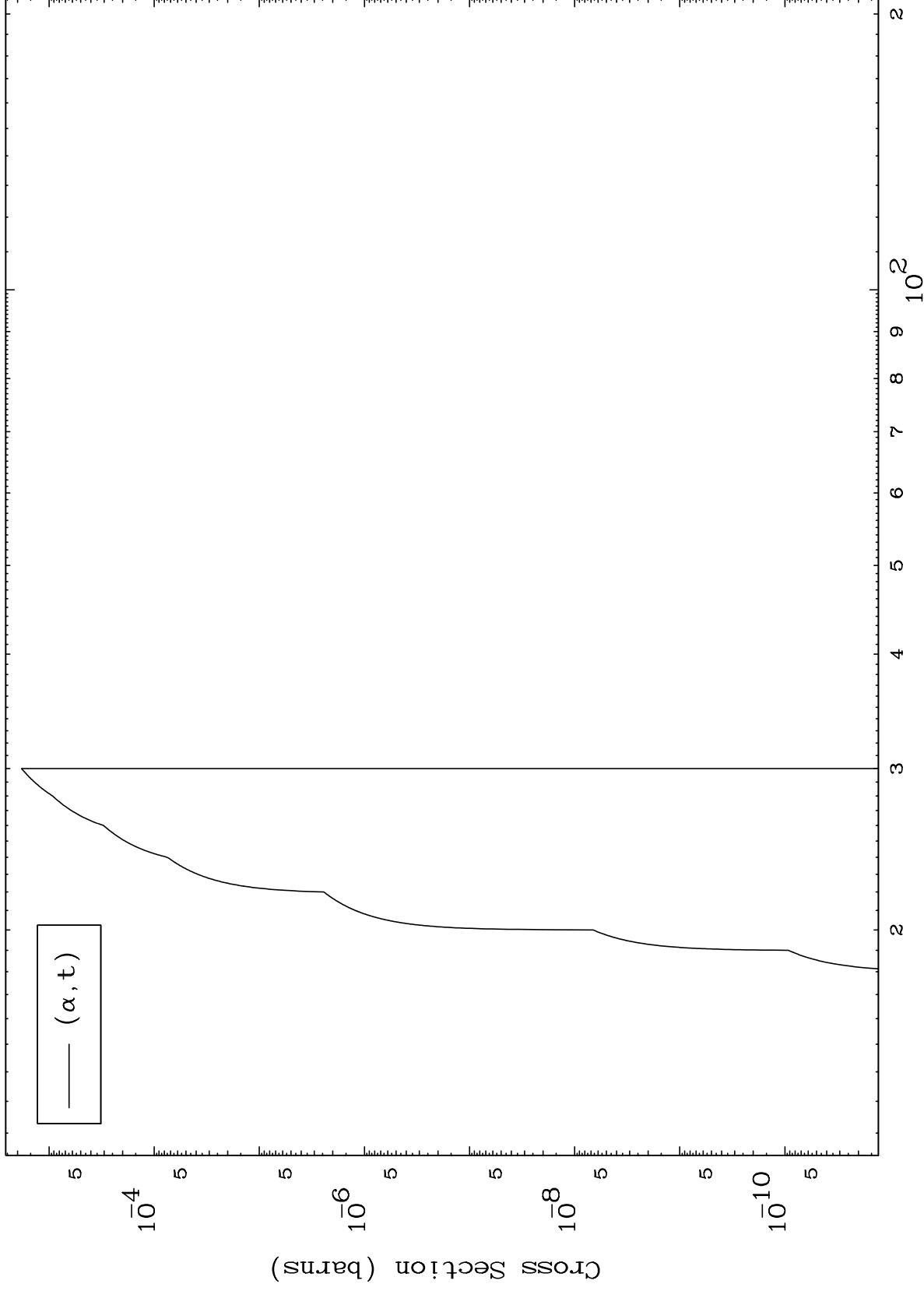
77-Ir-195



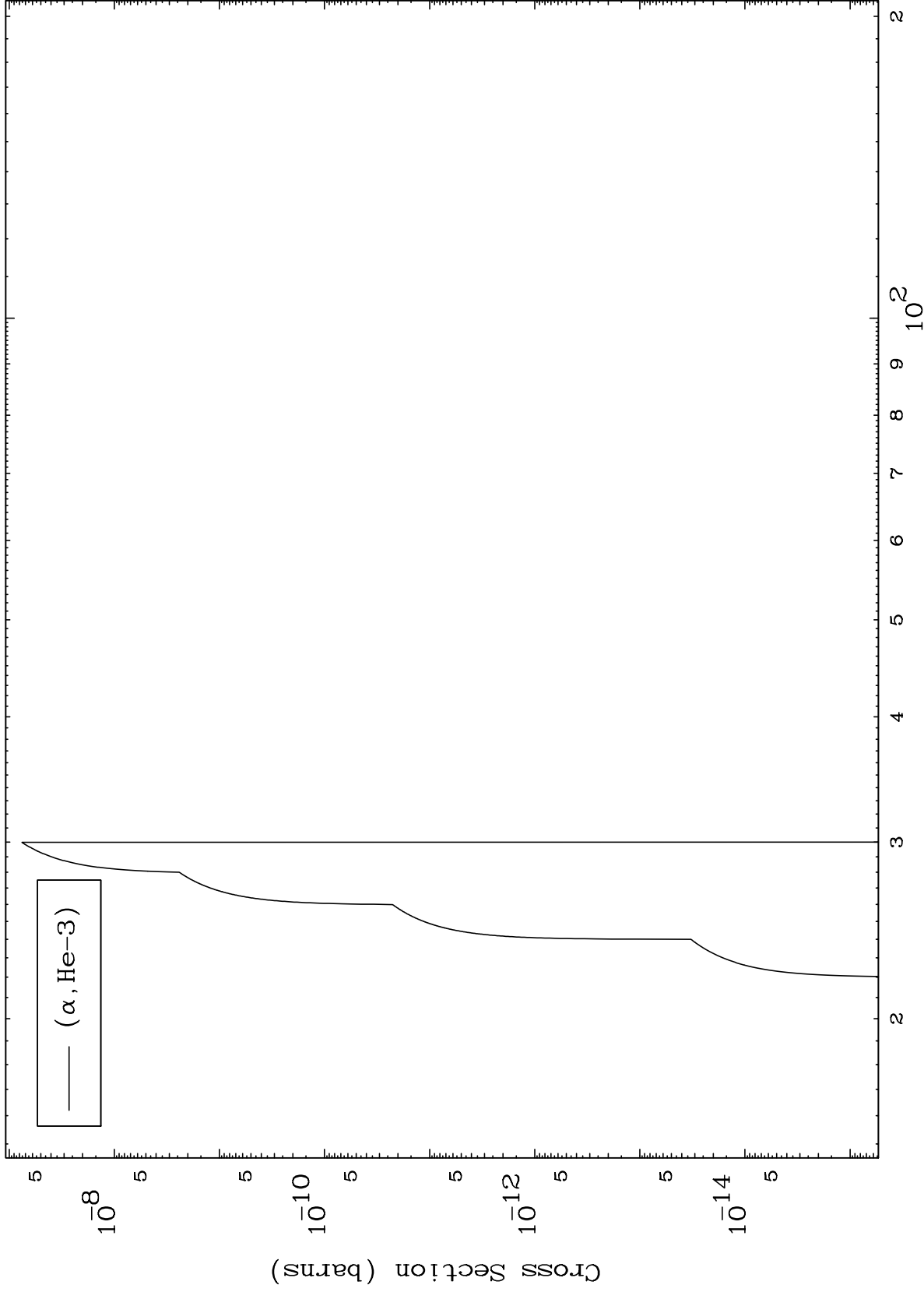
MAT 7737

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-195





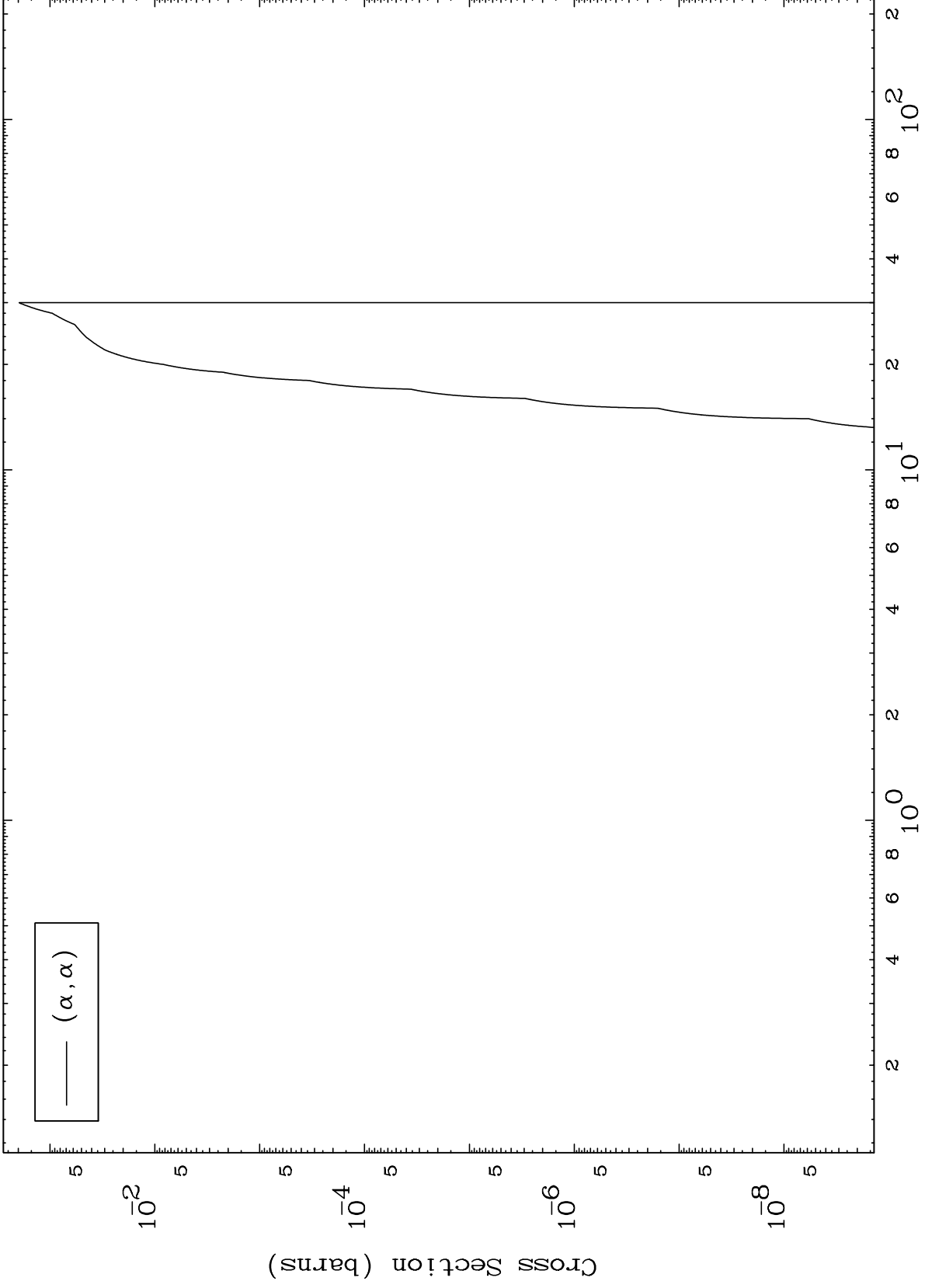


MAT 7737

( $\alpha, \alpha$ ) Levels

77-Ir-195

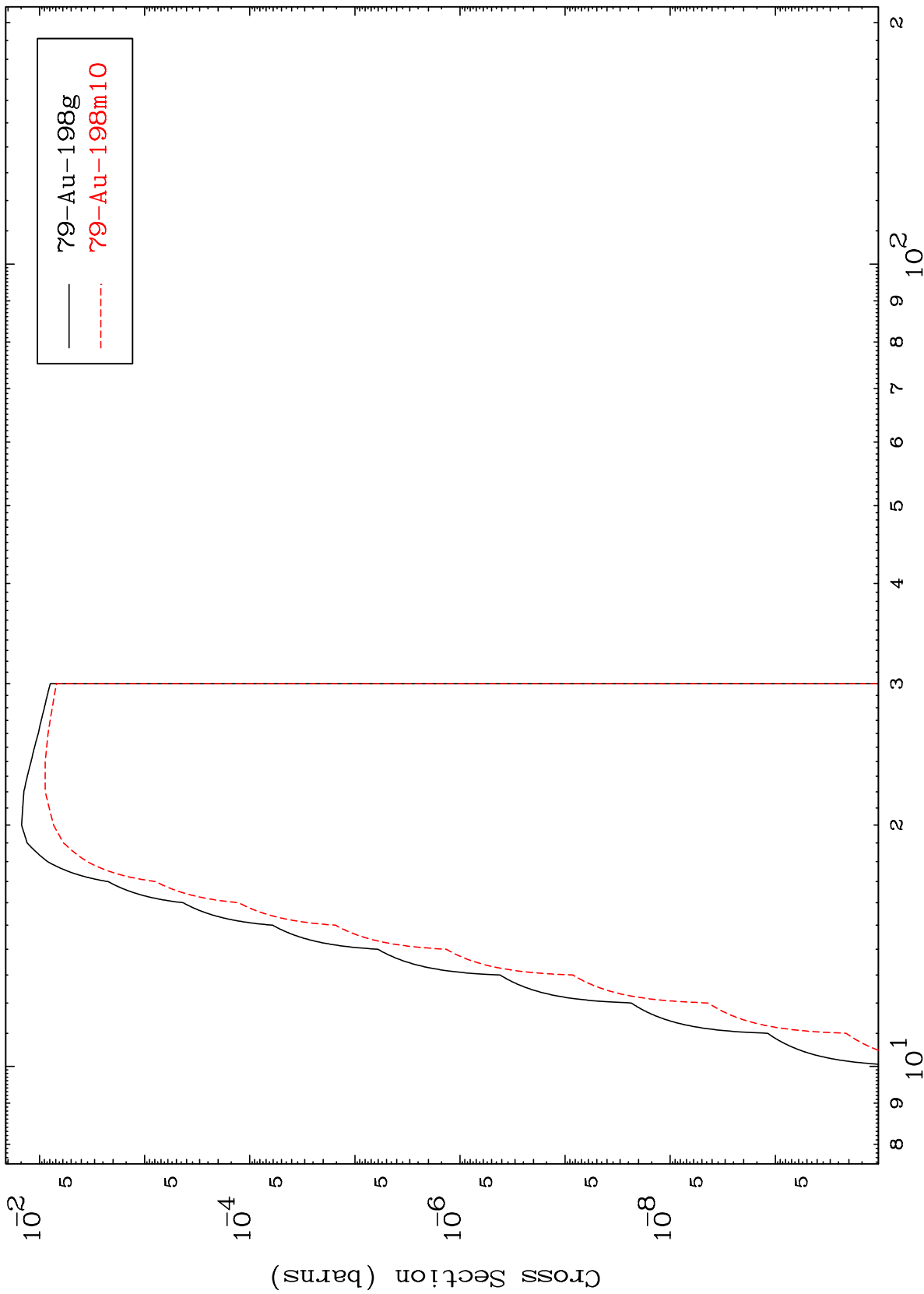
0 Kelvin Cross Sections



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77-Ir-195

$\alpha$  Inelastic  
Radionuclide Production Cross Section

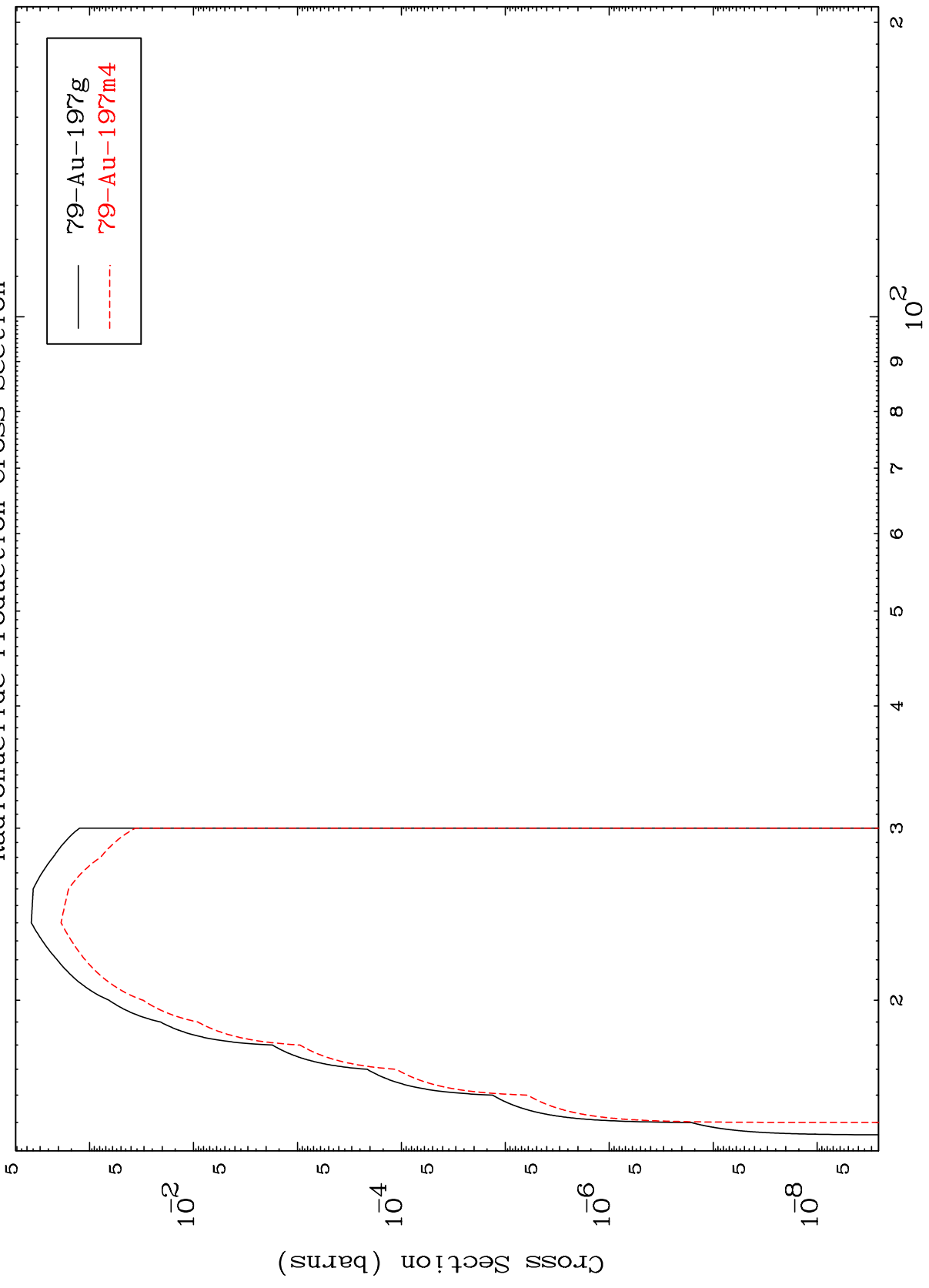


77-Ir-195

Incident Energy (MeV)

10

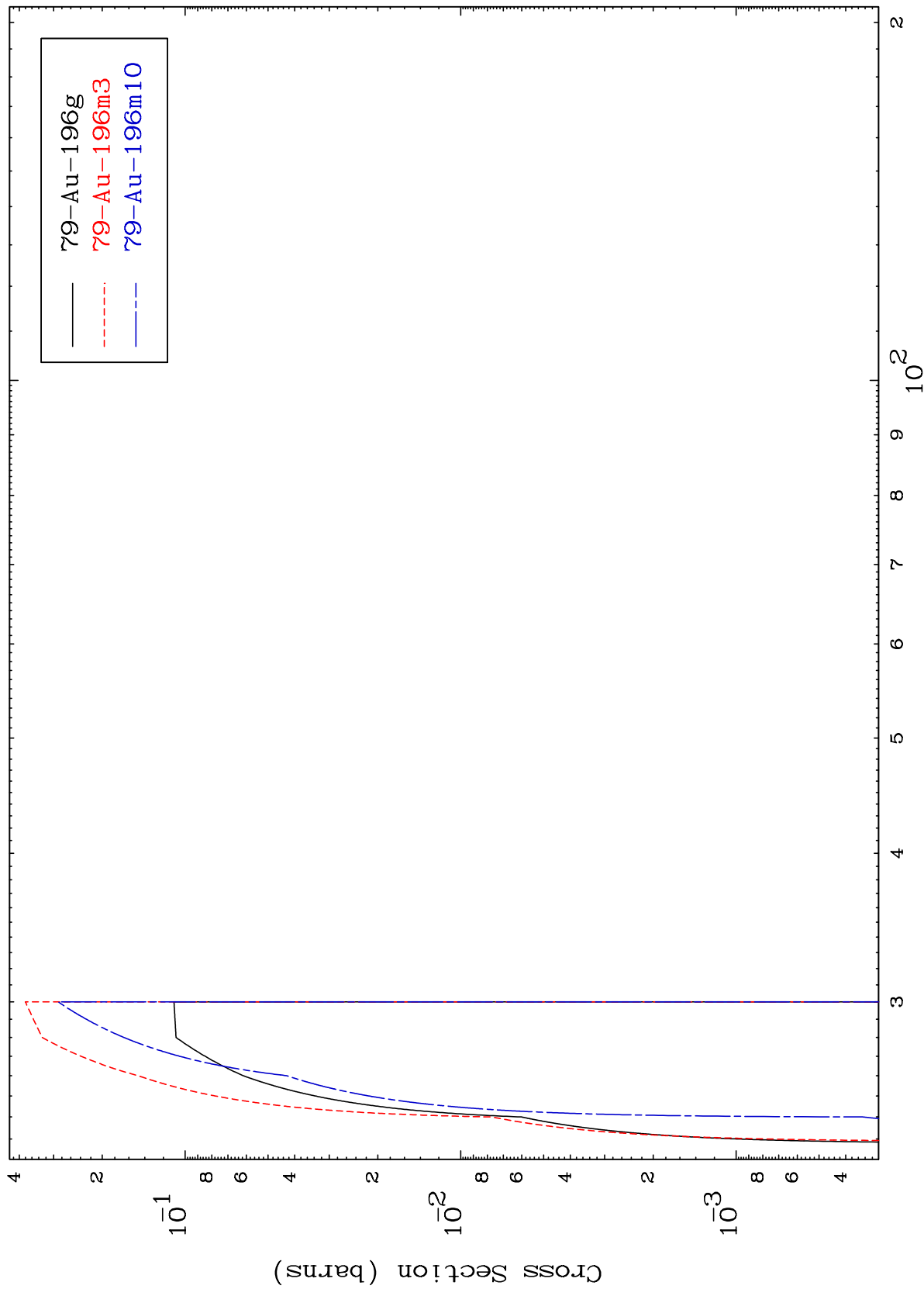
( $\alpha, 2n$ )  
Radionuclide Production Cross Section



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77-Ir-195

$(\alpha, 3n)$   
Radionuclide Production Cross Section



12

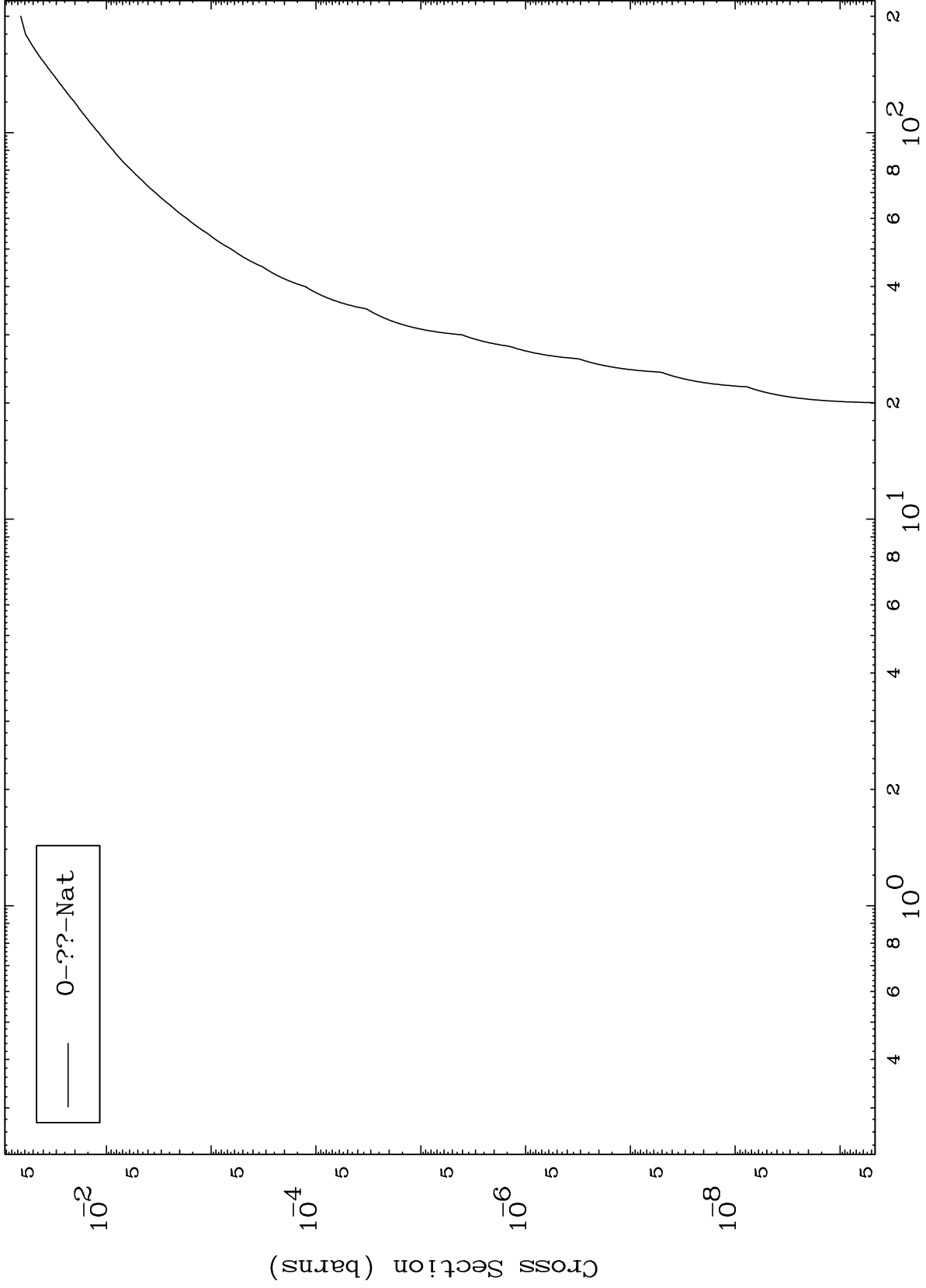
77-Ir-195

Incident Energy (MeV)

MAT 7737

77-Ir-195

$\alpha$  Fission  
Radionuclide Production Cross Section



13

Incident Energy (MeV)

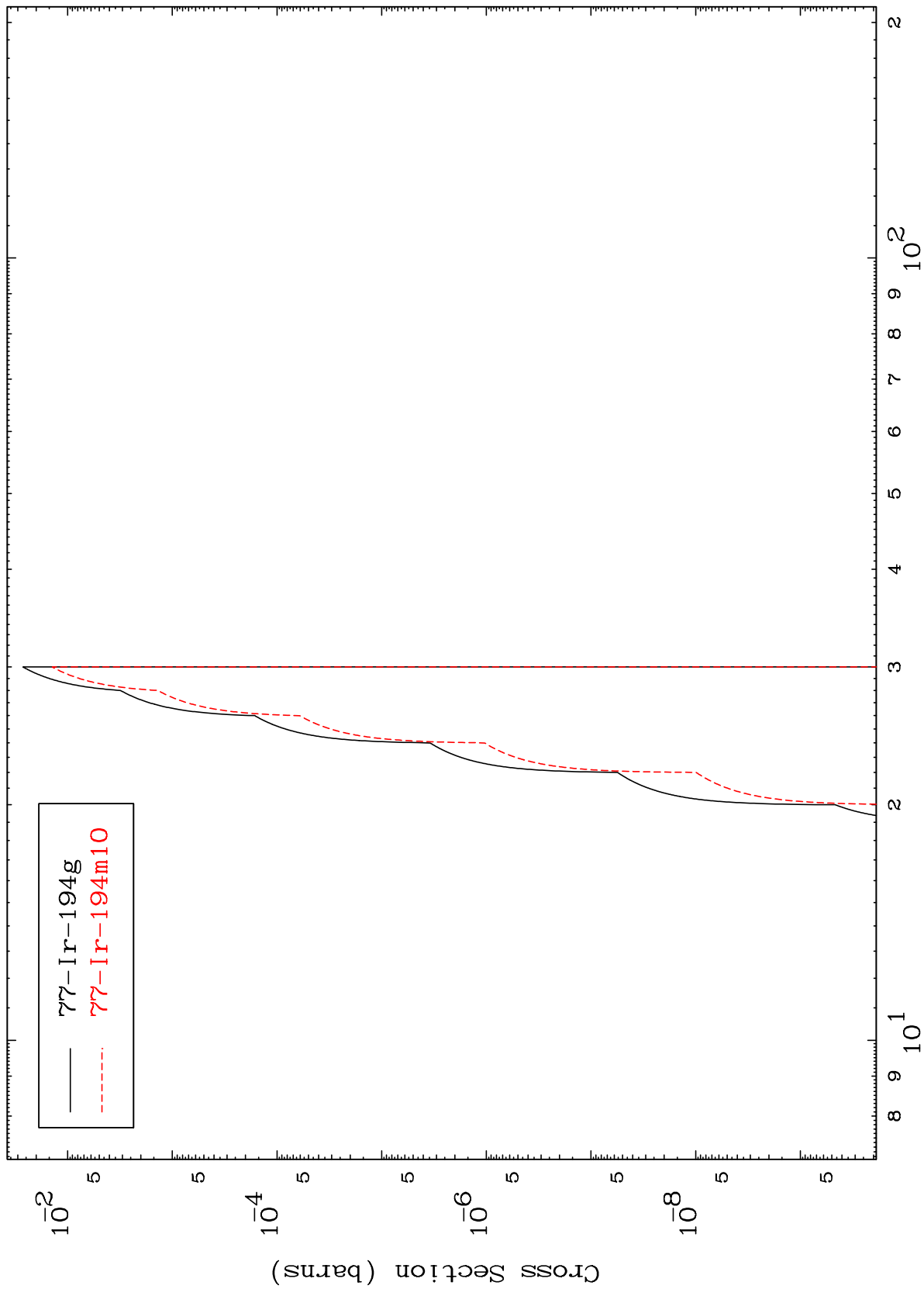
77-Ir-195

MAT 7737

<sup>77</sup>Ir-195

( $\alpha, n'$ )  $\alpha$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

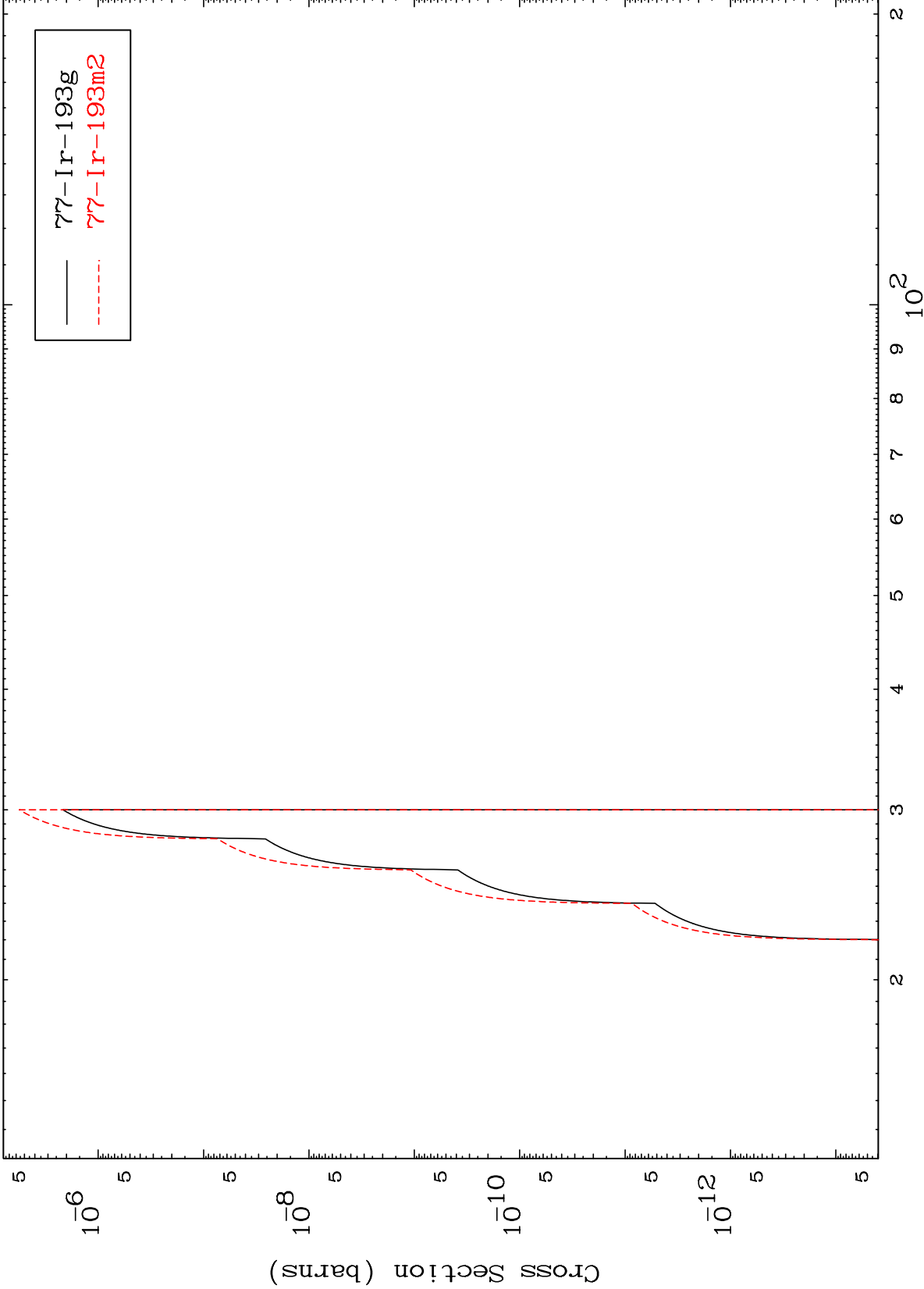
<sup>77</sup>Ir-195

MAT 7737

$(\alpha, 2n) \alpha$

77-Ir-195

Radionuclide Production Cross Section



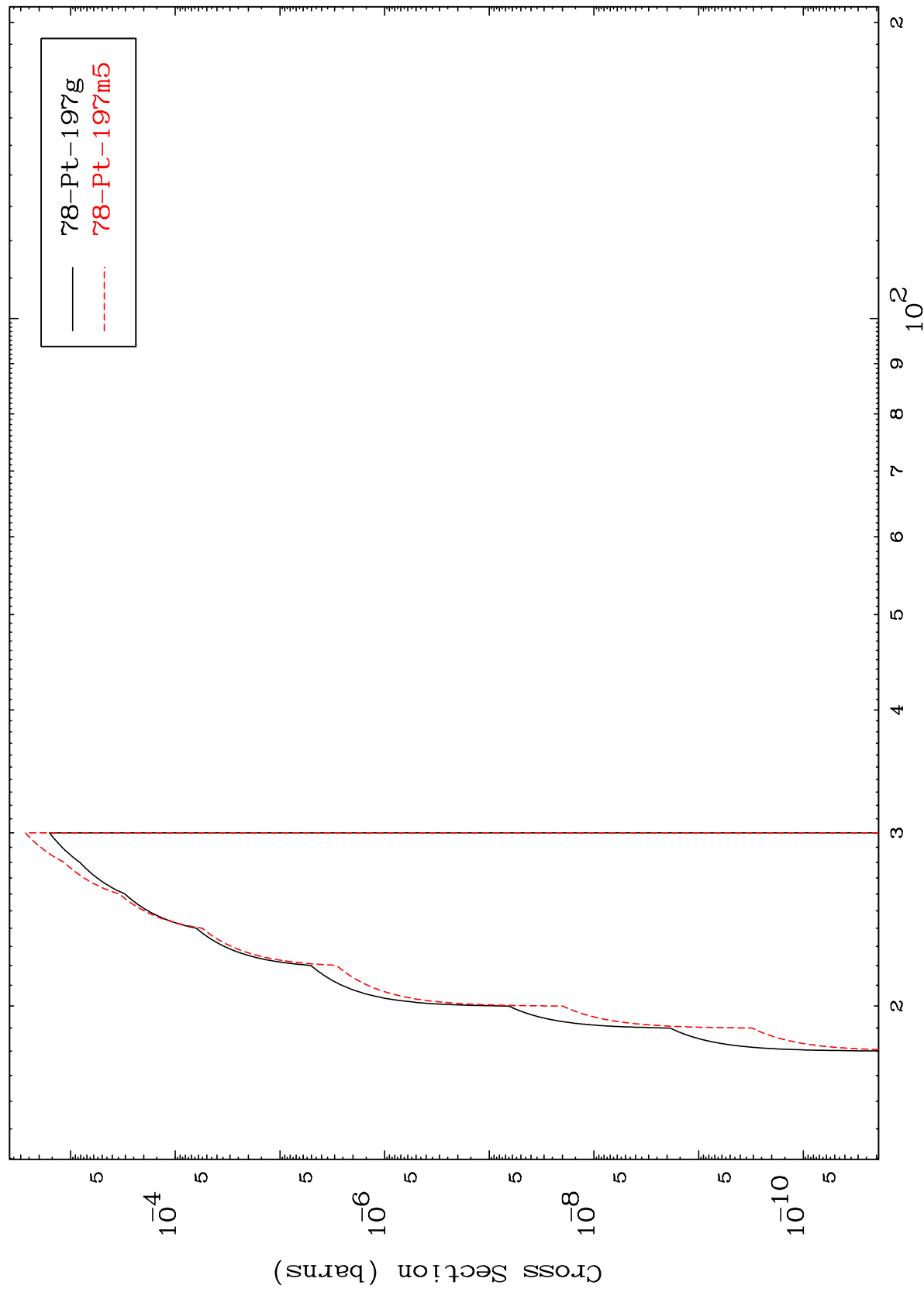
15

Incident Energy (MeV)

77-Ir-195



$(\alpha, n')$  p  
Radionuclide Production Cross Section

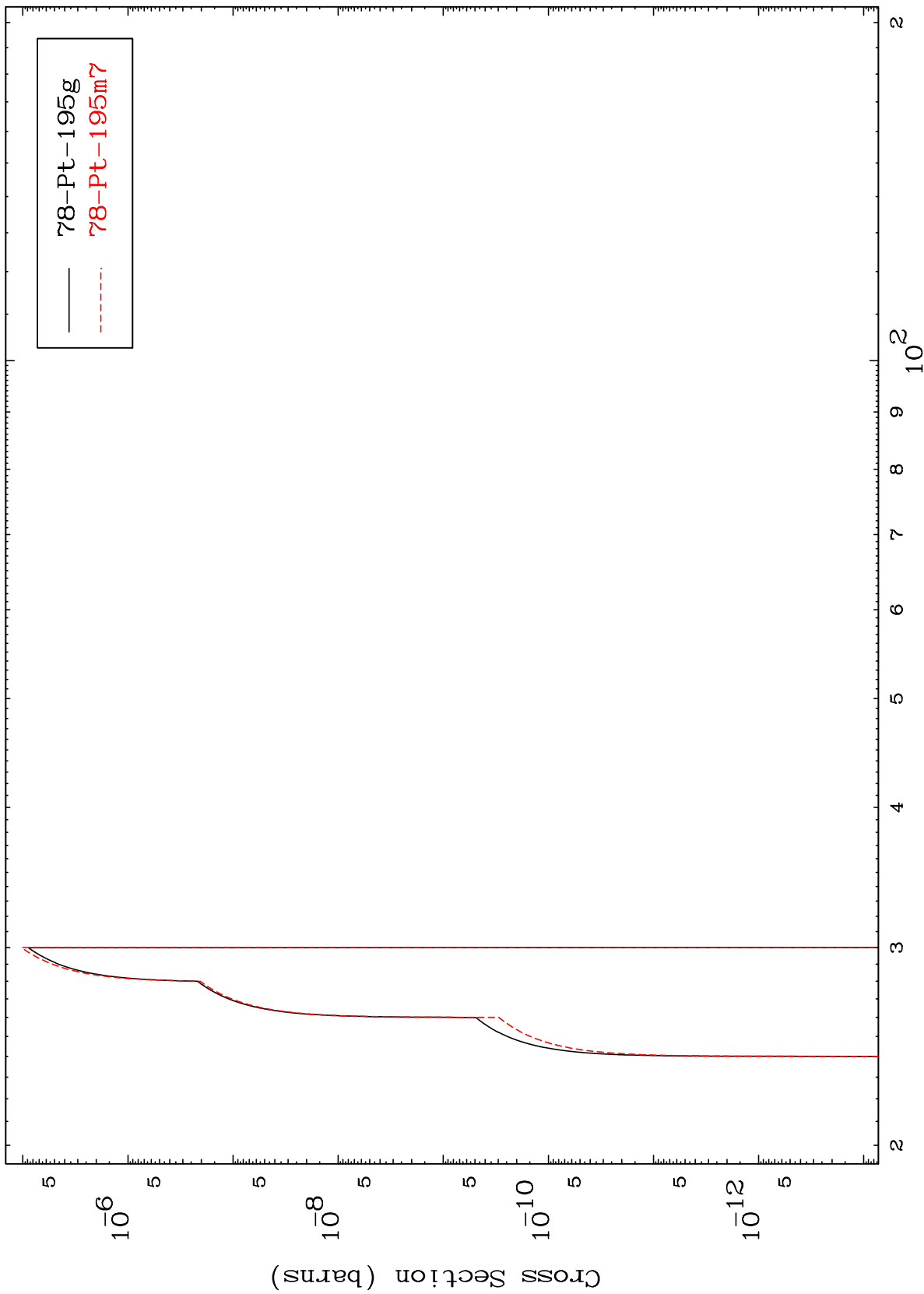


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$(\alpha, n')$  t

77-Ir-195

Radionuclide Production Cross Section



17

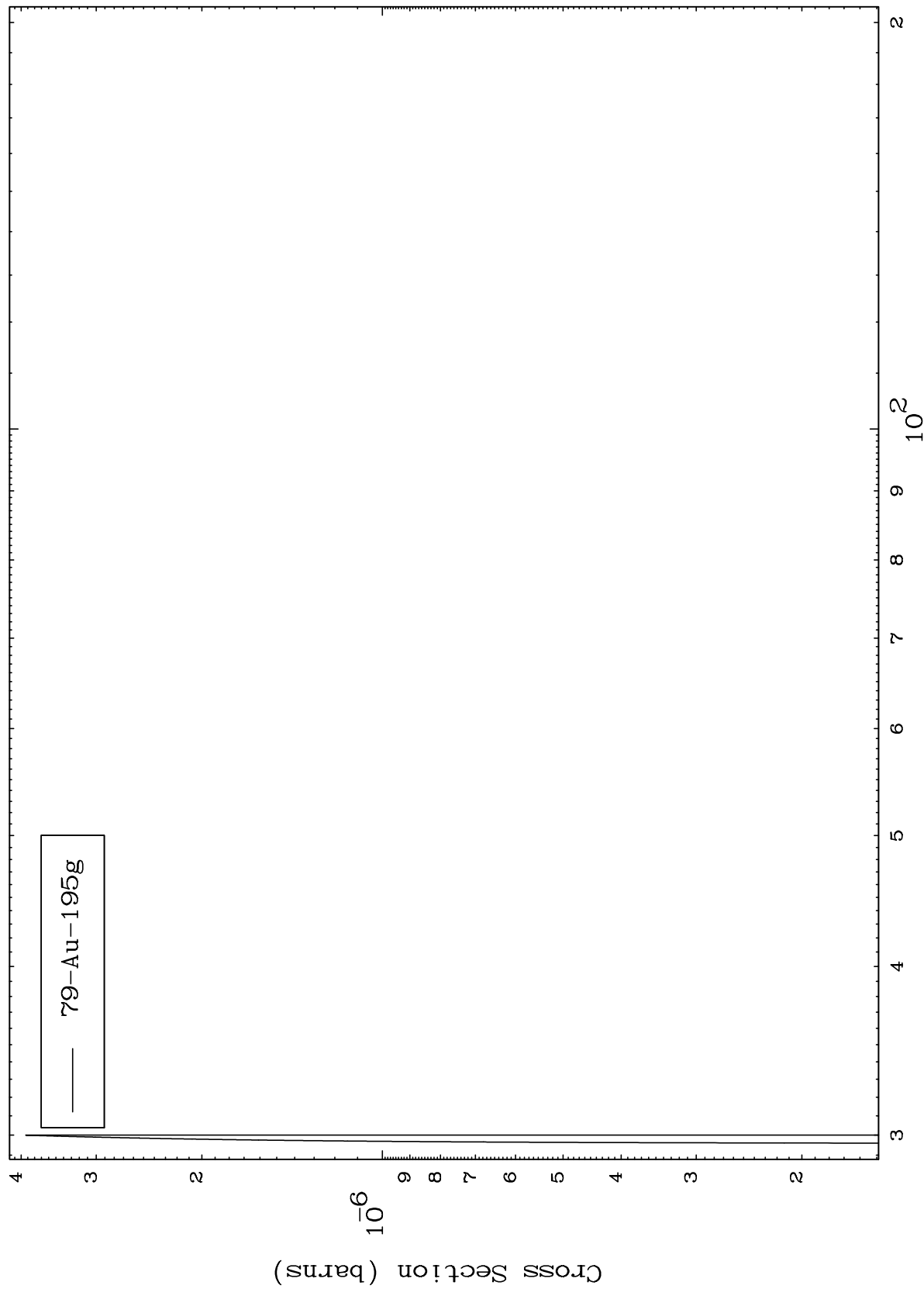
Incident Energy (MeV)

77-Ir-195

MAT 7737

77-Ir-195

( $\alpha, 4n$ )  
Radionuclide Production Cross Section

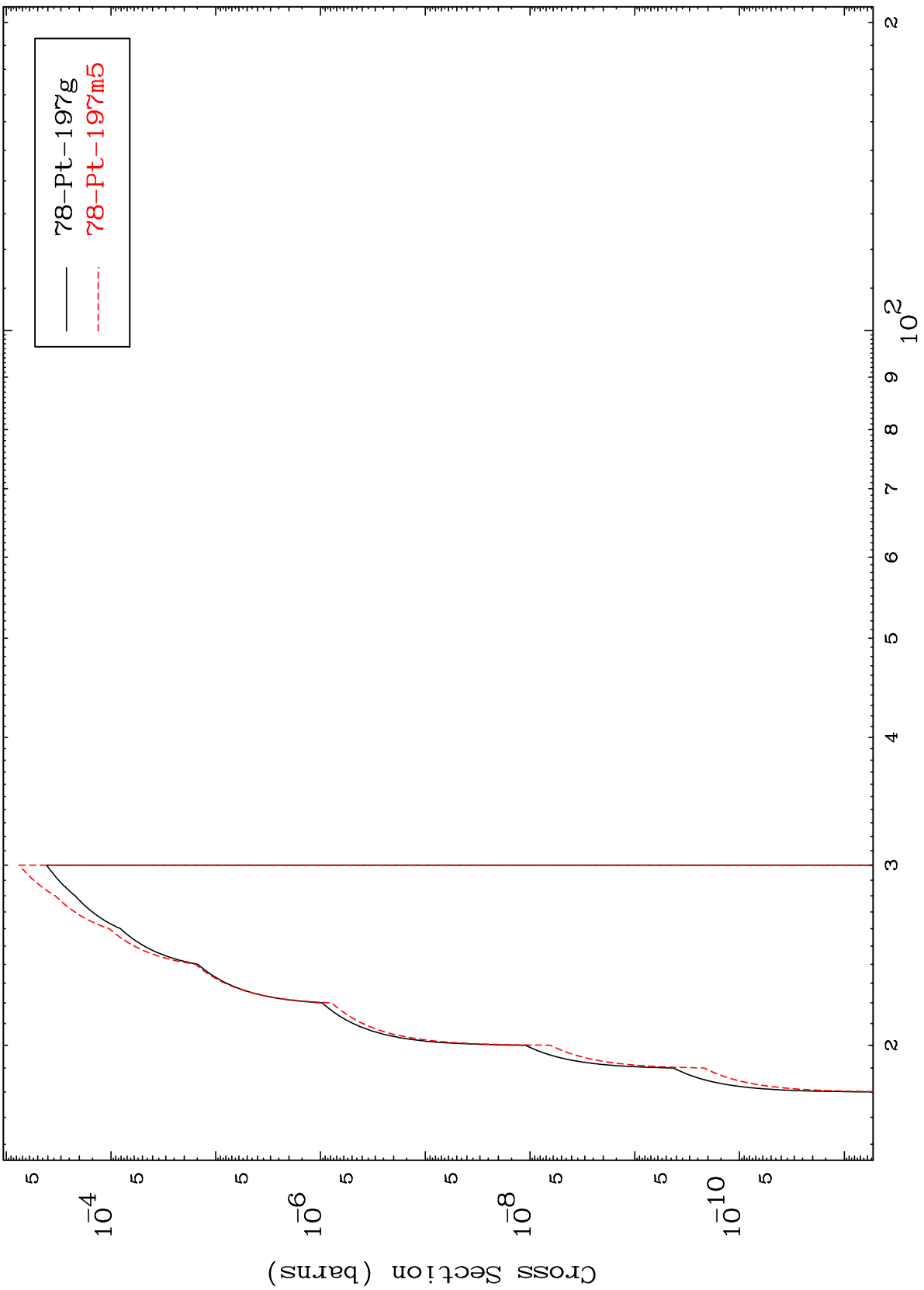


77-Ir-195

Incident Energy (MeV)

18

Radionuclide Production Cross Section

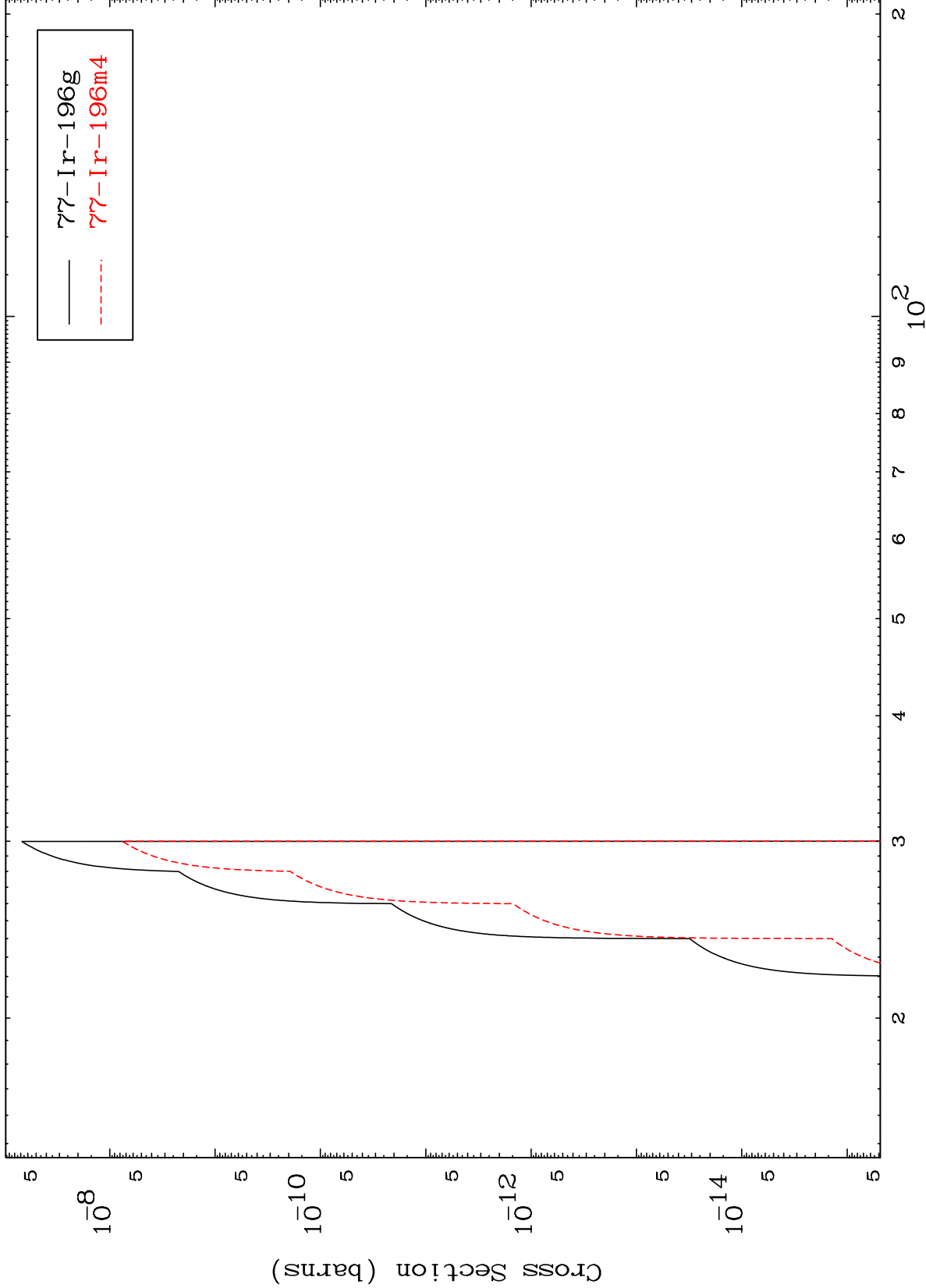


MAT 7737

( $\alpha, \text{He-3}$ )

77-Ir-195

Radionuclide Production Cross Section



77-Ir-196g  
77-Ir-196m4

20

Incident Energy (MeV)

77-Ir-195

MAT 7737

( $\alpha, \alpha$ )

<sup>77</sup>Ir-195

Radionuclide Production Cross Section

